

June 27, 2008

Mr. Michael D. Wadley
Site Vice President
Prairie Island Nuclear Generating Plant
Nuclear Management Company, LLC
1717 Wakonade Drive East
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: INCORPORATION OF TECHNICAL SPECIFICATION TASK FORCE TRAVELERS TSTF-479, TSTF-485 AND TSTF-497 (TAC NOS. MD5983 AND MD5984)

Dear Mr. Wadley:

The Commission has issued the enclosed Amendment No.185 to Facility Operating License No. DPR-42 and Amendment No.175 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated July 3, 2007.

The amendments revise the following technical specifications (TS): (1) TS 1.4, "Frequency," to modify the second paragraph of Example 1.4-1 to be consistent with the requirements of Surveillance Requirement (SR) 3.0.4 and incorporate the changes in Technical Specification Task Force (TSTF) industry traveler TSTF-485, "Correct Example 1.4-1"; (2) TS 5.5.7.a, to modify references to Section XI of the American Society of Mechanical Engineers (ASME) Code with references to the ASME Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code), to be consistent with TSTF-479, "Changes to Reflect Revision of 10 CFR [Code of Federal Regulations] 50.55a"; (3) TS 5.5.7.b, to restrict extension of Frequencies to those Frequencies specified as 2 years or less, and take exception to the limitation in SR 3.0.2 which does not apply the 1.25 times extension to Frequencies of 24 months, to be consistent with TSTF-479 and TSTF-497, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less"; and (4) TS 5.5.7.d, to modify the referenced ASME Code to be consistent with TSTF-479.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

M. Wadley

- 2 -

If you have any questions regarding this correspondence, please contact me at 301-415- .

Sincerely,

/RA/

Mahesh L. Chawla, Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures:

1. Amendment No.185 to DPR-42
2. Amendment No.175 to DPR-60
3. Safety Evaluation

cc w/encls: See next page

M. Wadley

- 2 -

If you have any questions regarding this correspondence, please contact me at 301-415- .

Sincerely,

/RA/

Mahesh L. Chawla, Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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cc w/encls: See next page

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OFFICE	NRR/LPL3-1/PM	NRR/LPL3-1/LA	ITSB/BC*	OGC	NRR/LPL3-1/BC
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DATE	6/17/08	6/13/08	05/05/08	6/25/08	6/27/08

*SE transmitted by memo of 5/5/08.

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NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.185
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated July 3, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.185, are hereby incorporated in the license. NMC shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by P. Tam for/

Lois M. James, Chief
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility Operating License
and Technical Specifications

Date of Issuance: June 27, 2008

NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.175
License No. DPR-60

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated July 3, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.175, are hereby incorporated in the license. NMC shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by P. Tam for /

Lois M. James, Chief
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility Operating License
and Technical Specifications

Date of Issuance: June 27, 2008

ATTACHMENT TO LICENSE AMENDMENT NOS.185 AND 175

FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

DOCKET NOS. 50-282 AND 50-306

Replace the following pages of the Facility Operating License No. DPR-42 and DPR-60 with the attached revised pages. The changed areas are identified by a marginal line.

REMOVE

INSERT

DPR-42, License Page 3
DPR-60, License Page 3

DPR-42, License Page 3
DPR-60, License Page 3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

1.4-3
5.0-11
5.0-12

1.4-3
5.0-11
5.0-12

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, NMC to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NMC to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
- (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NMC to transfer byproduct materials from other job sites owned by Northern States Power Company for the purpose of volume reduction and decontamination.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

NMC is authorized to operate the facility at steady state reactor core power levels not in excess of 1650 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 185, are hereby incorporated in the license. NMC shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

NMC shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains Safeguards Information protected under 10 CFR 73.21, is entitled: "Prairie Island Nuclear Generating Plant Security Plan, Training and Qualification Plan, Safeguards Contingency Plan, and Independent Spent Fuel Storage Installation Security Program," Revision 1, submitted by letters dated October 18, 2006, and January 10, 2007.

Unit 1
Amendment No. 185

July 2006

- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NMC to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
- (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NMC to transfer byproduct materials from other job sites owned by Northern States Power Company for the purposes of volume reduction and decontamination.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

NMC is authorized to operate the facility at steady state reactor core power levels not in excess of 1650 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 175, are hereby incorporated in the license. NMC shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

NMC shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains Safeguards Information protected under 10 CFR 73.21, is entitled: "Prairie Island Nuclear Generating Plant Security Plan, Training and Qualification Plan, Safeguards Contingency Plan, and Independent Spent Fuel Storage Installation Security Program," Revision 1, submitted by letters dated October 18, 2006, and January 10, 2007.

