MISCELLANEOUS CORRESPONDENCE

FOR THE

CLINTON POWER STATION INITIAL EXAMINATION

AUGUST 2007

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

August 7, 2007

Mr. Christopher M. Crane
President and Chief Nuclear Officer
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: OPERATOR LICENSING EXAMINATION APPROVAL

Dear Mr. Crane:

The purpose of this letter is to confirm the final arrangements for the upcoming operator licensing examinations at the Clinton Power Station.

The U. S. Nuclear Regulatory Commission (NRC) has completed its review of the operator license applications in connection with this examination and will separately provide a list of approved applicants to Mr. John P. Lindsey, Training Director. Note that any examination waivers and application denials have been addressed in separate correspondence.

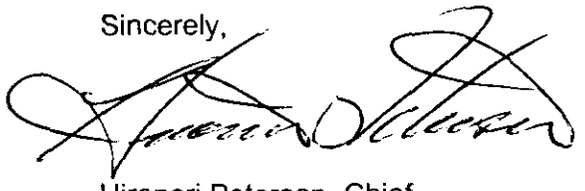
The NRC has approved the subject examinations and hereby authorizes you to administer the written examinations in accordance with Revision 9 of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," during the week of August 20, 2007. The NRC staff will administer the operating tests during the weeks of August 13 and August 20, 2007.

Please contact Mr. Dell McNeil at 630-829-9737 if you have any questions or identify any errors or changes in the license level (RO or SRO) or type of examination (partial or complete written examination and/or operating test) specified for each applicant.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records System (PARS) component of NRC's Agencywide

Documents Access and Management System (ADAMS), accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,



Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket No. 50-461
License No. NPF-62

cc: Site Vice President - Clinton Power Station
Plant Manager - Clinton Power Station
Regulatory Assurance Manager - Clinton Power Station
Chief Operating Officer
Senior Vice President - Nuclear Services
Vice President - Operations Support
Vice President - Licensing and Regulatory Affairs
Manager Licensing - Clinton Power Station
Senior Counsel, Nuclear, Mid-West Regional Operating Group
Document Control Desk - Licensing
Assistant Attorney General
Illinois Emergency Management Agency
State Liaison Officer, State of Illinois
Chairman, Illinois Commerce Commission
J. P. Lindsey, Training Director

Documents Access and Management System (ADAMS), accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,

/RA/

Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket No. 50-461
License No. NPF-62

cc: Site Vice President - Clinton Power Station
Plant Manager - Clinton Power Station
Regulatory Assurance Manager - Clinton Power Station
Chief Operating Officer
Senior Vice President - Nuclear Services
Vice President - Operations Support
Vice President - Licensing and Regulatory Affairs
Manager Licensing - Clinton Power Station
Senior Counsel, Nuclear, Mid-West Regional Operating Group
Document Control Desk - Licensing
Assistant Attorney General
Illinois Emergency Management Agency
State Liaison Officer, State of Illinois
Chairman, Illinois Commerce Commission
J. P. Lindsey, Training Director

DISTRIBUTION:
TEB
RAG1
SPS1
MMT
RidsNrrDirsIrib
GEG
KGO
RML2
SAM9
SRI Clinton
CAA1
DRSIII
DRPIII
PLB1
TXN
NLS

DOCUMENT NAME: G:\DRS\Work in Progress\CLI 07__07 DRS OL Exam Approval.wpd
☐ Publicly Available ☐ Non-Publicly Available ☐ Sensitive ☐ Non-Sensitive
To receive a copy of this document, indicate in the concurrence box "C" = Copy without attach/encl "E" = Copy with attach/encl "N" = No copy

OFFICE	RIII:DRS	C	RIII:DRS	N	RIII:DRS	N		
NAME	JRKweiser:jrk		DMcNeil		HPeterson			
DATE	07/23/2007		07/26/2007		07/27/2007			

OFFICIAL RECORD COPY

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

April 25, 2007

Mr. Christopher M. Crane
President and Chief Nuclear Officer
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Dear Mr. Crane:

In a telephone conversation on April 9, 2007, between Mr. T. Pickley, Senior Instructor, and Mr. D. McNeil, Senior Operations Engineer, arrangements were made for the administration of licensing examinations at the Clinton Power Station the weeks of August 13 and August 20, 2007. In addition, the NRC will make an examination validation visit to your facility the week of July 23, 2007.

As agreed during the telephone conversation, your staff will prepare the examinations based on the guidelines in Revision 9 of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors." The NRC regional office will discuss with your staff any changes that might be necessary before the examinations are administered.

Your staff has already provided the examination outline to the assigned chief examiner. To meet the above schedule, it will be necessary for your staff to furnish the written examinations, operating tests, and the supporting reference materials identified in Attachment 2 to ES-201 by July 2, 2007. Pursuant to 10 CFR 55.40(b)(3), of the Code of Federal Regulations (10 CFR 55..40(b)(3)), an authorized representative of the facility licensee shall approve the outlines, examinations, and tests before they are submitted to the NRC for review and approval. All materials shall be complete and ready to use. We request that any personal, proprietary, sensitive unclassified, or safeguards information in your response be contained in a separate enclosure and appropriately marked. Any delay in receiving the required examination and reference materials, or the submittal of inadequate or incomplete materials, may cause the examinations to be rescheduled.

In order to conduct the requested written examinations and operating tests, it will be necessary for your staff to provide adequate space and accommodations in accordance with ES-402, and to make the simulation facility available on the dates noted above. In accordance with ES-302, your staff should retain the original simulator performance data (e.g., system pressures, temperatures, and levels) generated during the dynamic operating tests until the examination results are final.

Appendix E of NUREG-1021 contains a number of NRC policies and guidelines that will be in effect while the written examinations and operating tests are being administered.

