

July 16, 2008

Dr. Jay F. Kunze
Reactor Administrator
Idaho State University
P.O. Box 8060
Pocatello, ID 83209-8060

SUBJECT: NRC INSPECTION REPORT NO. 50-284/2008-201

Dear Dr. Kunze:

On June 23-26, 2008, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at the Idaho State University AGN-201M Research Reactor Facility. The enclosed report documents the inspection results, which were discussed on June 26, 2008, with you and members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspector reviewed selected procedures and records, observed activities, and interviewed personnel. Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.390 of the NRC's, "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Should you have any questions concerning this inspection, please contact Craig Bassett at 404-358-6515.

Sincerely,

/RA/

Johnny H. Eads, Branch Chief
Research and Test Reactors Branch B
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

Docket No. 50-284
License No. R-110

Enclosure: NRC Inspection Report No. 50-284/2008-201

cc w/enclosure: See next page

Idaho State University

Docket No. 50-284

cc:

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Test, Research and Training
Reactor Newsletter
202 Nuclear Sciences Center
University of Florida
Gainesville, FL 32611

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U. S. NUCLEAR REGULATORY COMMISSION
OFFICE OF NUCLEAR REACTOR REGULATION

Docket No: 50-284

License No: R-110

Report No: 50-284/2008-201

Licensee: Idaho State University

Facility: AGN-201M Research Reactor Facility

Location: Pocatello, Idaho

Dates: June 23-26, 2008

Inspector: Craig Bassett

Approved by: Johnny H. Eads, Jr., Branch Chief
Research and Test Reactors Branch B
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

EXECUTIVE SUMMARY

Idaho State University
AGN-201M Research Reactor Facility
NRC Inspection Report No.: 50-284/2008-201

The primary focus of this routine, announced inspection included onsite review of selected aspects of the licensee's Class II research reactor safety program including: organizational structure and staffing, review and audit and design control functions, procedures, radiation protection, environmental protection, and transportation of radioactive materials since the last NRC inspection of these areas. The licensee's programs were acceptably directed toward the protection of public health and safety and in compliance with NRC requirements.

Organizational Structure and Staffing

- The facility organization and staffing remain in compliance with the requirements specified in the Technical Specifications.

Review and Audit and Design Control Functions

- The review and audit program was being conducted acceptably by the Reactor Safety Committee.
- The last changes that had been completed at the facility were reviewed by the Reactor Safety Committee using the criteria specified in Section 50.59, "Changes, tests, and experiments," of Title 10 of the *Code of Federal Regulations* and were determined to be acceptable.

Procedures

- Facility procedural review, revision, control, and implementation satisfied Technical Specifications requirements.

Radiation Control

- Surveys were being completed and the results documented acceptably.
- Postings met the regulatory requirements.
- Personnel dosimetry was being worn as required and doses were well within NRC regulatory limits.
- Radiation monitoring equipment was being maintained and calibrated as required.
- Acceptable radiation protection training was being provided to staff members.
- The Radiation Protection and As Low As Reasonably Achievable Programs were being acceptably implemented.

Effluent and Environmental Monitoring

- Effluent monitoring satisfied license and regulatory requirements.

- Releases were within the specified regulatory and Technical Specifications limits.

Transportation of Radioactive Materials

- No radioactive material had been shipped from the reactor facility under the NRC license during the past several years.

REPORT DETAILS

Summary of Plant Status

The Idaho State University (ISU) Aerojet General Nucleonics-201M (AGN-201M) Research Reactor Facility, licensed to operate at a maximum steady-state thermal power of 5 Watts, continued to be operated in support of operator training, surveillance, experiments, and laboratory work. During the inspection the reactor was operated for demonstration purposes.

