

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

Table 1. Generic Communications Matrix Master Table

Rev.1

Key: NA= Not Applicable; A-O=Applicable, therefore Open; C= Closed; CI=Closed/Implementation; CT= Closed/Technical Specifications; OT=Open/Technical Specifications; OV=Open/Validation; O= Open

Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
IEB 71-01	Involves Main Steam isolation valves	NA	NA		
IEB 71-02	PWR Reactor Trip Circuit Breakers	NA	NA		
IEB 71-03	Catastrophic failure of Main steam line Relief valve Headers	NA	NA		
IEB 72-01	Failed hangers for Emergency Core Cooling System Suction header	NA	NA		
IEB 72-02	Simultaneous Actuation of a Safety Injection Signal on Both units of a Dual unit Facility	NA	NA		
IEB 72-03	Limitorque Valve Operator Failures	NA	NA		
IEB 73-01	Faulty Overcurrent Trip Delay Device in Circuit Breakers for Engineered Safety Systems	C	C	TVA letter dated 04/04/1973 - IR390/391 75-5	
IEB 73-02	Malfunction of Containment Purge Supply Valve Switch	C	C	TVA letter dated 08/22/1973 - IR390/391 75-5	
IEB 73-03	Defective Hydraulic Snubbers and Restraints	C	C	IR 390/391 85-08	
IEB 73-04	Defective Bergen-Patterson Hydraulic Shock Absorbers	C	C	IR 390/391 85-08	
IEB 73-05	Manufacturing Defect in BWR Control Rods	NA	NA		
IEB 73-06	Inadvertent Criticality in a BWR	NA	NA		
IEB 74-01	Valve Deficiencies	C	C	TVA letter dated 04/15/1974 - IR390/391 75-5	
IEB 74-01a	Abnormal Occurrences in Piping Systems	C	C	TVA letter dated 04/15/1974 - IR390/391 75-5	
IEB 74-02	Truck Strike Possibility	NA	NA		
IEB 74-03	Failure of Structural or Seismic Support Bolts on Class I Components	CI	CI	Implement NUREG-0577 on Unit 2 similar as Unit 1	
IEB 74-03a	Failure of Structural or Seismic Support Bolts on Class I Components	C	C		
IEB 74-04	Malfunction of Target Rock Safety Relief Valves	NA	NA		
IEB 74-04a	Malfunction of Target Rock Safety Relief Valves	NA	NA		
IEB 74-05	Shipment of an Improperly Shielded Source	NA	NA		
IEB 74-06	Defective Westinghouse (W) Type W-2 Control Switch Component	C	C	TVA letter dated 10/18/1974 - IR390/391 75-6	
IEB 74-07	Personnel Exposure - Irradiation Facility	NA	NA		
IEB 74-08	Deficiency in the ITE Molded Case Circuit Breakers, Type HE-3	C	C	TVA letter dated 08/21/1974 - IR390/391 75-5	
IEB 74-09	Deficiencies in GE Model 4KV Magne-Blast Circuit Breakers	C	C	TVA letter dated 09/20/1974 - IR390/391 76-6	
IEB 74-10	Failures in 4 inch Bypass Pipe at Dresden 2	NA	NA		
IEB 74-10a	Failures in 4 inch Bypass Pipe at Dresden 2	NA	NA		
IEB 74-10b	Failures in 4 inch Bypass Pipe at Dresden 2	NA	NA		
IEB 74-11	Improper Wiring of Safety Injection Logic at Zion 1 & 2	C	C	IR 390/391 75-6	

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		TVA	NRC		
IEB 74-12	Incorrect Coils in W Type SG Relays at Trojan	C	C	TVA letter dated 12/12/1974 - IR390/391 75-5	
IEB 74-13	Improper Factory Wiring on GE Motor Control Centers at Fort Calhoun	C	C	TVA letter dated 12/24/1974 - IR390/391 75-5	
IEB 74-14	BWR Relief Valve Discharge to Suppression Pool	NA	NA		
IEB 74-15	Misapplication of Cutler-Hammer Three Position Maintained Switch Model No. 10250T	CI	CI	Install modified A3 Cutler-Hammer 10250T switches	
IEB 74-16	Improper Machining of Pistons in Colt Industries (Fairbanks-Morse) Diesel Generators	C	C	TVA letter dated 01/02/1975 - IR390/391 75-3	
IEB 75-01	Through-Wall Cracks in Core Spray Piping at Dresden-2	NA	NA		
IEB 75-02	Defective Radionics Radiograph Exposure Devices and Source Changes	NA	NA		
IEB 75-03	Incorrect Lower Disc Spring and Clearance Dimension in Series 8300 and 8302 ASCO Solenoid Valves	CI	CI	Modify valves not modified at factory	
IEB 75-04	Cable Fire at BFNPP	CI	CI	Part of fire protection CAP	
IEB 75-05	Operability of Category I Hydraulic Shock and Sway Suppressors	CI	CI	Unit 2 action: Install proper suppressors	
IEB 75-06	Defective W Type OT-2 Control Switches	CI	CI		
IEB 75-07	Exothermic Reaction in Radwaste Shipment	NA	NA		
IEB 75-08	PWR Pressure Instrumentation	CT	CT	NRC IR 390/391 85-08	
IEB 76-01	BWR Isolation Condenser Tube Failure	NA	NA		
IEB 76-02	Relay Coil Failures	CI	CI	Repair or Replace relays before preoperational tests	
IEB 76-03	Relay Malfunctions - GE Type STD Relays	C	C	TVA letter dated 05/17/1976 - IR390/391 76-6	
IEB 76-04	Cracks in Cold Water Piping at BWRs	NA	NA		
IEB 76-05	Relay Failures	C	C	TVA letter dated 06/07/1976-IR 390/391 85-08	
IEB 76-06	Diaphragm Failures in Air Operated Auxiliary Actuators for Safety/Relief Valves	C	C	TVA memo dated 01/25/1985- IR 390/391 85-05	
IEB 76-07	Crane Hoist Control Circuit Modifications	C	C	TVA letter dated 10/29/1976 - IR 390/391 85-08	
IEB 76-08	Teletherapy Units	NA	NA		
IEB 77-01	Pneumatic Time Delay Relay Setpoint Drift	C	C	TVA letter dated 07/01/1977-IR 390/391 85-08	
IEB 77-02	Potential Failure Mechanism in Certain W AR Relays with Latch Attachments	C	C	TVA letter dated 11/11/1977-IR 390/391 85-08	
IEB 77-03	On-Line Testing of the W Solid State Protection System	CI	CI	Include necessary periodic testing in test procedures	

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		TVA	NRC		
IEB 77-04	Calculation Error Affecting Performance of a System for Controlling pH of Containment Sump Water Following a LOCA	CT	CT	NRC IR 391/78-09	
IEB 77-05	Electrical Connector Assemblies	C	C	TVA letter dated 01/17/1978 - IR391 78-9	
IEB 77-05A	Electrical Connector Assemblies	C	C	TVA letter dated 01/17/1978 - IR391 78-9	
IEB 77-06	Potential Problems with Containment Electrical Penetration Assemblies	C	C	IR 390/391 85-08	
IEB 77-07	Containment Electrical Penetration Assemblies at Nuclear Power Plants under Construction	C	C	TVA letter dated 01/20/1978 - IR391 78-9	
IEB 77-08	Assurance of Safety and Safeguards During an Emergency - Locking Systems	C	C	TVA letter dated 03/01/1978 - IR391 78-9	
IEB 78-01	Flammable Contact - Arm Retainers in GE CR120A Relays	C	C	TVA letter dated 03/20/1978 - IR391 78-9	
IEB 78-02	Terminal Block Qualification	C	C	TVA letter dated 03/01/1978 - IR391 78-9	
IEB 78-03	Potential Gas Mixture Accumulations Associated with BWR Offgas System Operations	NA	NA		
IEB 78-04	Environmental Qualification of Certain Stem Mounted Limit Switches Inside Reactor Containment	CI	CI	Ensure NAMCO switches have been approved- IR50-391/82-10	
IEB 78-05	Malfunctioning of Circuit Breaker Auxiliary Contact Mechanism	C	C	TVA letter dated 06/12/1978 - IR391 78-15	
IEB 78-06	Defective Cutler-Hammer Type M Relays and DC Coils	C	C	IR 391/78-19	ML072530128
IEB 78-07	Protection Afforded by Air-Line Respirators and Supplied-Air Hoods	NA	NA		ML072610768
IEB 78-08	Radiation Levels from Fuel Element Transfer Tubes	NA	NA		
IEB 78-09	BWR Drywell Leakage Paths Associated with Inadequate Drywell Closures	NA	NA		
IEB 78-10	Bergen-Patterson Hydraulic Shock Suppressor Accumulator Spring Coils	C	C	TVA letter dated 08/14/1978 IR 391/ 78-19	ML072530128
IEB 78-11	Examination of Mark I Containment Torus Welds	NA	NA		
IEB 78-12	Atypical Weld Material in Reactor Pressure Vessel Welds	C	C	TVA Westinghouse letter dated 10/29/1979 - IR390/391 81-04	
IEB 78-12a	Atypical Weld Material in Reactor Pressure Vessel Welds	C	C	TVA Westinghouse letter dated 10/29/1979 - IR390/391 81-04	
IEB 78-12b	Atypical Weld Material in Reactor Pressure Vessel Welds	C	C	TVA Westinghouse letter dated 10/29/1979 - IR390/391 81-04	
IEB 78-13	Failures in Source Heads Kay Ray Gauge Models 7050, 7050B, 7051, 7051B, 7060, 7060B, 7061 and 7061B	NA	NA		
IEB 78-14	Deterioration of Buna-N Components in ASCO Solenoids	NA	NA		
IEB 79-01	Environmental Qualification of Class 1E Equipment	C	C	IR 391/80-05	
IEB 79-01r1	Environmental Qualification of Class 1E Equipment	C	C	IR 391/80-05	
IEB 79-01a	Environmental Qualification of Class 1E Equipment	C	C	IR 391/80-05	

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IEB 79-01b	Environmental Qualification of Class 1E Equipment	C	C	IR 391/80-05	
IEB 79-01bs1	Environmental Qualification of Class 1E Equipment	C	C	IR 391/80-05	
IEB 79-01bs2	Environmental Qualification of Class 1E Equipment	C	C	IR 391/80-05	
IEB 79-01bs3	Environmental Qualification of Class 1E Equipment	C	C	IR 391/80-05	
IEB 79-02	Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts	CI	CI	NRC review of HAAUP program in NUREG-1232, SSER6 & SSER8	
IEB 79-02r1	Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts	CI	CI	NRC review of HAAUP program in NUREG-1232, SSER6 & SSER8	
IEB 79-02r1s1	Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts	CI	CI	NRC review of HAAUP program in NUREG-1232, SSER6 & SSER8	
IEB 79-02r2	Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts	CI	CI	NRC review of HAAUP program in NUREG-1232, SSER6 & SSER8	
IEB 79-03	Longitudinal Weld Defects in ASME SA-312 Type 304 SS Pipe Spools Fabricated by Youngstown Welding & Engineering	C	C	TVA letter dated 07/16/1981-IR391/82-17 and IR391/84-33	
IEB 79-03a	Longitudinal Weld Defects in ASME SA-312 Type 304 SS Pipe Spools Fabricated by Youngstown Welding & Engineering	C	C	TVA letter dated 07/16/1981-IR391/82-17 and IR391/84-33	
IEB 79-04	Incorrect Weights for Swing Check Valves Manufactured by Velan Engineering	C	C	TVA letter dated 10/20/1980- IR391/83-11	
IEB 79-05	Nuclear Incident at TMI	NA	NA		
IEB 79-05a	Nuclear Incident at TMI	NA	NA		
IEB 79-05b	Nuclear Incident at TMI	NA	NA		
IEB 79-05c	Nuclear Incident at TMI	NA	NA		
IEB 79-06	Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	C	C	IR 391/80-05	
IEB 79-06a	Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	C	C	IR 391/80-05	
IEB 79-06aR1	Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	C	C	IR 391/80-05	
IEB 79-06b	Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	C	C	IR 391/80-05	
IEB 79-06c	Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	C	C	IR 391/80-05	
IEB 79-07	Seismic Stress Analysis of Safety-Related Piping	C	C	TVA letter dated 05/31/1979 - IR391/79-25	
IEB 79-08	Events Relevant to BWRs Identified During TMI Incident	NA	NA		
IEB 79-09	Failure of GE Type AK-2 Circuit Breaker in Safety Related Systems	CI	CI	Complete preservice preventive maintenance on AK-2 Circuit Breakers	
IEB 79-10	Requalification Training Program Statistics	C	C	IR states that IEB is NA & requires no action	ML072560606.
IEB 79-11	Faulty Overcurrent Trip Device in Circuit Breakers for Engineering Safety Systems	C	C	TVA letter dated 07/20/1979- IR391/79-25	

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		TVA	NRC		
IEB 79-12	Short Period Scrams at BWR Facilities	NA	NA		
IEB 79-13	Cracking in Feedwater Piping	C	C	TVA letter dated 12/01/1983-IR 390/391 85-08	
IEB 79-13r1	Cracking in Feedwater Piping	C	C	TVA letter dated 12/01/1983-IR 390/391 85-08	
IEB 79-13r2	Cracking in Feedwater Piping	C	C	TVA letter dated 12/01/1983-IR 390/391 85-08	
IEB 79-14	Seismic Analysis for As-Built Safety-Related Piping Systems	CI	CI	NRC review of HAAUP Program in NUREG-1232, SSER6 & SSER8	
IEB 79-14r1	Seismic Analysis for As-Built Safety-Related Piping Systems	CI	CI	NRC review of HAAUP Program in NUREG-1232, SSER6 & SSER8	
IEB 79-14r1s1	Seismic Analysis for As-Built Safety-Related Piping Systems	CI	CI	NRC review of HAAUP Program in NUREG-1232, SSER6 & SSER8	
IEB 79-14r1s2	Seismic Analysis for As-Built Safety-Related Piping Systems	CI	CI	NRC review of HAAUP Program in NUREG-1232, SSER6 & SSER8	
IEB 79-15	Deep Draft Pump Deficiencies	C	C	TVA letter dated 01/24/1992-IR 390/391 95-70	ML072610753
IEB 79-15s1	Deep Draft Pump Deficiencies	C	C	TVA letter dated 01/24/1992-IR 390/391 95-70	ML072610753
IEB 79-16	Vital Area Access Controls	C	C	IR states that IEB is NA & requires no action	ML072560606
IEB 79-17	Pipe Cracks in Stagnant Borated Water Systems at PWR Plants	NA	NA		
IEB 79-17r1	Pipe Cracks in Stagnant Borated Water Systems at PWR Plants	NA	NA		
IEB 79-18	Audibility Problems Encountered on Evacuation of Personnel from High-Noise Areas	NA	NA		

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		TVA	NRC		
IEB 79-19	Packaging of Low-Level Radioactive Waste for Transport and Burial	NA	NA		
IEB 79-20	Packaging, Transport and Burial of Low-Level Radioactive Waste	NA	NA		
IEB 79-21	Temperature Effects on Level Measurements	CI	CI	Reviewed in 7.2.5 of both original 1982 SER and SSER14	
IEB 79-22	Possible Leakage of Tubes of Tritium Gas Used in Time Pieces for Luminosity	NA	NA		
IEB 79-23	Potential Failure of Emergency Diesel Generator Field Exciter Transformer	C	C	TVA letter dated 10/29/1979- IR391/80-05	
IEB 79-24	Frozen Lines	CI	CI	Insulate section of piping in containment spray full flow test line	
IEB 79-25	Failures of W BFD Relays in Safety Related Systems	C	C	IR 391/80-02	
IEB 79-26	Boron Loss from BWR Control Blades	NA	NA		
IEB 79-26r1	Boron Loss from BWR Control Blades	NA	NA		
IEB 79-27	Loss of Non-Class 1E I & C Power System Bus During Operation	CI	CI	Reviewed in 7.5.3 of original 1982 SER	
IEB 79-28	Possible Malfunction of NAMCO Model EA180 Limit Switches at Elevated Temperatures	C	C	TVA letter dated 04/01/1993-IR 390/391 93-32	ML072680504
IEB 80-01	Operability of ADS Valve Pneumatic Supply	NA	NA		
IEB 80-02	Inadequate QA for Nuclear Supplied Equipment	NA	NA		
IEB 80-03	Loss of Charcoal from Standard Type II, 2 Inch Tray Adsorber Cells	C	C	TVA letter dated 30/21/1980- IR 391/80-12	ML072560654
IEB 80-04	Analysis of a PWR Main Steam Line Rupture with Continued Feedwater Addition	CI	CI	IR 50-391/85-49-Complete analysis for Unit 2	ML072500505
IEB 80-05	Vacuum Condition Resulting in Damage to Chemical Volume Control System Holdup Tanks	CI	CI	Closed in IR50-391/84-45-Complete surveillance procedures for Unit 2	
IEB 80-06	Engineered Safety Features Reset Control	CI	CI	Reviewed in 7.3.5 of 1982 SER. Perform verification during preoperational testing	
IEB 80-07	BWR Jet Pump Assembly Failure	NA	NA		
IEB 80-07s1	BWR Jet Pump Assembly Failure	NA	NA		
IEB 80-08	Examination of Containment Liner Penetration Welds	C	C	TVA letter dated 07/08/1980- IR 391/80-12	
IEB 80-09	Hydramotor Actuator Deficiencies	C	C	TVA letter dated 01/15/1981- NUREG/CR 5291, IR390/391 85-08 & IR 391/85-49	
IEB 80-10	Contamination of Non-radioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment	CI	CI	1. Correct deficiencies involving monitoring of systems. 2. Include proper monitoring of non-radioactive systems in procedures.	