ML 071210232

To permit timely NRC review and evaluation, your staff should submit preliminary reactor operator and senior reactor operator license applications (Office of Management and Budget (OMB) approval number 3150-0090), medical certifications (OMB approval number 3150-0024), and waiver requests (if any) (OMB approval number 3150-0090) at least 30 days before the first examination date. If the applications are not received at least 30 days before the examination date, a postponement may be necessary. Signed applications certifying that all training has been completed should be submitted at least 14 days before the first examination date.

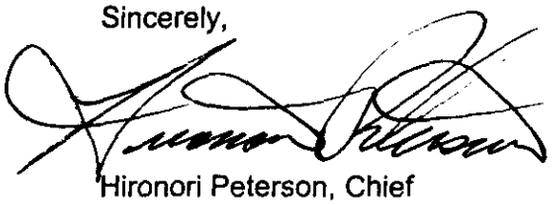
This letter contains information collections that are subject to the *Paperwork Reduction Act of 1995* (44 U.S.C. 3501 et seq.). These information collections were approved by the Office of Management and Budget, approval number 3150-0018, which expires on June 30, 2009. The public reporting burden for this collection is estimated to average 500 hours per response, including the time for reviewing instructions, gathering and maintaining the data needed, and completing and reviewing the collection of information. Send comments on any aspect of this collection of information, including suggestions for reducing the burden, to the Information and Records Management Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, D.C. 20555-0001, or by Internet electronic mail at BJS1@nrc.gov; and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0018), Office of Management and Budget, Washington, D.C. 20503-0001.

The NRC may neither conduct nor sponsor, and a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

Thank you for your cooperation in this matter. Mr. Pickley has been advised of the policies and guidelines referenced in this letter. If you have any questions regarding the NRC's examination procedures and guidelines, please contact Dell McNeil at 630-829-9737, or me at 630-829-9707.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC's Public Document Room or from the Publicly Available Records (PARS) component of the NRC's Agencywide Documents Access and Management System (ADAMS). ADAMS is accessible from the Electronic Reading Room page of the NRC's public Web site at http://www.nrc.gov/reading-rm/adams.html.

Sincerely,



Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket No. 50-461
License No. NPF-62

To permit timely NRC review and evaluation, your staff should submit preliminary reactor operator and senior reactor operator license applications (Office of Management and Budget (OMB) approval number 3150-0090), medical certifications (OMB approval number 3150-0024), and waiver requests (if any) (OMB approval number 3150-0090) at least 30 days before the first examination date. If the applications are not received at least 30 days before the examination date, a postponement may be necessary. Signed applications certifying that all training has been completed should be submitted at least 14 days before the first examination date.

This letter contains information collections that are subject to the *Paperwork Reduction Act of 1995* (44 U.S.C. 3501 et seq.). These information collections were approved by the Office of Management and Budget, approval number 3150-0018, which expires on June 30, 2009. The public reporting burden for this collection is estimated to average 500 hours per response, including the time for reviewing instructions, gathering and maintaining the data needed, and completing and reviewing the collection of information. Send comments on any aspect of this collection of information, including suggestions for reducing the burden, to the Information and Records Management Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, D.C. 20555-0001, or by Internet electronic mail at BJS1@nrc.gov; and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0018), Office of Management and Budget, Washington, D.C. 20503-0001.

The NRC may neither conduct nor sponsor, and a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

Thank you for your cooperation in this matter. Mr. Pickley has been advised of the policies and guidelines referenced in this letter. If you have any questions regarding the NRC's examination procedures and guidelines, please contact Dell McNeil at 630-829-9737, or me at 630-829-9707.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC's Public Document Room or from the Publicly Available Records (PARS) component of the NRC's Agencywide Documents Access and Management System (ADAMS). ADAMS is accessible from the Electronic Reading Room page of the NRC's public Web site at http://www.nrc.gov/reading-rm/adams.html.

Sincerely,

/RA/

Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket No. 50-461
License No. NPF-62

See Attached Distribution

DOCUMENT NAME: G:\DRS\Work in Progress\CLI 04__07 DRS OL.wpd
☐ Publicly Available ☐ Non-Publicly Available ☐ Sensitive ☐ Non-Sensitive
To receive a copy of this document, indicate in the concurrence box "C" = Copy without attach/encl "E" = Copy with attach/encl "N" = No copy

OFFICE	RIII:DRS	N	RIII:DRS			
NAME	DMcNeil:co		HPeterson			
DATE	04/24/07		04/25/07			

OFFICIAL RECORD COPY

cc: Site Vice President - Clinton Power Station
Plant Manager - Clinton Power Station
Regulatory Assurance Manager - Clinton Power Station
Chief Operating Officer
Senior Vice President - Nuclear Services
Vice President - Operations Support
Vice President - Licensing and Regulatory Affairs
Manager Licensing - Clinton Power Station
Senior Counsel, Nuclear, Mid-West Regional Operating Group
Document Control Desk - Licensing
Assistant Attorney General
Illinois Emergency Management Agency
State Liaison Officer, State of Illinois
Chairman, Illinois Commerce Commission
J. Lindsey, Training Department

DISTRIBUTION:
TEB
RAG1
SPS1
RidsNrrDirsIrib
GEG
KGO
GLS
RML2
SAM9
SRI Clinton
CAA1
DRSIII
DRPIII
PLB1
TXN
NLS

AmerGen

An Exelon Company

Clinton Power Station
R. R. 3, Box 228
Clinton, IL 61727

U-603799
February 2, 2007

U.S. Nuclear Regulatory Commission, Region III
2443 Warrenville Road
Lisle, IL 60532-4352
ATTN: D. R. McNeil

Clinton Power Station, Unit 1
Facility Operating License No. NPF-62
Docket No. 50-461

Subject: Submittal of Knowledge and Abilities (K/A) Statements That Will be Excluded From the Random Exam Generation Process

Enclosed are the K/A statements that have been excluded from the random exam generation process which Clinton Power Station (CPS) is submitting for review, comment, and approval for the Initial License Examination scheduled to start the week of August 13, 2007, for two weeks at CPS.