1. Organizational Structure and Staffing

a. Inspection Scope (Inspection Procedure (IP) 69001)

The inspector reviewed the following regarding the licensee's organization and staffing to ensure that the requirements of Technical Specifications (TS) Sections 6.1 and 6.2 were being met:

- Organization and staffing for reactor operation
- Administrative controls and management responsibilities
- ISU AGN-201M Reactor Facility Master Log Numbers (Nos.) 4 and 5
- TS for Idaho State University (ISU) AGN-201M Reactor, Amendment No. 6, dated August 14, 2006
- ISU AGN-201M General Rules, Revision (Rev.) 4, dated September 19, 1994
- ISU AGN-201M Operating Procedures No. 1 (OP-1), Rev. 3, dated April 26, 1994
- ISU AGN-201M Operating Procedures No. 2 (OP-2), Rev. 3, dated April 26, 1994
- ANS 15.4-1988, "Standards for Selection and Training of Personnel for Research Reactors"

b. Observations and Findings

Through discussions with licensee representatives the inspector determined that management responsibilities and the organization at the ISU AGN-201M Research Reactor Facility had not changed since the previous NRC inspection (Inspection Report No. 50-284/2007-201). The inspector determined that the Reactor Administrator retained direct control and overall responsibility for management of the facility as specified in TS Section 6.1. The Reactor Administrator reported to the designated University Officer at ISU who was the University Vice President.

The licensee's current operational organization consisted of the Reactor Administrator, the Reactor Supervisor, and an Engineer. The Reactor Supervisor and Engineer were licensed to operate the reactor and were both Senior Reactor Operators as required by TS Section 6.1. There were no licensed Reactor Operators at the facility at the time of the inspection. The inspector confirmed that the Reactor Administrator and Supervisor also met the qualifications specified in TS Section 6.2. It was noted that students and others were sometimes employed on a part-time basis. This organization was consistent with that specified in Figure 1 of TS Section 6.1.

c. Conclusions

The licensee's staffing and organization met the requirements specified in TS Sections 6.1 and 6.2.

2. Review and Audit and Design Control Functions

a. Inspection Scope (IP 69001)

To verify that the licensee had established and conducted reviews and audits as required in TS Section 6.4 and to verify that modifications to the facility were being reviewed in accordance with the stipulations in 10 CFR 50.59 and reviewed and approved as required by TS Section 6.5, the inspector reviewed:

- ISU Reactor Safety Committee Charter
- Completed audits and reviews since June 2006
- ISU AGN-201M Reactor Facility Master Log Nos. 4 and 5
- Reactor Safety Committee meeting minutes since August 2005
- TS for ISU AGN-201M Reactor, Amendment No. 6, dated August 14, 2006

b. Observations and Findings

(1) Review and Audit Functions

The inspector reviewed the Reactor Safety Committee (RSC) meeting minutes from August 2005 to the present. These meeting minutes showed that, as required by TS Section 6.4.1, the committee met at least once per calendar year and that a quorum was present. The topics considered during the meetings were appropriate and as stipulated in TS Section 6.1.8.

Through review of the RSC minutes the inspector confirmed that the RSC reviewed proposed changes in the license, audit reports, and any reported violations as required by TS Section 6.4.2. The inspector also noted that, since the last NRC inspection, members of the safety committee had completed audits of various aspects of the reactor facility operations, programs, and procedures as required by TS Section 6.4.3. The audits were structured so that the various aspects of the licensee's operations and safety programs were reviewed annually. The inspector noted that the audit findings were acceptable and that the licensee responded and took corrective actions as needed.

(2) Design Change Function

No current changes were being implemented at the facility except for one involving the installation of a new reactor console. However, the 50.59 Review Package for that project had not been finalized as of the date of the inspection so that project was still pending.