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		TVA	NRC		
IEB 80-11	Masonry Wall Design	CI	CI	Complete corrective actions in 50-391/93-01	
IEB 80-12	Decay Heat Removal System Operability	CI	CI	Implement operating instructions & abnormal operating instructions for RHR	
IEB 80-13	Cracking in Core Spray Spargers	NA	NA		
IEB 80-14	Degradation of Scram Discharge Volume Capability	CT	CT	NRC reviewed in SSER6 1.14.2	
IEB 80-15	Possible Loss of Emergency Notification System with Loss of Offsite Power	C	C	IR 390/391 85-08	
IEB 80-16	Potential Misapplication of Rosemount Models 1151 and 1152 Pressure Transmitters With Either "A" or "D" Output Codes	C	C	TVA letter dated 08/29/1980- IR390/391 81-17	ML072570467
IEB 80-17	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR	NA	NA		
IEB 80-17s1	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR	NA	NA		
IEB 80-17s2	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR	NA	NA		
IEB 80-17s3	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR	NA	NA		
IEB 80-17s4	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR	NA	NA		
IEB 80-17s5	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR	NA	NA		
IEB 80-18	Maintenance of Adequate Minimum Flow Thru Centrifugal Charging Pumps Following a Secondary Side High Energy Rupture	CI	CI	IR 50-391/85-49-Implement design and procedure changes.	
IEB 80-19	Mercury-Wetted Matrix Relay in Reactor Protective Systems	C	C	TVA letter dated 09/04/1980- NUREG/CR 4933, IR390/391 81-17	
IEB 80-19r1	Mercury-Wetted Matrix Relay in Reactor Protective Systems	C	C	TVA letter dated 09/04/1980- NUREG/CR 4933, IR390/391 81-17	
IEB 80-20	Failure of W Type W-2 Spring Return to Neutral Control Switches	CI	CI	Modify switches	
IEB 80-21	Valve Yokes Supplied by Malcolm Foundry Co.	C	C	TVA letter dated 05/06/1981-IR390/391 85-08	
IEB 80-22	Automation Industries, Model 200-520-008 Sealed-Source Connectors	NA	NA		
IEB 80-23	Failures of Solenoid Valves Manufactured by Valcor Eng. Corp	C	C	TVA letter dated 03/31/1981- NUREG/CR 5292, IR390/391 81-17	ML072570467
IEB 80-24	Prevention of Damage Due to Water Leakage Inside Containment	CI	CI	Confirm reactor cavity cannot be flooded, resulting in partial/total submergence	

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		TVA	NRC		
IEB 80-25	Operating Problems with Target Rock SR Valves at BWRs	NA	NA		
IEB 81-01	Surveillance of Mechanical Snubbers	NA	NA	NRC: IR 390/391 81-17. Report states bulletin is NA	
IEB 81-01r1	Surveillance of Mechanical Snubbers	NA	NA	NRC: IR 390/391 81-17. Report states bulletin is NA	
IEB 81-02	Failure of Gate Type Valves to Close Against Differential Pressure	C	C	TVA letter dated 09/30/1983-IR390/391 84-03	
IEB 81-02s1	Failure of Gate Type Valves to Close Against Differential Pressure	C	C	TVA letter dated 09/30/1983-IR390/391 84-03	
IEB 81-03	Flow Blockage to Cooling Water by Asiatic Clams and Mussels	C	C	TVA letter dated 07/21/1981- IR390/391 81-17	ML072570467
IEB 82-01	Alteration of Radiographs of Welds in Piping Subassemblies	C	C	IR 390/391 85-08	
IEB 82-01r1	Alteration of Radiographs of Welds in Piping Subassemblies	C	C	IR 390/391 85-08	
IEB 82-01r1s1	Alteration of Radiographs of Welds in Piping Subassemblies	C	C	IR 390/391 85-08	
IEB 82-02	Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of PWR Plants	CI	CI	Approach accepted in IR50-391/85-08	
IEB 82-03	Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants	NA	NA	Applies only to BWR's	ML031210724
IEB 82-03r1	Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants	NA	NA	Applies only to BWR's	ML031210724
IEB 82-04	Deficiencies in Primary Containment Electrical Penetration Assemblies	C	C	TVA letter dated 01/24/1983- IR391/83-08	
IEB 83-01	Failure of Trip Breakers (Westinghouse DB-50) to Open on Automatic Trip Signal	C	C	IR 390/391 85-08 and 391/92-13	
IEB 83-02	Stress Corrosion Cracking in Large-Diameter Stainless Steel Recirculation System Piping at BWR Plants	NA	NA		
IEB 83-03	Check Valve Failures in Raw Water Cooling Systems of Diesel Generators	NA	NA		
IEB 83-04	Failure of the Undervoltage Trip Function of Reactor Trip Breakers	CI	CI	Install new undervoltage attachment with wider grooves on reactor trip breakers	
IEB 83-05	ASME Nuclear Code Pumps and Spare Parts Manufactured by Hayward Tyler Pump Co	C	C	TVA letter dated 09/07/1983-IR 391/85-04 and NUREG/CR 5297	
IEB 83-06	Nonconforming Material Supplied by Tube-Line	CI	CI	IR 390/391 84-03; NUREG/CR 4934	
IEB 83-07	Apparently Fraudulent Products Sold by Ray Miller, Inc	C	C	TVA letter dated 03/22/1984-IR391/85-04	
IEB 83-07s1	Apparently Fraudulent Products Sold by Ray Miller, Inc	C	C	TVA letter dated 03/22/1984-IR391/85-04	
IEB 83-07s2	Apparently Fraudulent Products Sold by Ray Miller, Inc	C	C	TVA letter dated 03/22/1984-IR391/85-04	
IEB 83-08	Electrical Circuit Breakers With Undervoltage Trip in Safety Related Applications other than the Reactor Trip System	C	C	TVA letter dated 03/29/1984-IR391/84-33	
IEB 84-01	Cracks in BWR Mark 1 Containment Vent Headers	NA	NA		

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IEB 84-02	Failure of GE Type HFA Relays In Use In Class 1E Safety Systems	C	C	TVA letter dated 07/10/1984-IR390/391 84-42 and IR391/84-54	ML072570623
IEB 84-03	Refueling Cavity Water Seal	CI	CI	Reviewed in IR390/391/93-11	
IEB 85-01	Steam Binding of Auxiliary Feedwater Pumps	CI	CI	IR 390/391 90-20. See App EE of SSER 16	
IEB 85-02	Undervoltage Trip Attachment of W DB-50 Type Reactor Trip Breakers	CI	CI	Install automatic shut trip on Westinghouse DS-416 reactor trip breakers	appear as C & CI
IEB 85-03	Motor Operated Valve Common Mode Failures During Plant Transients due to Improper Switch Settings	C	C	Superseded by GL89-10	ML072690589
IEB 85-03s1	Motor Operated Valve Common Mode Failures During Plant Transients due to Improper Switch Settings	C	C	Superseded by GL89-10	ML072690589
IEB 86-01	Minimum Flow Logic Problems That Could Disable RHR Pumps	NA	NA		
IEB 86-02	Static "O" Ring Differential Pressure Switches	C	C	TVA letter dated 11/20/1986-IR390/391 90-24	ML072610318
IEB 86-03	Potential Failure of Multiple ECCS Pumps Due to Single Failure of Air-Operated Valve in Minimum Flow Recirculation Line	C	C	TVA letter dated 11/14/1986-IR390/391 87-03	ML072530767
IEB 86-04	Defective Teletherapy Timer That May Not Terminate Treatment Dose	NA	NA		
IEB 87-01	Thinning of Pipe Walls in Nuclear Power Plants	C	C		
IEB 87-02	Fastener Testing to Determine Conformance with Applicable Material Specifications	CI	CI	NRC letter dated 08/18/1989	
IEB 87-02s1	Fastener Testing to Determine Conformance with Applicable Material Specifications	CI	CI	NRC letter dated 08/18/1989	
IEB 87-02s2	Fastener Testing to Determine Conformance with Applicable Material Specifications	CI	CI	NRC letter dated 08/18/1989	
IEB 88-01	Defects in W Circuit Breakers	C	C	TVA letter dated 11/15/1991-IR 390/391 93-01	ML072680775
IEB 88-02	Rapidly Propagating Fatigue Cracks in Steam Generator Tubes	CI	CI	NRC letter 06/07/1990-Evaluate E/C data to determine anti vibration bar depth	
IEB 88-03	Inadequate Latch Engagement in HFA Type Latching Relays Manufactured by General Electric (GE) Company	C	C	TVA letter dated 04/13/1992-IR 390/391 92-13	ML072680594
IEB 88-04	Potential Safety-Related Pump Loss	CI	CI	NRC letter 05/24/1990-Perform calculations & install check valves	
IEB 88-05	Nonconforming Materials Supplied by Piping Supplies, Inc. and West Jersey Manufacturing Company	CI	CI	NRC reviewed in App. EE of SSER16	
IEB 88-05s1	Nonconforming Materials Supplied by Piping Supplies, Inc. and West Jersey Manufacturing Company	CI	CI		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
IEB 88-05s2	Nonconforming Materials Supplied by Piping Supplies, Inc. and West Jersey Manufacturing Company	CI	CI		
IEB 88-06	Actions to be Taken for the Transfer of Model No. SPEC 2-T Radiographic Exposure Device	NA	NA		
IEB 88-07	Power Oscillations in BWRs	NA	NA		
IEB 88-07s1	Power Oscillations in BWRs	NA	NA		
IEB 88-08	Thermal Stresses in Piping Connected to Reactor Cooling Systems	CI	CI	Implement program to prevent thermal stratification	
IEB 88-08s1	Thermal Stresses in Piping Connected to Reactor Cooling Systems	CI	CI	Implement program to prevent thermal stratification	
IEB 88-08s2	Thermal Stresses in Piping Connected to Reactor Cooling Systems	CI	CI	Implement program to prevent thermal stratification	
IEB 88-08s3	Thermal Stresses in Piping Connected to Reactor Cooling Systems	CI	CI	Implement program to prevent thermal stratification	
IEB 88-09	Thimble Tube Thinning in Westinghouse Reactors	CI	CI	TVA letter 03/11/1994 commitment to establish program and inspect thimble tubes	
IEB 88-10	Nonconforming Molded-Case Circuit Breakers	CI	CI	Replace circuits not traceable to a circuit breaker manufacturer	
IEB 88-10s1	Nonconforming Molded-Case Circuit Breakers	CI	CI	Replace circuits not traceable to a circuit breaker manufacturer	
IEB 88-11	Pressurizer Surge Line Thermal Stratification	CI	CI	NRC reviewed in SSER16 App EE. Complete mods to accommodate surge line	
IEB 89-01	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	CI	CI	NRC letter 09/26/1991-Remove SG tube plugs	
IEB 89-01s1	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	CI	CI	NRC letter 09/26/1991-Remove SG tube plugs	
IEB 89-01s2	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	CI	CI	NRC letter 09/26/1991-Remove SG tube plugs	
IEB 89-02	Stress Corrosion Cracking of High-Hardness Type 410 Stainless Steel Preloaded Bolting in Anchor Darling Model S350W Swing Check Valves or Valves of Similar Nature	CI	CI	NRC reviewed in App EE of SSER16	ML073371330
IEB 89-03	Potential Loss of Required Shutdown Margin During Refueling Operations	CI	CI	IR 390/391 94-04. NRC letter dated 06/22/1990	
IEB 90-01	Loss of Fill-Oil in Transmitters Manufactured by Rosemount	CI	CI	Implement & Id. Potentially defective transmitters & enhanced surveillance program	

