2/19/08

SUSQUEHANNA WRITTEN RETAKE EXAM

OUTLINE COMMENTS

<u>Tier 1, Grp 1& 2</u> Q #58 & 63 - ensure these Qs test different topics

Tier 2 Grp 1

Q # 3 & 4 - direct power supply type Qs okay in limited numbers. However, a better method is to try to ensure questions don't just ask power supplies to components directly but attempt to ask question in a manner that tests operator understanding.

Q #12 - Keep fill system is unique has passive head tank for back-up.

Tier 2 Grp 2

Q # 28 - direct power supply type Qs okay in limited numbers. However, a better method is to try to ensure questions don't just ask power supplies to components directly but attempt to ask question in a manner that tests operator understanding.

Q #30 - may be too basic.

Susquehanna Learning Center 769 Salem Boulevard Berwick, PA 18603-0467 570-542-3353



February 15, 2008

Mr. John Caruso USNRC Chief Examiner USNRC Region 1 475 Allendale Road King of Prussia, PA 19406-1415

Susquehanna Learning Center Examination Outlines PLA 006334 File A14-13D

Dear Mr. Caruso:

Enclosed for your review and approval are the outlines for the PPL Susquehanna, LLC Initial Licensed Operator Examination scheduled on May16, 2008. This is the written retest examination for the three (3) RO candidates who failed only the NRC Written Examination portion of the exam on December 7, 2007. These outlines are submitted in accordance with NUREG 1021, Operator Licensing Examination Standards for Power Reactors (Revision 9 Supplement 1). The following materials are enclosed:

- Form ES-201-2, Examination Outline Quality Checklist Rev. 0 (Signed)
- Form ES-201-3, Examination Security Agreement (Copy)
- Form ES-401-1, BWR Examination Outline Rev. 0
- Form ES-401-3, Generic Knowledge and Abilities Outline Tier 3 Rev. 0
- Form ES-401-4, Record of Rejected K/A's Rev. 0

The written examination outline was developed using the electronic random outline generator developed by Western Technical Services, Inc. The software, using the facility suppressed K/A profile, provided a written examination outline in accordance with the criteria contained in NUREG 1021, Revision 9 Supplement 1. Rejected K/As were documented on Form ES-401-4, Record of Rejected K/As.

Since this is a NRC RO Written retest only, this letter and attachments constitutes the 75 Day Submittal. Per our telephone discussion, the RO retest exam with supporting documents will be submitted for your review no later than April 18, 2008.

Also to confirm our prior discussion, a freeze of the Emergency Operating Procedures (EOPs) flowcharts and bases was completed as of February 11, 2008 to support both the candidate remediation and retest exam. This will prevent conflict with the station initiative currently underway to revise the EOPs. Following the NRC written retest examination, candidates will be trained on the updated EOPs along with the incumbent SSES licensees.

Expected additions to the Exam Security Agreement are additional Operation's Department Validation Team Members.

We request these materials be withheld from public disclosure until after the completion of the exam. The enclosed materials have been reviewed for "Safeguards Material" content. None of these materials are deemed to be "Safeguards Material."

Resubmittals of the NRC Form 398 applications with waivers are forthcoming in accordance with ES-202 Preparing and Reviewing Operator Licensing Applications. It is expected that conditions of the NRC Form 396 Medical applications previously submitted will be maintained. However, we also understand that a waiver will be required if more than 6 months have passed since the date of the applicant's last medical examination, including a certification that on the NRC Form 398 that the applicant has not developed any physical or mental conditions (10CFR 55.25) during this time. We will communicate any changes to the applicant's medical condition, as appropriate.

If you have any questions, please feel free to contact me at 570-542-3126, or Sid Morgan or Chris Michaels at 570-542-1891.