Records and observations showed that the changes that had been proposed at the facility in the past were acceptably reviewed in accordance with applicable administrative controls. Two of the past changes involved dashpot modifications and correction of a problem with the Channel No. 1 detector dust cover and travel stop. The licensee determined that the changes did not meet any of the criteria specified in 10 CFR 50.59 (c) (2) Paragraphs (i) – (viii). The changes were then reviewed by the RSC and found to be acceptable. None of the changes

constituted a safety question or required a change to the TS. The change review and approval process appeared to be acceptable.

c. Conclusions

The review and audit program was being conducted acceptably by the RSC. The last changes completed by the licensee were reviewed using the criteria specified in 10 CFR 50.59, reviewed by the RSC, and determined to be acceptable.

3. Procedures

a. Inspection Scope (IP 69001)

The inspector reviewed selected aspects of the following to verify that the licensee was complying with the requirements of TS Sections 6.5 and 6.6:

- Selected forms and checklists
- Selected maintenance and surveillance procedures
- ISU AGN-201M Reactor Facility Master Log Nos. 4 and 5
- Selected Operations Procedures and Radiation Protection Procedures
- ISU AGN-201M General Rules, Rev. 4, dated September 19, 1994
- ISU AGN-201M Surveillance Procedure No. 4 (SP-4) "Shield Tank Water Level Interlock Calibration," Rev. 1, dated June 23, 1998

b. Observations and Findings

The licensee's procedures were found to be acceptable for current facility operations and the current staffing level. It was noted that the procedures specified the responsibilities of the various members of the staff. The inspector determined that the various facility procedures were being updated as needed and that substantive revisions to procedures, checklists, and forms were presented to the RSC for review and approval as required by TS.

The inspector was able to observe reactor operations activities during this inspection. The inspector was also able to observe the completion of two surveillance activities. One involved the biennial inspection of the reactor tank and the other involved the shield tank water level interlock calibration. It was noted that all required checks, verification activities, and radiological surveys were conducted and completed in accordance with the appropriate procedure. Also, the activities were properly documented on the appropriate procedural forms and in facility logs as required. These observations and related records review indicated that training of personnel on procedures and changes was acceptable. It was also noted that procedures for potential malfunctions and emergencies (e.g., radioactive releases, contaminations, and reactor equipment problems) had been developed and were implemented as required.

During the inspection, the licensee indicated that a new procedure had been developed for removing the Channel No.3 detector can. It was noted that the procedure was scheduled to be reviewed during the next RSC meeting. That meeting was scheduled for the week of July 1, 2008.

c. Conclusions

Procedural review, revision, control, and implementation satisfied TS requirements.

4. Radiation Protection

a. Inspection Scope (IP 69001)

The inspector reviewed the following to verify compliance with 10 CFR Parts 19 and 20, and TS Sections 3.4 and 4.4.c:

- Radiological signs and posting
- Personnel dosimetry records for 2006 to present
- ISU Radiation Protection Program and ALARA Policy
- ISU Radiation Safety Policy Manual (RSM), Rev. 3, dated September 2000
- Contamination and radiation survey records for reactor from April 2006 to present
- ISU AGN-201M Reactor Facility Master Log Nos. 4 and 5 documenting radiation surveys and reactor operations
- Records documenting the maintenance and calibration of radiation monitoring equipment from 2006 to present
- Annual Operating Report for the Idaho State University AGN-201M Reactor, License No. R-110; Docket No. 50-284, for calendar years 2005 and 2006
- AGN-201 Operations Log, Form ROL-101, Rev. 3, dated April 26, 1994, for the period from January 20, 2007 to the present documenting Pre-Start checks, surveys, and reactor operations
- TS for ISU AGN-201M Reactor, Amendment No. 6, dated August 14, 2006
- ISU AGN-201M General Rules, Rev. 4, dated September 19, 1994
- ISU AGN-201M Operating Procedures No. 1 (OP-1), Rev. 3, dated April 26, 1994
- ISU AGN-201M Operating Procedures No. 2 (OP-2), Rev. 3, dated April 26, 1994
- ISU AGN-201M Visitor Log and Register for Organized Groups and Tours
- ISU AGN-201M Experimental Plan 8 (EP-8), "Health Physics Survey," Rev. 1, dated May 3, 1979
- ISU Technical Safety Office (TSO) Radiation Safety Procedure Number 1 (RPR 1), "Radiation User Personal Data," Rev. 3, dated June 2000
- ISU TSO RPR 2, "Radiation Use Application," Rev. 3, dated June 2000
- ISU TSO RPR 10, "Radionuclide Data," Rev. 3, dated June 2000
- ISU TSO RPR 11 "Radioisotope Laboratory Safety Procedure," Rev. 3, dated June 2000
- ISU TSO RPR 44, "Radiation Safety Training," Rev. 3, dated June 2000
- ISU TSO RPR 61, "Calibration of the Radiation Monitoring Instruments," Rev. 3, dated June 2000

