



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS  
WASHINGTON, DC 20555 - 0001

July 1, 2008

MEMORANDUM TO: Sherry Meador, Technical Secretary  
Advisory Committee on Reactor Safeguards

FROM: Cayetano Santos, Chief */RA/*  
Reactor Safety Branch  
Advisory Committee on Reactor Safeguards

SUBJECT: MINUTES OF THE 553rd MEETING OF THE ADVISORY  
COMMITTEE ON REACTOR SAFEGUARDS (ACRS),  
June 4-6, 2008

I certify that based on my review of the minutes from the 553rd ACRS Full Committee meeting, and to the best of my knowledge and belief, I have observed no substantive errors or omissions in the record of this proceeding subject to the comments noted below.

OFFICE	ACRS	ACRS:RSB
NAME	SMeador	CSantos/sam
DATE	05/ 22 /08	05/22/08

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Date Issued: 7/1/2008  
Date Certified: 7/16/2008

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Minutes of the 553<sup>rd</sup> ACRS Meeting  
June 4-6, 2008

During its 553<sup>rd</sup> meeting, June 4-6, 2008, the Advisory Committee on Reactor Safeguards (ACRS) discussed several matters and completed the following report and memoranda.

REPORT

Report to Dale E. Klein, Chairman, NRC, from William J. Shack, Chairman, ACRS:

- ARTIST Test Program, dated June 13, 2008

MEMORANDA

Memoranda to R. W. Borchardt, Executive Director for Operations, NRC, from Frank P. Gillespie, Executive Director, ACRS:

- Draft Final Revision to 10 CFR 50.55a, "Codes and Standards," dated June 9, 2008
- Draft Final Regulatory Guide 4.21, "Minimization of Contamination and Radioactive Waste Generation: Life-Cycle Planning," dated June 9, 2008
- Withdrawal of Regulatory Guide 1.139, "Guidance for Residual Heat Removal," dated June 9, 2008
- Draft Regulatory Guides 1186, 4013, and 3034, dated June 9, 2008

MINUTES OF THE 552<sup>nd</sup> MEETING OF THE  
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS  
May 8-10, 2008  
ROCKVILLE, MARYLAND

The 553<sup>rd</sup> meeting of the Advisory Committee on Reactor Safeguards (ACRS) was held in Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on June 4-6, 2008. Notice of this meeting was published in the *Federal Register* on May 20, 2008 (72 FR 29169-29170) (Appendix I). The purpose of this meeting was to discuss and take appropriate action on the items listed in the meeting schedule and outline (Appendix II). The meeting was open to public attendance.

A transcript of selected portions of the meeting is available in the NRC's Public Document Room at One White Flint North, Room 1F-19, 11555 Rockville Pike, Rockville, Maryland. Copies of the transcript are available for purchase from Neal R. Gross and Co., Inc., 1323 Rhode Island Avenue, NW, Washington, DC 20005. Transcripts are also available at no cost to download from, or review on, the Internet at <http://www.nrc.gov/ACRS/ACNW>.

#### ATTENDEES

ACRS Members: Dr. William J. Shack (Chairman), Dr. Mario V. Bonaca (Vice-Chairman), Dr. Said Abdel-Khalik (Member-at-Large), Dr. Joseph Armijo, Dr. Dennis Bley, Dr. George E. Apostolakis, Mr. Charles Brown, Dr. Michael Corradini, Mr. Otto L. Maynard, Dr. Dana A. Powers, Mr. John Sieber, and Mr. John Stetkar. Mr. Charles Brown was unable to attend this meeting. For a list of other attendees, see Appendix III.

#### I. Chairman's Report (Open)

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

Dr. William J. Shack, Committee Chairman, convened the meeting at 8:30 a.m. In his opening remarks he announced that the meeting was being conducted in accordance with the provisions of the Federal Advisory Committee Act. He reviewed the agenda items for discussion and noted that no written comments or requests for time to make oral statements from members of the public had been received. Dr. Shack also noted that a transcript of the open portions of the meeting was being kept and speakers were requested to identify themselves and speak with clarity and volume. Various administrative announcements were made.