Sincerely,

Ŕ. M. Peal Manager - Nuclear Training

Response: No

Enclosures: Listed (Page 1)

- cc: R. M. Fry R. R. Sgarro Ops Letter File Nuc Records
- exam outlines pla

CGM/RMP/nlk

ES-401

Facility:	SSES 2	008 I	N OF	/ritte	n	Dat	e of l	Exam	ı:		5/1	6/20	08					
	_				RO I	(/A (Categ	ory P	oints	;				SR	0-0	nly Po	oints	
Tier	Group	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A	.2	G	\$ {c	Total
1.	1	3	3	3				4	4			3	20	N	A	N/	4	7
Emergency &	2	1	2	1				1	1			1	7	N	A	NA	4	3
Plant Evolutions	Tier Totals	4	5	4				5	5			4	27	N	A	NÆ	λ	10
	1	3	2	3	3	3	2	2	2	3	1	2	26	N	A	NA	4	5
2. Plant	2	2	1	0	1	0	1	1	3	1	1	1	12	NA	NA	NA	4	3
Systems	Tier Totals	5	3	3	4	3	3	3	5	4	2	3	38	N	A	NA	4	8
3. Generic K	nowledge	e & A	bilit	ies			2	2	-	3	4	Ļ	10	1	2	3	4	7
(Categories	8				4		2		2		2	10	NA	NA	NA	NA	,
Note: 1.	Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two). The point total for each group and tier in the proposed outline must match that specified in the table. The																	
2.	The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ± 1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.															The IRC		
3.	Systems/ not apply not inclu eliminati	evolu at th ded o on of	tions e faci n the inapp	withi lity sl outlir oropri	n eacl hould he sho ate K	h grou be de uld b /A sta	ip are leted e adde temer	ident and ju ed. R nts.	ified 1stifie efer te	on the cd; op o sect	e asso eratio ion D	ciatec nally .1.b o	l outline; importar f ES-401	syste: it, site , for §	ms or -spec guidar	evolu ific sy nce reş	tions stems gardir	that do that are g
4.	Select to before se	pics f	rom a Ig a se	s mar cond	ny sys topic	tems for a	and ev ny sys	voluti stem o	ons a or evo	s poss olutior	sible; 1.	samp	le every s	systen	n or e	volutio	on in	the group
5.	Absent a selected.	plant Use	speci the R	ific pr O and	riority 1 SRC	, only) ratin	those those	e KAs the H	s havi RO ar	ng an d SR	impo O-onl	rtanco y por	e rating (tions, res	IR) of pectiv	2.5 o vely.	r high	er sha	all be
6.	Select SI	RO to	pics f	or Tie	ers 1 a	nd 2	from (the sh	aded	systei	ms an	d K/A	categori	ies.				
7.*	The gene must be r K/A's	eric (C releva	G) K/A nt to	As in ' the ap	Tiers oplica	1 and ble ev	2 sha volutio	ll be on or :	select syster	ed fro n. Rei	om Se fer to	ction Section	2 of the l on D.1.b	K/A C of ES	atalo -401	g, but for the	the to appl	pics icable
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9.	For Tier point tota	3, sel als (#)	ect to on F	pics f orm I	rom S ES-40	ectio 1-3.	n 2 of Limit	the K SRO	K/A C selec	atalog tions	g, and to K//	enter As tha	the K/A at are link	numb red to	ers, d 10CF	lescrip R55.4	tions 3	, IRs, and

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SSES 2008 RO Written Exam Written Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1 Group 1