This letter is submitted for the K/A suppression as required by NUREG 1021, Revision 9, Supplement 1, "Operator Licensing Examination Standards for Power Reactors".

Should you have any questions concerning this letter, please contact Mr. L. T. Pickley, Senior Training Instructor, at (217) 937-4118.

Respectfully,

Bryan Hanson
Site Vice President
Clinton Power Station

EET/blf

Enclosure: Suppressed K/A Statements

cc: Chief, NRC Operator Licensing Branch – w/o enclosure
NRC Senior Resident Inspector - Clinton Power Station – w/o enclosure

CPS Suppressed K & As

Clinton Power
Suppressed K/As Report

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
Control Rod Drive Hydraulic System			
201001	A1.04	Ability to predict and/or monitor changes in parameters associated with operating the CONTROL ROD DRIVE HYDRAULIC SYSTEM controls including:	Head spray flow: BWR-3
		Clinton is a BWR 6 and does not have this configuration	
201001	K1.04	Knowledge of the physical connections and/or cause- effect relationships between CONTROL ROD DRIVE HYDRAULIC SYSTEM and the	Head spray: BWR-3
		Clinton is a BWR 6 and does not have this configuration	
201001	K1.05	Knowledge of the physical connections and/or cause- effect relationships between CONTROL ROD DRIVE HYDRAULIC SYSTEM and the	Feedwater (or reactor water cleanup)-CRD return to vessel: Plant-Specific
		CRD does not return to the vessel through Feedwater or Reactor Water Cleanup.	
201001	K3.04	Knowledge of the effect that a loss or malfunction of the CONTROL ROD DRIVE HYDRAULIC SYSTEM will have on following:	Head spray: BWR-3
		Clinton is a BWR 6 and does not have this configuration	
Reactor Manual Control System			
201002	2.1.27	Conduct of Operations	Knowledge of system purpose and/or
		Clinton is a BWR 6 and does not utilize this system.	
201002	2.1.28	Conduct of Operations	Knowledge of the purpose and function of major system components and controls.
		Clinton is a BWR 6 and does not utilize this system.	
201002	2.1.32	Conduct of Operations	Ability to explain and apply system limits and precautions.
		Clinton is a BWR 6 and does not utilize this system.	
201002	2.1.33	Conduct of Operations	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.
		Clinton is a BWR 6 and does not utilize this system.	
201002	2.2.22	Equipment Control	Knowledge of limiting conditions for operations and safety limits.
		Clinton is a BWR 6 and does not utilize this system.	
201002	A1.01	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR MANUAL CONTROL SYSTEM	CRD drive water flow
		Clinton is a BWR 6 and does not utilize this system.	
201002	A1.02	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR MANUAL CONTROL SYSTEM	Control rod position
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201002	A1.03	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR MANUAL CONTROL SYSTEM	Rod movement sequence lights
		Clinton is a BWR 6 and does not utilize this system.	
201002	A1.04	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR MANUAL CONTROL SYSTEM	Overall reactor power
		Clinton is a BWR 6 and does not utilize this system.	
201002	A1.05	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR MANUAL CONTROL SYSTEM	Local reactor power
		Clinton is a BWR 6 and does not utilize this system.	
201002	A2.01	Ability to (a) predict the impacts of the following on the REACTOR MANUAL CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Rod movement sequence timer
		Clinton is a BWR 6 and does not utilize this system.	
201002	A2.02	Ability to (a) predict the impacts of the following on the REACTOR MANUAL CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Rod drift alarm
		Clinton is a BWR 6 and does not utilize this system.	
201002	A2.03	Ability to (a) predict the impacts of the following on the REACTOR MANUAL CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Select block
		Clinton is a BWR 6 and does not utilize this system.	
201002	A2.04	Ability to (a) predict the impacts of the following on the REACTOR MANUAL CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Control rod block
		Clinton is a BWR 6 and does not utilize this system.	
201002	A3.01	Ability to monitor automatic operations of the REACTOR MANUAL CONTROL SYSTEM including:	Control rod block actuation
		Clinton is a BWR 6 and does not utilize this system.	
201002	A3.02	Ability to monitor automatic operations of the REACTOR MANUAL CONTROL SYSTEM including:	Rod movement sequence lights
		Clinton is a BWR 6 and does not utilize this system.	
201002	A3.03	Ability to monitor automatic operations of the REACTOR MANUAL CONTROL SYSTEM including:	Rod drift alarm
		Clinton is a BWR 6 and does not utilize this system.	
201002	A3.04	Ability to monitor automatic operations of the REACTOR MANUAL CONTROL SYSTEM including:	Rod movement sequence timer malfunction alarm: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201002	A4.01	Ability to manually operate and/or monitor in the control room:	Rod movement control switch
201002	A4.02	*Clinton is a BWR 6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Emergency in/notch override switch
201002	A4.03	*Clinton is a BWR 6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Rod drift test switch
201002	A4.04	*Clinton is a BWR 6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Timer malfunction test switch: Plant-Specific
201002	A4.05	*Clinton is a BWR 6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Rod select matrix
201002	A4.06	*Clinton is a BWR 6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Rod select matrix power switch
201002	K1.01	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:	Control rod drive hydraulic system
201002	K1.02	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:	Control rod and drive mechanism
201002	K1.03	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:	Control rod block interlocks/power operation refueling
201002	K1.04	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:	Rod block monitor: Plant-Specific
201002	K1.05	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:	Rod worth minimizer: Plant-Specific
201002	K1.06	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:	Rod sequence control system:
201002	K1.07	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:	Process computer: Plant-Specific
201002	K1.08	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:	Refueling interlocks: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201002	K2.01	Knowledge of electrical power supplies to the following:	Select matrix
		Clinton is a BWR 6 and does not utilize this system.	
201002	K2.02	Knowledge of electrical power supplies to the following:	CRD HCU directional control valves
		Clinton is a BWR 6 and does not utilize this system.	
201002	K3.01	Knowledge of the effect that a loss or malfunction of the REACTOR MANUAL CONTROL SYSTEM will have on following:	Ability to move control rods
		Clinton is a BWR 6 and does not utilize this system.	
201002	K3.02	Knowledge of the effect that a loss or malfunction of the REACTOR MANUAL CONTROL SYSTEM will have on following:	Rod block monitor: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
201002	K3.03	Knowledge of the effect that a loss or malfunction of the REACTOR MANUAL CONTROL SYSTEM will have on following:	Ability to process rod block signals
		Clinton is a BWR 6 and does not utilize this system.	
201002	K4.01	Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Detection of sequence timer malfunction
		Clinton is a BWR 6 and does not utilize this system.	
201002	K4.02	Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Control rod blocks
		Clinton is a BWR 6 and does not utilize this system.	
201002	K4.03	Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Detection of drifting control rods
		Clinton is a BWR 6 and does not utilize this system.	
201002	K4.04	Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Single notch rod withdrawal and insertion
		Clinton is a BWR 6 and does not utilize this system.	
201002	K4.05	Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Notch override rod withdrawal
		Clinton is a BWR 6 and does not utilize this system.	
201002	K4.06	Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Emergency In rod insertion
		Clinton is a BWR 6 and does not utilize this system.	
201002	K4.07	Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Timing of rod insert and withdrawal cycles (rod movement sequence timer)
		Clinton is a BWR 6 and does not utilize this system.	
201002	K4.08	Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Continuous In rod insertion