The inspector also toured the licensee's facility and observed the use of dosimetry and radiation monitoring equipment. Licensee personnel were interviewed as well.

b. Observations and Findings

(1) Surveys

Various periodic and Prestart-Up contamination and radiation surveys were completed by reactor staff and TSO personnel. Through records reviews and interviews with reactor and TSO staff members, the inspector determined that the contamination and radiation surveys were performed as required by TS Section 4.4.c, RSM Sections 6.3 and 7.2, and Radiation Safety procedures. The inspector also verified that results were evaluated and corrective actions taken and documented as required when contamination levels exceeded the established limits. Radiation survey results were used to verify the location of radiation and high radiation areas and ensure 10 CFR 20.1902 postings were accurate.

During the inspection, the inspector conducted a radiation survey of the Reactor Room compared the readings detected with those found by the licensee. With one exception, the results were comparable to those of the licensee. At one location in the Reactor Room the inspector noted a reading of four (4) millirem per hour (mr/hr) while the survey map showed a reading of 183.5 mr/hr. This discrepancy in radiation readings was discussed with licensee personnel. The licensee indicated that the problem was apparently the result of a recording error; i.e., the person who recorded the radiation reading on the survey map recorded the wrong number. This was subsequently corrected. No other discrepancies were noted.

(2) Postings and Notices

During tours of the facility, the inspector observed that caution signs and postings in place and controls established for the controlled areas were acceptable for the hazards involving radiation, high radiation, and contamination and were posted as required by 10 CFR Part 20, Subpart J. Through observations of and interviews with licensee staff, the inspector confirmed that personnel complied with the signs, postings, and controls. The facility's radioactive material storage areas were noted to be properly posted. No unmarked radioactive material was detected in the facility.

Copies of current notices to workers were posted in various areas in the facility. Radiological signs were typically posted at the entrances to controlled areas. Other postings also characterized the industrial hygiene hazards that were present in the areas as well. During one facility tour, the inspector noted that various copies of NRC Form-3, "Notice to Employees," which were posted at the facility as required by 10 CFR Part 19.11, were not the current version. This issue was brought to the attention of the licensee and copies of the correct version were immediately retrieved from the Internet. The copies were then posted on the Bulletin Board by the main entrance to the Reactor Room and at other locations in the facility. With the current version of NRC Form 3 posted, notices, caution signs, postings, and controls for radiation areas were as required in 10 CFR Parts 19 and 20.

(3) Dosimetry

The licensee was supplied dosimetry by the ISU TSO. The TSO used a National Voluntary Laboratory Accreditation Program accredited vendor, Landauer, to process the whole body and extremity thermoluminescent dosimeters (TLDs) supplied to facility personnel. Through direct observation, the inspector determined that dosimetry was acceptably used by facility personnel. Exit frisking practices were also observed and determined to be in accordance with radiation protection requirements.

An examination of the TLD results, indicating exposure to radiation at the facility for the past two years, showed that the highest occupational doses, as well as doses to the public, were within 10 CFR Part 20 limits. The records showed that the highest annual whole body exposure received by a single individual for 2006 was 19 millirem (mr) deep dose equivalent (DDE). The records indicated that no one received a measurable extremity exposure in 2006. The highest annual whole body exposure received by a single person for 2007 was 34 mr DDE. The records again indicated that no one received a measurable extremity exposure in 2007.