EAPE # / Name Safety Function	K1	K2	кз	A1	A2	G	K/A Topic(s)	lmp.	Q#
295024 High Drywell Pressure / 5	x						EK1.01 - Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL PRESSURE : Drywell integrity: Plant-Specific	4.1	39
295021 Loss of Shutdown Cooling / 4	×						AK1.04 - Knowledge of the operational implications of the following concepts as they apply to LOSS OF SHUTDOWN COOLING : Natural circulation	3.6	40
295004 Partial or Total Loss of DC Pwr / 6	x						AK1.05 - Knowledge of the operational implications of the following concepts as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER : Loss of breaker protection	3.3	41
295028 High Drywell Temperature / 5		x					EK2.01 - Knowledge of the interrelations between HIGH DRYWELL TEMPERATURE and the following: Drywell spray: Mark-I&II	3.7	42
295003 Partial or Complete Loss of AC / 6		x					AK2.02 - Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF A.C. POWER and the following: Emergency generators	4.1	43
295031 Reactor Low Water Level / 2		x					EK2.15 - Knowledge of the interrelations between REACTOR LOW WATER LEVEL and the following: A.C. distribution: Plant-Specific	3.2	44
295018 Partial or Total Loss of CCW / 8			x				AK3.05 - Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER : Placing standby heat exchanger in service	3.2	45
600000 Plant Fire On-site / 8			x	n de ser anna an a			AK3.04 - Knowledge of the reasons for the following responses as they apply to PLANT FIRE ON SITE: Actions contained in the abnormal procedure for plant fire on site	2.8	46
295005 Main Turbine Generator Trip / 3			x				AK3.03 - Knowledge of the reasons for the following responses as they apply to MAIN TURBINE GENERATOR TRIP: Feedwater temperature decrease	2.8	47
295037 SCRAM Conditions Present and Reactor Power Above APRM Downscale or Unknown / 1				x			EA1.05 - Ability to operate and/or monitor the following as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN : CRD hydraulics systems	3.9	48
295019 Partial or Total Loss of Inst. Air / 8				x			AA1.02 - Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR : Instrument air system valves: Plant-Specific	3.3	49
295026 Suppression Pool High Water Temp. / 5				×			EA1.01 - Ability to operate and/or monitor the following as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE: Suppression pool cooling	4.1	50
295025 High Reactor Pressure / 3					X		EA2.02 - Ability to determine and/or interpret the following as they apply to HIGH REACTOR PRESSURE: Reactor power	4.2	51

SSES 2008 RO Written Exam Written Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1 Group 1

EAPE # / Name Safety Function	K1	K2	КЗ	A1	A2	G	K/A Topic(s)	lmp.	Q#
295023 Refueling Acc Cooling Mode / 8					X		AA2.02 - Ability to determine and/or interpret the following as they apply to REFUELING ACCIDENTS : Fuel pool level	3.4	52
295030 Low Suppression Pool Water Level / 5					×		EA2.02 - Ability to determine and/or interpret the following as they apply to LOW SUPPRESSION POOL WATER LEVEL : Suppression pool temperature	3.9	53
295028 High Drywell Temperature / 5						X	2.4.6 - Emergency Procedures / Plan: Knowledge of EOP mitigation strategies.	3.7	54
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4						X	2.1.31 - Conduct of Operations: Ability to locate control room switches, controls, and indications, and to determine that they correctly reflect the desired plant lineup.	4.6	55
295016 Control Room Abandonment / 7						X	2.1.23 - Conduct of Operations: Ability to perform specific system and integrated plant procedures during all modes of plant operation.	4.3	56
700000 Generator Voltage and Electric Grid Disturbances				x			AA1.02 - Ability to operate and/or monitor the following as they apply to GENERATOR VOLTAGE AND ELECTRIC GRID DISTURBANCES: Turbine/generator controls.	3.8	57
295006 SCRAM / 1					x		AA2.02 - Ability to determine and/or interpret the following as they apply to SCRAM : Control rod position	4.3	58
K/A Category Totals:	3	3	3	4	4	3	Group Point Total:		20

SSES 2008 RO Written Exam Written Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1 Group 2

EAPE # / Name Safety Function	K1	K2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Q#
295020 Inadvertent Cont. Isolation / 5 & 7	x						AK1.02 - Knowledge of the operational implications of the following concepts as they apply to INADVERTENT CONTAINMENT ISOLATION : Power/reactivity control	3.5	59
295002 Loss of Main Condenser Vac / 3		x					AK2.04 - Knowledge of the interrelations between LOSS OF MAIN CONDENSER VACUUM and the following: Reactor/turbine pressure regulating system	3.2	60
295013 High Suppression Pool Temperature / 5			x		것은 이 아파 인터스 Maria		AK3.02 - Knowledge of the reasons for the following responses as they apply to HIGH SUPPRESSION POOL TEMPERATURE : Limiting heat additions	3.6	61
295007 High Reactor Pressure / 3				x			AA1.02 - Ability to operate and/or monitor the following as they apply to HIGH REACTOR PRESSURE : HPCI: Plant- Specific	3.5	62
295015 Incomplete SCRAM / 1					×		AA2.02 - Ability to determine and/or interpret the following as they apply to INCOMPLETE SCRAM : Control rod position	4.1	63
295036 Secondary Containment High Sump/Area Water Level / 5						x	2.4.20 - Emergency Procedures / Plan: Knowledge of operational implications of EOP warnings, cautions, and notes.	3.8	64
295034 Secondary Containment Ventilation High Radiation / 9		x					EK2.06 - Knowledge of the interrelations between SECONDARY CONTAINMENT VENTILATION HIGH RADIATION and the following: PCIS/NSSSS: Plant-Specific	3.9	65
K/A Category Totals:	1	2	1	1	1. 1.	1	Group Point Total:		7

