

May 29, 2008

Mr. David A. Christian
President and Chief Nuclear Officer
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: SURRY POWER STATION, UNIT NO. 2, REQUEST FOR ADDITIONAL
INFORMATION (RAI) REGARDING THE REVISION OF TECHNICAL
SPECIFICATION 4.4, PERTAINING TO THE CONTAINMENT LEAKAGE RATE
TESTING PROGRAM (TAC NO. MD7535)

Dear Mr. Christian:

By letter dated December 17, 2007, Virginia Electric and Power Company (the licensee) proposed Technical Specification (TS) changes to permit a one-time 5-year exception to the 10-year frequency of the performance-based leakage rate testing program for Type A tests, as required by Regulatory Guide 1.163 for Surry Power Station, Unit No. 2. The Nuclear Regulatory Commission (NRC) staff is reviewing the submittal and has determined that additional information is required to complete its evaluation.

The NRC staff's RAI is enclosed. The licensee is required to provide a response to the RAI within 30 days.

Sincerely,

/RA/

Siva P. Lingam, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-281

Enclosure:
RAI

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING REVISION OF TECHNICAL SPECIFICATION
PERTAINING TO THE CONTAINMENT LEAKAGE
RATE TESTING PROGRAM
SURRY POWER STATION, UNIT NO. 2
DOCKET NO. 50-281

By application dated December 17, 2007 (Agencywide Documents Access and Management System (ADAMS) Accession Number ML073511682), Virginia Electric and Power Company (the licensee) proposed a license amendment to change Surry Power Station, Unit No. 2 (Surry 2) Technical Specifications (TSs). The proposed change would permit a one-time 5-year exception to the 10-year frequency of the performance based leakage rate testing program for Type A tests, as required by Regulatory Guide (RG) 1.163.

To complete our review of the proposed TS changes, the Nuclear Regulatory Commission staff requests the licensee's response to the following:

1. Identify the Probabilistic Risk Analysis (PRA) version used to support the integrated leak rate testing extension request. Discuss the reasons for the significant change in the core damage frequency (CDF) and the intact containment frequency relative to the PRA version used to support license renewal (i.e., a reduction in CDF from 3.8E-5 to 2.93E-5 per year and a reduction in intact containment frequency from 2.2E-5 to 3.17E-6 per year.)
2. Table A-2 of Attachment 2 to your letter dated December 17, 2007, provides the estimated population dose for each Electric Power Research Institute (EPRI) accident class based on the frequency-weighted sum of the doses provided in Table A-1 of your letter dated December 17, 2007. However, the population doses reported in Table A-2 for EPRI Classes 2, 7, and 8 are inconsistent with the frequency-weighted sum of the doses from Table A-1. Although the correct dose values were used for Classes 7 and 8 in the main report (e.g., Table 2 of your letter dated December 17, 2007), the incorrect dose value was used for Class 2. Please reconcile the population dose values provided in Tables A-1, A-2, and 2, and update the risk assessment as appropriate.
3. The total large early release frequency (LERF) for Surry Power Station, including the requested change, is stated to be 9.79E-7 per year (page 18 of Attachment 2 of your letter dated December 17, 2007). However, this value does not include the contribution to LERF from external events. As stated in Section 2.2.4 of RG 1.174, the risk-acceptance guidelines (in this case, for LERF) are intended for comparison with a full-scope assessment risk assessment, including internal and external events. Consistent with this guidance, and to the extent supportable by the available risk models for Surry Power Station, provide an estimate of the total LERF when external events and the impact of the requested change are included within the assessment.

Enclosure

Surry Power Station, Units 1 & 2

cc:

Mr. David A. Christian
President and Chief Nuclear Officer
Virginia Electrical and Power Company
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

Ms. Lillian M. Cuoco, Esq.
Senior Counsel
Dominion Resources Services, Inc.
120 Tredegar Street, RS-2
Richmond, VA 23219

Mr. Donald E. Jernigan
Site Vice President
Surry Power Station
Virginia Electric and Power Company
5570 Hog Island Road
Surry, Virginia 23883-0315

Senior Resident Inspector
Surry Power Station
U. S. Nuclear Regulatory Commission
5850 Hog Island Road
Surry, Virginia 23883

Chairman
Board of Supervisors of Surry County
Surry County Courthouse
Surry, Virginia 23683

Dr. W. T. Lough
Virginia State Corporation Commission
Division of Energy Regulation
Post Office Box 1197
Richmond, Virginia 23218

Dr. Robert B. Stroube, MD, MPH
State Health Commissioner
Office of the Commissioner
Virginia Department of Health
Post Office Box 2448
Richmond, Virginia 23218

Office of the Attorney General

Commonwealth of Virginia
900 East Main Street
Richmond, Virginia 23219

Mr. Chris L. Funderburk, Director
Nuclear Licensing & Operations Support
Dominion Resources Services, Inc.
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, Virginia 23060-6711

Mr. Matt Adams, Director
Nuclear Safety & Licensing
Surry Power Station
Virginia Electric and Power Company
5570 Hog Island Road
Surry, Virginia 23883-0315