#### II. ARTIST Test Program

[Note: Mr. David Bessette was the Designated Federal Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff to review the results of the tests conducted by Paul Scherrer Institut (PSI) at the Aerosol Trapping in a Steam Generator Test (ARTIST) facility in Switzerland. The objective was to investigate the retention of aerosols representative of those produced during several core damage accidents as they pass through a

ruptured steam generator tube and transported through the secondary side of the steam generator. The ARTIST test program was sponsored by an international consortium, including NRC. This was a four-year program and completed testing in 2007. During this four-year program, a number of separate effects tests as well as large-scale integral tests were performed. It was found that there was limited deposition of aerosols in the steam generators. The staff plans to use the test data obtained from the ARTIST test program to refine the MELCOR code. The Committee issued a report to the NRC Chairman on this matter dated June 13, 2008, concurring with the staff that the ARTIST test program has provided sufficient experimental data to closeout Item 3.3a, "Development of Experimental Information on Aerosol Source Term Attenuation on the Secondary Side of Steam Generators," of the NRC Steam Generator Action Plan.

### III. Risk Assessment Standardization Project

[Note: Mr. Harold J. VanderMolen was the Designated Federal Officer for this portion of the meeting.]

The Committee met with representatives of the NRC staff to discuss the Risk Assessment Standardization Project (RASP) and related matters. This Project is intended to provide consistent methods between the Accident Sequence Precursor (ASP) calculations, the Significance Determination Process (SDP) Phase 3 programs, and incident investigation programs. Thus, the focus of RASP is to standardize the event assessment programs.

Based on the user need requests, the RASP project was divided into four tasks:

- Task 1:** Develop guides for the analysis of internal events during power operations.
- Task 2:** Develop new methods and guides for the analysis of external events, internal events during low power and shutdown operations
- Task 3:** Enhance the Standardized Plant Analysis Risk (SPAR) models and SAPHIRE/GEM code package
- Task 4:** Provide on-going technical support

The RASP handbook was issued in January 2008. This handbook provides guidelines for performing internal and external event analysis to address the first two tasks. To address Task 3, the SPAR models are under active development. There are now internal events models for essentially all operating plants. In addition, there are 15 external events models and five shutdown events models. Also, two Level II (LERF) models are under development. Finally, a new version of the SAPHIRE/GEM code package is being developed, including a new user interface and improved features and capabilities. RES continues to provide technical support to address Task 4. The staff briefly discussed the future activities that it plans to perform in this area. This was an information briefing. No Committee action was necessary at this time. The Committee plans to discuss this matter with the staff during future meetings.

### IV. Overview of the U.S. Evolutionary Power Reactor (EPR) Design

[Note: Mr. Derek Widmayer was the Designated Federal Official for this portion of the meeting.]

The Committee was briefed by representatives of the NRC staff and AREVA NP Inc. regarding the design concepts, major safety systems, and components of the U.S. EPR design, as well as the main differences in design between the U.S. EPR and a standard Pressurized Water Reactor (PWR) now in operation in the U.S.

The staff discussed the schedule for reviewing the Design Certification Application for the US EPR, which was accepted for review on March 26, 2008. The target date for completing the Safety Evaluation Report (SER) with Open Items is March 5, 2010. The staff plans to group several chapters of the SER with Open Items and submit to the ACRS for review. The staff also stated that the Reference Combined License Application (RCOLA) for the US EPR at the Calvert Cliffs site in Lusby, MD is under review.

AREVA provided information on the design objectives for the US EPR, the general plant layout, the core design, digital instrumentation and control systems, severe accident mitigation, steam generator tube rupture and small-break loss-of-coolant accident mitigation, and the probabilistic risk assessment prepared for the EPR design. Major design differences between the U.S. EPR and an uprated 4-loop PWR were presented. The major components and systems for safety were discussed, including the radial design of the safeguards buildings, the four trains of safety systems, aircraft impact protection of the reactor and safeguards buildings, and the main safety systems for the primary and secondary sides of the reactor. This was an information briefing. No Committee action was necessary at this time. The ACRS Subcommittee on EPR plans to review parts of the Safety Analysis Report submitted by AREVA and the associated Chapters of NRC staff's SER.

V. Status of the Development of Rules and Regulatory Guidance in the Areas of Safeguards and Security

[Note: Ms. Maitri Banerjee was the Designated Federal Official for this portion of the meeting.]

The Committee was briefed by the staff regarding the status of the ongoing rulemaking and regulatory guidance development activities in the area of Safeguards and Security. The staff was previously informed that consistent with the Commission direction in the October 31, 2003

Staff Requirements Memorandum, the Committee is reviewing primarily four parts of the rule. These are: safety and security interface; cyber security; mitigative strategies for large area fire/explosion and aircraft threat; and aircraft impact assessment. The last subject was not included in the current briefing. The staff had already provided the draft final rulemaking package on aircraft impact assessment to the Committee for review and this part of the rulemaking is scheduled to be submitted to the Commission in September while the other parts will be delivered in July 2008.

