

BellefonteRAIsPEm Resource

From: Ravindra Joshi
Sent: Wednesday, July 16, 2008 4:29 PM
To: BellefonteRAIsPEm Resource
Subject: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 074 RELATED TO SRP SECTION 05.02.04 FOR THE BELLEFONTE UNITS 3 and 4 COMBINED LICENSE APPLICATION
Attachments: BEL-RAI-LTR-074.doc

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Subject: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 074 RELATED TO SRP SECTION 05.02.04 FOR THE BELLEFONTE UNITS 3 and 4 COMBINED LICENSE APPLICATION
Sent Date: 7/16/2008 4:29:10 PM
Received Date: 7/16/2008 4:29:12 PM
From: Ravindra Joshi

Created By: Ravindra.Joshi@nrc.gov

Recipients:
"BellefonteRAIsPEm Resource" <BellefonteRAIsPEm.Resource@nrc.gov>
Tracking Status: None

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Priority: Standard
Return Notification: No
Reply Requested: No
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Expiration Date:
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July 16, 2008

Ms. Andrea L. Sterdis
Manager, Nuclear Licensing & Industry Affairs
Nuclear Generation Development & Construction
Tennessee Valley Authority
1101 Market Street
Chattanooga, Tennessee 37402-2801

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 074 RELATED TO
SRP SECTION 05.02.04 FOR THE BELLEFONTE UNITS 3 and 4 COMBINED
LICENSE APPLICATION

Dear Ms. Sterdis:

By letter dated October 30, 2007, as supplemented by letters dated November 2, 2007, January 8, 2008 and January 14, 2008, Tennessee Valley Authority (TVA) submitted its application to the U. S. Nuclear Regulatory Commission (NRC) for a combined license (COL) for two AP1000 advance passive pressurized water reactors pursuant to 10 CFR Part 52. The NRC staff is performing a detailed review of this application to enable the staff to reach a conclusion on the safety of the proposed application.

The NRC staff has identified that additional information is needed to continue portions of the review. The staff's request for additional information (RAI) is contained in the enclosure to this letter.

To support the review schedule, you are requested to respond within 45 days of the date of this letter. If changes are needed to the final safety analysis report, the staff requests that the RAI response include the proposed wording changes.

If you have any questions or comments concerning this matter, you may contact me at 301-415-6191 or you may contact Joseph Sebrosky, the lead project manager for the Bellefonte combined license at 301-415-1132.

Sincerely,

/RA/

Ravindra G. Joshi, Project Manager
AP1000 Projects Branch 1
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-014
52-015

Enclosure:
Request for Additional Information

If you have any questions or comments concerning this matter, you may contact me at 301-415-6191 or you may contact Joesph Sebrosky, the lead project manager for the Bellefonte combined license at 301-415-1132.

Sincerely,

/RA/

Ravindra G. Joshi, Project Manager
AP1000 Projects Branch 1
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-014
52-015
eRAI Tracking No. 77

Enclosure:
Request for Additional Information

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NRO-002

OFFICE	CIB1/BC	NWE1/PM	OGC	NWE1/L-PM
NAME	DTerao *	RJoshi*	AHodgdon*	JSebrosky*
DATE	5/26/08	6/11/08	7/7/08	7/7/08

*Approval captured electronically in the electronic RAI system.

OFFICIAL RECORD COPY

**Bellefonte Units 3 and 4
Tennessee Valley Authority
Docket No. 52-014 and 52-015**

**SRP Section: 05.02.04- Reactor Coolant Pressure Boundary Inservice Inspection and Testing
Application Section: FSAR 5.2.4**

QUESTION from Component Integrity, Performance and Testing Branch 1 (CIB1)

05.02.04-5

The supplemental information STD COL 5.2-2 added at the end of DCD Subsection 5.2.4.1 discusses Order EA-03-009 and ASME Code Case N-729-1, as amended by the NRC staff, as guidance to the system boundary subject to inspection of the reactor vessel head. Due to CRDM J-Groove weld cracking, the staff believes it is important that the most recent inspection guidance be applied during operation. The current NRC position applicable to inspection guidance for the reactor vessel is presented in the proposed amendments to 10 CFR 50.55a(g)(6)(ii)(D) related to reactor vessel head inspections (72 FR 16740). Discuss TVA's plans for augmenting its inservice inspection program for the reactor vessel top head by implementing ASME Code Case N-729-1 as amended.