

# NQA-1 Based QA Program Conversion

Duke Energy & NRC Meeting July 15, 2008



### Agenda

- Introductions
- Meeting Purpose
- Conversion Reasons
- Conversion Methodology
- Submittal Package
- Schedule
- Closing Remarks



### **Conversion Team**

- Jim Fisicaro NRC Interface
- Bob Gill Licensing Manager
- Jerry Standridge QA SPOC
- Kevin Rhyne Team Lead
- 40 Contributors QA Criteria SMEs.
- Management Review Team



### Meeting Purpose

- Communicate Duke's plan to convert the operating plant fleet to an ASME NQA-1-1994 based QA program
- Share QA program conversion methodology
- Obtain NRC feedback prior on QAPD package completion and submittal



### **Conversion Reasons**

- Adopt a more current standard, incorporating years of industry operating experience
- Use a single, comprehensive, and actively maintained standard
- Benefit from a simpler, easier to understand standard
- Synchronize with the currently approved standard for new plant construction and operation
- Obtain consistency with the QAPD for new plant licensing (e.g. Lee Nuclear)
- Conform with the utility conversion trend



## Conversion Methodology

- Meet 10CFR50, Appendix B requirements for a QA Program Description (QAPD)
- Use NQA-1 standard and NEI 06-14A template as proposed QAPD basis to establish an equivalent QA program
- Address differences between current QATR and proposed QAPD documents



- NEI 06-14A QAPD revisions 4 & 5 used as a template for the operating fleet program
- Duke specifics replaced bracketed template text with additional information inserted as needed to fully describe the program
- Comparison matrix developed to show where current QATR commitments are addressed within proposed QAPD



	Quality Assurance	Program Description	Comparison Matrix	
Column 1	Column 2	Column 3	Column 4	Column 5
Current QATR Sections/Paragraphs (Note 1)	Proposed QAPD and NQA-1 Sections/Paragraphs (Note 2)	Differences (Note 3)	<b>R/N/I</b> (Note 4)	Basis for Acceptability (Note 5)
17.3.2.12 Inspection	Section 10 – Inspection	The paragraph title was changed to a section title.	N .	
In order to assure safe and reliable operation, a program of inspections for QA Condition 1 structures, systems, and components is established at each nuclear station. Inspection procedures for those activities affecting QA Condition 1 structures, systems and components are established by Nuclear Generation personnel.	Part II, Section 10 Supplement 10S-1	N/A	N	N/A



#### **Comparison Matrix Format and Information Description**

Notes 1 - 5 describe information found in columns 1-5, respectively.

- Column 1 Provides verbatim text from the current QA Topical Report that is displayed in a paragraph-by-paragraph format (headings, list items too).
- Column 2 References locations where <u>equivalent</u> requirement statements are found within the proposed QAPD, NQA-1, or other specifically committed standards and regulatory documents.
- Column 3 Describes significant differences between the requirement statements found in the current and proposed QAPDs, clarifying statements, and references to notes.
- Column 4 Notes one of three requirement statement evaluation result designations: R =
   Reduction in Commitment, N = No reduction in Commitment, and I = Increase in Commitment.
- Column 5 Provides clarifying or justifying statements for commitment changes (e.g., reductions, exceptions, alternatives).



#### **Comparison Matrix Format and Information Description**

The following evaluation notes are typically referenced within Column 3 to minimize redundant wording entry.

- Note 6 Descriptive requirement wording within the proposed QAPD, NQA-1 standard, or a regulatory document establishes <u>equivalent</u> requirements even though there may be a reduction in the level of methodology detail from the current QATR.
- Note 7 A requirement or commitment within the current QATR is <u>redundant</u> to one found in a regulation or other licensing document, therefore, the requirement is not repeated in the proposed QAPD.
- Note 8 A statement within the current QATR that provides only descriptive or clarifying information about the QA program is not considered a commitment to a regulatory requirement and is not repeated in the proposed QAPD.



#### **QAPD Differences Evaluation Process**

- 1. Benchmark other utility submittals
- 2. Initial evaluation by Team Lead
- 3. Validation by Subject Matter Experts
- 4. QA SPOC 10CFR50.54 review
- 5. Management review



#### **QAPD Differences Evaluation Criteria**

#### 1. No Reduction in Commitment:

- a. Statements found within the proposed QAPD, NQA-1 standard, or other regulatory document establishes equivalent requirements and commitments.
- b. Detailed statements on <u>how</u> requirements are met may not be carried over into the proposed QAPD, provided the related requirements are addressed.
- c. A commitment to a requirement within the current QATR is redundant to one already made to a regulation within another licensing document.
- d. A statement that provides only descriptive or clarifying information about the QA program may not be carried over.
- e. Document format changes.



#### **QAPD Differences Evaluation Criteria** (continued)

#### 2. Increase in Commitment:

 Commitment statements made within the proposed QAPD are beyond those found in the current QATR.

#### 3. Reduction in Commitment:

 Commitment statements made within the current QATR are deliberately not included within the proposed QAPD.