The inspector verified that NRC Form-5 reports had been completed and provided to each employee who had received exposure at the facility during 2006 and 2007.

(4) Radiation Monitoring Equipment

Examination of selected survey meters indicated that the instruments had the acceptable up-to-date calibration sticker attached. The instrument calibration records indicated calibration of portable survey meters was typically completed by TSO personnel and/or a contractor. Calibration frequency met the specified TS requirements and records were maintained as required. Area radiation monitors were also being calibrated as required.

During the inspection, the inspector observed the calibration range maintained by the TSO. The calibration range appeared to be adequate. Proper precautions had been established to maintain doses ALARA.

(5) Radiation Protection Program

The licensee's Radiation Protection Program was established in the Idaho State University Radiation Safety Policy Manual, Revision 3, with approval by the Radiation Safety Committee dated September 2000. The program included requirements that all personnel who worked with radioactive materials receive training in radiation protection, policies, procedures, requirements, and facilities. Completion of this training was verified by each person's supervisor and by TSO personnel. The program appeared to be acceptable and was being reviewed annually as required by the Radiation Safety Officer (RSO).

(6) ALARA Program

The ALARA Program was outlined and established in RSM Section 3.5. The ALARA program provided guidance for keeping doses as low as reasonably achievable and was consistent with the guidance in 10 CFR Part 20.

(7) Radiation Protection Training

The inspector reviewed the radiation worker (or rad worker) training given to reactor staff members, to those who were not on staff but who were authorized to use the experimental facilities of the reactor, and to student assistants working at the facility on a part-time basis. The inspector verified that rad worker training was given upon initial employment and refresher training was offered annually thereafter. The inspector reviewed documentation of the training provided to licensee staff members. The documents indicated that all current staff members had received the required training. The inspector determined that the personnel training program satisfied requirements in 10 CFR 19.12.

(8) Facility Tours

The inspector toured the Reactor Room, adjacent laboratories, and support areas. Control of radioactive material and control of access to radiation areas was acceptable. As noted above, the inspector also conducted an independent radiation survey of the Reactor Room and determined that radiation levels recorded on licensee survey maps were generally representative and accurate.

c. Conclusions

The inspector determined that the Radiation Protection and ALARA Programs satisfied regulatory requirements because: 1) surveys were being completed and documented acceptably, 2) postings met regulatory requirements, 3) personnel dosimetry was being worn as required and doses were well within the NRC's regulatory limits, 4) radiation monitoring equipment was being maintained and calibrated as required, and, 5) acceptable radiation protection training was being provided.

5. Effluent and Environmental Monitoring

a. Inspection Scope (IP 69001)

The inspector reviewed the following to verify compliance with the requirements of 10 CFR Part 20 and TS Sections 3.4, 4.2.3, 5.4, 6.6, and 6.7.2:

- Records of selected airborne releases
- Records documenting the maintenance and calibration of radiation monitoring equipment from 2006 to present
- Annual Operating Report for the Idaho State University AGN-201M Reactor, License No. R-110; Docket No. 50-284, for calendar years 2005 and 2006
- AGN-201 Operations Log, Form ROL-101, Rev. 3, dated April 26, 1994, for the period from January 20, 2007 to the present, documenting Pre-Start checks, surveys, and reactor operations

- ISU TSO RPR 54, "Radioactive Waste Management," Rev. 3, dated June 2000

b. Observation and Findings

The inspector reviewed the records documenting airborne releases to the environment for the past two years. The inspector determined that gaseous releases continued to be calculated as required by procedure and were adequately documented. The releases were determined to be within the annual dose constraints of 10 CFR 20.1101 (d), 10 CFR Part 20 Appendix B concentrations, and TS limits. The inspector confirmed that there had been no liquid or solid waste radioactive releases from the reactor facility during the past two years. Through observation of the facility, the inspector found no new potential release paths.