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201002	K6.01	Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR MANUAL CONTROL SYSTEM:	Select matrix power
		Clinton is a BWR 6 and does not utilize this system.	
Rod Sequence Control System (Plant Specific)			
201004	2.1.27	Conduct of Operations	Knowledge of system purpose and/or
		Clinton is a BWR-6 and does not utilize this system.	
201004	2.1.28	Conduct of Operations	Knowledge of the purpose and function of major system components and controls.
		Clinton is a BWR-6 and does not utilize this system.	
201004	2.1.32	Conduct of Operations	Ability to explain and apply system limits and precautions.
		Clinton is a BWR-6 and does not utilize this system.	
201004	2.1.33	Conduct of Operations	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.
		Clinton is a BWR-6 and does not utilize this system.	
201004	2.2.22	Equipment Control	Knowledge of limiting conditions for operations and safety limits.
		Clinton is a BWR-6 and does not utilize this system.	
201004	A1.01	Ability to predict and/or monitor changes in parameters, associated with operating the ROD SEQUENCE CONTROL SYSTEM controls including:	Reactor manual control system: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A2.01	Ability to (a) predict the impacts of the following on the ROD SEQUENCE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Loss of rod position information: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A2.02	Ability to (a) predict the impacts of the following on the ROD SEQUENCE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Attempting to move a stuck control rod: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A2.03	Ability to (a) predict the impacts of the following on the ROD SEQUENCE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Turbine trip: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A3.01	Ability to monitor automatic operations of the ROD SEQUENCE CONTROL SYSTEM including:	Rod select switch light: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A3.02	Ability to monitor automatic operations of the ROD SEQUENCE CONTROL SYSTEM including:	Rod select bottom lamp dimmer logic:
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201004	A3.03	Ability to monitor automatic operations of the ROD SEQUENCE CONTROL SYSTEM including:	Back panel indicators: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A3.04	Ability to monitor automatic operations of the ROD SEQUENCE CONTROL SYSTEM including:	Annunciator and alarm signals: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A3.05	Ability to monitor automatic operations of the ROD SEQUENCE CONTROL SYSTEM including:	Verification of proper function/ operability: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A4.01	Ability to manually operate and/or monitor in the control room:	System bypass switches: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A4.02	Ability to manually operate and/or monitor in the control room:	RSCS console switches and indicators: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	A4.03	Ability to manually operate and/or monitor in the control room:	RSCS back panel switches and indicators: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K1.01	Knowledge of the physical connections and/or cause- effect relationships between ROD SEQUENCE CONTROL SYSTEM and the	Reactor manual control system: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K1.02	Knowledge of the physical connections and/or cause- effect relationships between ROD SEQUENCE CONTROL SYSTEM and the	Turbine generator system: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K1.03	Knowledge of the physical connections and/or cause- effect relationships between ROD SEQUENCE CONTROL SYSTEM and the	Rod position information system: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K1.04	Knowledge of the physical connections and/or cause- effect relationships between ROD SEQUENCE CONTROL SYSTEM and the	Rod worth minimizer: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K3.01	Knowledge of the effect that a loss or malfunction of the ROD SEQUENCE CONTROL SYSTEM will have on following:	Reactor manual control: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K4.01	Knowledge of ROD SEQUENCE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Select blocks: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K4.02	Knowledge of ROD SEQUENCE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Insert rod blocks: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K4.03	Knowledge of ROD SEQUENCE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Withdraw rod blocks: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201004	K4.04	Knowledge of ROD SEQUENCE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	RSCS bypass as reactor power increases: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K4.05	Knowledge of ROD SEQUENCE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Rod movement, direction, and selection information: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K4.06	Knowledge of ROD SEQUENCE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Group notch control: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K4.07	Knowledge of ROD SEQUENCE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Minimizing rod worth: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K5.01	Knowledge of the operational implications of the following concepts as they apply to ROD SEQUENCE CONTROL SYSTEM:	Prevention of clad damage if a control rod drop accident (CRDA) occurs: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K5.02	Knowledge of the operational implications of the following concepts as they apply to ROD SEQUENCE CONTROL SYSTEM:	Sequences and groups: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K5.03	Knowledge of the operational implications of the following concepts as they apply to ROD SEQUENCE CONTROL SYSTEM:	Group notch control limits and rod density: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K6.01	Knowledge of the effect that a loss or malfunction of the following will have on the ROD SEQUENCE CONTROL SYSTEM:	Rod position information: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K6.02	Knowledge of the effect that a loss or malfunction of the following will have on the ROD SEQUENCE CONTROL SYSTEM:	Rod direction information: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K6.03	Knowledge of the effect that a loss or malfunction of the following will have on the ROD SEQUENCE CONTROL SYSTEM:	Rod movement information: BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
201004	K6.04	Knowledge of the effect that a loss or malfunction of the following will have on the ROD SEQUENCE CONTROL SYSTEM:	Turbine generator (1st stage shell pressure): BWR-4, 5
		Clinton is a BWR-6 and does not utilize this system.	
Rod Worth Minimizer System (RWM) (Plant			
201006	2.1.27	Conduct of Operations	Knowledge of system purpose and/or
		Clinton is a BWR 6 and does not utilize this system.	
201006	2.1.28	Conduct of Operations	Knowledge of the purpose and function of major system components and controls.
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category *Suppression Basis*	K/A Statement
201006	2.1.32	Conduct of Operations	Ability to explain and apply system limits and precautions.
201006	2.1.33	*Clinton is a BWR 6 and does not utilize this system.* Conduct of Operations	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.
201006	2.2.22	*Clinton is a BWR 6 and does not utilize this system.* Equipment Control	Knowledge of limiting conditions for operations and safety limits.
201006	A1.01	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the ROD WORTH MINIMIZER SYSTEM (RWM) controls including:	Rod position: P-Spec(Not-BWR6)
201006	A1.02	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the ROD WORTH MINIMIZER SYSTEM (RWM) controls including:	Status of control rod movement blocks; P-Spec(Not-BWR6)
201006	A1.03	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the ROD WORTH MINIMIZER SYSTEM (RWM) controls including:	Latched group indication:
201006	A2.01	*Clinton is a BWR 6 and does not utilize this system.* Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Power supply loss: P-Spec(Not-BWR6)
201006	A2.02	*Clinton is a BWR 6 and does not utilize this system.* Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Loss of steam flow input:
201006	A2.03	*Clinton is a BWR 6 and does not utilize this system.* Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Rod drift: P-Spec(Not-BWR6)