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		TVA	NRC		
IEB 90-01s1	Loss of Fill-Oil in Transmitters Manufactured by Rosemount	CI	CI	Implement & Id. Potentially defective transmitters & enhanced surveillance program	
IEB 90-02	Loss of Thermal Margin Caused by Channel Box Bow	NA	NA		
IEB 91-01	Reporting Loss of Criticality Safety Controls	NA	NA		
IEB 91-01s1	Reporting Loss of Criticality Safety Controls	NA	NA		
IEB 92-01	Failure of Thermo-Lag 330 Fire Barrier System to Perform its Specified Fire Endurance Function	OV	OV		
IEB 92-01s1	Failure of Thermo-Lag 330 Fire Barrier System to Perform its Specified Fire Endurance Function	OV	OV		
IEB 92-02	Safety Concerns Related to "End of Life" of Aging Theratronics Teletherapy Units	NA	NA		
IEB 92-03	Release of Patients After Brachytherapy	NA	NA		
IEB 93-01	Release of Patients After Brachytherapy Treatment with Remote Afterloading Devices	NA	NA		
IEB 93-02	Debris Plugging of Emergency Core Cooling Suction Strainers	NA	NA		ML072690589
IEB 93-02s1	Debris Plugging of Emergency Core Cooling Suction Strainers	NA	NA		
IEB 93-03	Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWRs	NA	NA		
IEB 94-01	Potential Fuel Pool Draindown Caused by Inadequate Maintenance Practices at Dresden	NA	NA		
IEB 94-02	Corrosion Problems in Certain Stainless Steel Packaging's Used to Transport Uranium Hexafluoride	NA	NA		
IEB 95-01	Quality Assurance Program for Transportation of Radioactive Material	NA	NA		
IEB 95-02	Unexpected Clogging of a Residual Heat Removal Pump Strainer While Operating in Suppression Pool Cooling Mode	NA	NA		
IEB 96-01	Control Rod Insertion Problems (PWR)	OV	OV		
IEB 96-02	Movement of Heavy Loads over Spent Fuel, over Fuel in the Reactor, or over Safety-Related Equipment	CI	CI	NRC letter 05/20/1998	
IEB 96-03	Potential Plugging of ECCS Suction Strainers by Debris in BWRs	NA	NA		
IEB 96-04	Chemical, Galvanic, or Other Reactions in Spent Fuel Storage and Transportation Casks	NA	NA		
IEB 97-01	Potential for Erroneous Calibration, Dose Rate, or Radiation Exposure Measurements with Certain Victoreen Model 530 and 531SI Electrometers, Dose Meters	NA	NA		
IEB 97-02	Puncture Testing of Shipping Packages Under 10 CFR Part 71	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
IEB 01-01	Circumferential Cracking of Reactor Pressure Vessel (RPV) Head Penetration Nozzles	OV	OV		
IEB 02-01	RPV Head Degradation and Reactor Coolant Pressure Boundary Integrity	OV	OV		
IEB 02-02	RPV Head and Vessel Head Penetration Nozzle Inspection Program	OV	OV		
IEB 03-01	Potential Impact of Debris Blockage on Emergency Sump Recirculation	NA	NA	TVA: Letter dated September 7, 2007	ML0725706761
IEB 03-02	Leakage from RPV Lower Head Penetrations and Reactor Coolant Pressure Boundary Integrity	OV	OV		
IEB 03-03	Potentially Deficient 1-inch Valves for Uranium Hexafluoride Cylinders	NA	NA		
IEB 03-04	Rebaselining of Data in the Nuclear Management and Safeguards System	C	C	NRC letter dated 12/18/2003	
IEB 04-01	Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at PWRs	OV	OV		
IEB 05-01	Material Control and Accounting of Reactors and Spent Fuel Storage Facilities	C	C	TVA letter dated 03/21/2005 and 05/11/2005. NRC letter dated 11/30/2005	ML052560158
IEB 05-02	Emergency Preparedness and Response Actions for Security Based Events	C	C	TVA letters dated 01/20/2006 & 08/16/2006	
IEB 07-01	Security Officer Attentiveness	OV	OV		
cr76001	Crane Hoist Control Circuit Modifications	C	C		
cr76002	Relay Failures - Westinghouse BF (ac) and BFD (dc) Relays	C	C	Compliance achieved 12/12/1976- IR 50/391/76 11	ML071760483
cr76003	Radiation Exposures in Reactor Cavities	NA	NA		
cr76004	Neutron Monitor and Flow Bypass Switch Malfunctions	NA	NA		
cr76005	Hydraulic Shock and Sway Suppressors - Maintenance of Bleed and Lock-up Velocities on ITT Grinnell Models...	C	C	TVA letter 01/07/1977	
cr76006	Stress Corrosion Cracks in Stagnant, Low Pressure Stainless Piping Containing Boric Acid Solution at PWRs	NA	NA		
cr76007	Inadequate Performance by Reactor Operating and Support Staff Members	NA	NA		
cr77001	Malfunctions of Limitorque Valve Operators	NA	NA		
cr77002	Potential Heavy Spring Flooding (OL)	NA	NA		
cr77002a	Potential Heavy Spring Flooding (CP)	NA	NA		
cr77003	Fire Inside a Motor Control Center	NA	NA		
cr77004	Inadequate Lock Assemblies	NA	NA		
cr77005	Fluid Entrapment in Valve Bonnets	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
cr77006	Effects of Hydraulic Fluid on Electrical Cables	NA	NA		
cr77007	Short Period During Reactor Startup	NA	NA		
cr77008	Failure of Feedwater Sample Probe	NA	NA		
cr77009	Improper Fuse Coordination in BWR Standby Liquid Control System Control Circuits	NA	NA		
cr77010	Vacuum Conditions Resulting in Damage to Liquid Process Tanks	NA	NA		
cr77011	Leakage of Containment Isolation Valves with Resilient Seats	NA	NA		
cr77012	Dropped Fuel Assemblies at BWR Facilities	NA	NA		
cr77013	Reactor Safety Signals Negated During Testing	NA	NA		
cr77014	Separation of Contaminated Water Systems from Noncontaminated Plant Systems	NA	NA		
cr77015	Degradation of Fuel Oil Flow to the Emergency Diesel Generator	NA	NA		
cr77016	Emergency Diesel Generator Electrical Trip Lock-Out Features	NA	NA		
cr78001	Loss of Well Logging Source	NA	NA		
cr78002	Proper Lubricating Oil for Terry Turbines	NA	NA		
cr78003	Packaging Greater than Type A Quantities of Low Specific Activity Radioactive Material for Transport	NA	NA		
cr78004	Installation Errors That Could Prevent Closing of Fire Doors	NA	NA		
cr78005	Inadvertent Safety Injection During Cooldown	NA	NA		
cr78006	Potential Common Mode Flooding of ECCS Equipment Rooms at BWR Facilities	NA	NA		
cr78007	Damaged Components of a Bergen-Paterson Series 25000 Hydraulic Test Stand	NA	NA		
cr78008	Environmental Qualification of Safety-Related Electrical Equipment at Nuclear Power Plants	NA	NA		
cr78009	Arcing of General Electric Company Size 2 Contactors	NA	NA		
cr78010	Control of Sealed Sources in Radiation Therapy	NA	NA		
cr78011	Recirculation MG Set Overspeed Stops	NA	NA		
cr78012	HPCI Turbine Control Valve Lift Rod Bending	NA	NA		
cr78013	Inoperability of Service Water Pumps	NA	NA		
cr78014	HPCI Turbine Reversing Chamber Hold Down Bolting	NA	NA		
cr78015	Tilting Disc Check Valves Fail to Close with Gravity in Vertical Position	NA	NA		
cr78016	Limitorque Valve Actuators	NA	NA		
cr78017	Inadequate Guard Training/Qualification and Falsified Training Records	NA	NA		
cr78018	UL Fire Test	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
cr78019	Manual Override (Bypass) of Safety System Actuation Signals	NA	NA		
cr79001	Administration of Unauthorized Byproduct Material to Humans	NA	NA		
cr79002	Failure of 120 Volt Vital AC Power Supplies	NA	NA		
cr79003	Inadequate Guard Training-Qualification and Falsified Training Records	NA	NA		
cr79004	Loose Locking Nut on Limitorque Valve Operators	NA	NA		
cr79005	Moisture Leakage in Stranded Wire Conductors	NA	NA		
cr79006	Failure to Use Syringe and Bottle Shields in Nuclear Medicine	NA	NA		
cr79007	Unexpected Speed Increase of Reactor Recirculation MG Set Resulted in Reactor Power Increase	NA	NA		
cr79008	Attempted Extortion - Low Enriched Uranium	NA	NA		
cr79009	Occurrences of Split or Punctured Regulator Diaphragms in Certain Self Contained Breathing Apparatus	NA	NA		
cr79010	Pipefittings Manufactured from Unacceptable Material	NA	NA		
cr79011	Design/Construction Interface Problem	NA	NA		
cr79012	Potential Diesel Generator Turbocharger Problem	NA	NA		
cr79013	Replacement of Diesel Fire Pump Starting Contactors	NA	NA		
cr79014	Unauthorized Procurement and Distribution of XE-133	NA	NA		
cr79015	Bursting of High Pressure Hose and Malfunction of Relief Valve O-Ring in Certain Self-Contained Breathing Apparatus	NA	NA		
cr79016	Excessive Radiation Exposures to Members of the General Public and a Radiographer	NA	NA		
cr79017	Contact Problem in SB-12 Switches on General Electric Company Metalclad Circuit Breakers	NA	NA		
cr79018	Proper Installation of Target Rock Safety-Relief Valves	NA	NA		
cr79019	Loose Locking Devices on Ingersoll-Rand Pumps	NA	NA		
cr79020	Failure of GTE Sylvania Relay Type PM Bulletin 7305 Catalog 5U12 11-AC with a 120V AC Coil	NA	NA		
cr79021	Prevention of Unplanned Releases of Radioactivity	NA	NA		
cr79022	Stroke Times for Power Operated Relief Valves	NA	NA		
cr79023	Motor Starters and Contactors Failed to Operate	NA	A-O	N.R.R But defective shipment was received and should be addressed	
cr79024	Proper Installation and Calibration of Core Spray Pipe Break Detection Equipment on BWRs	NA	NA		
cr79025	Shock Arrestor Strut Assembly Interference	NA	A-O	N.R.R but inspections & C.A expected on behalf of WBN 2	
cr80001	Service Advice for GE Induction Disc Relays	NA	NA		
cr80002	Nuclear Power Plant Staff Work Hours	NA	NA		
cr80003	Protection from Toxic Gas Hazards	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
cr80004	Securing of Threaded Locking Devices on Safety-Related Equipment	NA	NA		
cr80005	Emergency Diesel-Generator Lubricating Oil Addition and Onsite Supply	NA	NA		
cr80006	Control and Accountability Systems for Implant Therapy Sources	NA	NA		
cr80007	Problems with HPCI Turbine Oil System	NA	NA		
cr80008	BWR Technical Specification Inconsistency - RPS Response Time	NA	NA		
cr80009	Problems with Plant Internal Communications Systems	NA	NA		
cr80010	Failure to Maintain Environmental Qualification of Equipment	NA	NA		
cr80011	Emergency Diesel Generator Lube Oil Cooler Failures	NA	NA		
cr80012	Valve-Shaft-to-Actuator Key May Fall Out of Place when Mounted Below Horizontal Axis	NA	NA		
cr80013	Grid Strap Damage in Westinghouse Fuel Assemblies	NA	NA		
cr80014	Radioactive Contamination of Plant Demineralized Water System and Resultant Internal Contamination of Personnel	NA	NA		
cr80015	Loss of Reactor Coolant Pump Cooling and Natural Circulation Cooldown	NA	NA		
cr80016	Operational Deficiencies in Rosemount Model 510DU Trip Units and Model 1152 Pressure Transmitters	NA	NA		
cr80017	Fuel Pin Damage Due to Water Jet from Baffle Plate Corner	NA	NA		
cr80018	10 CFR 50.59 Safety Evaluations for Changes to Radioactive Waste Treatment Systems	NA	NA		
cr80019	Noncompliance with License Requirements for Medical Licensees	NA	NA		
cr80020	Changes in Safe-Slab Tank Dimensions	NA	NA		
cr80021	Regulation of Refueling Crews -	NA	NA		
cr80022	Confirmation of Employee Qualifications	NA	NA		
cr80023	Potential Defects in Beloit Power Systems Emergency Generators	NA	NA		
cr80024	AECL Teletherapy Unit Malfunction	NA	NA		
cr80025	Case Histories of Radiography Events	NA	NA		
cr81001	Design Problems Involving Indicating Pushbutton Switches Manufactured by Honeywell Incorporated	NA	NA		
cr81002	Performance of NRC-Licensed Individuals while on Duty	NA	NA		
cr81003	Inoperable Seismic Monitoring Instrumentation	NA	NA		
cr81004	The Role of Shift Technical Advisors and Importance of Reporting Operational Events	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
cr81005	Self-Aligning Rod End Bushings for Pipe Supports	NA	NA		
cr81006	Potential Deficiency Affecting Certain Foxboro 10 to 50 Milliampere Transmitters	NA	NA		
cr81007	Control of Radioactively Contaminated Material	NA	NA		
cr81008	Foundation Materials	NA	NA		
cr81009	Containment Effluent Water that Bypasses Radioactivity Monitor	NA	NA		
cr81010	Steam Voiding in the Reactor Coolant System During Decay Heat Removal Cooldown	NA	NA		
cr81011	Inadequate Decay Heat Removal During Reactor Shutdown	NA	NA		
cr81012	Inadequate Periodic Test Procedure of PWR Reactor Protection System	NA	NA		
cr81013	Torque Switch Electrical Bypass Circuit for Safeguard Service Valve Motors	NA	A-O	Report required if discrepancies found. Clarify if discrepancies were found.	
cr81014	Main Steam Isolation Valve Failures to Close	NA	NA		
cr81015	Unnecessary Radiation Exposures to the Public and Workers During Events Involving Thickness and Level Measuring Devices	NA	NA		
GL 77-01	Intrusion Detection Systems Handbook	NA	NA		
GL 77-02	Fire Protection Functional Responsibilities	NA	NA		
GL 77-03	Transmittal of NUREG-0321, "A Study of the Nuclear Regulatory Commission Quality Assurance Program"	NA	NA	GL states no copies of NUREG found	
GL 77-04	Shipments of Contaminated Components From NRC Licensed Nuclear Power Facilities to Vendors and Service Companies	NA	NA		
GL 77-05	Nonconformity of Addresses of Items Directed to the Office of Nuclear Reactor Regulation	NA	NA		
GL 77-06	Questionnaire Related to Steam Generators	NA	NA		
GL 77-07	Reliability of Standby Diesel Generator Units	NA	NA		
GL 77-08	Revised Intrusion detection handbook and Entry Control Systems Handbook	NA	NA		
GL 78-01	Correction To Letter Of 12/15/77	NA	NA		
GL 78-02	Asymmetric Loads Background and Revised Request for Additional Information	C	C	Reviewed in SSER15-App C. Resolved by approval of leak-before-break analysis	
GL 78-03	Request for Information on Cavity Annulus Seal Ring	NA	NA		
GL 78-04	Steam Generator Operating History	NA	NA		
GL 78-05	Internal Distribution Of Correspondence - Asking For Comments On Mass Mailing System	NA	NA		
GL 78-06	This Generic Letter Was Never Issued	NA	NA		
GL 78-07	This Generic Letter Was Never Issued	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 78-08	Enclosing NUREG-0408 RE Mark I Containments, And Granting Exemption From GDC 50 And Enclosing Sample Notice	NA	NA		
GL 78-09	Multiple Subsequent Actuations of Safety Relief Valves Following an Isolated Event	NA	NA		
GL 78-10	Guidance On Radiological Environmental Monitoring	NA	NA		
GL 78-11	Guidance on Spent Fuel Pool Modifications	NA	NA		
GL 78-12	Notice Of Meeting Regarding "Implementation of 10 CFR 73.55 Requirements And Status Of Research..."	NA	NA		
GL 78-13	Forwarding of NUREG-0219	NA	NA		
GL 78-14	Transmittal Of Draft NUREG-0219 For Comment	NA	NA		
GL 78-15	Request for Information on Control of Heavy Loads Near Spent Fuel	NA	NA		
GL 78-16	Request For Information On Control Of Heavy Loads Near Spent Fuel Pools	NA	NA		
GL 78-17	Corrected Letter On Heavy Loads Over Spent Fuel	NA	NA		
GL 78-18	Corrected Letter On Heavy Loads Over Spent Fuel	NA	NA		
GL 78-19	Enclosing Sandia Report SAND 77-0777, "Barrier Technology Handbook"	NA	NA		
GL 78-20	Enclosing - "A Systematic Approach To The Conceptual Design Of Physical Protection Systems For Nuclear Facilities"	NA	NA		
GL 78-21	Transmitting NUREG/CR-0181 Concerning Barrier & Penetration Data Needed For Physical Security System Assessment	NA	NA		
GL 78-22	Revision To Intrusion Detection Systems And Entry Control Handbooks And Nuclear Safeguards Technology Handbook	NA	NA		
GL 78-23	Manpower Requirements For Operating Reactors	NA	NA		
GL 78-24	Model Appendix I Technical Specifications And Submittal Schedule For BWRs	NA	NA		
GL 78-25	This Generic Letter Was Never Issued	NA	NA		
GL 78-26	Excessive Control Rod Guide Tube Wear	NA	NA		
GL 78-27	Forwarding of NUREG-0181	NA	NA		
GL 78-28	Forwarding pages omitted from 07/11/78 letter	NA	NA		
GL 78-29	Notice of PWR steam generator conference	NA	NA		
GL 78-30	Forwarding of NUREG-0219	NA	NA		
GL 78-31	Notice of Steam Generator Conference Agenda	NA	NA		
GL 78-32	Reactor Protection System Power Supplies	NA	NA		
GL 78-33	Meeting Schedule And Locations For Upgraded Guard Qualification	NA	NA		