4

Form ES-401-1

System # / Name	К I 1 2	к к 2 3	К 4	К 5	К 6	A 1	A 2	A 3	A 4	G		lmp.	Q#
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212000 RPS	x								K1.05 - Knowledge of the physical connections and/or cause- effect relationships between REACTOR PROTECTION SYSTEM and the following: Process radiation monitoring system	3.3	1
259002 Reactor Water Level Control	x								K1.02 - Knowledge of the physical connections and/or cause- effect relationships between REACTOR WATER LEVEL CONTROL SYSTEM and the following: Main steam flow	3.2	2
205000 Shutdown Cooling		x				ŀ			K2.02 - Knowledge of electrical power supplies to the following: Motor operated valves	2.5	3
209001 LPCS		x							K2.01 - Knowledge of electrical power supplies to the following: Pump power	3.0	4
264000 EDGs			x				-		K3.02 - Knowledge of the effect that a loss or malfunction of the EMERGENCY GENERATORS (DIESEL/JET) will have on following: A.C. electrical distribution	3.9	5
400000 Component Cooling Water			x						K3.01 - Knowledge of the effect that a loss or malfunction of the CCWS will have on the following: Loads cooled by CCWS	2.9	6
206000 HPCI				x					K4.13 - Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following: Turbine and pump lubrication: BWR-2,3,4	3.0	7
300000 Instrument Air				x					K4.02 - Knowledge of (INSTRUMENT AIR SYSTEM) design feature(s) and or interlocks which provide for the following: Cross-over to other air systems	3.0	8
215004 Source Range Monitor					x				K5.03 - Knowledge of the operational implications of the following concepts as they apply to SOURCE RANGE MONITOR (SRM) SYSTEM : Changing detector position	2.8	9
212000 RPS					x				K5.01 - Knowledge of the operational implications of the following concepts as they apply to REACTOR PROTECTION SYSTEM : Fuel thermal time constant	2.7	10
262001 AC Electrical Distribution						x			K6.03 - Knowledge of the effect that a loss or malfunction of the following will have on the A.C. ELECTRICAL DISTRIBUTION: Generator trip	3.5	11

4

Form ES-401-1

System # / Name	К 1	к 2	К 3	К 4	К 5	К 6	A 1	A 2	A 3	A 4	G		Imp.	Q#
203000 RHR/LPCI: Injection Mode						x						K6.04 - Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: INJECTION MODE (PLANT SPECIFIC) : Keep fill system	3.3	12
262002 UPS (AC/DC)							×					A1.02 - Ability to predict and/or monitor changes in parameters associated with operating the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) controls including: Motor generator outputs	2.5	13
263000 DC Electrical Distribution							x					A1.01 - Ability to predict and/or monitor changes in parameters associated with operating the D.C. ELECTRICAL DISTRIBUTION controls including: Battery charging/discharging rate	2.5	14
223002 PCIS/Nuclear Steam Supply Shutoff								X				A2.05 - Ability to (a) predict the impacts of the following on the PRIMARY CONTAINMENT ISOLATION SYSTEM/NUCLEAR STEAM SUPPLY SHUT-OFF; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abn cond or ops. Nuclear boiler instrumentation failures	3.3	15
217000 RCIC												A2.10 - Ability to (a) predict the impacts of the following on the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC) ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Turbine control system failures	3.1	16
218000 ADS									×			A3.04 - Ability to monitor automatic operations of the AUTOMATIC DEPRESSURIZATION SYSTEM including: Primary containment pressure	3.7	17
239002 SRVs									×			A3.04 - Ability to monitor automatic operations of the RELIEF/SAFETY VALVES including: Acoustical monitor noise: Plant-Specific	3.6	18
259002 Reactor Water Level Control		T		T						x		A4.02 - Ability to manually operate and/or monitor in the control room: All individual component controllers in the automatic mode	3.7	19
215005 APRM / LPRM									×			A3.08 - Ability to monitor automatic operations of the AVERAGE POWER RANGE MONITOR/LOCAL POWER RANGE MONITOR SYSTEM including: Control rod block status	3.7	20