### Preliminary Evaluation Results

#### N = No Reduction in Commitment

(100% of preliminary paragraph comparison results fall into this category.)

#### I = Increase in Commitment

(0% of preliminary paragraph comparison results fall into this category.)

#### R = Reduction in Commitment

(0% of preliminary paragraph comparison results fall into this category.)



Current QATR Sections/Paragraphs (Note 1)	Proposed QAPD and NQA-1 Sections/Paragraphs (Note 2)	Differences (Note 3)	<b>R/N/I</b> (Note 4)	Basis for Acceptability (Note 5)
17.3.2.8 Test Control	Section 11 – Test Control	The paragraph title was changed to a section title.	N	
a) Requirements and acceptance limits contained in applicable design and vendor documents.	QAPD Part II, Section 11  NQA-1 Supplement 11S-1	None .	N	N/A
	Supplement 11S-1 excerpt provided for this example:			
	"Test requirements and acceptance criteria shall be provided or approved by the organization responsible for the design of the item to be tested Test requirements and acceptance criteria shall be based upon specified			
	requirements contained in applicable design and other pertinent technical documents."			-



Current QATR Sections/Paragraphs (Note 1)	Proposed QAPD and NQA-1 Sections/Paragraphs (Note 2)	Differences (Note 3)	<b>R/N/I</b> (Note 4)	Basis for Acceptabili (Note 5)
17.3.2.2 Design Control	Section 3 – Design Control	The paragraph title was changed to a section title.	N	
Abbreviated QAPD excerpt provided for this example:  'Final approval prior to implementation of each station engineering change shall be by the Nuclear Station Manager or the Manager of Engineering; or for the Nuclear Station Manager by the Operations Superintendent, the Maintenance Superintendent, the Work Control Superintendent, or the On-Duty Emergency Coordinator as previously designated by the Nuclear Station Manager."	QAPD Part II, Section 1 NQA-1 Basic Requirement 1  QAPD Part II, Section 2  QAPD Part II, Section 3 NQA-1 Basic Requirement 3  QAPD Part II, Section 5 NQA-1 Basic Requirement 5	Requirements for establishing QA program responsibilities, control, and procedural prescription and performance of activities affecting quality are found within the proposed QAPD and NQA-1 standard.  However, no statements are included within the proposed QAPD that specify detailed administrative statements on how final approval of engineering changes are made. Such details are provided within QA program implementation directives and procedures.	N	N/A



Current QATR Sections/Paragraphs	Proposed QAPD and NQA-1 Sections/Paragraphs	Differences (Note 3)	<b>R/N/I</b> (Note 4)	Basis for Acceptability (Note 5)
(Note 1)	(Note 2)		- •	
17.3.2.12 Inspection	Section 10 – Inspection	The paragraph title was changed to a section title.	N	
For inspections of concrete containments, personnel culfilling the role of Responsible Engineer, shall be a Registered Professional Engineer experienced in evaluating the enservice condition of structural concrete and knowledgeable of the design and construction codes and other criteria used in the design and construction of the concrete containment estructure. The Responsible Engineer may also perform inspections as discussed in this section.	QAPD Part II, Section 10 NQA-1 Basic Requirement 10	Note 7  No detailed statements are included within the proposed QAPD that specify Responsible Engineer qualifications and responsibilities.  The Responsible Engineer qualifications and responsibilities are described in the ASME Boiler and Pressure Vessel Code, Section XI, Subsection IWL, Requirements for Class CC Concrete Components of Light-water Cooled Plants, paragraph IWL-2320, Responsible Engineer. Section XI of the ASME Code is endorsed by the NRC via reference within 10CFR50.55a(b)(2).	N	N/A



Current QATR Sections/Paragraphs (Note 1)	Proposed QAPD and NQA-1 Sections/Paragraphs (Note 2)	Differences (Note 3)	<b>R/N/I</b> (Note 4)	Basis for Acceptability (Note 5)
The Group Executive, Nuclear Generation and Chief Nuclear Officer, appoints a Nuclear Safety Review Board (NSRB) to serve as a nuclear safety review and audit backup to the normal operating organization.	QAPD Part II, Section 2	Note 6  No statements are included within the proposed QAPD that specify the appointment or use of a Nuclear Safety Review Board (NSRB).  Rather, Duke commits to Option 1 - Independent Review Body (IRB) as described in Part II, Section 2 of the NEI 06-14A QAPD template. Standard Review Plan 17.5 references approval of this option via SE (Accession No. ML050210276). Approval of the NEI template was via SE (Project No. 689, TAC No. MD3406, dated April 25, 2007).	N	N/A



# Proposed Submittal Package

#### Submittal Pursuant to 10CFR50.54

- Duke QAPD Conversion Submittal Letter
- Enclosure 1 Marked Template Change Copy of Proposed QAPD
- Enclosure 2 Final Unmarked Copy of Proposed QAPD
- Enclosure 3 Old Versus New QAPD Comparison Matrix
- Enclosure 4 Summary of Comparison Results
- Enclosure 5 Summary of Proposed QAPD Alternatives/Exceptions



