



FPL Energy

Point Beach Nuclear Plant

June 27, 2008

NRC 2008-0048
TS 5.5.13
10 CFR 50.71(e)

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

Technical Specification Bases Change Summary

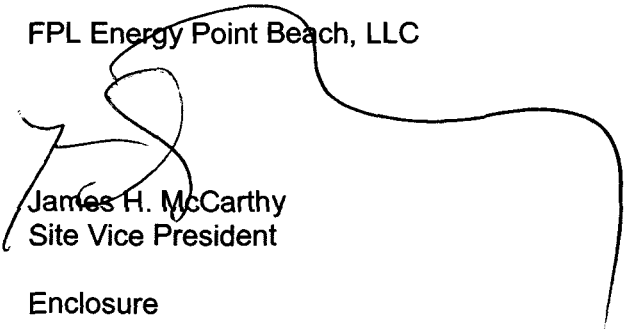
In accordance with the requirements of Technical Specification 5.5.13 and 10 CFR 50.71(e), FPL Energy Point Beach, LLC, is submitting changes made to the Point Beach Nuclear Plant (PBNP) Technical Specification Bases.

Enclosure 1 contains the Technical Specification (TS) Bases change summary from January 1, 2007, through June 30, 2008.

This letter contains no new commitments and no revisions to existing commitments.

Very truly yours,

FPL Energy Point Beach, LLC


James H. McCarthy
Site Vice President

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW

ENCLOSURE 1

FPL ENERGY POINT BEACH, LLC POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

TECHNICAL SPECIFICATIONS BASES CHANGE SUMMARY

<u>Date of Change</u>	<u>Affected Section</u>	<u>Summary</u>
01/08/07	B 3.1.4	<p>In the last sentence of the Applicable Safety Analyses section, "Criterion 2 of the NRC Policy Statement" was changed to "Criterion 2 of 10 CFR 50.36(c)(2)(ii)."</p> <p>In the first paragraph of the Actions section, a sentence was added to clarify the note is applicable to rods moved within one hour.</p>
03/15/07	B 3.3.5	<p>The requirements for operation of control room ventilation became applicable to only movement of irradiated fuel; applicability to core alterations was removed from Applicable Safety Analysis section, Applicability section and Actions section via Amendments 224/230.</p>
	B 3.6.6	<p>Throughout the Background section generic references to release limits and DBA transient were replaced with the phrase "LOCA or SLB."</p> <p>Specific descriptions of LOCA and SLB were added to the Background section.</p> <p>The Applicable Safety Analysis section was rewritten to specifically address LOCA and SLB events.</p> <p>The changes resulted in the addition of several pages, and page renumbering, to the Basis document.</p>
	B 3.7.2	<p>The second paragraph of Applicable Safety Analyses section was changed to identify the containment pressure that would be reached by analysis, and statement of the containment pressure limit.</p> <p>The final sentence of Applicable Safety Analysis section was changed to include a 10 CFR 50.36 subsection reference rather than an NRC policy statement number.</p> <p>Section SR 3.7.2.1 had the specific MSIV stroke time value replaced by a reference to the ASME code-based inservice testing program via License Amendments 230/235.</p> <p>The References section was changed to clarify that OM Code is part of ASME Boiler and Pressure Vessel Code, the reference for NUREG-800 was removed, and reference to TM 4.7, Inservice Testing Program was added.</p>

<u>Date of Change</u>	<u>Affected Section</u>	<u>Summary</u>
B 3.7.3		<p>The second paragraph of Applicable Safety Analyses was changed to identify the containment pressure that would be reached by analysis, and statement of the containment pressure limit. A paragraph was added that identifies the need to perform a safety function determination program (SFDP) review, and that the SFDP must consider containment spray and containment fan cooler operability. This same paragraph was added to the LCO section.</p>
B 3.7.9		<p>In the Actions section, the action B.4 was removed, reflecting Amendments 224/230 to the Technical Specifications. Also, the term "Core Alterations" was removed as the Specification only applies during movement of irradiated fuel.</p>
B 3.8.4		<p>In the Background section, fourth paragraph, the reference to Safety Injection Reset, has been removed because it does not apply.</p> <p>In the Background section, beginning on Page 2, the value of battery voltage is removed and a corrected FSAR reference included with figure numbers removed.</p> <p>The second paragraph of the Surveillance Requirements section on Page 8 has had details of the surveillance testing removed.</p>
B 3.8.9		<p>In the Background section, Page 2, sixth paragraph, the reference to Safety Injection Reset, has been removed because it does not apply.</p>
B 3.8.10		<p>For the first paragraph of the Actions section, the specific limitations of applicability to Core Alterations and fuel movement are removed as a result of Amendments 224/230. Applicability is as specified in the Technical Specifications.</p>
B 3.9.1		<p>In the Actions section, removed reference to Action A.2 as it was deleted from Technical Specifications. Also the reference to Core Alterations due to Amendments 224/230 was removed.</p>
B 3.9.2		<p>Throughout the Action section, removed the reference to Core Alterations due to Amendments 224/230.</p>
B 3.9.3		<p>Page 2 was revised for typographical change and relocation of a revision bar from left to right margin.</p> <p>In the Actions section, the reference to Action A.2 was removed as it no longer applies.</p>