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		TVA	NRC		
GL 78-34	Reactor Vessel Atypical Weld Material	C	C	TVA Westinghouse letter dated 10/29/1979-IR 390/391 81-04	
GL 78-35	Regional Meetings To Discuss Upgraded Guard Qualifications	NA	NA		
GL 78-36	Cessation of Plutonium Shipments By Air Except In NRC Approved Containers	NA	NA		
GL 78-37	Revised Meeting Schedule & Locations For Upgraded Guard Qualifications	NA	NA		
GL 78-38	Forwarding Of 2 Tables Of Appendix I, Draft Radiological Effluent Technical Specifications, PWR, and NUREG-0133	NA	NA		
GL 78-39	Forwarding of 2 Tables of Appendix I, Draft Radiological Effluent Technical Specifications, BWR, And NUREG-0133	NA	NA		
GL 78-40	Training & Qualification Program Workshops	NA	NA		
GL 78-41	Mark II Generic Acceptance Criteria For Lead Plants	NA	NA		
GL 78-42	Training and Qualification Program Workshops	NA	NA		
GL 79-01	Interservice Procedures for Instructional Systems Development - TRADOC	NA	NA		
GL 79-02	Transmitting Rev. to Entry Control Systems Handbook (SAND 77-1033), Intrusion Detection Handbook (SAND 76-0554), and Barrier Penetration Database	NA	NA		
GL 79-03	Offsite Dose Calculation Manual	NA	NA		
GL 79-04	Referencing 4/14/78 Letter - Modifications to NRC Guidance "Review and Acceptance of Spent Fuel Pool Storage and Handling"	NA	NA		
GL 79-05	Information Relating to Categorization of Recent Regulatory Guides by the Regulatory Requirements Review Committee	NA	NA		
GL 79-06	Contents of the Offsite Dose Calculation Manual	NA	NA		
GL 79-07	Seismic (SSE) and LOCA Responses (NUREG-0484)	NA	NA	No copies of GL are available	
GL 79-08	Amendment to 10 CFR 73.55	NA	NA		
GL 79-09	Staff Evaluation of Interim Multiple-Consecutive Safety-Relief Valve Actuations	NA	NA		
GL 79-10	Transmitting Regulatory Guide 2.6 for Comment	NA	NA		
GL 79-11	Transmitting "Summary of Operating Experience with Recalculating Steam Generators, January 1979" NUREG-0523	NA	NA		
GL 79-12	ATWS - Enclosing Letter to GE, with NUREG-0460, Vol. 3	NA	NA		
GL 79-13	Schedule for Implementation and Resolution of Mark I Containment Long Term Program	NA	NA		
GL 79-14	Pipe Crack Study Group - Enclosing NUREG-0531 and Notice	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 79-15	Steam Generators-Enclosing Summary of Operating Experience with Recirculating Steam Generators, NUREG-0523	NA	NA		
GL 79-16	Meeting Re Implementation Of Physical Security Requirements	NA	NA		
GL 79-17	Reliability of Onsite Diesel Generators at Light Water Reactors	NA	NA		
GL 79-18	Westinghouse Two-Loop NSSS	NA	NA		
GL 79-19	NRC Staff Review of Responses to I&E Bulletins 79-06 and 79-06a	NA	NA		
GL 79-20	Cracking in Feedwater Lines	C	C		
GL 79-21	Enclosing NUREG/CR 0660, "Enhancement of on Site Emergency Diesel Generator Reliability"	NA	NA		
GL 79-22	Enclosing NUREG-0560, "Staff Report on the Generic Assessment of Feedwater Transients in PWRs Designed by B&W"	NA	NA		
GL 79-23	NRC Staff Review of Responses to I&E Bulletin 79-08	NA	NA		
GL 79-24	Multiple Equipment Failures in Safety-Related Systems	NA	A-O	Applies to ALL PWR's. Requests submittal of reports	
GL 79-25	Information Required to Review Corporate Capabilities	NA	NA		
GL 79-26	Upgraded Standard Technical Specification Bases Program	NA	NA		
GL 79-27	Operability Testing of Relief and Safety Relief Valves	NA	NA		
GL 79-28	Evaluation of Semi-Scale Small Break Experiment	NA	NA		
GL 79-29	Transmitting NUREG-0473, Revision 2, Draft Radiological Effluent Technical Specifications	NA	NA		
GL 79-30	Transmitting NUREG-0472, Revision 2, Draft Radiological Technical Specifications	NA	NA		
GL 79-31	Submittal of Copies of Response to 6/29/79 NRC Request	NA	NA		
GL 79-32	Transmitting NUREG-0578, "TMI-2 Lessons Learned"	NA	NA		
GL 79-33	Transmitting NUREG-0576 - Security Training and Qualification Plans	NA	NA		
GL 79-34	New Physical Security Plans (FR 43280-285)	NA	NA		
GL 79-35	Regional Meetings to Discuss Impacts on Emergency Planning	NA	NA		
GL 79-36	Adequacy of Station Electric Distribution Systems Voltages	CI	CI	GL compliance w/ BTP PSB-1, "Adequacy of Station Electric Distribution System"	
GL 79-37	Amendment to 10 CFR 73.55 Deferral from 8/1/79 to 11/1/79	NA	NA		
GL 79-38	BWR Off-Gas Systems - Enclosing NUREG/CR-0727	NA	NA		
GL 79-39	Transmitting Division 5 Draft Regulatory Guide and Value Impact Statement	NA	NA		
GL 79-40	Follow-up Actions Resulting from the NRC Staff Reviews Regarding the TMI-2 Accident	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 79-41	Compliance with 40 CFR 190, EPA Uranium Fuel Cycle Standard	NA	NA		
GL 79-42	Potentially Unreviewed Safety Question on Interaction Between Non-Safety Grade Systems and Safety Grade Systems	NA	NA		
GL 79-43	Reactor Cavity Seal Ring Generic Issue	NA	NA		
GL 79-44	Referencing 6/29/79 Letter Re Multiple Equipment Failures	NA	NA		
GL 79-45	Transmittal of Reports Regarding Foreign Reactor Operating Experiences	NA	NA		
GL 79-46	Containment Purge and Venting During Normal Operation	NA	NA		
GL 79-47	Radiation Training	NA	NA		
GL 79-48	Confirmatory Requirements Relating to Condensation Oscillation Loads for the Mark I Containment Long Term Program	NA	NA		
GL 79-49	Summary of Meetings Held on 9/18-20/79 to Discuss Potential Unreviewed Safety Question on Systems Interaction for B&W PI	NA	NA		
GL 79-50	Emergency Plans Submittal Dates	NA	NA		
GL 79-51	Follow-up Actions Resulting from the NRC Staff Reviews Regarding the TMI-2 Accident	NA	A-O	Applies to all C.P holders and requires implementation of actions	
GL 79-52	Radioactive Release at No. Anna Unit 1 and Lessons Learned	NA	NA		
GL 79-53	ATWS	NA	NA		
GL 79-54	Containment Purging and Venting During Normal Operation	NA	NA		
GL 79-55	Summary of Meeting Held on October 12, 1979 to Discuss Responses to IE Bulletins 79-05C and 79-06C and HPI Termination Criteria	NA	NA		
GL 79-56	Discussion of Lessons Learned Short Term Requirements	NA	NA		
GL 79-57	Acceptance Criteria for Mark I Long Term Program	NA	NA		
GL 79-58	ECCS Calculations on Fuel Cladding	NA	NA		
GL 79-59	GL was never issued.	NA	NA		
GL 79-60	Discussion of Lessons Learned Short Term Requirements	NA	NA		
GL 79-61	Discussion of Lessons Learned Short Term Requirements	NA	NA		
GL 79-62	ECCS Calculations on Fuel Cladding	NA	NA		
GL 79-63	Upgraded Emergency Plans	NA	A-O	Check Applicability. Header (yes) content (No). Request add. Info.	
GL 79-64	Suspension of All Operating Licenses	NA	NA		
GL 79-65	Radiological Environmental Monitoring Program Requirements - Enclosing Branch Technical Position, Revision 1	NA	NA		
GL 79-66	Additional Information Re 11/09/79 Letter on ECCS Calculations	NA	NA		

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		TVA	NRC		
GL 79-67	Estimates for Evacuation of Various Areas Around Nuclear Power Reactors	NA	NA		
GL 79-68	Audit of Small Break LOCA Guidelines	NA	NA		
GL 79-69	Cladding Rupture, Swelling, and Coolant Blockage as a Result of a Reactor Accident	NA	NA		
GL 79-70	Environmental Monitoring for Direct Radiation	NA	NA		
GL 80-01	NUREG-0630, "Cladding, swelling and Rupture-Models for LOCA Analysis"	NA	NA		
GL 80-02	QA Requirements Regarding Diesel Generator Fuel Oil	C	C	FSAR Section 9.5.4.2. Verified through review.	
GL 80-03	BWR Control Rod Failures	NA	NA		
GL 80-04	IEB 80-01 Operability Of ADS Valve Pneumatic Supply	NA	NA		
GL 80-05	IEB 79-01b Environmental Qualification Of Class 1E Equipment	NA	NA		
GL 80-06	Issuance Of NUREG-0313, Rev 1, "Technical Report On Material Selection And Processing Guidelines For BWR Coolant Pressure	NA	NA		
GL 80-07	Information Regarding The Program For Environmental Qualification Of Safety-Related Electrical Equipment	NA	NA		
GL 80-08	IEB 80-02 Inadequate Quality Assurance For Nuclear Supplied Equipment	NA	NA		
GL 80-09	Low Level Radioactive Waste Disposal	NA	NA		
GL 80-10	Issuance Of NUREG-0588, "Interim Staff Position On Equipment Qualifications Of Safety-Related Electrical Equipment	NA	NA		
GL 80-11	IEB 80-03 Loss Of Charcoal From Standard Type II, 2 Inch, Tray Absorber Cells	NA	A-O	Requires action on behalf of C.P holder	
GL 80-12	IEB 80-04 Analysis Of A PWR Main Steam Sine Break With Continued Feedwater Addition	NA	NA		
GL 80-13	Qualification Of Safety Related Electrical Equipment	NA	NA		
GL 80-14	LWR Primary Coolant System Pressure Isolation Valves	CT	CT	NRC reviewed in 1.14.2 of SSER 6	
GL 80-15	Request For Additional Management And Technical Resources Information	NA	NA		
GL 80-16	IEB 79-01b Environmental Qualification Of Class 1E Equipment	NA	NA		
GL 80-17	Modifications To Boiling Water Reactor Control Rod Drive Systems	NA	NA		
GL 80-18	Crystal River 3 Reactor Trip From Approximately 100% Full Power	NA	NA		
GL 80-19	Resolution Of Enhanced Fission Gas Release Concern	NA	NA		

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		TVA	NRC		
GL 80-20	Actions Req.. From OL Applicants Of NSSS Designs By W and CE Resulting From NRC B&O Task Force Review Of TMI2 Accident	NA	NA		
GL 80-21	Vacuum Condition Resulting in Damage to Chemical Volume Control System Holdup Tanks	CI	CI	Complete surveillance procedures for Unit 2. IR 50/391/84-45	
GL 80-22	Transmittal Of NUREG-0654 "Criteria For Preparation And Evaluation Of Radiological Emergency Response Plans And	NA	NA		
GL 80-23	Change Of Submittal Date For Evaluation Time Estimates	NA	NA		
GL 80-24	Transmittal Of Information On NRC "Nuclear Data Link Specifications"	NA	NA		
GL 80-25	IEB 80-06 Engineering Safety Feature (ESF) Reset Controls	NA	NA		
GL 80-26	Qualifications Of Reactor Operators	NA	NA		
GL 80-27	IEB 80-07 BWR Jet Pump Assembly Failure	NA	NA		
GL 80-28	IEB 80-08 Examination Of Containment Liner Penetration Welds	NA	A-O	Info expected to be submitted 90 days after issuance	
GL 80-29	Modifications To Boiling Water Reactor Control Rod Drive Systems	NA	NA		
GL 80-30	Clarification Of The Term "Operable" As It Applies To Single Failure Criterion For Safety Systems Required By TS	NA	NA		
GL 80-31	IEB 80-09 Hydramotor Actuator Deficiencies	NA	NA		
GL 80-32	Information Request on Category I Masonry Walls Employed by Plants Under CP and OL Review	NA	A-O	B80-11 requests actions from all C.P facilities	
GL 80-33	Actions Req. From OL Applicants Of B&W Designed NSSS Resulting From NRC B&O Task Force Review Of TMI2 Accident	NA	NA		
GL 80-34	Clarification of NRC Requirements for Emergency Response Facilities at Each Site	NA	NA		
GL 80-35	Effect Of A DC Power Supply Failure On ECCS Performances	NA	NA		
GL 80-36	IEB 80-10 Contamination Of Non-Radioactive System And Resulting Potential For Unmonitored, Uncontrolled Release To Envir	NA	NA		
GL 80-37	Five Additional TMI-2 Related Requirements To Operating Reactors	NA	NA		
GL 80-38	Summary of Certain Non-Power Reactor Physical Protection Requirements	NA	NA		
GL 80-39	IEB 80-11 Masonry Wall Design	NA	NA		

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		TVA	NRC		
GL 80-40	Transmittal Of NUREG-0654 "Report Of The B&O Task Force " Appropriate NUREG-0626 "Generic Eval Of FW Transient And Sbl	NA	NA		
GL 80-41	Summary Of Meetings Held On April 22 &23, 1980 With Representatives Of The Mark I Owners Group	NA	NA		
GL 80-42	IEB 80-12 Decay Heat Removal System Operability	NA	NA		
GL 80-43	IEB 80-13 Cracking In Core Spray Spargers	NA	NA		
GL 80-44	Reorganization Of Functions And Assignments Within ONRR/SSPB	NA	NA		
GL 80-45	Fire Protection Rule	NA	NA		
GL 80-46	A-12, Fracture Toughness and Additional Guidance on Potential for Low Fracture Toughness and Laminar Tearing on PWR Steam Generator Coolant Pump Supports	C	C	N.R.R.-NUREG-0577 states laminar issue was resolved.	
GL 80-47	Additional Guidance On "Potential For Low Fracture Toughness And Laminar Tearing On PWR SG Reactor Coolant Pump Suprts	C	C	N.R.R.-NUREG-0577 states laminar issue was resolved.	
GL 80-48	Revision To 5/19/80 Letter On Fire Protection	NA	NA		
GL 80-49	Nuclear Safeguards Problems	NA	NA		
GL 80-50	Generic Activity A-10 BWR Cracks	NA	NA		
GL 80-51	On-Site Storage Of Low-Level Waste	NA	NA		
GL 80-52	Five Additional TMI-2 Related Requirements - Erata Sheets To 5/7/80 Letter	NA	NA		
GL 80-53	Decay Heat Removal Capability	NA	NA		
GL 80-54	IEB 80-14 Degradation Of Scram Discharge Volume Capability	NA	NA		
GL 80-55	IEB 80-15 Possible Loss Of Hotline With Loss Of Off-Site Power	NA	NA		
GL 80-56	Commission Memorandum And Order On Equipment Qualification	NA	NA		
GL 80-57	Further Commission Guidance For Power Reactor Operating Licenses NUREG-0660 And NUREG-0694	NA	NA		
GL 80-58	IEB 80-16 Potential Misapplication Of Rosemount Inc. Models 1151/1152 Pressure Transmitters With "A" Or "D" Output Codes	NA	NA		
GL 80-59	Transmittal Of Federal Register Notice RE Regional Meetings To Discuss Environmental Qualification Of Elec. Equipment	NA	NA		
GL 80-60	Request for Information Regarding Evaluation Times	NA	NA		
GL 80-61	TMI-2 Lessons Learned	NA	NA		
GL 80-62	TMI-2 Lessons Learned	NA	NA		