The staff discussed the major changes made to the rulemaking package in response to the public comments, and the logic behind restructuring the rule in various parts of the 10 CFR, including moving parts of the rule outside 10 CFR Part 73. The mitigative strategies and response procedure for potential and actual aircraft attacks duplicate what was imposed on the operating reactors via orders, and are now in 10 CFR 50.54, as they will be imposed as license conditions.

Draft regulatory guides have been developed for safety/security interface and protection of digital computers and communication system (cyber security). While these draft guides have been made available to the ACRS, the draft guidance on mitigative strategies will not be

available before July 2008. The staff requested ACRS comments on the draft final rule prior to the Committee's review of the draft guides. ACRS members commented that given the very general wordings in some draft rules, having the draft guidance along with the rule package will help in reviewing the draft final rule. The Chairman of the ACRS Subcommittee on Safeguards and Security noted the difficulty in preparing a report during the July 9-11, 2008 meeting, given the anticipated submittal of the rule package to the ACRS at the end of June.

This was an information briefing. No Committee action was necessary at this time. The Committee plans to review the draft final rulemaking package during the July 9-11, 2008 ACRS meeting.

VI. Status of the Quality Assessment of Selected Research Projects

[Note: Dr. Hossein Nourbakhsh was the Designated Federal Official for this portion of the meeting.]

The Committee discussed the status of the quality assessment of the research projects selected for FY 2008. The Committee agreed that the panel review of research project on FRAPCON/FRAPTRAN code work at the Pacific Northwest National Laboratory (PNNL) should be focused on the revised draft NUREG/CR report entitled, "Assessment of Predictive Bias and the Influence of Manufacturing, Model, and Power Uncertainties in NRC Fuel Performance Code Predictions." The Committee plans to discuss the draft report on quality assessment of the selected research projects during September 4- 6, 2008 ACRS meeting.

VI. Overview of the US-Advanced Pressurized Water Reactor (US-APWR) Design

[Note: Mr. David Bessette was the Designated Federal Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff and Mitsubishi Heavy Industries, LTD (MHI) to discuss the design features and preliminary design certification review schedule for the US-APWR design. MHI submitted the application for US-APWR standard design certification on December 31, 2007. The staff acceptance review was completed and the application was docketed on February 29, 2008. The staff's review of the design certification application is currently under way along with the preparation of the SER with Open Items. The staff's proposed dates for ACRS review of the SER with Open Items and the final SER are June 2010 and August 2011, respectively. Luminant Generation Company, LLC has selected the US-APWR design for proposed new units at the Comanche Peak Nuclear Power Plant site. A COL application is expected to be submitted in September 2008.

Representatives of MHI provided an overview of the US-APWR design features. The US-APWR design is similar to the Japanese APWR that is currently undergoing licensing review in Japan. The MHI presentation included information about the fuel and core design, details of the system design and safety features, instrumentation and control (I&C) systems and architecture, and a discussion of MHI experience with digital I&C applications and reliability. MHI has developed a simulation facility near Pittsburgh, PA and stated that members of the ACRS were welcome to tour this facility.

This was an information briefing. No Committee action was necessary. During upcoming meetings the Committee will discuss and prioritize which US-APWR reports will be reviewed in detail. The ACRS staff will follow-up with NRC staff to schedule future briefings. The Committee plans to send representatives to visit MHI's Pittsburgh simulation facility later this year.

VII. Status of NRC Staff Activities Associated with the Resolution of Generic Safety Issue (GSI)- 191, "Assessment of Debris Accumulation on Pressurized-Water Reactor (PWR) Sump Performance"

[Note: Mr. David Bessette was the Designated Federal Official for this portion of the meeting.]

The Committee met with the NRC staff to discuss the status of staff activities associated with the resolution of GSI-191. GSI-191 addresses the impact and consequence of debris generated during design basis-LOCA on the capability and performance of the emergency core cooling and containment spray systems in the recirculation mode. The staff discussed the areas that still remain challenging as the licensees work towards completion of the actions related to PWR sump performance.