201006	A2.04	*Clinton is a BWR 6 and does not utilize this system.* Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Stuck rod: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201006	A2.05	Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Out of sequence rod movement; P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A2.06	Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Loss of reactor water level control input: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A2.07	Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	RWM hardware/software failure: P-Spec(Not-BWR6)
		Clinton is a BWR-6 and does not utilize this system.	
201006	A3.01	Ability to monitor automatic operations of the ROD WORTH MINIMIZER SYSTEM (RWM)	System window and light indication: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A3.02	Ability to monitor automatic operations of the ROD WORTH MINIMIZER SYSTEM (RWM)	Verification of proper functioning/ operability: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A3.03	Ability to monitor automatic operations of the ROD WORTH MINIMIZER SYSTEM (RWM)	Annunciator and alarm signals: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A3.04	Ability to monitor automatic operations of the ROD WORTH MINIMIZER SYSTEM (RWM)	Control rod movement blocks: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A3.05	Ability to monitor automatic operations of the ROD WORTH MINIMIZER SYSTEM (RWM)	Latched group indication:
		Clinton is a BWR 6 and does not utilize this system.	
201006	A4.01	Ability to manually operate and/or monitor in the control room:	System bypass switch: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A4.02	Ability to manually operate and/or monitor in the control room:	Pushbutton indicating switches: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A4.03	Ability to manually operate and/or monitor in the control room:	Latched group indication:
		Clinton is a BWR 6 and does not utilize this system.	
201006	A4.04	Ability to manually operate and/or monitor in the control room:	Rod withdrawal error indication: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	A4.05	Ability to manually operate and/or monitor in the control room:	Rod insert error indication:
		Clinton is a BWR 6 and does not utilize this system.	
201006	A4.06	Ability to manually operate and/or monitor in the control room:	Selected rod position indication: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201006	K1.01	Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the	Reactor manual control: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K1.02	Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the	Rod position indication system: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K1.03	Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the	Reactor water level control (feed flow): P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K1.04	Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the	Steam flow/reactor power:
		Clinton is a BWR 6 and does not utilize this system.	
201006	K1.05	Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the	Control rod drop accident:
		Clinton is a BWR 6 and does not utilize this system.	
201006	K1.06	Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the	Rod sequence control system: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K1.07	Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the	Process computer: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K1.08	Knowledge of the physical connections and/or cause- effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the	Reactor power (turbine first stage pressure): P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K2.01	Knowledge of electrical power supplies to the following:	Rod worth minimizer: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K3.01	Knowledge of the effect that a loss or malfunction of the ROD WORTH MINIMIZER SYSTEM (RWM) will have on following:	Reactor manual control system: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K4.01	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	Insert blocks/errors: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K4.02	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	Withdraw blocks/errors: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K4.03	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	Select blocks/errors: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201006	K4.04	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	System bypass: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K4.05	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	Substitute rod position data:
		Clinton is a BWR 6 and does not utilize this system.	
201006	K4.06	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	Correction of out of sequence rod positions: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K4.07	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	Display of out of position control rods without rod blocks (transition zone):
		Clinton is a BWR 6 and does not utilize this system.	
201006	K4.08	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	System testing: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K4.09	Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:	System initialization: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K5.01	Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Minimize clad damage if a control rod drop accident (CRDA) occurs:
		Clinton is a BWR 6 and does not utilize this system.	
201006	K5.02	Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Low power set point: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K5.03	Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Low power alarm point (LPAP): P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K5.04	Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Transition zone: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K5.05	Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	High power set point: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K5.06	Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Rod groups and steps: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	
201006	K5.07	Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Latch groups: P-Spec(Not-BWR6)
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
201006	K5.08	Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Operating sequence: P-Spec(Not-BWR6)
201006	K5.09	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Select error: P-Spec(Not-BWR6)
201006	K5.10	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Withdraw error: P-Spec(Not-BWR6)
201006	K5.11	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Insert error: P-Spec(Not-BWR6)
201006	K5.12	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Withdraw block: P-Spec(Not-BWR6)
201006	K5.13	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Insert block: P-Spec(Not-BWR6)
201006	K5.14	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):	Alternate withdraw and insert limits: P-Spec(Not-BWR6)
201006	K6.01	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ROD WORTH MINIMIZER SYSTEM (RWM):	RWM power supply: P-Spec(Not-BWR6)
201006	K6.02	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ROD WORTH MINIMIZER SYSTEM (RWM):	Reactor water level control input: P-Spec(Not-BWR6)
201006	K6.03	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ROD WORTH MINIMIZER SYSTEM (RWM):	Rod position indication: P-Spec(Not-BWR6)
201006	K6.04	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ROD WORTH MINIMIZER SYSTEM (RWM):	Process computer: P-Spec(Not-BWR6)
201006	K6.05	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ROD WORTH MINIMIZER SYSTEM (RWM):	Steam flow input: P-Spec(Not-BWR6)
Recirculation System		*Clinton is a BWR 6 and does not utilize this system.*	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
202001	A2.09	Ability to (a) predict the impacts of the following on the RECIRCULATION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Recirculation scoop tube lockup:
		Clinton is a BWR 6 and does not utilize this system.	
202001	K1.14	Knowledge of the physical connections and/or cause- effect relationships between RECIRCULATION SYSTEM and the following:	Rod block monitor: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
202001	K1.24	Knowledge of the physical connections and/or cause- effect relationships between RECIRCULATION SYSTEM and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
202001	K2.05	Knowledge of electrical power supplies to the following:	MG set oil pumps: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