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		TVA	NRC		
GL 80-63	IEB 80-17 Failure Of Control Rods To Insert During A Scram At A BWR	NA	NA		
GL 80-64	Scram Discharge Volume Designs	NA	NA		
GL 80-65	Request For Estimated Construction Completion And Fuel Load Schedules	NA	NA		
GL 80-66	IEB 80-17 Supplement 1 Failure Of Control Rods To Insert During A Scram At A BWR	NA	NA		
GL 80-67	Scram Discharge Volume	NA	NA		
GL 80-68	IEB 80-17 Supplement 2 Failures Revealed By Testing Subsequent To Failure Of Control Rods To Insert During A Scram At A	NA	NA		
GL 80-69	IEB 80-18 Maintenance Of Adequate Minimum Flow Through Centrifugal Charging Pumps Following Secondary Side Helb	NA	NA		
GL 80-70	IEB 80-19 Failures Of Mercury-Wetted Matrix Relays In RPS Of Operating Nuclear Power Plants Designed By GE	NA	NA		
GL 80-71	IEB 80-20 Failures Of Westinghouse Type W-2 Spring Return To Neutral Control Switches	NA	NA		
GL 80-72	Interim Criteria For Shift Staffing	NA	NA		
GL 80-73	Functional Criteria For Emergency Response Facilities", NUREG-0696	NA	NA		
GL 80-74	Notice Of Forthcoming Meeting With Representatives Of EPRI To Discuss Program For Resolution Of USI A-12, Fracture Tough	NA	NA		
GL 80-75	Lessons Learned Tech Specs	NA	NA		
GL 80-76	Notice Of Forthcoming Meeting With GE To Discussed Proposed BWR Feedwater Nozzle Leakage Detection System	NA	NA		
GL 80-77	Refueling Water Level - Technical Specifications Changes	CT	CT	Address in Tech Specs as appropriate.	
GL 80-78	Mark I Containment Long-Term Program	NA	NA		
GL 80-79	IEB 80-17 Supplement 3 Failures Revealed By Testing Subsequent To Failure Of Control Rods To Insert During A Scram At A	NA	NA		
GL 80-80	Preliminary Clarification Of TMI Action Plan Requirements	NA	NA		
GL 80-81	Preliminary Clarification Of TMI Action Plan Requirements - Addendum To 9/5/80 Letter	NA	NA		
GL 80-82	IEB 79-01b Supplement 2 Environmental Qualification Of Class 1E Equipment	NA	NA		
GL 80-83	Environmental Qualification Of Safety-Related Equipment	NA	NA		
GL 80-84	BWR Scram System	NA	NA		

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		TVA	NRC		
GL 80-85	Implementation Of Guidance From USI A-12 "Potential For LOW Fracture Toughness And Lamellar Tearing On Component Support	NA	NA		
GL 80-86	Notice Of Meeting To Discuss Final Resolution Of USI A-12	NA	NA		
GL 80-87	Notice Of Meeting To Discuss Status Of EPRI-Proposed Resolution Of The USI A-12 Fracture Toughness Issue	NA	NA		
GL 80-88	Seismic Qualification Of Auxiliary Feedwater Systems	NA	NA		
GL 80-89	IEB 79-01b Supplement 3 Environmental Qualification Of Class 1E Equipment	NA	NA		
GL 80-90	NUREG-0737, TMI (Prior and future GLs, with the exception of certain discrete scopes, have been screened into NUREG list for those applicable to Watts Bar 2)	CI	CI		
GL 80-91	Odyn Code Calculation	NA	NA		
GL 80-92	IEB 80-21 Valve Yokes Supplied By Malcolm Foundry Company, Inc.	NA	A-O	Report was to be submitted within 60 days	
GL 80-93	Emergency Preparedness	NA	NA		
GL 80-94	Emergency Plan	NA	NA		
GL 80-95	A-10, NUREG-0619, "BWR Feedwater Nozzle and Control Rod Drive Return Line Cracking"	NA	NA		
GL 80-96	Fire Protection	NA	NA		
GL 80-97	IEB 80-23 Failures Of Solenoid Valves Manufactured By Valcor Engineering Corporation	NA	NA		
GL 80-98	IEB 80-24 Prevention Of Damage Due To Water Leakage Inside Containment	NA	NA		
GL 80-99	Technical Specification Revisions For Snubber Surveillance	NA	NA		
GL 80-100	Appendix R to 10 CFR 50 Regarding Fire Protection-Federal Register Notice	NA	NA		
GL 80-101	Inservice Inspection Programs	NA	NA		
GL 80-102	Commission Memorandum And Order Of May 23, 1980 (Referencing IEB 79-01b Supplement 2 - q.2 & 3 - Sept 30, 1980)	NA	NA		
GL 80-103	Fire Protection - Revised Federal Register Notice	NA	NA		
GL 80-104	Orders On Environmental Qualification Of Safety Related Electrical Equipment	NA	NA		
GL 80-105	Implementation Of Guidance For USI A-12, Potential For Low Fracture Toughness And Lamellar Tearing On Component Supports	NA	NA		
GL 80-106	Report On ECCS Cladding Models, NUREG-0630	NA	NA		
GL 80-107	BWR Scram Discharge System	NA	NA		

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		TVA	NRC		
GL 80-108	Emergency Planning	NA	NA		
GL 80-109	Guidelines For Systematic Evaluation Program Soil Structure Interaction Reviews	NA	NA		
GL 80-110	Periodic Updating Of FSARS	NA	NA		
GL 80-111	IEB 80-17 Supplement 4, Failure Of Control Rods To Insert During A Scram At A BWR	NA	NA		
GL 80-112	IEB 80-25 Operating Problems With Target Rock Safety Relief Valves	NA	NA		
GL 80-113	Control of Heavy Loads	C	C	Superseded by GL 81-007	
GL 81-01	Qualification of Inspection, Examination, Testing and Audit Personnel	NA	NA		
GL 81-02	Analysis, Conclusions and Recommendations Concerning Operator Licensing	NA	NA		
GL 81-03	NUREG-0313, Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping	NA	NA		
GL 81-04	Emergency Procedures and Training for Station Blackout	C	C	Superseded by Station Blackout Rule	
GL 81-05	Information Regarding The Program For Environmental Qualification Of Safety-Related Electrical Equipment	NA	NA		
GL 81-06	Periodic Updating of Final Safety Analysis Reports (FSARS)	NA	NA		
GL 81-07	Control of Heavy Loads	CI	CI	Unit 2 Heavy Loads program will be in compliance with NUREG-0612	
GL 81-08	Odyn Code	NA	NA		
GL 81-09	BWR Scram Discharge System	NA	NA		
GL 81-10	Post-TMI Requirements For The Emergency Operations Facility	NA	NA		
GL 81-11	BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking (NUREG-0619)	NA	NA		
GL 81-12	Fire Protection Rule (and prior related GLs)	NA	NA		
GL 81-13	SER For GEXL Correlation For 8X8R Fuel Reload Applications For Appendix D Submittals of The GE Topical Report	NA	NA		
GL 81-14	Seismic Qualification of Auxiliary Feedwater Systems	CI	CI	TVA: FSAR 10.4.9	
GL 81-15	Environmental Qualification of Class 1E Electrical Equipment-Clarification of Staffs Handling of Proprietary Information	NA	NA		
GL 81-16	NUREG-0737 Item I.C.1 SER on Abnormal Transient Operating Guidelines (ATOG)	NA	NA		
GL 81-17	Functional Criteria for Emergency Response Facilities	NA	NA		
GL 81-18	BWR Scram Discharge System-Clarification of Diverse Instrumentation Requirements	NA	NA		

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		TVA	NRC		
GL 81-19	Thermal Shock to Reactor Pressure Vessels	NA	NA		
GL 81-20	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	NA	NA		
GL 81-21	Natural Circulation Cooldown	CI	CI	TVA responded 12/03/1981- Unit 2 action: Issue operating procedures	
GL 81-22	Engineering Evaluation of The H.B. Robinson Reactor Coolant System Leak on 1/29/81	NA	NA		
GL 81-23	INPO Plant Specific Evaluation Reports	NA	NA		
GL 81-24	Multi-Plant Issue B-56 Control Rods Fail To Fully Insert	NA	NA		
GL 81-25	Change in Implementing Schedule For Submission and Evaluation of Upgraded Emergency Plans	NA	NA		
GL 81-26	Licensing Requirements for Pending Construction Permit and Manufacturing License Applications	NA	NA		
GL 81-27	Privacy and Proprietary Material in Emergency Plans	NA	NA		
GL 81-28	Steam Generator Overfill	NA	NA		
GL 81-29	Simulator Examinations	NA	NA		
GL 81-30	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	NA	NA		
GL 81-31	Small Break LOCA Confirmatory Integral Systems Experiments for B&W Designed Plants	NA	NA		
GL 81-32	NUREG-0737, Item II.K.3.44-Evaluation of Anticipated Transients Combined With Single Failure	NA	NA		
GL 81-33	Technical Specification for Station Batteries Multiplant Action	NA	NA		
GL 81-34	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	NA	NA		
GL 81-35	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	NA	NA		
GL 81-36	Revised Schedule for Completion of TMI Action Plan Item II.D.1, Relief and Safety Valve Testing	NA	NA		
GL 81-37	ODYN Code Reanalysis Requirements	NA	NA		
GL 81-38	Storage of Low Level Radioactive Wastes at Power Reactor Sites	NA	NA		
GL 81-39	NRC Volume Reduction Policy	NA	NA		
GL 81-40	Qualifications of Reactor Operators	NA	NA		
GL 82-01	New Applications Survey	NA	NA		
GL 82-02	Commission Policy on Overtime	NA	NA		
GL 82-03	High Burnup MAPLHGR Limits	NA	NA		
GL 82-04	Use of INPO See-in Program	NA	NA		
GL 82-05	Post-TMI Requirements	NA	NA		

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		TVA	NRC		
GL 82-06	NEVER ISSUED	NA	NA		
GL 82-07	Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	NA	NA		
GL 82-08	Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	NA	NA		
GL 82-09	Environmental Qualification of Safety Related Electrical Equipment	NA	NA		
GL 82-10	Post-TMI Requirements	NA	NA		
GL 82-11	Transmittal of NUREG-0916 Relative to the Restart of R.E. Ginna Nuclear Power Plant	NA	NA		
GL 82-12	Nuclear Power Plant Staff Working Hours	NA	NA		
GL 82-13	Reactor Operator and Senior Reactor Operator Examinations	NA	NA		
GL 82-14	Submittal of Documents to the NRC	NA	NA		
GL 82-15	NEVER ISSUED	NA	NA		
GL 82-16	NUREG-0737 Technical Specifications	NA	NA		
GL 82-17	Inconsistency of Requirements Between 50.54(T) and 50.15	NA	NA		
GL 82-18	Reactor Operator and Senior Reactor Operator Requalification Examinations	NA	NA		
GL 82-19	Submittal of Copies of Documentation to NRC-Copy Requirements for Emergency Plans and Physical Security Plans	NA	NA		
GL 82-20	Guidance for Implementing the Standard Review Plan Rule	NA	NA		
GL 82-21	Fire Protection Audits	NA	NA		
GL 82-22	Congressional Request for Information Concerning Steam Generator Tube Integrity	NA	NA		
GL 82-23	Inconsistency Between Requirements of 10CFR 73.40(D) and Standard Tech Specs For Performing Audits of Safeguards Contingency	NA	NA		
GL 82-24	Safety Relief Valve Quencher Loads: BWR MARK II and III Containments	NA	NA		
GL 82-25	Integrated IAEA Exercise for Physical Inventory at LWRS	NA	NA		
GL 82-26	NUREG-0744, REV. 1 - Pressure Vessel Material Fracture Toughness	NA	NA		
GL 82-27	Transmittal of NUREG-0763 - Guidelines For Confirmatory In-Plant Tests of Safety-Relief Valve Discharge for BWR Plants	NA	NA		
GL 82-28	Inadequate Core Cooling Instrumentation System	O	O		
GL 82-29	This Generic Letter Was Never Issued	NA	NA		
GL 82-30	Filings Related to 10 CFR 50 Production and Utilization Facilities	NA	NA		
GL 82-31	NEVER ISSUED	NA	NA		
GL 82-32	Draft Steam Generator Report (SAI)	NA	NA		

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		TVA	NRC		
GL 82-33	Supplement to NUREG-0737, Requirements for Emergency Response Capability	CI	CI	NRC reviewed in SSER5,SSER6 & 18.2.2 of SSER15. Install SPDS prior start up	
GL 82-34	NEVER ISSUED	NA	NA		
GL 82-35	NEVER ISSUED	NA	NA		
GL 82-36	NEVER ISSUED	NA	NA		
GL 82-37	NEVER ISSUED	NA	NA		
GL 82-38	Meeting to Discuss Developments for Operator Licensing Examinations	NA	NA		
GL 82-39	Problems With Submittals of Subsequent Information of CURT 73.21 For Licensing Reviews	NA	NA		
GL 83-01	Operator Licensing Examination Site Visit	NA	NA		
GL 83-02	NUREG-0737 Technical Specifications	NA	NA		
GL 83-03	Regulatory Guide 1.150, "Ultrasonic Testing of Reactor Vessel Welds During Pre-Service and Inservice Examinations"	NA	NA		
GL 83-04	Regional Workshops Regarding Supplement 1 to NUREG-0737, Requirements For Emergency Response Capability	NA	NA		
GL 83-05	Safety Evaluation of "Emergency Procedure Guidelines, Revision 2," June 1982	NA	NA		
GL 83-06	Certificates and Revised Format For Reactor Operator and Senior Reactor Operator Licenses	NA	NA		
GL 83-07	The Nuclear Waste Policy Act of 1982	NA	NA		
GL 83-08	Modification of Vacuum Breakers on Mark I Containments	NA	NA		
GL 83-09	Review of Combustion Engineering Owners' Group Emergency Procedures Guideline Program	NA	NA		
GL 83-10a	Resolution of TMI Action items II.K.3.5	NA	NA		
GL 83-10b	Resolution of TMI Action items II.K.3.5	NA	NA		
GL 83-10c	Automatic Trip of Reactor Coolant Pumps (GL 83-10 a & b: CE, GL 83-10 d: W licensees, GL 83-10 e & f: B&W)	CI	CI	NRC letter 06/08/1990-Incorporate emergency response guidelines	
GL 83-10d	Resolution of TMI Action items II.K.3.5	NA	NA		
GL 83-10e	Resolution of TMI Action items II.K.3.5	NA	NA		
GL 83-10f	Resolution of TMI Action items II.K.3.5	NA	NA		
GL 83-11	Licensee Qualification for Performing Safety Analyses in Support of Licensing Actions	NA	NA		
GL 83-12	Issuance of NRC FORM 398 - Personal Qualifications Statement - Licensee	NA	NA		
GL 83-13	Clarification of Survell. Req's for HEPA Filters and Charcoal Absorber Units In STD. Tech Specs on ESF Cleanup Systems	NA	NA		
GL 83-14	Definition of "Key Maintenance Personnel," (Clarification of Generic Letter 82-12)	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 83-15	Implement. of Reg. Guide 1.150, "Ultrasonic Testing of RX Vessel Welds During Preservice & Inservice Examinations, REV.1	NA	NA		
GL 83-16	Transmittal of NUREG-0977 Relative to the ATWS Events at Salem Generating Station, Unit NO. 1	NA	NA		
GL 83-16a	Transmittal of NUREG-0977 Relative to the ATWS Events at Salem Generating Station, Unit NO. 1	NA	NA		
GL 83-17	Integrity of Requalification Examinations for Renewal of Reactor Operator and Senior Reactor Operator Licenses	NA	NA		
GL 83-18	NRC Staff Review of the BWR Owners' Group (BWROG) Control Room Survey Program	NA	NA		
GL 83-19	New Procedures for Providing Public Notice Concerning Issuance of Amendments To Operating Licenses	NA	NA		
GL 83-20	Integrated Scheduling for Implementation of Plant Modifications	NA	NA		
GL 83-21	Clarification of Access Control Procedures for Law Enforcement Visits	NA	NA		
GL 83-22	Safety Evaluation of "Emergency Response Guidelines"	NA	NA		
GL 83-23	Safety Evaluation of "Emergency Procedure Guidelines"	NA	NA		
GL 83-24	TMI Task Action Plan Item I.G.1, "Special Low Power Testing and Training," Recommendations for BWRS	NA	NA		
GL 83-25	Issuance of NRC Form 398 Personal Qualifications Statement - Licensee	NA	NA		
GL 83-26	Clarification Of Surveillance Requirements For Diesel Fuel Impurity Level Tests	NA	NA		
GL 83-27	Surveillance Intervals in Standard Technical Specifications	NA	NA		
GL 83-28	Required Actions Based on Generic Implications of Salem ATWS Events (Prior and future GLs, with the exception of				
GL 83-28	certain discrete scopes, have been screened into the following list):				
GL 83-28	1.2 - Post Trip Review Data and Information Capability	C	C	TVA letters dated 11/07/1983 & 12/04/1987. IR 391/86-04	ML072500568-ML072750564
GL 83-28	2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	CI	CI	NRC letter 10/20/1986 & 06/18/1990	ML072500568
GL 83-28	2.2 - Equipment Classification and Vendor Interface (All SR Components)	CI	CI		
GL 83-28	3.1 - Post-Maintenance Testing (Reactor Trip System Components)	CT	CT	NRC letter 12/10/1985; 10/27/1986 & 07/02/1990. IR 391/86-04	ML072610753
GL 83-28	3.2 - Post-Maintenance Testing (All SR Components)	CT	CT	NRC letter 12/10/1985; 10/27/1986 & 07/02/1990. IR 391/86-04	ML072610753