### **Proposed Schedule**

- Duke Submittal October 2008
- NRC Approval Spring 2009?
- Implementation 6 to 9 Months After Approval



### Summary

## Closing Remarks

M. Wong

- 2 -

PARTICIPANTS:

- ALL PRE-REGISTERED Participants from the NRC include members of the Office of Nuclear

Reactor Regulation (NRR)

NRR

→ D. Thatcher P. Prescott

C. Roquecruz L. Olshan

J\_Stand-- VIC. FIALL

DUKE ✓J. Fisicaro

√R. Gill

K. Rhyne B. Travis

- VI. Standridge -JBRRP

Docket Nos. 50-269, 50-270, 50-287, 50-369, 50-370, 50-413, and 50-414

Enclosure: Agenda

cc w/encl: See next page

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### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

.

July 1, 2008

MEMORANDUM TO: Melanie C. Wong, Branch Chief

Plant Licensing Branch II-1

Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

FROM:

Leonard N. Olshan, Project Manager

Plant Licensing Branch II-1

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

SUBJECT:

FORTHCOMING MEETING WITH DUKE ENERGY CAROLINAS, LLC, REGARDING OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, MCGUIRE NUCLEAR STATION, UNITS 1 AND 2, AND CATAWBA

NUCLEAR STATION, UNITS 1 AND 2

DATE & TIME:

Tuesday, July 15, 2008

11:00 a.m. – 1:00 p.m.

LOCATION:

U.S. Nuclear Regulatory Commission (NRC)

One White Flint North

11555 Rockville Pike, Room O-1F22

Rockville, Maryland

**PURPOSE:** 

To discuss Duke's transition to American Society for Mechanical

Engineers NQA [Nuclear Quality Assurance]-1-1994

CATEGORY 1: \*

This is a Category 1 Meeting. The public is invited to observe this

meeting and will have one or more opportunities to communicate with the NRC after the business portion, but before the meeting is adjourned.

MEETING CONTACTS:

Leonard N. Olshan, NRR

John F. Stang, NRR

301-415-1419

301-415-1345

Ino@nrc.gov

ifs2@nrc.gov

<sup>\*</sup> Commission's Policy Statement on "Enhancing Public Participation in NRC Meetings" (67 FR 36920), May 28, 2002

PARTICIPANTS:

Participants from the NRC include members of the Office of Nuclear Reactor Regulation (NRR)

<u>NRR</u>	DUKE
D. Thatcher	J. Fisicaro
P. Prescott	R. Gill
C. Roquecruz	K. Rhyne
L. Olshan	B. Travis
J. Stand	

Docket Nos. 50-269, 50-270, 50-287, 50-369, 50-370, 50-413, and 50-414

Enclosure: Agenda

cc w/encl: See next page

### FORTHCOMING MEETING ON JULY 15, 2008, TO DISCUSS DUKE ENERGY CAROLINAS, LLC (DUKE) TRANSITION TO ASME NQA-1-1994

#### JULY 15, 2008

#### **AGENDA**

11:00 a.m. – 11:10 a.m. Introductions and purpose of meeting
 11:10 a.m. – 11:55 a.m. Discussion of transition from Duke's current quality assurance program (QA) program to American Society of Mechanical Engineers NQA [Nuclear QA]-1-1994
 11:55 a.m. – 12:50 p.m. Discuss comparison of NQA-1-1994 to Nuclear Energy Institute QA Topical Report
 12:50 p.m. – 1:00 p.m. Closing remarks
 1:00 p.m. Adjourn

Oconee Nuclear Station, Units 1, 2, and 3

cc:

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#### McGuire Nuclear Station, Units 1 & 2

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Charlotte, NC 28202

#### **Leonard Olshan**

From:

Paul Prescott

Sent:

Friday, June 27, 2008 10:28 AM

To:

Leonard Olshan

Subject:

RE:

NQA - Nuclear Quality Assurance

QATR - Quality Assurance Topical Report

From: Leonard Olshan

Sent: Friday, June 27, 2008 10:27 AM

**To:** Paul Prescott **Subject:** RE:

Whoops, there is one question. What do "NQA" and "QATR" stand for? (I'll guess that NQA is nuclear quality assurance.)

From: Paul Prescott

Sent: Friday, June 27, 2008 10:18 AM

To: Leonard Olshan

Cc: Carla Roquecruz; Dale Thatcher

**Subject:** 

Hi Lenny,

Just talked to Jim Fisicaro again. He had nothing planned for an agenda, so how's this: "The meeting is to discuss Duke's transition from their current quality assurance program to ASME NQA-1-1994 and format of this program to the NEI QATR template that was developed for new reactors." NRC will do introduction. Duke will provide discussion. We will close with any questions.

Just let me know if you need anything else.

Paul Prescott Senior Operations Engineer NRR/DE/EQVB (301)415-3026 MS: 0-9E3

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