202001	K3.02	Knowledge of the effect that a loss or malfunction of the RECIRCULATION SYSTEM will have on following:	Load following capabilities: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
202001	K3.12	Knowledge of the effect that a loss or malfunction of the RECIRCULATION SYSTEM will have on following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
202001	K4.08	Knowledge of RECIRCULATION System design feature(s) and/or interlocks which provide for the following:	Oil pump automatic starts: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Recirculation Flow Control System			
202002	A2.05	Ability to (a) predict the impacts of the following on the RECIRCULATION FLOW CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Scoop tube lockup: BWR-2, 3, 4
		Clinton is a BWR 6 and does not utilize this system.	
202002	A3.03	Ability to monitor automatic operations of the RECIRCULATION FLOW CONTROL SYSTEM including:	Scoop tube operation: BWR-2, 3, 4
		Clinton is a BWR 6 and does not utilize this system.	
202002	A4.06	Ability to manually operate and/or monitor in the control room:	Scoop tube power: BWR-2, 3, 4
		Clinton is a BWR 6 and does not utilize this system.	
202002	K4.01	Knowledge of RECIRCULATION FLOW CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Scoop tube break: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
202002	K4.04	Knowledge of RECIRCULATION FLOW CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Automatic load following: Plant-Specific
		Clinton does not utilize this function.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
202002	K4.08	Knowledge of RECIRCULATION FLOW CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Automatic flow control valve positioning: BWR-5, 6
		Clinton does not utilize this function.	
202002	K5.01	Knowledge of the operational implications of the following concepts as they apply to RECIRCULATION FLOW CONTROL SYSTEM:	Fluid coupling: BWR-3, 4
		Clinton is a BWR 6 and does not utilize this configuration.	
RHR/LPCI: Injection Mode (Plant Specific)			
203000	A2.15	Ability to (a) predict the impacts of the following on the RHR/LPCI: INJECTION MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Loop selection logic failure: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
203000	A3.07	Ability to monitor automatic operations of the RHR/LPCI: INJECTION MODE including:	Loop selection: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
203000	A4.12	Ability to manually operate and/or monitor in the control room:	Condensate storage tank level:
		Clinton is a BWR 6 and does not utilize this function.	
203000	A4.14	Ability to manually operate and/or monitor in the control room:	Testable check valves
		Clinton is a BWR 6 and does not utilize this configuration.	
203000	K1.05	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: INJECTION MODE and the following:	Recirculation system: BWR-3, 4
		Clinton is a BWR 6 and does not utilize this configuration.	
203000	K4.11	Knowledge of RHR/LPCI: INJECTION MODE design feature(s) and/or interlocks which provide for the following:	Loop selection logic: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
203000	K5.01	Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: INJECTION MODE:	Testable check valve operation
		Clinton is a BWR 6 and does not utilize this configuration.	
Reactor Water Cleanup System			
204000	A2.02	Ability to (a) predict the impacts of the following on the REACTOR WATER CLEANUP SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Pressure control valve failure: LP-RWCU
		Clinton utilizes a HP RWCU system.	
204000	A3.01	Ability to monitor automatic operations of the REACTOR WATER CLEANUP SYSTEM	System pressure downstream of the pressure regulating valve: LP-RWCU
		Clinton utilizes a HP RWCU system.	
204000	K4.08	Knowledge of REACTOR WATER CLEANUP SYSTEM design feature(s) and/or interlocks which provide for the following:	Reducing reactor pressure upstream of low pressure piping: LP-RWCU
		Clinton utilizes a HP RWCU system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
Shutdown Cooling System (RHR Shutdown			
205000	A2.11	Ability to (a) predict the impacts of the following on the SHUTDOWN COOLING SYSTEM/MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Recirculation pump trips: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
205000	K1.09	Knowledge of the physical connections and/or cause- effect relationships between SHUTDOWN COOLING SYSTEM (RHR SHUTDOWN COOLING MODE) and the following:	Auxiliary steam supply: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
205000	K1.11	Knowledge of the physical connections and/or cause- effect relationships between SHUTDOWN COOLING SYSTEM (RHR SHUTDOWN COOLING MODE) and the following:	Nitrogen: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
205000	K1.12	Knowledge of the physical connections and/or cause- effect relationships between SHUTDOWN COOLING SYSTEM (RHR SHUTDOWN COOLING MODE) and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
205000	K6.07	Knowledge of the effect that a loss or malfunction of the following will have on the SHUTDOWN COOLING SYSTEM/MODE:	Nitrogen: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
High Pressure Coolant Injection System			
206000	2.1.27	Conduct of Operations	Knowledge of system purpose and/or
		Clinton is a BWR-6 and does not utilize this system.	
206000	2.1.28	Conduct of Operations	Knowledge of the purpose and function of major system components and controls.
		Clinton is a BWR-6 and does not utilize this system.	
206000	2.1.32	Conduct of Operations	Ability to explain and apply system limits and precautions.
		Clinton is a BWR-6 and does not utilize this system.	
206000	2.1.33	Conduct of Operations	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.
		Clinton is a BWR-6 and does not utilize this system.	
206000	2.2.22	Equipment Control	Knowledge of limiting conditions for operations and safety limits.
		Clinton is a BWR-6 and does not utilize this system.	
206000	A1.01	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	Reactor water level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	A1.02	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	Reactor pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A1.03	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	Condensate storage tank level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A1.04	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	Suppression pool level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A1.05	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	Suppression pool temperature: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A1.06	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	System flow: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A1.07	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	System discharge pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A1.08	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	System lineup: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A1.09	Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:	Turbine speed: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A2.01	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Turbine trips: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A2.02	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Valve closures: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	A2.03	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR-6 and does not utilize this system.*	Valve openings: BWR-2, 3, 4
206000	A2.04	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR-6 and does not utilize this system.*	A.C. failures: BWR-2, 3, 4
206000	A2.05	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR-6 and does not utilize this system.*	D.C. failures: BWR-2, 3, 4
206000	A2.06	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR-6 and does not utilize this system.*	Inadequate system flow: BWR-2, 3, 4
206000	A2.07	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR-6 and does not utilize this system.*	Low suppression pool level: BWR-2, 3, 4
206000	A2.08	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR-6 and does not utilize this system.*	High suppression pool temperature: BWR-2, 3, 4
206000	A2.09	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR-6 and does not utilize this system.*	Low condensate storage tank level: BWR-2,
206000	A2.10	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR-6 and does not utilize this system.*	System isolation: BWR-2, 3, 4