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 83-28	4.1 - Reactor Trip System Reliability (Vendor Related Modifications)	CI	CI		
GL 83-28	4.2 - Reactor Trip System Reliability (Preventive Maintenance and Surveillance Program for Reactor Trip Breakers)	CT	CT	NRC letters 07/26/1985 and 06/18/1992	ML072750564
GL 83-28	4.3 - Reactor Trip System Reliability (Automatic Actuation of Shunt Trip Attachment)	C	C	TVA letters dated 11/07/1983 & 03/22/1985. IR 50-391/86-04	ML072750564
GL 83-28	4.5 - Reactor Trip System Reliability (Automatic Actuation of Shunt Trip Attachment)	CT	CT	NRC letters 06/28/1990 and 10/09/1990	ML072610753
GL 83-29	This Generic Communication was not issued	NA	NA		
GL 83-30	Deletion of STD. Tech Spec Surveillance Requirement 4.8.1.1.2.d.6 For Diesel Generator Testing	NA	NA		
GL 83-31	Safety Evaluation of "Abnormal Transient Operating Guidelines"	NA	NA		
GL 83-32	NRC Staff Recommendations Regarding Operator Action for Reactor Trip and ATWS	NA	NA		
GL 83-33	NRC Positions on Certain Requirements of Appendix R to 10 CFR 50	NA	NA		
GL 83-34	Important to Safety	NA	NA		
GL 83-35	Clarification of TMI Action Plan Item II.K.3.31	NA	NA		
GL 83-36	NUREG-0737 Technical Specifications	NA	NA		
GL 83-37	NUREG-0737 Technical Specifications	NA	NA		
GL 83-38	NUREG-0965, "NRC Inventory of Dams"	NA	NA		
GL 83-39	Voluntary Survey of Licensed Operators	NA	NA		
GL 83-40	Operator Licensing Examination	NA	NA		
GL 83-41	Fast Cold Starts of Diesel Generators	NA	NA		
GL 83-42	Clarification to GL 81-07 Regarding Response to NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants"	NA	NA		
GL 83-43	Reporting Requirements of 10 CFR 50, Sections 50.72 and 50.73, and Standard Technical Specifications	NA	NA		
GL 83-44	Availability of NUREG-1021, "Operator Licensing Examiner Standards"	NA	NA		
GL 84-01	NRC Use Of The Terms "Important To Safety" and "Safety Related"	NA	NA		
GL 84-02	Notice of Meeting Regarding Facility Staffing	NA	NA		
GL 84-03	Availability of NUREG-0933, "A Prioritization of Generic Safety Issues"	NA	NA		
GL 84-04	Safety Evaluation of Westinghouse Topical Reports Dealing with Elimination of Postulated Pipe Breaks in PWR Primary Main Loops (Generic Letter 84-04)	NA	NA		

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		TVA	NRC		
GL 84-05	Change to NUREG-1021, "Operator Licensing Examiner Standards"	NA	NA		
GL 84-06	Operator and Senior Operator License Examination Criteria For Passing Grade	NA	NA		
GL 84-07	Procedural Guidance for Pipe Replacement at BWRs	NA	NA		
GL 84-08	Interim Procedures for NRC Management of Plant-Specific Backfitting	NA	NA		
GL 84-09	Recombiner Capability Requirements of 10 CFR 50.44(c)(3)(ii)	NA	NA		
GL 84-10	Administration of Operating Tests Prior to Initial Criticality	NA	NA		
GL 84-11	Inspections of BWR Stainless Steel Piping	NA	NA		
GL 84-12	Compliance W 10CFR61 and Implementation of Rad Effluent Tech Specs, Attendant Process Control Program	NA	NA		
GL 84-13	Technical Specification for Snubbers	NA	NA		
GL 84-14	Replacement and Regualification Training Program	NA	NA		
GL 84-15	Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability	NA	NA		
GL 84-16	Adequacy of On-Shift Operating Experience for Near Term Operating License Applicants	NA	NA		
GL 84-17	Annual Meeting to Discuss Recent Developments Regarding Operator Training, Qualifications, and Examinations	NA	NA		
GL 84-18	Filing of Applications for Licenses and Amendments	NA	NA		
GL 84-19	Availability of Supplement 1 to NUREG-0933, "A Prioritization of Generic Safety Issues"	NA	NA		
GL 84-20	Scheduling Guidance for Licensee Submittals of Reloads That Involve Unreviewed Safety Questions	NA	NA		
GL 84-21	Long Term Low Power Operation in Pressurized Water Reactors	NA	NA		
GL 84-22	10 CFR 20.408 Termination Reports - Format	NA	NA		
GL 84-23	Reactor Vessel Water Level Instrumentation in BWRs	NA	NA		
GL 84-24	Cert of Compliance to 10CFR50.49, Eq of Electric Equipment Important to Safety for Npps	CI	CI	See Special Program for Environmental Qualification	
GL 85-01	Fire Protection Policy Steering Committee Report	NA	NA		
GL 85-02	Staff Recommended Actions Regarding Steam Generator Tube Integrity	CI	CI	TVA letter 06/17/1985-Perform SG inspection	
GL 85-03	Clarification of Equivalent Control Capacity for Standby Liquid Control Systems	NA	NA		
GL 85-04	Operating Licensing Examinations	NA	NA		
GL 85-05	Inadvertent Boron Dilution Events	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 85-06	Quality Assurance Guidance for ATWS Equipment That Is Not Safety-Related	NA	NA		
GL 85-07	Implementation of Integrated Schedules for Plant Modifications	NA	NA		
GL 85-08	10 CFR 20.408 Termination Reports - Format	NA	NA		
GL 85-09	Technical Specifications For Generic Letter 83-28, Item 4.3	NA	NA		
GL 85-10	Technical Specification For Generic Letter 83-28, Items 4.3 and 4.4	NA	NA		
GL 85-11	Completion of Phase II of "Control Of Heavy Loads At Nuclear Power Plants" NUREG-0612	C	C	Verified. No response required.	
GL 85-12	Implementation Of TMI Action Item II.K.3.5, "Automatic Trip Of Reactor Coolant Pumps	CI	CI	Reviewed in 15.5.4 of 1982 SER; became license condition #35.Resolved SSER4	
GL 85-13	Transmittal Of NUREG-1154 Regarding The Davis-Besse Loss Of Main And Auxiliary Feedwater Event	NA	NA		
GL 85-14	Commercial Storage At Power Reactor Sites Of Low Level Radioactive Waste Not Generated By The Utility	NA	NA		
GL 85-15	Information On Deadlines For 10CFR50.49, "EQ Of Electric Equipment Important To Safety At Npps"	NA	NA		
GL 85-16	High Boron Concentrations	NA	NA		
GL 85-17	Availability Of Supplements 2 and 3 To NUREG-0933, "A Prioritization Of Generic Safety Issues"	NA	NA		
GL 85-18	Operator Licensing Examinations	NA	NA		
GL 85-19	Reporting Requirements On Primary Coolant Iodine Spikes	NA	NA		
GL 85-20	Resolution Of Generic Issue 69: High Pressure Injection/Make-up Nozzle Cracking In Babcock And Wilcox Plants	NA	NA		
GL 85-21	This Generic Communication was not issued	NA	NA		
GL 85-22	Potential For Loss Of Post-LOCA Recirculation Capability Due To Insulation Debris Blockage	NA	NA		
GL 86-01	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	NA	NA		
GL 86-02	Technical Resolution of Generic Issue B-19 - Thermal Hydraulic Stability	NA	NA		
GL 86-03	Applications For License Amendments	NA	NA		
GL 86-04	Policy Statement On Engineering Expertise On Shift	NA	A-O	Applies to O.L.A and requires actions. Additional info needed.	
GL 86-05	Implementation Of TMI Action Item II.K.3.5, "Automatic Trip Of Reactor Coolant Pumps"	NA	NA		
GL 86-06	Implementation Of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps"	NA	NA		

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		TVA	NRC		
GL 86-07	Transmittal of NUREG-1190 Regarding The San Onofre Unit 1 Loss of Power And Water Hammer Event	NA	NA		
GL 86-08	Availability of Supplement 4 to NUREG-0933, "A Prioritization of Generic Safety Issues"	NA	NA		
GL 86-09	Technical Resolution of Generic Issue B-59, (n-1) Loop Operation in BWRs and PWRs	CT	CT	Confirm Tech Specs prohibit (N-1) Loop Operation	
GL 86-10	Implementation of Fire Protection Requirements	NA	NA		
GL 86-10S1	Fire Endurance Test Acceptance Criteria for Fire Barrier Systems Used to Separate Redundant Safe shutdown Trains within same fire area.	NA	NA		
GL 86-11	Distribution of Products Irradiated in Research Reactors	NA	NA		
GL 86-12	Criteria for Unique Purpose Exemption From Conversion From The Use of Heu Fuel	NA	NA		
GL 86-13	Potential Inconsistency Between Plant Safety Analyses and Technical Specifications	NA	NA		
GL 86-14	Operator Licensing Examinations	NA	NA		
GL 86-15	Info ... Compliance W 10CFR50.49 "Eq of Electric Equipment Important to Safety For Nuclear Power Plants"	NA	NA		
GL 86-16	Westinghouse ECCS Evaluation Models	NA	NA		
GL 86-17	Availability of NUREG-1169, "Technical Findings Related to Generic Issue C-8, BWR MSIC Leakage And Treatment Methods"	NA	NA		
GL 87-01	Public Availability Of The NRC Operator Licensing Examination Question Bank	NA	NA		
GL 87-02	Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors, USI A-46	NA	NA		
GL 87-02s1	Transmits SSER No. 2 on SQUG Generic Implementation Procedure, Revision 2, as Corrected February 14, 1992 (GIP-2)	NA	NA		
GL 87-03	Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors, USI A-46	NA	NA		
GL 87-04	Temporary Exemption From Provisions Of The FBI Criminal History Rule For Temporary Workers	NA	NA		
GL 87-05	Request for Additional Information on Assessment of License Measures to	NA	NA		
	Mitigate and/or Identify Potential Degradation of Mark I Drywells				
GL 87-06	Periodic Verification of Leak Tight Integrity of Pressure Isolation Valves	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 87-07	Information Transmittal of Final Rulemaking For Revisions To Operator Licensing - 10 CFR 55 And Confirming Amendments	NA	NA		
GL 87-08	Implementation of 10 CFR 73.55 Miscellaneous Amendments and Search Requirements	NA	NA		
GL 87-09	Sections 3.0 And 4.0 of Standard Tech Specs on Limiting Conditions For Operation And Surveillance Requirements	NA	NA		
GL 87-10	Implementation of 10 CFR 73.57, Requirements For FBI Criminal History Checks	NA	NA		
GL 87-11	Relaxation in Arbitrary Intermediate Pipe Rupture Requirements	NA	NA		
GL 87-12	Loss of Residual Heat Removal While The Reactor Coolant System is Partially Filled	C	C	Superseded by GL88-17	
GL 87-13	Integrity requalification Examinations at Non-Power Reactors	NA	NA		
GL 87-14	Operator Licensing Exams	NA	NA		
GL 87-15	Policy Statement on deferred plants	NA	NA		
GL 87-16	Transmittal of NUREG-1262, "Answers to Questions On Implementation of 10CFR 55 on Operator's Licenses"	NA	NA		
GL 88-01	NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping	NA	NA		
GL 88-02	Integrated Safety Assessment Program II (ISAP II)	NA	NA		
GL 88-03	Resolution of GSI 93, Steam Binding of Auxiliary Feedwater Pumps	CI	CI	See App EE of SSER 16	
GL 88-04	Distribution of Gems Irradiated in Research Reactors	NA	NA		
GL 88-05	Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWR plants	CI	CI	NRC Acceptance letter 08/08/1990	
GL 88-06	Removal of Organization Charts from Technical Specification Administrative Control Requirements	NA	NA		
GL 88-07	Modified Enforcement Policy Relating to 10 CFR 50.49, EQ of Electrical Equipment Important to Safety for Nuclear Power Plants	CI	CI	See Special Program for Environmental Qualification	
GL 88-08	Mail Sent or Delivered to the Office of Nuclear Reactor Regulation	NA	NA		
GL 88-09	Pilot Testing of Fundamentals Examination	NA	NA		
GL 88-10	Purchase of GSA Approved Security Containers	NA	NA		
GL 88-11	NRC Position on Radiation Embrittlement of Reactor Vessel Material and its Impact on Plant Operations	CI	CI	NRC Acceptance letter 06/29/1989 for both Units.	
GL 88-12	Removal of Fire Protection Requirements from Technical Specification	NA	NA		
GL 88-13	Operator Licensing Examinations	NA	NA		

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		TVA	NRC		
GL 88-14	Instrument Air Supply System Problems Affecting Safety-Related Equipment	CI	CI	NRC letter 07/26/1990. Unit 2 action: Complete implementation.	
GL 88-15	Electric Power Systems - Inadequate Control Over Design Process	NA	NA		
GL 88-16	Removal of Cycle-Specific Parameter Limits from Technical Specifications	NA	NA		
GL 88-17	Loss of Decay Heat Removal	CI	CI	Implement mods to provide RCS temp, RV level and RHR system performance.	ML073230823
GL 88-18	Plant Record Storage on Optical Disks	NA	NA		
GL 88-19	Use of Deadly Force by Licensee Guards to Prevent Theft of Special Nuclear Material	NA	NA		
GL 88-20	Individual Plant Examination for Severe Accident Vulnerability	O	O		
GL 88-20s1	Initiation of the Individual Plant Examination for Severe Accident Vulnerabilities - 10 CFR 50.54	O	O		
GL 88-20s2	Accident Management Strategies for Consideration in the Individual Plant Examination Process	O	O		
GL 88-20s3	Completion of Containment Performance Improvement Program and Forwarding of Insights for Use in the IPE for SAV	O	O		
GL 88-20s4	Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities	O	O		
GL 88-20s5	Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities - 10 CFR 50.54(f)	O	O		
GL 89-01	Implementation of Programmatic and Procedural Controls for Radiological Effluent Technical Specifications	NA	NA		
GL 89-02	Actions to Improve the Detection of Counterfeit and Fraudulently Marketed Products	NA	A-O	N.R.R but actions should be taken. Applies to all C.P holders	
GL 89-03	Operator Licensing Examination Schedule	NA	NA		
GL 89-04	Guidelines on Developing Acceptable Inservice Testing Programs	OV	OV		
GL 89-05	Pilot testing of the Fundamentals Examination	NA	NA		
GL 89-06	TMI Action Plan Item I.D.2 - Safety Parameter Display System	CI	CI	NRC reviewed in SSER5,SSER6 & SSER15(18.2.2)-Install SPDS prior start up	
GL 89-07	Power Reactor Safeguards Contingency Planning for Surface Vehicle Bombs	C	C	TVA letter dated 10/31/1989 & NRC memo 06/26/1990. IR50-391/95-70	ML072610753
GL 89-08	Erosion/Corrosion-Induced Pipe Wall Thinning	CI	CI	Prepare procedure & perform baseline inspections	
GL 89-09	ASME Section III Component Replacements	NA	NA		
GL 89-10	Safety-Related Motor-Operated Valve Testing and Surveillance	CI	CI	NRC accepted review 09/14/1990 and reviewed App. SSER16.	

