

July 1, 2008

Mr. J. Randy Johnson  
Vice President - Farley  
Joseph M. Farley Nuclear Plant  
7388 North State Highway 95  
Columbia, AL 36319

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT (FNP), UNITS 1 AND 2,  
REQUEST FOR ADDITIONAL INFORMATION REGARDING RELIEF  
REQUESTS INCLUDED WITHIN FOURTH TEN-YEAR INTERVAL  
INSERVICE TESTING PROGRAM UPDATE (TAC NOS. MD8746,  
MD8747, MD8748, MD8749, MD8750, MD8751, MD8752 AND MD8753

Dear Mr. Johnson:

By letter dated August 14, 2007, (Agencywide Documents Access and Management System (ADAMS) Accession No. ML072280494), Southern Nuclear Operating Company, Inc. submitted requests identifying proposed alternatives or requests for relief within the FNP Units 1 and 2 Fourth 10-Year Interval Inservice Testing (IST) Program.

After reviewing the request, the U.S. Nuclear Regulatory Commission staff determined that additional information is needed to complete the review. The attached enclosure identifies the requested information. Your attention and prompt response to this matter is requested.

Please contact me at 301-415-2728 with any questions.

Sincerely,

*/ra/*

R. A. Jervy, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-364

Enclosure:  
RAI

cc w/encl: See next page

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R. A. Jervey, Project Manager  
Plant Licensing Branch II-1  
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Office of Nuclear Reactor Regulation

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NAME	RJervey	GLappert	MWong
DATE	6/24/08	6/24/08	7/1/08

OFFICIAL RECORD COPY

OFFICE OF NUCLEAR REACTOR REGULATION  
REQUEST FOR ADDITIONAL INFORMATION  
THE PROPOSED ALTERNATIVE  
IN-SERVICE INSPECTION (ISI)-ALT-01  
JOSEPH M. FARLEY NUCLEAR PLANT (FNP), UNIT 2  
SOUTHERN NUCLEAR OPERATING COMPANY, INC.  
DOCKET NO. 50-364

The U.S. Nuclear Regulatory Commission staff requests the following additional information.

1. Relief Request RR-P-1

RR-P-1-01

The relief request states that analytical methods have been developed which can be used to determine individual pump flow rates. Please provide a detailed description of the analytical methods utilized to determine individual pump flow rates and describe in detail the test methodology and acceptance criteria utilized to determine acceptable pump performance.

RR-P-1-02

The relief request states that flow instrumentation is installed only to measure the flow from each of the two service water trains. Please discuss the feasibility of installing permanent flow instrumentation or temporary flow instrumentation in the pump discharge lines to allow compliance with the ASME Code requirements.

RR-P-1-03

The relief request states that combined flow, differential pressure and vibration will be measured quarterly and compared to reference values. Please describe in detail the test methodology and acceptance criteria utilized to determine acceptable parallel pump performance.

RR-P-1-04

The relief request states that whenever combined flow measurements are not in the acceptable range, individual pump evaluations will be performed and that if three pumps are not immediately available, then tests will be performed as soon as three pumps are available. Please discuss why individual pump evaluations should not be performed quarterly as required by the ASME Code. Please discuss the technical justification to allow continued operation for an indefinite time period with a service water system in the required action range if three pumps are not available to allow individual pump evaluations.

RR-P-1-05

The ASME OM Code has an acceptable range of 0.95 to 1.10 times the reference value for vertical line shaft pumps. Please discuss the ramifications of using this acceptance band for parallel pump operation versus single pump operation (i.e., How does the alternative ensure individual pump degradation greater than 5 percent is identified).

Enclosure

2. Relief Request RR-P-3

RR-P-3-01

The relief request states that the 2 inch minimum flow fixed resistance line limits flow to 50 gpm which provides only minimal data for determining pump operational readiness. The relief request also states that each pump is provided with a 4 inch return line to the condensate storage tank but that significant piping vibration was experienced when attempts were made to utilize the piping for full flow testing of the AFW pumps. Please discuss the ramifications of using the 4 inch return line at flow rates greater than 50 gpm but less than 350 gpm for the quarterly Group A test.

3. Relief Request RR-V-1

RR-V-1-01

The 2001 edition of the ASME Code allows check valves to be incorporated into a check valve condition monitoring program in accordance with Appendix II. Please discuss why these valves can not be incorporated into a condition monitoring program in accordance with the ASME Code.

RR-V-1-02

Please discuss the feasibility of utilizing non-intrusive techniques to verify valve obturator movement (without reference to INPO SOER 86-03 inspection programs).

RR-V-1-03

Please discuss the refueling outage schedules for Unit 1 and 2 with respect to the time intervals between the proposed Q1P16V0661 and Q2P16V0661 valve inspections.

RR-V-1-04

The relief request states that if a generic problem is determined to exist, then the valve on the other unit will be disassembled and inspected at the next refueling outage. The ASME Code requires that all check valves in the group be inspected during the same refueling outage. Please discuss the technical justification to allow continued operation for an indefinite time period with a service water system check valve in an unknown condition with respect to valve operability.

RR-V-1-05

Please discuss the planned maintenance windows scheduled during the operating cycle and discuss the possibility of performing the disassembly and inspection of the individual check valves outside of a refueling outage.

RR-V-1-06

The relief request states that without a sampling disassembly and inspection plan, each train of diesel emergency power will be required to be removed from service each outage. Please provide additional information on electrical bus and service water outage schedules during refueling and the maintenance windows available during outages where valve disassembly could possibly be performed.

Joseph M. Farley Nuclear Plant, Units 1 & 2

cc:

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