

May 21, 2008

Mr. Charles G. Pardee
Chief Nuclear Officer
and Senior Vice President
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Chief Nuclear Officer
AmerGen Energy Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: CLINTON POWER STATION, UNIT NO. 1; DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3; LASALLE COUNTY STATION, UNITS 1 AND 2; LIMERICK GENERATING STATION, UNITS 1 AND 2; OYSTER CREEK NUCLEAR GENERATING STATION; PEACH BOTTOM ATOMIC POWER STATION, UNITS 2, AND 3; AND QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2 – CORRECTION OF SAFETY EVALUATION SUPPORTING RELIEF REQUEST AUTHORIZATION TO USE BOILING-WATER REACTOR VESSEL AND INTERNALS PROJECT GUIDELINES IN LIEU OF SPECIFIC AMERICAN SOCIETY OF MECHANICAL ENGINEERS CODE REQUIREMENTS (TAC NOS. MD5352 THRU MD5363)

Dear Mr. Pardee,

On April 30, 2008, the Nuclear Regulatory Commission (NRC) staff issued a letter authorizing relief from the requirements of Title 10 of the *Code of Federal Regulations*, Section 50.55a, for the subject plants in response to your letter dated April 19, 2007, as supplemented by letters dated October 5, 2007, January 8, and April 4, 2008.

Subsequent to its issuance, Mr. Tom Loomis of your staff identified a typographical error on page 2 of Attachment 1, wherein the marking for Note 2 (i.e., ⁽²⁾) was omitted from the table cell containing the "BWRVIP Exam Scope" for the "Shroud Support Weld (Weld H9)." The NRC staff has verified that the note was included in the application for relief and was considered by the NRC staff as part of its review, and that omission from the April 30, 2008, letter, was a typographical error. The error has been corrected in the attached page 2 of Attachment 1; please use it to replace corresponding page in the April 30, 2008, safety evaluation (SE).

These corrections do not change the conclusion reached in the SE. We regret any inconvenience these errors may have caused.

C. Pardee

- 2 -

If you have any questions, please contact me at (301) 415-1055.

Sincerely,

/RA/

Christopher Gratton, Sr. Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-461, 50-237, 50-249,
50-373, 50-374, 50-352, 50-353, 50-219,
50-277, 50-278, 50-254 and 50-265

Enclosure:
As stated

cc w/encl: See next page

C. Pardee

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50-373, 50-374, 50-352, 50-353, 50-219,
50-277, 50-278, 50-254 and 50-265

Enclosure:
As stated

cc w/encl: See next page

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NAME	CGratton	EWhitt (T.Harris for)	MMitchell	HChernoff	RGibbs
DATE	05/19/08	05/16/08	05/19/08	05/19/08	05/21/08

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ATTACHMENT 1 (cont.)

ASME Item No. Table IWB-2500-1	Component	ASME Exam Scope	ASME Exam	ASME Frequency	Authorized Alternative	BWRVIP Exam Scope	BWRVIP Exam	BWRVIP Frequency
B13.30	Interior Attachments Beyond Beltline – Steam Dryer Hold-down Brackets	Accessible Welds	VT-3	Each 10-year Interval	BWRVIP-48-A Table 3-2	Bracket Attachment	VT-3	Each 10-year Interval
	Guide Rod Brackets				BWRVIP-48-A Table 3-2	Bracket Attachment	VT-3	Each 10-year Interval
	Steam Dryer Support Brackets				BWRVIP-48-A Table 3-2	Bracket Attachment	EVT-1	Each 10-year Interval
	Feedwater Sparger Brackets				BWRVIP-48-A Table 3-2	Bracket Attachment	EVT-1	Each 10-year Interval
	Core Spray Piping Brackets				BWRVIP-48-A Table 3-2	Bracket Attachment	EVT-1	Every 4 Refueling Cycles
	Upper and Middle Surveillance Specimen Holder Brackets				BWRVIP-48-A Table 3-2	Bracket Attachment	VT-3	Each 10-year Interval
	Shroud Support (Weld H9) including gussets where applicable				BWRVIP-38, 3.1.3.2 Figures 3-2 and 3-5	Weld H9 ⁽²⁾	EVT-1 or UT	Maximum of 6 years for one sided EVT-1, Maximum of 10 years for UT
	Weld H12 Shroud Support Legs	BWRVIP-38, 3.2.3	Weld H12	Per BWRVIP-38 NRC SER (7-24-2000) inspect with appropriate method ⁽⁴⁾	When accessible			
	Rarely Accessible							