16-5, KONAN 2-CHOME, MINATO-KU TOKYO, JAPAN

June 27, 2008

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

Attention: Mr. Jefrey A. Ciocco,

Docket No. 52-021 MHI Ref: UAP-HF-08108

Subject: MHI's Response to US-APWR DCD RAI No.7

References: 1) "Request for Additional Information No.7 Revision 0, SRP Section: 14.02 -

Initial Plant Test Program - Design Certification and New License

Applicants, Application Section: 14.2," dated May 28, 2008.

With this letter, Mitsubishi Heavy Industries, Ltd. ("MHI") transmits to the U.S. Nuclear Regulatory Commission ("NRC") a document entitled "Response to Request for Additional Information No.7 Revision 0."

Enclosed is the response to 1 RAI that is contained within Reference 1.

Please contact Dr. C. Keith Paulson, Senior Technical Manager, Mitsubishi Nuclear Energy Systems, Inc. if the NRC has questions concerning any aspect of the submittals. His contact information is below.

Sincerely,

Yoshiki Ogata,

General Manager- APWR Promoting Department

Mitsubishi Heavy Industries, LTD.

**Enclosures:** 

1. Response to Request for Additional Information No.7 Revision 0

CC: J. A. Ciocco C. K. Paulson

**Contact Information** 

C. Keith Paulson, Senior Technical Manager Mitsubishi Nuclear Energy Systems, Inc. 300 Oxford Drive, Suite 301 Monroeville, PA 15146 E-mail: ck\_paulson@mnes-us.com

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# Enclosure 1

UAP-HF-08108 Docket No. 52-021

Response to Request for Additional Information No.7 Rev.0

June 27, 2008

### RESPONSE TO REQUEST FOR ADITTIONAL INFORMATION

6/27/2008

## **US-APWR Design Certification** Mitsubishi Heavy Industries **Docket No.52-021**

RAI NO .:

NO.7 REVISION 0

**SRP SECTION:** 

14.02 - Initial Plant Test Program - Design Certification and New

License Applicants

**APPLICATION SECTION: 14.2** 

DATE OF RAI ISSUE:

5/28/2008

### **QUESTION NO.: 14.02-1**

The following information in the application needs to be revised:

1. The issue date of RG 1.68.2, Revision 1, listed in Table 14.2-2 of Tier 2 of the DCD should be 707/1978, not April, 1978.

2. The issue date of RG 1.206 listed in Table 14.2-2 of Tier 2 of the DCD should be 06/2007, not May 2007.

The title of RG 1.9, Revision 4, listed in Table 14.2-2 of Tier 2 of the DCD should be "Application and Testing of Safety-Related Diesel Generators in Nuclear Power Plants." not "Selection, Design, and Qualification of Diesel-Generator Units Used as Onsite Electric Power Systems at Nuclear Power Plants."

Table 14.2-2 of Tier 2 of the DCD lists Revision 1 of RG 1.37, "Quality Assurance Requirements for Cleaning of Fluid Systems and Associated Components of Water- Cooled "Nuclear Power Plants." However, Table 1.9.1-1 of the DCD notes that the RG is "Not \*Applicable." "RG applies to a site-specific operational program." Clarification is needed on the applicability of this RG to the COL applicant(s).

### ANSWER:

The following is the response to the question on Section 14.2.

- 1-3. MHI will revise the issue dates and title of RG in Table 14.2-2 as instructed.
- MHI will revise Table 1.9.1-1 to indicate the conformance with RG 1.37 and reference Subsection 14.2.7.

### Impact on DCD

This revision impacts Table 14.2-2 and Table 1.9.1-1 of the DCD. These tables of the DCD will be modified to provide in substance as follows:

Table 14.2-2, page 14.2-160:

# Table 14.2-2 Regulatory Guides Associated with the ITP Regulatory Guide 1.9, "Selection, Design, Application and Qualification Testing of Safety-related Diesel-Generator Units Used as Onsite Electric Power Systems at Generators in Nuclear Power Plants." Rev. 4, March 2007 RG 1.68.2, "Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-Cooled Nuclear Power Plants," Rev. 1, April July 1978 RG 1.206, "Combined License Applications for Nuclear Power Plants" (LWR Edition), May-June 2007

Table 1.9.1-1 (sheet 3 of 15), page 1.9-5:

Reg Guide Number	Title	Status	Corresponding Chapter/Section/ Subsection
1.37	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated Components of Water-Cooled Nuclear Power Plants (Rev. 1, March 2007)	Not applicable. RG-applies to a site-specific- operational program. Conformance with no exceptions identified.	N/A 14.2.7

### Impact on COLA

There is no impact on COLA.

### Impact on PRA

There is no impact on PRA.