The staff acknowledged the substantial work done by licensees so far, including the installation of new strainers with larger surface areas and better pressure drop performance under strainer clogging conditions. The licensees changed the buffers and removed insulations in the zone of influence (ZOI) that adversely contribute to the debris fibers and chemical effects. The staff had by large reviewed and accepted or commented on most of the testing protocols that are intended to demonstrate adequate strainer functions under conditions representative of the plant-specific characteristics. In reference to the December 31, 2007 implementation deadline, the staff stated that most licensees requested and were granted extension for completion of certain corrective actions such as downstream effects analyses, integrated head loss testing, and plant modifications.

The staff approved the topical reports related to chemical effects and ex-vessel downstream effects. Although the draft safety evaluation of the in-vessel (core blockage) topical report was issued, the staff and the PWR owners group are currently addressing the concerns raised during the March 19, 2008 ACRS Subcommittee meeting.

The staff discussed some of the still pending challenges in resolution of GSI-191 and projected that the GSI-191 issue will be resolved by 2009. However, the staff also pointed out that the actions implemented so far reduced the risk of strainer clogging significantly. This was an information briefing. No Committee action was necessary at this time. The Committee plans to review the proposed resolution of GSI-191 during future meetings.

VI. Executive Session

[Note: Mr. Frank Gillespie was the Designated Federal Official for this portion of the meeting.]

A. Reconciliation of ACRS Comments and Recommendations/EDO Commitments

- The Committee considered the EDO's response of May 27, 2008 to comments and recommendations included in the May 2, 2008 ACRS report on the, Hope Creek Generating Station Extended Power Uprate Application. The Committee decided that it was satisfied with the EDO's response.



- The Committee considered the EDO's response of May 28, 2008 to comments and recommendations included in the April 29, 2008 ACRS report on the review of Digital Instrumentation and Control Systems Interim Staff Guidance. The Committee decided that it was satisfied with the EDO's response.
- The Committee considered the EDO's response of May 20, 2008 to comments and recommendations included in the April 21, 2008 ACRS report concerning State-of-the-Art Reactor Consequences Analyses (SOARCA) Project. The staff has restated its justifications for the current SOARCA approach. The staff plans to clarify the SOARCA methodology. The staff's intent is not clear. The Committee plans to discuss this matter during future meetings.

B. Report of the Planning and Procedures Subcommittee Meeting

Review of the Member Assignments and Priorities for ACRS Reports and Letters for the June ACRS Meeting

Member assignments and priorities for ACRS reports and letters for the June ACRS meeting were discussed. Reports and letters that would benefit from additional consideration at a future ACRS meeting were also discussed.

Anticipated Workload for ACRS Members

The anticipated workloads for ACRS members through September 2008 were discussed. The objectives were:

- Review the reasons for the scheduling of each activity and the expected work product and to make changes, as appropriate
- Manage the members' workload for these meetings
- Plan and schedule items for ACRS discussion of topical and emerging issues

Visit to the Braidwood Nuclear Plant and Meeting with the Region III Administrator

During its May 2008 meeting, the members decided to visit the Braidwood Nuclear Plant, and meet with the Region III Administrator to discuss items of mutual interest. A proposed schedule is as follows:

- Tuesday, July 22, 2008 – travel to Braidwood
- Wednesday, July 23, 2008 – plant visit
- Thursday, July 24, 2008 – meet with the Regional Administrator

The Committee requested that Maitri Banerjee provide detailed arrangements for the trip and Mr. Sieber propose a list of topics for meeting with the Regional Administrator. Accordingly, arrangements for this trip and proposed topics for meeting with the licensee and Region III Administrator were discussed.

## Staff Requirements Memorandum

In a Staff Requirements Memorandum (SRM) dated May 8, 2008 resulting from the Commission meeting with an industry Panel and the staff regarding the status of new reactor issues, the Commission stated the following:

- The ACRS should advise the staff and the Commission on the adequacy of the design basis long-term core cooling approach for each new reactor design based, as appropriate, on either its review of the design certification or the first license application referencing the reactor design.

## Draft Final Revision to 10 CFR 50.55a, "Codes and Standards"

The staff provided a copy of the draft final revision to 10 CFR 50.55a to the ACRS requesting that the Committee decide whether it wants to hear a briefing on this rule. The current version of this rule reflects incorporation of public comments, as appropriate. In the revision, the staff requires the use of:

- ASME Code Case N-722, "Additional Inspections for PWR Pressure Retaining Welds in Class 1 Pressure Boundary Components Fabricated with Alloy 60/82/182 Materials, Section XI, Division 2."
- ASME Code Case N-729-1, "Alternative Examination Requirements for PWR Reactor Vessel Upper heads with Nozzles Having Pressure-Retaining Partial- Penetration Welds, Section XI, Division 1."