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		TVA	NRC		
GL 89-10s1	Results of the Public Workshops	CI	CI	NRC accepted review 09/14/1990 and reviewed App. SSER16.	
GL 89-10s2	Availability of Program Descriptions	CI	CI	NRC accepted review 09/14/1990 and reviewed App. SSER16.	
GL 89-10s3	Consideration of the Results of NRC-Sponsored Tests of Motor-Operated Valves	CI	CI	NRC accepted review 09/14/1990 and reviewed App. SSER16.	
GL 89-10s4	Consideration of Valve Mispositioning in Boiling Water Reactors	CI	CI	NRC accepted review 09/14/1990 and reviewed App. SSER16.	
GL 89-10s5	Inaccuracy of Motor-Operated Valve Diagnostic Equipment	CI	CI	NRC accepted review 09/14/1990 and reviewed App. SSER16.	
GL 89-10s6	Information on Schedule and Grouping, and Staff Responses to Additional Public Questions	CI	CI	NRC accepted review 09/14/1990 and reviewed App. SSER16.	
GL 89-10s7	Consideration of Valve Mispositioning in Pressurized-Water Reactors	CI	CI	NRC accepted review 09/14/1990 and reviewed App. SSER16.	
GL 89-11	Resolution of Generic Issue 101 "Boiling Water Reactor Water Level Redundancy"	NA	NA		
GL 89-12	Operator Licensing Examination	NA	NA		
GL 89-13	Service Water System Problems Affecting Safety Related Equipment	CI	CI	NRC letters 07/09/1990 & 06/13/1997	
GL 89-14	Line-Item Improvements in Technical Specifications - Removal of 3.25 Limit on Extending Surveillance Intervals	NA	NA		
GL 89-15	Emergency Response Data System	NA	C		ML072610803
GL 89-16	Installation of a Hardened Wetwell Vent	NA	NA		
GL 89-17	Planned Administrative Changes to the NRC Operator Licensing Written Examination Process	NA	NA		
GL 89-18	Resolution of Unresolved Safety Issues A-17, "Systems Interactions in Nuclear Power Plants"	NA	NA		
GL 89-19	Request for Actions Related to Resolution of Unresolved Safety Issue A-47 "Safety Implication of Control Systems in LWR Nuclear Power Plants" Pursuant to 10 CFR 50.54(f)	CI	CI	NRC acceptance letter 10/24/1990 Perform evaluation of common mode failures due to fire	
GL 89-20	Protected Area Long Term Housekeeping	NA	NA		
GL 89-21	Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements	NA	A-O	NRC requests review and subsequent report	
GL 89-22	Potential For Increased Roof Load Due to Changes in Maximum Precipitation	C	C	TVA letter dated 12/16/1981	
GL 89-23	NRC Staff Responses to Questions Pertaining to Implementation of 10 CFR Part 26	NA	NA		

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 90-01	Request for Voluntary Participation in NRC Regulatory Impact Survey	NA	NA		
GL 90-02	Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications	NA	NA		
GL 90-02s1	Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications	NA	NA		
GL 90-03	Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2 "Vendor Interface for Safety-Related Components"	NA	NA		
GL 90-03s1	Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2, "Vendor Interface for Safety-Related Components"	NA	NA		
GL 90-04	Request for Information on the Status of Licensee Implementation of GSIs Resolved with Imposition of Requirements or CAs	CI	CI	NRC letter 12/09/1994. Actions: Implement modification and testing.	
GL 90-05	Guidance for Performing Temporary Non-Code Repair of ASME Code Class 1, 2, and 3 Piping	NA	NA		
GL 90-06	Resolution of Generic Issues 70, "PORV and Block Valve Reliability," and 94, "Additional LTOP Protection for PWRs"	CT	CT	NRC letter 01/09/1991 accepted TVA's response for both units.	
GL 90-07	Operator Licensing National Examination Schedule	NA	NA		
GL 90-08	Simulation Facility Exemptions	NA	NA		
GL 90-09	Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions	NA	NA		
GL 91-01	Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specifications	NA	NA		
GL 91-02	Reporting Mishaps Involving LLW Forms Prepared for Disposal	NA	NA		
GL 91-03	Reporting of Safeguards Events	NA	NA		
GL 91-04	Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle	NA	NA		
GL 91-05	Licensee Commercial-Grade Procurement and Dedication Programs	NA	NA		
GL 91-06	Resolution of Generic Issue A-30, "Adequacy of Safety Related DC Power Supplies"	NA	NA		
GL 91-07	GI-23, "Reactor Coolant Pump Seal Failures" and Its Possible Effect on Station Blackout	NA	NA		
GL 91-08	Removal of Component Lists from Technical Specifications	NA	NA		
GL 91-09	Modification of Surveillance Interval for the Electrical Protective Assemblies in Power Supplies for the Reactor Protection System	NA	NA		
GL 91-10	Explosives Searches at Protected Area Portals	NA	NA		

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

Table 1. Generic Communications Matrix Master Table

Rev.1

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 91-11	Resolution of Generic Issues A-48, "LCOs for Class 1E Vital Instrument Buses", and 49, "Interlocks and LCOs for 1E Tie Breakers"	NA	NA		
GL 91-12	Operator Licensing National Examination Schedule	NA	NA		
GL 91-13	Request for Information Related to Resolution of Generic Issue 130, "Essential Service Water System Failures @ Multi-Unit Sites"	NA	NA		
GL 91-14	Emergency Telecommunications	NA	NA		
GL 91-15	Operating Experience Feedback Report, Solenoid-Operated Valve Problems at U.S. Reactors	NA	NA		
GL 91-16	Licensed Operators' and Other Nuclear Facility Personnel Fitness for Duty	NA	NA		
GL 91-17	Generic Safety Issue 29, "Bolting Degradation or Failure in Nuclear Power Plants"	NA	NA		
GL 91-18	Information to Licensees Regarding Two NRC Inspection Manual Sections on Resolution of Degraded and Nonconforming Conditions and on Operability [GL 91-18 has been superseded RIS 2005-20]	NA	NA	Superseded by RIS 2005-20	
GL 91-18r1	NRC Generic Letter 91-18, Revision 1: Information to Licensees Regarding NRC Inspection Manual Section on Resolution of Degraded and Nonconforming Conditions [GL 91-18 R1 has been superseded RIS 2005-20]	NA	NA	Superseded by RIS 2005-20	
GL 91-19	Information to Addressees Regarding New Telephone Numbers for NRC Offices Located in One White Flint North	NA	NA		
GL 92-01	Reactor Vessel Structural Integrity	NA	NA		
GL 92-01r1		NA	NA		
GL 92-01r1s1		NA	NA		
GL 92-02	Resolution of Generic Issue 79, "Unanalyzed Reactor Vessel (PWR) Thermal Stress During Natural Convection Cooldown"	NA	NA		
GL 92-03	Compilation of the Current Licensing Basis: Request for Voluntary Participation in Pilot Program	NA	NA		
GL 92-04	Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs Pursuant to 10 CFR 50.54(f)	NA	NA		
GL 92-05	NRC Workshop on the Systematic Assessment of Licensee Performance (SALP) Program	NA	NA		
GL 92-06	Operator Licensing National Examination Schedule	NA	NA		
GL 92-07	Office of Nuclear Reactor Regulation Reorganization	NA	NA		
GL 92-08	Thermo-Lag 330-1 Fire Barriers	OV	OV		

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 92-09	Limited Participation by NRC in the IAEA International Nuclear Event Scale	NA	NA		
GL 93-01	Emergency Response Data System Test Program	NA	C	IR 390/391-95-78	ML072610803
GL 93-02	NRC Public Workshop on Commercial Grade Procurement and Dedication	NA	NA		
GL 93-03	Verification of Plant Records	NA	NA		
GL 93-04	Rod Control System Failure and Withdrawal of Rod Control Cluster Assemblies, 10 CFR 50.54(f)	CI	CI	NRC letter dated 12/09/1994	
GL 93-05	Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation	NA	NA		
GL 93-06	Research Results on Generic Safety Issue 106, "Piping and the Use of Highly Combustible Gases in Vital Areas"	NA	NA		
GL 93-07	Modification of the Technical Specification Administrative Control Requirements for Emergency and Security Plans	NA	NA		
GL 93-08	Relocation of Technical Specification Tables of Instrument Response Time Limits	NA	NA		
GL 94-01	Removal of Accelerated Testing and Special Reporting Requirements for Emergency Diesel Generators	NA	NA		
GL 94-02	Long Term Solutions and Upgrade of Interim Operating Recommendations for Thermal/Hydraulic Instabilities in BWRs	NA	NA		
GL 94-03	IGSCC of Core Shrouds in BWRs	NA	NA		
GL 94-04	Voluntary Reporting of Additional Occupational Radiation Exposure Data	NA	NA		
GL 95-01	NRC Staff Technical Position on Fire Protection for Fuel Cycle Facilities	NA	NA		
GL 95-02	Use of Numarc/epri Report TR-102348, "Guideline on Licensing Digital Upgrades," in Determining the Acceptability of Performing Analog-to-Digital Replacements under 10 CFR 50.59	NA	NA		
GL 95-03	Circumferential cracking of Steam Generator (SG) Tubes	OV	OV		
GL 95-04	Final Disposition of the Systematic Evaluation Program Lessons-Learned Issues	NA	NA		
GL 95-05	Voltage Based Repair Criteria for W SG Tubes Affected by Outside Diameter Stress Corrosion Cracking	C	C	N.R.R. TVA letter 09/07/2007	
GL 95-06	Changes in the Operator Licensing Program	NA	NA		
GL 95-07	Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves	CI	CI	Evil. pressure locking & thermal binding of safety related power gate valves	
GL 95-08	10 CFR 50.54(p) Process for Changes to Security Plans Without Prior NRC Approval	NA	NA		

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
GL 95-09	Monitoring and Training of Shippers and Carriers of Radioactive Materials	NA	NA		
GL 95-10	Relocation of Selected Technical Specifications Requirements Related to Instrumentation	NA	NA		
GL 96-01	Testing of Safety-Related Circuits	CI	CI	TVA responded 04/18/1996. Implement recommendations	
GL 96-02	Reconsideration of Nuclear Power Plant Security Requirements Associated with an Internal Threat	NA	NA		
GL 96-03	Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits	CI	CI	N.R.R.-Submit temp. pressure limits & incorporate into licensee-controlled doc.	
GL 96-04	Boraflex Degradation in Spent Fuel Pool Storage Racks	NA	NA		
GL 96-05	Periodic Verification of Design-Basis Capability of Safety-Related Motor-Operated Valves	CI	CI	TVA response on 07/21/1999-Implement joint owner's group MOV PV program	
GL 96-06	Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions	OV	OV	NRC letter 04/06/1999-Implement & provide containment penetration relief.	
GL 96-06s1	NRC Generic Letter 96-06, Supplement 1: Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions	OV	OV	NRC letter 04/06/1999-Implement & provide containment penetration relief.	
GL 96-07	Interim Guidance on Transportation of Steam Generators	NA	NA		
GL 97-01	Degradation of Control Rod Drive Mechanism Nozzle and Other Vessel Closure Head Penetrations	OV	OV		
GL 97-02	Revised Contents of the Monthly Operating Report	NA	NA		
GL 97-03	Annual Financial Update of Surety Requirements for Uranium Recovery Licensees	NA	NA		
GL 97-04	Assurance of Sufficient Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal Pumps	OV	OV		
GL 97-05	SG Tube Inspection Techniques	OV	OV		
GL 97-06	Degradation of SG Internals	OV	OV		
GL 98-01	Year 2000 Readiness of Computer Systems at Nuclear Power Plants	NA	NA	IR 50-390/391 99-09	ML993620597
GL 98-02	Loss of Reactor Coolant Inventory and Associated Potential for Loss of Emergency Mitigation Functions While in a Shutdown Condition	OV	OV		
GL 98-03	NRC Generic Letter 98-03: NMSS Licensees' and Certificate Holders' Year 2000 Readiness Programs	NA	NA		
GL 98-04	Potential for Degradation of the ECCS and the Containment Spray System After a	OV	OV		

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
	LOCA Because of Construction and Protective Coating Deficiencies and Foreign Material in Containment				
GL 98-05	NRC Generic Letter 98-05: Boiling Water Reactor Licensees Use of the BWRVIP-05 Report to Request Relief from Augmented Examination Requirements on Reactor Pressure Vessel Circumferential Shell Welds	NA	NA		
GL 99-01	NRC Generic Letter 99-01: Recent Nuclear Material Safety and Safeguards Decision on Bundling Exempt Quantities	NA	NA		
GL 99-02	Laboratory Testing of Nuclear Grade Activated Charcoal	NA	NA	WB closeout letter	ML003681388
GL 03-01	Control Room Habitability	OT	OT		
GL 04-01	Requirements for SG Tube Inspection	OV	OV		
GL 04-02	Potential Impact of Debris Blockage on Emergency Recirculation during Design Basis Accidents at PWRs	OV	OV		
GL 06-01	SG Tube Integrity and Associated Technical Specifications	OT	OT		
GL 06-02	Grid Reliability and the Impact on Plant Risk and the Operability of Offsite Power	OV	OV		
GL 06-03	Potentially Nonconforming Hemyc and MT Fire Barrier Configurations	OV	OV		
GL 07-01	Independent Safety Engineering Group	OV	OV	Complete testing of four additional cables.	
GL 07-02	Managing Gas Accumulation in Emergency Core Cooling, Decay Heat Removal, and Containment Spray systems	O	O		
GL 08-01	Inaccessible or Underground Power Cable Failures that Disable Accident Mitigation Systems or Cause Plant Transients	O	O		
NUREG 0737	TMI Items (TVA letter dated September 14, 1981 applies to all):				
NUREG 0737	I.A.1.3 Shift Manning	C	C	Closed in SSER 16.	
NUREG 0737	I.A.2.1 Immediate upgrade of RO and SRO Training and qualifications	C	C	Closed in SSER 16.	
NUREG 0737	I.A.2.3 Administration of Training programs	C	C	Closed in SSER 16.	
NUREG 0737	I.A.3.1 Revise scope and criteria for licensing exams	C	C	Closed in SSER 16.	
NUREG 0737	I.B.1.2 Independent Safety Engineering Group	OV	OV		
NUREG 0737	I.C.1 Short term accident and procedure review	CI	CI	NRC reviewed App. EE of SSER16. Implement Emergency Operating Procedures	
NUREG 0737	I.C.2 Shift and relief turnover procedures	C	C	Closed in SSER 16.	
NUREG 0737	I.C.3 Shift supervisor responsibility	C	C	Closed in SSER 16.	
NUREG 0737	I.C.4 Control Room Access	C	C	Closed in SSER 16.	
NUREG 0737	I.C.5 Feedback of Operating experience	C	C	Closed in SSER 16.	
NUREG 0737	I.C.6 Verify correct performance of operating activities	C	C	Closed in SSER 16.	