4

Form ES-401-1

System # / Name	к 1	к 2	К 3	К 4	K 5	К 6	A 1	A 2	A 3	A 4	G		Imp.	Q#
261000 SGTS											x	2.2.42 - Equipment Control:: Ability to recognize system parameters that are entry-level conditions for Technical Specifications.	3.9	21
259002 Reactor Water Level Control						are as an a company more than and it with the state of the state of the state of the					×	2.2.44 - Equipment Control: Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives effect plant and system conditions.	4.2	22
203000 RHR/LPCI: Injection Mode				-	x							K5.02 - Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: INJECTION MODE (PLANT SPECIFIC) : Core cooling methods	3.5	23
264000 EDGs	x											K1.04 - Knowledge of the physical connections and/or cause- effect relationships between EMERGENCY GENERATORS (DIESEL/JET) and the following: Emergency generator cooling water system	3.2	24
209001 LPCS				x								K4.10 - Knowledge of LOW PRESSURE CORE SPRAY SYSTEM design feature(s) and/or interlocks which provide for the following: Testability of all operable components	2.8	25
212000 RPS			x									K3.09 - Knowledge of the effect that a loss or malfunction of the REACTOR PROTECTION SYSTEM will have on following: The magnitude of heat energy that must be absorbed by the containment during accident/transient conditions	3.2	26
K/A Category Totals:	3	2	3	3	3	2	2	2	3	1	2	Group Point Total:		26

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Form ES-401-1

System # / Name	К 1	К 2	К 3	К 4	к 5	к 6	A 1	A 2	А 3	A 4	G	Imp.	Q#
		2	3	4	э		'	2	3	4		·	

	_			 				_			
272000 Radiation Monitoring				x					K6.02 - Knowledge of the effect that a loss or malfunction of the following will have on the RADIATION MONITORING SYSTEM : D.C. power	2.5	27
233000 Fuel Pool Cooling/Cleanup		x						1	K2.02 - Knowledge of electrical power supplies to the following: RHR pumps	2.8	28
201006 RWM	x								K1.02 - Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) (PLANT SPECIFIC) and the following: Rod position indication system: P-Spec(Not-BWR6)	3.4	29
239001 Main and Reheat Steam			x						K4.04 - Knowledge of MAIN AND REHEAT STEAM SYSTEM design feature(s) and/or interlocks which provide for the following: Limits steam flow during a steam line rupture to 200%	3.4	30
259001 Reactor Feedwater					11日本の2011年1月2日	i kultur a met Santa II.		1. The second s second second se second second sec second second sec	A2.04 - Ability to (a) predict the impacts of the following on the REACTOR FEEDWATER SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Loss of extraction steam	3.3	31
216000 Nuclear Boiler Inst.	x								K1.01 - Knowledge of the physical connections and/or cause- effect relationships between NUCLEAR BOILER INSTRUMENTATION and the following: Reactor protection system	3.9	32
256000 Reactor Condensate					X				A1.07 - Ability to predict and/or monitor changes in parameters associated with operating the REACTOR CONDENSATE SYSTEM controls including: System lineup	3.1	33
259001 Reactor Feedwater									A2.01 - Ability to (a) predict the impacts of the following on the REACTOR FEEDWATER SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Pump trip	3.7	34
271000 Off-gas						· · · · ·	(Trodad	A3.02 - Ability to monitor automatic operations of the OFFGAS SYSTEM including: System flows	2.9	35
234000 Fuel Handling Equipment							>	<	A4.01 - Ability to manually operate and/or monitor in the control room: Neutron monitoring system	3.7	36