Based on his review of the draft final revision of this rule, Dr. Shack recommended that the Committee not hear a briefing on this rule.

## Regulatory Guide (RG) 1.139, "Guidance for Residual Heat Removal"

During its May 2008 meeting, the Committee discussed the staff's proposal to withdraw RG 1.139. The Committee deferred action pending receipt of additional information from the staff. In May, John Flack sent information to the members regarding this subject. He also sent a list of questions to the staff. The staff's response to these questions was discussed.

This Guide describes an overly conservative and prescriptive method for complying with the regulations. A draft version of this Guide was issued for public comment in 1975. However, it has never been issued final. Existing plant licensees have developed alternatives, without reliance on this Guide, for complying with the regulations; these alternatives were approved by the staff on a case-by-case basis. Since alternatives, acceptable to the staff, have been developed by the existing plant licensees without relying on this Guide and guidance for the staff reviewers is provided in the SRP, the staff has decided that there is no further use for this Guide.

### Draft Final Regulatory Guide

The staff plans to issue RG 4.21, "Minimization of Contamination and Radioactive Waste Generation: Life-Cycle Planning," as final. RG 4.21 provides guidance to licensees in meeting the requirements of 10 CFR 20.1406, "Minimization of Contamination."

The Advisory Committee on Nuclear Waste & Materials (ACNW&M) was briefed on the draft version of this Guide (DG-4012) at its June 2007 meeting. The ACNW&M wrote a report to the Chairman dated June 28, 2007, with its recommendations to improve the draft Guide. Based on the ACNW&M report, the staff modified DG-4012 before it was issued for public comment, as identified in the Executive Director for Operation's (EDO) August 7, 2007, response.

ACNW&M reviewed the EDO response and the revised DG-4012 and wrote another report to the Chairman dated November 27, 2007, primarily because the Committee thought the modifications made to DG-4012 were insufficient. The staff considered the additional comments, but disagreed with the major changes recommended by the ACNW&M. The EDO replied with a letter dated January 14, 2008. DG-4012 was issued for public comment in July 2007. This final version of RG 4.21 addresses the comments received from the public.

The two ACNW&M recommendations made in its November 27, 2007 report were that (1) DG-4012 should be modified so that it is only applicable to reactors, and (2) additional and better guidance should be issued in a separate guide for other radioactive material licensees. The staff responded to these comments in its resolution of comments document available with the final RG 4.21.

The staff responded that RG 4.21 was useful to other licensees because it included a graded approach for other radioactive material licensees that were favorably reviewed by the State of Washington (an Agreement State that regulates many radioactive materials users). The staff responded that if it revised RG 4.21 to apply only to reactors that other licensees would be left with no guidance. The staff further stated that, despite a great deal of effort to obtain comments on the merits of the approach in this Guide for other licensees, only the State of Washington replied, indicating there was not a great deal of need for improvement of that part of the guidance. The vast majority of the comments on DG-4012 addressed reactor-specific guidance, and most of the revisions in the final RG 4.21 improve this Guide for new reactor licensees.

### Proposed Regulatory Guides

The staff plans to issue the following Regulatory Guides for public comment:

- Proposed Revision 2 to RG 1.21 (DG-1186), "Measuring, Evaluating, and Reporting Radioactive Materials in Liquid and Gaseous Effluents and Solid Wastes"
- Proposed Revision 2 to RG 4.1 (DG-4013), "Environmental Monitoring for Nuclear Power Plants"

On March 10, 2006, the EDO established the Liquid Radioactive Release Lessons Learned Task Force in response to incidents at some nuclear power plants related to unplanned and unmonitored releases of radioactive liquids into the environment. The Task Force issued a final report, "Liquid Radioactive Release Lessons Learned Task Force Final Report," that recommended the revision of effluent and environmental monitoring program requirements and

guidance and the provision of additional guidance on detecting, evaluating, and monitoring unplanned and unmonitored releases of radioactive liquids into the environment.

Proposed Revision 2 to RG 1.21 updates the Guide to describe the improved methods of measuring, evaluating, and reporting radioactivity in solid waste and radioactivity in liquid and gaseous effluents, and incorporates other editorial corrections and revisions to enhance clarity.

Proposed Revision 2 to RG 4.1 updates the Guide to describe the improved methods of environmental monitoring, and incorporates other editorial corrections and revisions to enhance clarity.