On-site gamma radiation monitoring was completed using various TLDs placed around the facility in accordance with the applicable procedures. The data indicated that there were no measurable doses to the public above any regulatory limits.

c. Conclusions

Effluent monitoring satisfied license and regulatory requirements and releases were within the specified regulatory and TS limits.

6. Transportation

a. Inspection Scope (IP 86740)

The inspector reviewed the following to verify compliance with regulatory and procedural requirements for shipping or transferring licensed material:

- Annual Operating Report for the Idaho State University AGN-201M Reactor, License No. R-110; Docket No. 50-284, for calendar years 2005 and 2006
- AGN-201 Operations Log, Form ROL-101, Rev. 3, dated April 26, 1994, for the period from January 20, 2007 to the present documenting Pre-Start checks, surveys, and reactor operations
- ISU TSO RPR 13, "Radioisotope Acquisition and Disposition," Rev. 3, dated June 2000
- ISU TSO RPR 14, "Shipment of Excepted Quantities of Radioisotopes," Rev. 3, dated June 2000
- ISU TSO RPR 54, "Radioactive Waste Management," Rev. 3, dated June 2000
- ISU TSO RPR 55, "Transportation of Radioactive Materials," Rev. 3, dated June 2000

b. Observations and Findings

Through records review and discussions with licensee personnel, the inspector determined that the licensee had not shipped any radioactive material from the facility under the reactor license. It was noted that radioactive material produced in the reactor was either transferred to the campus broadscope license (ISU Type A Broadscope Radioactive Materials License 11-27380-1) and shipped under the

auspices of that license, or transferred to other authorized users on campus, or maintained at the reactor facility for use in laboratories in accordance with procedure

c. Conclusions

No radioactive material had been shipped from the reactor facility under the NRC license during the past several years.

7. Exit Meeting Summary

The inspection scope and results were summarized on June 26, 2008, with licensee representatives. The inspector discussed the findings for each area reviewed. No dissenting comments were received from the licensee.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

| | |
|------------|--|
| J. Bennion | Reactor Supervisor |
| K. Hart | Senior Reactor Operator and Associate Lecturer/Test Engineer |
| G. Imel | Chair, Nuclear Engineering Department |
| J. Kunze | Reactor Administrator |

Other Personnel

| | |
|-----------|---|
| R. Brey | Radiation Safety Officer and Director, TSO, ISU |
| F. Just | Chair, Reactor Safety Committee |
| D. Mecham | Radiation Safety Technician, TSO, ISU |

INSPECTION PROCEDURES USED

| | |
|----------|---|
| IP 69001 | Class II Research and Test Reactors |
| IP 86740 | Inspection of Transportation Activities |

ITEMS OPENED, CLOSED, AND DISCUSSED

OPENED:

None

CLOSED:

None

LIST OF ACRONYMS USED

| | |
|----------|---|
| ADAMS | Agencywide Documents Access and Management System |
| AGN-201M | Aerojet General Nucleonics-201M |
| ALARA | As Low As Reasonably Achievable |
| CFR | Code of Federal Regulations |
| DDE | Deep dose equivalent |
| EP | Experimental Plan |
| IFI | Inspection Follow-up Item |
| IP | Inspection Procedure |
| ISU | Idaho State University |
| MOU | Memorandum of Understanding |
| mr | millirem |
| mr/hr | millirem per hour |
| NRC | Nuclear Regulatory Commission |
| Rev. | Revision |

| | |
|-----|--------------------------------|
| RPR | Radiation Safety Procedure |
| RS | Reactor Supervisor |
| RSC | Reactor Safety Committee |
| RSM | Radiation Safety Policy Manual |
| TLD | Thermoluminescent dosimeter |
| TS | Technical Specification |
| TSO | Technical Safety Office |