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	A2.11	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Low reactor water level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A2.12	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Loss of room cooling: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A2.13	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Loss of applicable plant air systems: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A2.14	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Flow controller failure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A2.15	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Loss of control oil pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A2.16	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	High drywell pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A2.17	Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	HPCI inadvertent initiation: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A3.01	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	Turbine speed: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A3.02	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	System Flow: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	A3.03	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	System lineup: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A3.04	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	Reactor pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A3.05	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	Reactor water level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A3.06	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	System discharge pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A3.07	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	Lights and alarms: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A3.08	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	Condensate storage tank level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A3.09	Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:	Response to system isolation: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.01	Ability to manually operate and/or monitor in the control room:	Turbine speed controls: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.02	Ability to manually operate and/or monitor in the control room:	Flow controller: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.03	Ability to manually operate and/or monitor in the control room:	Turbine temperatures: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.04	Ability to manually operate and/or monitor in the control room:	Major system valves: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.05	Ability to manually operate and/or monitor in the control room:	Reactor water level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.06	Ability to manually operate and/or monitor in the control room:	Reactor pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.07	Ability to manually operate and/or monitor in the control room:	Condensate storage tank level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	A4.08	Ability to manually operate and/or monitor in the control room:	Suppression pool temperature: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.09	Ability to manually operate and/or monitor in the control room:	Suppression pool level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.10	Ability to manually operate and/or monitor in the control room:	System pumps: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.11	Ability to manually operate and/or monitor in the control room:	Turning gear: BWR-2, 3, 4(P-Spec)
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.12	Ability to manually operate and/or monitor in the control room:	Turbine trip controls: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.13	Ability to manually operate and/or monitor in the control room:	Turbine reset control: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	A4.14	Ability to manually operate and/or monitor in the control room:	System auto start control: BWR-2, 3,
		Clinton is a BWR-6 and does not utilize this system.	
206000	K1.01	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Reactor vessel: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K1.02	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Reactor water level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K1.03	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Reactor pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K1.04	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Reactor feedwater system: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K1.05	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Condensate storage system: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K1.06	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Suppression chamber: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K1.07	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	D.C. power: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	K1.08	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	A.C. power: BWR-2, 3, 4
206000	K1.09	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	ECCS keep fill system: BWR-2, 3,
206000	K1.10	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Condensate storage and transfer system: BWR-2, 3, 4
206000	K1.11	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	PCIS: BWR-2, 3, 4
206000	K1.12	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Nuclear boiler instrumentation: BWR-2, 3, 4
206000	K1.13	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Main condenser: BWR-2, 3, 4(P-Spec)
206000	K1.14	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	SBGT: BWR-2, 3, 4(P-Spec)
206000	K1.15	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Plant air systems: BWR-2, 3, 4
206000	K1.16	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM	Containment/Torus pressure: BWR-2, 3, 4
206000	K2.01	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of electrical power supplies to the following:	System valves: BWR-2, 3, 4
206000	K2.02	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of electrical power supplies to the following:	System pumps: BWR-2, 3, 4
206000	K2.03	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of electrical power supplies to the following:	Initiation logic: BWR-2, 3, 4
206000	K2.04	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of electrical power supplies to the following:	Turbine control circuits: BWR-2, 3, 4
206000	K3.01	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the HIGH PRESSURE COOLANT INJECTION SYSTEM will have on following:	Reactor water level control: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	K3.02	Knowledge of the effect that a loss or malfunction of the HIGH PRESSURE COOLANT INJECTION SYSTEM will have on following:	Reactor pressure control: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K3.03	Knowledge of the effect that a loss or malfunction of the HIGH PRESSURE COOLANT INJECTION SYSTEM will have on following:	Suppression pool level control: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.01	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Turbine trips: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.02	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	System isolation: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.03	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Resetting turbine trips: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.04	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Resetting system isolations: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.05	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Preventing water hammer in turbine exhaust line (procedural control): BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.06	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Preventing water hammer in pump discharge line (procedural control): BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.07	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Automatic system initiation: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.08	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Manual system initiation: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.09	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Automatic flow control: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.10	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Surveillance for all operable components: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.11	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Turbine speed control: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	K4.12	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Condensation of shaft sealing steam: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.13	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Turbine and pump lubrication: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.14	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Control oil to turbine speed controls: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.15	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Low speed turning of the turbine rotor: BWR-2, 3, 4(P-Spec)
