

RS-08-088
July 15, 2008

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. STN 50-456 and STN 50-457

Byron Station, Units 1 and 2
Facility Operating License Nos. NPF-37 and NPF-66
NRC Docket Nos. STN 50-454 and STN 50-455

Subject: Notice of Completion of Activities Related to NRC Generic Letter 2004-02,
"Potential Impact of Debris Blockage on Emergency Recirculation During Design
Basis Accidents at Pressurized-Water Reactors"

- References:
- (1) Generic Letter 2004-02, "Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors," dated September 13, 2004
 - (2) Letter from K. R. Jury (Exelon Generation Company, LLC and AmerGen Energy Company, LLC) to U. S. Nuclear Regulatory Commission, "Exelon/AmerGen Response to NRC Generic Letter 2004-02, 'Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors,'" dated March 7, 2005
 - (3) Letter from K. R. Jury (Exelon Generation Company, LLC) to U. S. Nuclear Regulatory Commission, "Supplement to Exelon Response to NRC Generic Letter 2004-02, 'Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors,'" dated May 31, 2006
 - (4) Letter from R. F. Kuntz (U. S. Nuclear Regulatory Commission) to C. M. Crane (Exelon Generation Company, LLC), "Byron Station, Unit No.1 and Braidwood Station, Unit 2 – Requested Extension of Completion Schedule for NRC Generic Letter 2004-02, 'Potential Impact of Debris Blockage on Emergency Recirculation During Design-Basis Accidents at Pressurized-Water Reactors,'" dated July 21, 2006

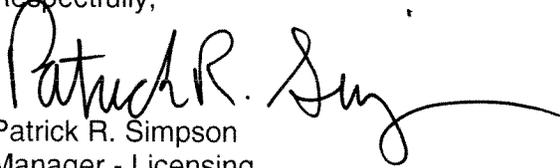
- (5) Letter from P. R. Simpson (Exelon Generation Company, LLC) to U. S. Nuclear Regulatory Commission, "Supplemental Response to NRC Generic Letter 2004-02, 'Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors'" dated December 31, 2007

The purpose of this letter is to provide notification of the completion of activities associated with Generic Letter (GL) 2004-02 (Reference 1). The U. S. Nuclear Regulatory Commission (NRC) issued Reference 1 to request that addressees perform an evaluation of the emergency core cooling system (ECCS) and containment spray system (CSS) recirculation functions in light of the information provided in the GL and, if appropriate, take additional actions to ensure system function. Additionally, the GL requested addressees to provide the NRC with a written response in accordance with 10 CFR 50.54(f). The request was based on identified potential susceptibility of the pressurized water reactor (PWR) recirculation sump screens to debris blockage during design basis accidents requiring recirculation operation of ECCS or CSS and on the potential for additional adverse effects due to debris blockage of flowpaths necessary for ECCS and CSS recirculation and containment drainage.

Reference 2 provided the initial Exelon Generation Company, LLC (EGC) response to the GL. In Reference 3, EGC requested an extension until Spring 2008 to complete the installation and testing of ECCS throttle valves and the potential replacement of CSS Pump Seal Flushing Line Cyclone Separators at Braidwood Station Unit 2 and Byron Station Unit 1. The extension was approved in Reference 4. In Reference 5, EGC provided a response to all outstanding requests for additional information regarding GL 2004-02 compliance at Braidwood and Byron Stations and confirmed that CSS Cyclone Separator replacements would not be required and therefore the ECCS valve modification remained the only outstanding GL 2004-02 activity for 2008. Since that submittal, the ECCS throttle valve installation and testing has been satisfactorily completed for Byron Station Unit 1 in the Spring 2008 refueling outage (B1R15) and also for Braidwood Station Unit 2 in the Spring 2008 refueling outage (A2R13). With installation of these modifications, Braidwood Station Units 1 and 2 and Byron Station Units 1 and 2 have completed all plant changes necessary to assure compliance with the requirements of GL 2004-02.

This letter contains no regulatory commitments. If you have any questions, please contact Ken Nicely at (630) 657-2803.

Respectfully,


Patrick R. Simpson
Manager - Licensing