- Proposed Revision 1 to RG 3.12 (DG-3034), “General Design Guide for Ventilation Systems of Plutonium Processing and Fuel Fabrication Plants.”

Regulatory Guide 3.12 was first issued in August 1973 to provide guidance for facilities processing plutonium. Since that time, there have been few commercial facility applications for plutonium processing and fuel fabrication. At this time, the NRC is licensing a mixed oxide fuel fabrication facility for use in processing surplus weapons materials, and it is expected that, in the future, additional facilities may be proposed for licensing. These future facilities may be either commercial facilities or facilities licensed through the U.S. Department of Energy’s Global Nuclear Energy Partnership. In 2000, the NRC made significant regulatory changes to 10 CFR Part 70 that require applicants to prepare Integrated Safety Assessments (ISAs), which are systematic evaluations of nuclear facility hazards using risk-informed approaches. In addition, new industry consensus standards are available that update previous guidance reflecting new experiences and state-of-the-art equipment.

Regulatory Guide 3.12 is being revised in its entirety to address changes in Part 70 regarding ISAs.

Proposed ACRS Meeting Dates for CY 2009 – CY 2012

In March 2008, the staff provided the ACRS a description of the Committee’s anticipated workload and a proposed schedule for Subcommittee and full committee meetings. The proposed ACRS meeting dates from CY 2009 through CY 2012 were discussed and summarized below. These meeting dates were provided to the Committee during the May ACRS meeting. Since May, the July 2009 meeting dates have been changed from July 15-17 to July 8-10, 2009.

Meeting Number	Dates	Days
---	January 2009	(No Meeting)
559	February 5-7, 2009	Thursday-Saturday
560	March 5-7, 2009	Thursday-Saturday
561	April 2-4, 2009	Thursday-Saturday
562	May 7-9, 2009	Thursday-Saturday
563	June 3-5, 2009	Wednesday-Friday
564	July 8-10, 2009	Wednesday-Friday
---	August 2009	(No Meeting)
565	September 10-12, 2009	Thursday-Saturday
566	October 8-10, 2009	Thursday-Saturday

567	November 5-7, 2009	Thursday-Saturday
568	December 3-5, 2009	Thursday-Saturday
---	January 2010	(No Meeting)
569	February 4-6, 2010	Thursday-Saturday
570	March 4-6, 2010	Thursday-Saturday
571	April 8-10, 2010	Thursday-Saturday
572	May 6-8, 2010	Thursday-Saturday
573	June 9-11, 2010	Wednesday-Friday
574	July 14-16, 2010	Wednesday-Friday
---	August 2010	(No Meeting)
575	September 9-11, 2010	Thursday-Saturday
576	October 7-9, 2010	Thursday-Saturday
577	November 4-6, 2010	Thursday-Saturday
578	December 2-4, 2010	Thursday-Saturday
---	January 2011	(No Meeting)
579	February 10-12, 2011	Thursday-Saturday
580	March 10-12, 2011	Thursday-Saturday
581	April 7-9, 2011	Thursday-Saturday
582	May 12-14, 2011	Thursday-Saturday
583	June 8-10, 2011	Wednesday-Friday
584	July 13-15, 2011	Wednesday-Friday
---	August 2011	(No Meeting)
585	September 8-10, 2011	Thursday-Saturday
586	October 6-8, 2011	Thursday-Saturday
587	November 3-5, 2011	Thursday-Saturday
588	December 1-3, 2011	Thursday-Saturday
---	January 2012	(No Meeting)
589	February 9-11, 2012	Thursday-Saturday
590	March 8-10, 2012	Thursday-Saturday
591	April 12-14, 2012	Thursday-Saturday
592	May 10-12, 2012	Thursday-Saturday
593	June 6-8, 2012	Wednesday-Friday
594	July 11-13, 2012	Wednesday-Friday
---	August 2012	(No Meeting)
595	September 6-8, 2012	Thursday-Saturday
596	October 4-6, 2012	Thursday-Saturday
597	November 1-3, 2012	Thursday-Saturday
598	December 6-8, 2012	Thursday-Saturday

The proposed dates for Subcommittee meetings would be the following:

- two days before a full committee meeting,
- the second Thursday/Friday after a full committee meeting, or
- the Thursday/Friday during the third week of a month with no full committee meeting.

The meeting was adjourned at 1:00 p.m. on June 6, 2008.