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Form ES-401-1

System # / Name	К 1	K 2	К 3	K 4	K 5	К 6	A 1	A 2	A 3	A 4	G		Imp.	Q#
230000 RHR/LPCI: Torus/Pool Spray Mode											X	2.4.31 - Emergency Procedures / Plan: Knowledge of annunciator alarms, indications, or response procedures.	4.2	37
241000 Reactor/Turbine Pressure Regulator								X				A2.09 - Ability to (a) predict the impacts of the following on the REACTOR/TURBINE PRESSURE REGULATING SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Loss of generator load	3.4	38
K/A Category Totals:	2	1	0	1	0	1	1	3	1	1	1	Group Point Total:		12

Generic Knowledge and Abilities Outline (Tier 3)

Facility:	SSES 2	008 RO Written Exam Date:				
Category	17 1 8 #	T ania	RO		SRO-Only	
		i opic	IR	Q#	IR	Q#
1. Conduct of Operations	2.1.9	Ability to direct personnel activities inside the control room.	2.9	66		
	2.1.20	Ability to interpret and execute procedure steps.	4.6	67		
	2.1.41	Knowledge of the refueling process.	2.8	69		
	2.1.34	Knowledge of primary and secondary plant chemistry limits.	2.7	74		
	Subtotal			4		
					- <u>F</u> rènce	
2. Equipment Control	2.2.17	Knowledge of the process for managing maintenance activities during power operations, such as risk assessments, work prioritizaion, coordination with the	2.6	68		
		Knowledge of the process for conducting	· · ·			
	2.2.7	special or infrequent tests.	2.9	75		
	Subtotal			2		
3. Radiation Control					1	
	2.3.12	Knowledge of Radialogical Safety Principles pertaining to licensed operator duties, such as containment entry requirements, fuel handling responsibilities, access to locked high- radiation areas, aligning filters, etc.	3.2	70		
	2.3.4	Knowledge of radiation exposure limits under	3.2	71		
		normal or emergency conditions.				
					<u>+</u>	
	Subtotal	1		2	an a	
4.		Τ		4	$u_{i}^{1} + \frac{1}{2^{i}} \frac{1}$	
Emergency					+	
Procedures /	·····					

Generic Knowledge and Abilities Outline (Tier 3)

Plan					
	2.4.16	Knowledge of EOP implementation hierarchy and coordination with other support procedures or guidelines such as, operating procedures, AOP's and SAMG's.	3.5	72	
	2.4.42	Knowledge of emergency response facilities.	2.6	73	
	[
	Subtotal	dan, , , , , , , , , , , , , , , ,		2	
Tier 3 Point Total				10	7

Tier / Group	Randomly Selected K/A	Reason for Rejection
1/1	295038 / 2.2.38	(Q #54) KA for High Offsite Release Rate with Generic 2.2.38 raises this to the SRO level, making it difficult to write a discriminating question at the RO level. Randomly replaced with 295028 / 2.4.6.
2/1	211000 / K1.07	(Q #2) It isn't possible to prepare a psychometrically sound question related to the subject K/A. Randomly replaced with $295002 / K1.02$.
2 / 1	215003 / K5.03	(Q #10) Duplicate KA for nuclear Instrument detector position. Deselected the IRM, retained the SRM KA. Randomly replaced with 212000 / K5.01.
2/1	215005 / A4.03	(Q #20) Difficult to write a question on ability to operate or monitor in the control room; APRM back panel meters, switches and indicator lights. Randomly replaced with 215005 / A3.08
2/2	201004 / K3.01	(Q #29) Current plant procedure has this system bypassed in startup and shutdown, only used for indication following reactor scram. RSCS has limited use at SSES and has been removed from TS and TRM. Randomly replaced with 201006 / K1.02.
3/2	G2 / 2.2.23	(Q #69) The KA to track Technical Specification LCO is a SRO task and would be very difficult to write a discriminating RO question. Randomly replaced with 2.1.41.
2/2	286000 / K5.07	(Q #31) This KA topic has limited operational validity for RO applicants. Randomly replaced with 259001 / A2.04.
2/2	215002 / K6.03	(Q #32) Essential Power used for RBM Operator Display Assembly ODA only. Difficult to write discriminating question Randomly replaced with 216000 / K1.01.
2/2	223001 / K1.17	(Q #27) Over sampled K1 in group 2, the ability to write a discriminating question on the physical relationship between PC and Rx Bldg HVAC would be very difficult. Randomly replaced with 272000 / K 6.02.

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