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
NUREG 0737	I.C.7 NSSS vendor revision of procedures	CI	CI	Unit 2: Revise power ascension and emergency procedures	
NUREG 0737	I.C.8 Pilot monitoring of selected emergency procedures for NTOLs	CI	CI	Pilot monitor selected emergency procedures for NTOL	
NUREG 0737	I.D.1 CRDR	OV	OV		
NUREG 0737	I.D.2 Plant-safety-parameter-display console	CI	CI	NRC reviewed in SSER5,SSER6 & 18.2.2 of SSER15	
NUREG 0737	I.G.1 Training during low power testing	C	C	Closed in SSER 16.	
NUREG 0737	II.B.1 Reactor-coolant-vent-system	CI	CI	Verify installation of reactor coolant vents	
NUREG 0737	II.B.2 Plant shielding	CI	CI	Complete design review of EQ of equipment for spaces/systems of post accident	
NUREG 0737	II.B.3 Post-accident sampling	CT	CT	NRC reviewed in 9.3.2 of SSER16.	
NUREG 0737	II.B.4 Training for mitigating core damage	C	C	Closed in SSER 16.	
NUREG 0737	II.D.1 Relief and safety valve test requirements	CI	CI	NRC reviewed in Tech. Eval. Report attached to App. EE of SSER15	
NUREG 0737	II.D.3 Valve position indication	CI	CI	Verify installation of the acoustic monitoring system to PORV to indicate position	
NUREG 0737	II.E.1.1 Auxiliary feedwater system evaluation, modifications	CI	CI	Perform auxiliary feedwater system analysis. It pertains to syst. failure & flow rate	
NUREG 0737	II.E.1.2 Auxiliary feedwater system initiation and flow	CI	CI	IR 50-391/84-16 & letters dated 03/29/1985 & 10/31/1995	
NUREG 0737	II.E.3.1 Emergency power for pressurizer heaters	CI	CI	NRC letters 03/29/1985 & 10/31/1995- Implement procedures & testing	
NUREG 0737	II.E.4.1 Dedicated Hydrogen Penetrations	C	C	IR 50-391/83-19	
NUREG 0737	II.E.4.2 Containment isolation dependability	CT	CT	NRC letters 03/29/1985; 07/12/1990 & 10/31/1995	
NUREG 0737	II.F.1.2. Accident-monitoring instrumentation :				
NUREG 0737	A. - Noble gas	CI	CI	Install noble gas, iodine/particulate sampling & containment High Range monitors	
NUREG 0737	B. - Iodine/particulate sampling	CI	CI	Install noble gas, iodine/particulate sampling & containment High Range monitors	
NUREG 0737	C. - Containment high range monitoring	CI	CI	Install noble gas, iodine/particulate sampling & containment High Range monitors	
NUREG 0737	D. - Containment pressure	CI	CI	Verify installation of containment water level monitors	
NUREG 0737	E. - Containment water level	CI	CI	Verify installation of containment water level monitors	
NUREG 0737	F. - Containment hydrogen	CI	CI	Verify installation of containment hydrogen accident monitoring instrumentation	

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
NUREG 0737	II.F.2 Instrumentation for detection of inadequate core-cooling	O	O		
NUREG 0737	II.G.1 Power supplies for pressurizer relief valves, block valves and level indicators	CI	CI	Implement mods. So PORV & block valves powered same train but different bus	
NUREG 0737	II.K.1.5 Review ESF valves	C	C	NRC letter dated 03/29/1985	
NUREG 0737	II.K.1.10 Operability status	CI	CI	Confirm multi unit operation will have no impact on administrative procedures	
NUREG 0737	II.K.1.17 Trip per low-level B/S	C	C	NRC letter dated 03/29/1985	
NUREG 0737	II.K.2.13 Effect of High Pressure Injection for small Break LOCA with no Auxiliary Feedwater	C	C	In SSER4 staff concluded vessel integrity would be maintained for small breaks.	
NUREG 0737	II.K.2.17 Voiding in the reactor Coolant System	C	C	LICENSE CONDITION-Voiding the reactor coolant system.	
NUREG 0737	II.K.3.1 Auto PORV isolation	C	C	Reviewed in SSER5 and NRC letter 06/129/1990	
NUREG 0737	II.K.3.2 Report on PORV failures	C	C	Reviewed on SSER5 & NRC letter dated 06/29/1990	
NUREG 0737	II.K.3.3 Reporting SV/RV failures/challenges	CT	CT	NRC reviewed in App EE of SSER16	
NUREG 0737	II.K.3.5 Auto trip of RCPs	CI	CI	License condition #35. Item reviewed in App. EE of SSER 16.	
NUREG 0737	II.K.3.9 PID controller	CI	CI	Reviewed in original 1982 SER. Set derivative time constant to zero.	
NUREG 0737	II.K.3.10 Anticipatory trip at high power	CT	CT	NRC letter 10/31/1995; SSER 16	
NUREG 0737	II.K.3.12 Confirm existence of anticipatory reactor trip upon turbine trip	C	C	Closed in SSER16	
NUREG 0737	II.K.3.17 ECCS outages	C	C	In SSER3 NRC accepted commitment from 10/28/1983 letter.	
NUREG 0737	II.K.3.25 Power on pump seals	CI	CI	Ensure DG power is provided to pump sealing cooling system	
NUREG 0737	II.K.3.30 SB LOCA methods	CI	CI	NRC letters dated 03/29/1985 & 07/24/1986	
NUREG 0737	II.K.3.31 Plant specific analysis	CI	CI	Complete analysis for unit 2.	
NUREG 0737	III.A.1.1 Emergency Preparedness, Short term	C	C	NRC completed review of REP in SSER20	
NUREG 0737	III.A.1.2 Upgrade Emergency Support facilities	C	C	NRC completed review of REP in SSER20	
NUREG 0737	III.A.2 Emergency Preparedness	C	C	NRC completed review of REP in SSER20	
NUREG 0737	III.D.1.1 Primary coolant outside containment	OT	OT		
NUREG 0737	III.D.3.3 In-plant I2 radiation monitoring	CI	CI	NRC reviewed in App. EE of SSER16	
NUREG 0737	III.D.3.4 Control-room habitability	OV	OV		

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

Table 2. Generic Communications Status-Open (A-O & OT also Included)

Rev.1

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Item	Title	Position		Additional Comments	ADAMS
		TVA	NRC		Accession Number
cr79023	Motor Starters and Contactors Failed to Operate	NA	A-O	N.R.R But defective shipment was received and should be addressed	
cr79025	Shock Arrestor Strut Assembly Interference	NA	A-O	N.R.R but inspections & C.A expected on behalf of WBN 2	
cr81013	Torque Switch Electrical Bypass Circuit for Safeguard Service Valve Motors	NA	A-O	Report required if discrepancies found. Clarify if discrepancies were found.	
GL 79-24	Multiple Equipment Failures in Safety-Related Systems	NA	A-O	Applies to ALL PWR's. Requests submittal of reports	
GL 79-51	Follow-up Actions Resulting from the NRC Staff Reviews Regarding the TMI-2 Accident	NA	A-O	Applies to all C.P holders and requires implementation of actions	
GL 79-63	Upgraded Emergency Plans	NA	A-O	Check Applicability. Header (yes) content (No). Request add. Info.	
GL 80-11	IEB 80-03 Loss Of Charcoal From Standard Type II, 2 Inch, Tray Absorber Cells	NA	A-O	Requires action on behalf of C.P holder	
GL 80-28	IEB 80-08 Examination Of Containment Liner Penetration Welds	NA	A-O	Info expected to be submitted 90 days after issuance	
GL 80-32	Information Request on Category I Masonry Walls Employed by Plants Under CP and OL Review	NA	A-O	B80-11 requests actions from all C.P facilities	
GL 80-92	IEB 80-21 Valve Yokes Supplied By Malcolm Foundry Company, Inc.	NA	A-O	Report was to be submitted within 60 days	
GL 86-04	Policy Statement On Engineering Expertise On Shift	NA	A-O	Applies to O.L.A and requires actions. Additional info needed.	
GL 89-02	Actions to Improve the Detection of Counterfeit and Fraudulently Marketed Products	NA	A-O	N.R.R but actions should be taken. Applies to all C.P holders	
GL 89-21	Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements	NA	A-O	NRC requests review and subsequent report	
GL 82-28	Inadequate Core Cooling Instrumentation System	O	O		
GL 88-20	Individual Plant Examination for Severe Accident Vulnerability	O	O		
GL 88-20s1	Initiation of the Individual Plant Examination for Severe Accident Vulnerabilities - 10 CFR 50.54	O	O		

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

Table 2. Generic Communications Status-Open (A-O & OT also Included)

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Item	Title	Position		Additional Comments	ADAMS
		TVA	NRC		Accession Number
GL 88-20s2	Accident Management Strategies for Consideration in the Individual Plant Examination Process	O	O		
GL 88-20s3	Completion of Containment Performance Improvement Program and Forwarding of Insights for Use in the IPE for SAV	O	O		
GL 88-20s4	Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities	O	O		
GL 88-20s5	Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities - 10 CFR 50.54(f)	O	O		
GL 07-02	Managing Gas Accumulation in Emergency Core Cooling, Decay Heat Removal, and Containment Spray systems	O	O		
GL 08-01	Inaccessible or Underground Power Cable Failures that Disable Accident Mitigation Systems or Cause Plant Transients	O	O		
NUREG 0737	II.F.2 Instrumentation for detection of inadequate core-cooling	O	O		
GL 03-01	Control Room Habitability	OT	OT		
GL 06-01	SG Tube Integrity and Associated Technical Specifications	OT	OT		
NUREG 0737	III.D.1.1 Primary coolant outside containment	OT	OT		

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

Table 3. Generic Communication Status-Open for Validation

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Item	Title	Position		Additional Comments	ADAMS Accession Number
		TVA	NRC		
IEB 92-01	Failure of Thermo-Lag 330 Fire Barrier System to Perform its Specified Fire Endurance Function	OV	OV		
IEB 92-01s1	Failure of Thermo-Lag 330 Fire Barrier System to Perform its Specified Fire Endurance Function	OV	OV		
IEB 96-01	Control Rod Insertion Problems (PWR)	OV	OV		
IEB 01-01	Circumferential Cracking of Reactor Pressure Vessel (RPV) Head Penetration Nozzles	OV	OV		
IEB 02-01	RPV Head Degradation and Reactor Coolant Pressure Boundary Integrity	OV	OV		
IEB 02-02	RPV Head and Vessel Head Penetration Nozzle Inspection Program	OV	OV		
IEB 03-02	Leakage from RPV Lower Head Penetrations and Reactor Coolant Pressure Boundary Integrity	OV	OV		
IEB 04-01	Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at PWRs	OV	OV		
IEB 07-01	Security Officer Attentiveness	OV	OV		
GL 89-04	Guidelines on Developing Acceptable Inservice Testing Programs	OV	OV		
GL 92-08	Thermo-Lag 330-1 Fire Barriers	OV	OV		
GL 95-03	Circumferential cracking of Steam Generator (SG) Tubes	OV	OV		
GL 96-06	Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions	OV	OV	NRC letter 04/06/1999-Implement & provide containment penetration relief.	
GL 96-06s1	NRC Generic Letter 96-06, Supplement 1: Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions	OV	OV	NRC letter 04/06/1999-Implement & provide containment penetration relief.	
GL 97-01	Degradation of Control Rod Drive Mechanism Nozzle and Other Vessel Closure Head Penetrations	OV	OV		
GL 97-04	Assurance of Sufficient Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal Pumps	OV	OV		
GL 97-05	SG Tube Inspection Techniques	OV	OV		
GL 97-06	Degradation of SG Internals	OV	OV		
GL 98-02	Loss of Reactor Coolant Inventory and Associated Potential for Loss of Emergency Mitigation Functions While in a Shutdown Condition	OV	OV		

GL 98-04	Potential for Degradation of the ECCS and the Containment Spray System After a LOCA Because of Construction and Protective Coating Deficiencies and Foreign Material in Containment	OV	OV		
GL 04-01	Requirements for SG Tube Inspection	OV	OV		
GL 04-02	Potential Impact of Debris Blockage on Emergency Recirculation during Design Basis Accidents at PWRs	OV	OV		
GL 06-02	Grid Reliability and the Impact on Plant Risk and the Operability of Offsite Power	OV	OV		
GL 06-03	Potentially Nonconforming Hemyc and MT Fire Barrier Configurations	OV	OV		
GL 07-01	Independent Safety Engineering Group	OV	OV	Complete testing of four additional cables.	
NUREG 0737	I.B.1.2 Independent Safety Engineering Group	OV	OV		
NUREG 0737	I.D.1 CRDR	OV	OV		
NUREG 0737	III.D.3.4 Control-room habitability	OV	OV		

NRC Review: Generic Communications Status for TVA Watts Bar Unit 2.

Table 4. Generic Communication Status-Applicable therefore Open

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Item	Title	Position		Additional Comments	ADAMS
		TVA	NRC		Accession Number
cr79023	Motor Starters and Contactors Failed to Operate	NA	A-O	N.R.R But defective shipment was received and should be addressed	
cr79025	Shock Arrestor Strut Assembly Interference	NA	A-O	N.R.R but inspections & C.A expected on behalf of WBN 2	
cr81013	Torque Switch Electrical Bypass Circuit for Safeguard Service Valve Motors	NA	A-O	Report required if discrepancies found. Clarify if discrepancies were found.	
GL 79-24	Multiple Equipment Failures in Safety-Related Systems	NA	A-O	Applies to ALL PWR's. Requests submittal of reports	
GL 79-51	Follow-up Actions Resulting from the NRC Staff Reviews Regarding the TMI-2 Accident	NA	A-O	Applies to all C.P holders and requires implementation of actions	
GL 79-63	Upgraded Emergency Plans	NA	A-O	Check Applicability. Header (yes) content (No). Request add. Info.	
GL 80-11	IEB 80-03 Loss Of Charcoal From Standard Type II, 2 Inch, Tray Absorber Cells	NA	A-O	Requires action on behalf of C.P holder	
GL 80-28	IEB 80-08 Examination Of Containment Liner Penetration Welds	NA	A-O	Info expected to be submitted 90 days after issuance	
GL 80-32	Information Request on Category I Masonry Walls Employed by Plants Under CP and OL Review	NA	A-O	B80-11 requests actions from all C.P facilities	
GL 80-92	IEB 80-21 Valve Yokes Supplied By Malcolm Foundry Company, Inc.	NA	A-O	Report was to be submitted within 60 days	
GL 86-04	Policy Statement On Engineering Expertise On Shift	NA	A-O	Applies to O.L.A and requires actions. Additional info needed.	
GL 89-02	Actions to Improve the Detection of Counterfeit and Fraudulently Marketed Products	NA	A-O	N.R.R but actions should be taken. Applies to all C.P holders	
GL 89-21	Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements	NA	A-O	NRC requests review and subsequent report	