Unit 2
Amendment No. 175

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.185 TO FACILITY OPERATING LICENSE NO. DPR-42
AND AMENDMENT NO.175 TO FACILITY OPERATION LICENSE NO. DPR-60
NUCLEAR MANAGEMENT COMPANY, LLC
PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2
DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By application dated July 3, 2007, the Nuclear Management Company, LLC (the licensee), requested changes to the Technical Specifications (TSs) for the Prairie Island Nuclear Generating Plant, Units 1 and 2. The proposed changes would:

- (1) Revise TS 1.4, "Frequency" to modify the second paragraph of Example 1.4-1 to be consistent with the requirements of Surveillance Requirement (SR) 3.0.4 and incorporate the changes in Technical Specification Task Force (TSTF) industry traveler TSTF-485, "Correct Example 1.4-1";
- (2) Revise TS 5.5.7.a and d, to modify references to Section XI of the American Society of Mechanical Engineers (ASME) Code with references to the ASME Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code), to be consistent with TSTF-479, "Changes to Reflect Revision of 10 CFR [*Code of Federal Regulations*] 50.55a"; and,
- (3) Revise TS 5.5.7.b, to restrict extension of Frequencies to those Frequencies specified as 2 years or less, and take exception to the limitation in SR 3.0.2 which does not apply the 1.25 times extension to Frequencies of 24 months, to be consistent with TSTF-479 and TSTF-497, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less."

2.0 REGULATORY EVALUATION

The regulations in Title 10 of the *Code of Federal Regulations* (CFR) Part 50.36 specify the format and content of the TSs. Section 1.0 of the TS contains instructions regarding the Use and Application of the TS. Section 1.4 defines the proper use and application of Frequency requirements. The proposed change to TS 1.4 for consistency with the requirements in SR 3.0.4 and limiting condition for operation (LCO) 3.0.4 is administrative and has no effect on any regulatory requirements or criteria.

The regulations in 10 CFR 50.55a define the requirements for applying industry codes to each licensed nuclear powered facility. Licensees are required by 10 CFR 50.55a(f)(4)(i) to prepare programs to perform inservice testing of certain ASME Code Class 1, 2, and 3 pumps and valves during the initial 120-month interval. The regulations require that programs be developed using the latest edition and addenda incorporated into paragraph (b) of 10 CFR 50.55a on the date 12 months prior to the date of issuance of the operating license subject to the limitations and modification identified in paragraph (b). Nuclear Regulatory Commission (NRC) regulations also require that the inservice testing programs be revised during successive 120-month intervals to comply with the latest edition and addenda of the Code incorporated by reference in paragraph (b) 12 months prior to the start of the interval. NRC amended 10 CFR 50.55a(f)(4)(ii) to require licensees to update their Inservice Testing Program to the latest edition of the ASME OM Code, which was incorporated by reference into 10 CFR 50.55a(b). The final rule was published in the *Federal Register* on September 22, 1999 (64 FR 51370). This amendment request is for a successive 120-month interval.

The proposals in the license amendment request (LAR) to replace references to Section XI of the ASME Boiler and Pressure Vessel Code with updated references to the ASME Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code) in TS 5.5.7, "Inservice Testing Program," are in accordance with 10 CFR 50.55a, and are considered administrative.

In accordance with 10 CFR 50.36(d)(3), SR are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met.

3.0 TECHNICAL EVALUATION

Revision of TS 1.4, "Frequency"

Section 1.0 of the TS contains instructions regarding the Use and Application of the TS. Section 1.4 defines the proper use and application of Frequency requirements. The discussion in Example 1.4-1 currently states that if the interval as specified by SR 3.0.2 is exceeded while the unit is not in a MODE or other specified condition in the Applicability of the LCO for which performance of the SR is required, the Surveillance must be performed within the Frequency requirements of SR 3.0.2 prior to entry into the MODE or other specified condition. Failure to do so, would result in a violation of SR 3.0.4.