<u>Date of Change</u>	<u>Affected Section</u>	<u>Summary</u>
	B 3.9.6	Throughout the document, removed the term "Core Alterations" because Amendments 224/230 removed the condition of applicability.
04/07/07	B 3.3.4	<p>Title changed to delete "Load Sequence" and clearly identify that starting instrumentation only is addressed; references to undervoltage and degraded voltage are removed throughout the Bases.</p> <p>The value for loss of voltage was changed.</p> <p>The Background section was changed to explain how degraded voltage conditions initiate emergency diesel generator operation. An enhanced functional description of setpoints and logic was added.</p> <p>In the Allowable Values portion of Background section, the reference to the FSAR as a source has been replaced by electrical system analyses. Also, the surveillance requirement setpoint source is the electrical system analyses.</p> <p>The Applicable Safety Analyses section has been changed to identify the system design is based on GDC 39, the impacts of individual components on system performance is deleted, and the NRC policy statement reference is replaced by the applicable 10 CFR 50.36 criterion.</p> <p>In the Surveillance Requirement section for SR 3.3.4.3, the frequency changed from refueling to every 18 months, clarified the instrument loop by excluding the potential transformer sensor, and added a paragraph describing time delays.</p>
04/30/07	B 3.6.3	Changes throughout Bases to reflect the elimination of the inside containment valve as a containment isolation valve (CIV). Outside containment valve retained as CIV. Applicable to both Units 1 and 2 as physical modifications were completed.
	B 3.9.3	Changes made throughout Bases to reflect the elimination of inside containment valve as a CIV. The outside containment valve retained as CIV. Applicable to both Units 1 and 2 as physical modifications have been completed.
06/25/07	B 3.8.1	<p>Background section, second paragraph on Page 5, replaced "full load" with "design basis accident loads."</p> <p>Background section, sixth paragraph, fan air temperature limit was lowered to 80° F.</p>

<u>Date of Change</u>	<u>Affected Section</u>	<u>Summary</u>
11/06/07	B 3.4.3	In Background section, the ASME Code changed to Section XI from Section III. In Applicable Safety Analysis section "NRC Policy Statement" changed to 10 CFR 50.36 criterion. In Reference section, added one calculation and changed ASME code number.
12/19/07	B 3.7.2	In Surveillance Requirements section, for MSIV testing, removed specific time value and details on timing; replaced by reference number as a result of Amendments 230/235. In References section, ASME code name made formal and added reference to TRM 4.7.
01/22/08	B 3.8.1	In LCO section, Item a., added "or" between two paragraphs.
	B 3.8.2	In LCO section, Item a., added "or" between two paragraphs.
04/01/08	B 3.4.5, B 3.4.6, B 3.4.7	All locations of steam generator narrow range surveillance limit were reduced to 35%.
	B 3.4.9	All locations of pressurizer high and low-level indication values changed.
	B 3.5.1	All locations of accumulator level indication values changed.
	B 3.5.4	All locations of RWST volume limit (level value) changed.
04/07/08	B 3.7.6 B 3.1.7	All locations of indicated CST level limits changed. In Background section, the maximum rod position deviation allowed changed from units of inches to rod steps.
	B 3.6.7	All occurrences of high acceptable pH value changed to 10.5.
04/11/08	B 3.6.1	In the Background and LCO sections, added the identification of Purge Supply and Exhaust System inside containment blind flanges as part of the containment boundary. In the Reference section, added NRC SE for Amendments 231/236 associated with blind flanges.

**Date of
Change**

**Affected
Section**

Summary

B 3.6.3

In the Background section changes were made to describe the Purge Supply and Exhaust System configuration (one outside valve and one inside blind flange). Also identified that blind flange uses double O ring seal. (Amendments 231/236)

The Applicable Safety Analyses section was changed to reflect the removal of control switch positioning and status control.

The LCO section was changed to reflect the removal of control switch positioning and status control.

The Actions section was changed to remove the restrictions associated with opening a single valve because the penetration remains closed by the blind flange.

The Surveillance Requirements section was changed, identifying SR 3.6.3.1 as "deleted."

The References section was revised to add the NRC Safety Evaluation for Amendments 231/236.

B 3.9.3

The Background section was changed to identify correct LCO number, 3.6.1, in place of 3.6.3.

The References section was revised to add the NRC Safety Evaluation for Amendments 231/236.

Paragraph for SR 3.9.3.2 was changed to remove reference to SR 3.6.3.5 and valve timing; non-CIVs have no timing requirement.