SR 3.0.4 states that entry into a MODE or other specified condition in the Applicability of an LCO shall only be made when the LCO's Surveillances have been met within their specified Frequency, except as provided by SR 3.0.3. When an LCO is not met due to Surveillances not having been met, entry into a MODE or other specified condition in the Applicability shall only be made in accordance with LCO 3.0.4. LCO 3.0.4 provides conditions under which it is acceptable to enter the Applicability of the LCO with the LCO not met. Therefore, it is possible to enter the MODE or other specified condition in the Applicability of an LCO with a Surveillance not performed within the Frequency requirements of SR 3.0.2 and this does not result in a violation of SR 3.0.4. TS 1.4 Example 1.4-1 is being modified to indicate that if the interval as specified by SR 3.0.2 is exceeded while the unit is not in a MODE or other specified condition in the Applicability of the LCO for which performance of the SR is required, then SR 3.0.4 becomes applicable.

The proposed change to TS 1.4 SR Example 1.4-1 is considered administrative in that it modifies the example to demonstrate the proper application of SR 3.0.4 and LCO 3.0.4. SR 3.0.4 and LCO 3.0.4 are not modified by this proposed change. Additionally, the proposed change to modify the discussion of TS 1.4 SR Example 1.4-1 is consistent with TSTF-485-A, Revision 0, "Correct Example 1.4-1." Because the change is necessary for consistency with the requirements of SR 3.0.4 and LCO 3.0.4, the NRC staff finds the proposed change to TS 1.4 acceptable.

Revision to TS 5.5.7a and 5.5.7d - Incorporation of references to the OM Code (TSTF-479)

As stated in Section 2.0, "Regulatory Evaluation," 10 CFR 50.55a(f)(4)(ii) requires licensees during successive 120-month intervals to update their Inservice Testing Program to the latest edition of the ASME OM Code. The licensee's revision to TS 5.5.7, "Inservice Testing Program," parts a and d, is administrative in nature in that the TS are updated to incorporate the latest edition of the ASME OM Code, as required by 10 CFR 50.55a. The proposed change to update the Inservice Testing Program to the latest edition of the ASME OM Code is consistent with TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a." Because this change is necessary for compliance with the applicable regulations, the NRC staff finds the proposed changes to TS 5.5.7a and 5.5.7d acceptable.

Revision to Surveillance Frequency Extension (TSTF-497)

TS 5.5.7a currently lists various specific testing frequencies for the Inservice Testing Program. TS 5.5.7b currently states that the provisions of SR 3.0.2 are applicable to TS 5.5.7a. The Prairie Island SR 3.0.2 contains language that differs from the Standard Technical Specification in that interval extensions for SRs specified with a 24 month frequency are not allowed.

The licensee proposes to modify TS 5.5.7b so that the provisions of SR 3.0.2 are applicable to both the specific testing frequencies listed in TS 5.5.7a as well as other normal and accelerated testing frequencies that are not specifically listed in TS 5.5.7a, and are specified as less than 2 years in the Inservice Testing Program. SR 3.0.2 currently states that the surveillance frequency extension allowed in SR 3.0.2 does not apply to SRs with a specified frequency of 24 months, unless exceptions are stated in the individual specifications. Currently, no SR 3.0.2 exception exists in TS 5.5.7. The LAR modifies TS 5.5.7b to include an exception to SR 3.0.2, so that extensions are permitted for Inservice Testing Program surveillances with a frequency of 2 years. The proposed changes are consistent with the intent of TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a" and TSTF-497, Revision 0, "Limit Inservice Testing Program Surveillance Requirement 3.0.2 Application to Frequencies of 2 Years or Less."

The NRC staff has previously determined that the provisions of SR 3.0.2, when applied to normal and accelerated testing frequencies as specified as 2 years or less in the Inservice Testing Program, will continue to assure that SRs will maintain the necessary quality of systems and components so that facility operation will be within safety limits, and that the limiting conditions for operation will be met. Because the SRs will maintain the necessary quality of systems and components, and the changes are consistent with the changes in TSTF-479 and TSTF-497, the NRC staff finds the proposed change to TS 5.5.7b acceptable.

The licensee also submitted proposed changes to the TS Bases along with the proposed changes to the TS. The NRC staff has no objection to the proposed TS Bases changes.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes the requirements with respect to use of a facility component located within the restricted area as defined in 10 CFR Part 20, and changes surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (72 FR 49579). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Aron Lewin, NRR

Date: June 27, 2008

Prairie Island Nuclear Generating Plant,
Units 1 and 2

cc:

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When distributing documents that are not
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July 2006