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.16	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Minimizing fission product concentration in the condensate storage tank (valve closures on system initiation): BWR-2, 3, 4(P-Spec)
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.17	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Protection against draining the condensate storage tank to the suppression pool: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.18	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Pump minimum flow: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K4.19	Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Automatic transfer of HPCI pump suction: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K5.01	Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	Turbine operation: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K5.02	Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	Turbine shaft sealing: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K5.03	Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	GEMAC controllers: BWR-2, 3, 4(P-Spec)
		Clinton is a BWR-6 and does not utilize this system.	
206000	K5.04	Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	Indications of pump cavitation: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K5.05	Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	Turbine speed control: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	K5.06	Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	Turbine speed measurement: BWR-2, 3, 4
206000	K5.07	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	System venting: BWR-2, 3, 4
206000	K5.08	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	Vacuum breaker operation: BWR-2, 3, 4
206000	K5.09	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	Testable check valve operation: BWR-2, 3,
206000	K5.10	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:	Assist core cooling: BWR-2, 3, 4
206000	K6.01	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	Plant air systems: BWR-2, 3, 4
206000	K6.02	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	D.C. power: BWR-2, 3, 4
206000	K6.03	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	A.C. power: BWR-2, 3, 4
206000	K6.04	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	Condensate storage tank level: BWR-2, 3, 4
206000	K6.05	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	Suppression pool level: BWR-2, 3, 4
206000	K6.06	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	SBGTS: BWR-2, 3, 4(P-Spec)
206000	K6.07	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	ECCS keep fill system: BWR-2, 3,
206000	K6.08	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	Reactor pressure: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
206000	K6.09	Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	Condensate storage and transfer system: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K6.10	Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	PCIS: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K6.11	Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	Nuclear boiler instrumentation: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
206000	K6.12	Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION	Reactor water level: BWR-2, 3, 4
		Clinton is a BWR-6 and does not utilize this system.	
Isolation (Emergency) Condenser			
207000	2.1.27	Conduct of Operations	Knowledge of system purpose and/or
		Clinton is a BWR-6 and does not utilize this system.	
207000	2.1.28	Conduct of Operations	Knowledge of the purpose and function of major system components and controls.
		Clinton is a BWR-6 and does not utilize this system.	
207000	2.1.32	Conduct of Operations	Ability to explain and apply system limits and precautions.
		Clinton is a BWR-6 and does not utilize this system.	
207000	2.1.33	Conduct of Operations	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.
		Clinton is a BWR-6 and does not utilize this system.	
207000	2.2.22	Equipment Control	Knowledge of limiting conditions for operations and safety limits.
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.01	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Isolation condenser level: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.02	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Shell side water temperature: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.03	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Steam flow: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
207000	A1.04	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Condensate flow: BWR-2, 3(P-Spec)
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.05	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Reactor pressure: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.06	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Reactor water level: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.07	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Vent radiation level: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.08	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Cooldown rate: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.09	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Valve operations: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A1.10	Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:	Primary side temperature: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A2.01	Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal	Tube bundle leak: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A2.02	Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal	High vent radiation: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	A2.03	Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal	PCIS signal resulting in system isolation: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
207000	A2.04	Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal	Inadequate system flow: BWR-2, 3
207000	A2.05	*Clinton is a BWR-6 and does not utilize this system.* Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal	Insufficient shell side makeup flow: BWR-2,
207000	A2.06	*Clinton is a BWR-6 and does not utilize this system.* Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal	Valve openings: BWR-2, 3
207000	A2.07	*Clinton is a BWR-6 and does not utilize this system.* Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal	Valve closures: BWR-2, 3
207000	A2.08	*Clinton is a BWR-6 and does not utilize this system.* Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal	System initiation: BWR-2, 3
207000	A3.01	*Clinton is a BWR-6 and does not utilize this system.* Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER including:	Isolation condenser level: BWR-2, 3
207000	A3.02	*Clinton is a BWR-6 and does not utilize this system.* Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER including:	Reactor pressure: BWR-2, 3
207000	A3.03	*Clinton is a BWR-6 and does not utilize this system.* Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER including:	Reactor water level: BWR-2, 3
207000	A3.04	*Clinton is a BWR-6 and does not utilize this system.* Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER including:	Vent radiation levels: BWR-2, 3
207000	A3.05	*Clinton is a BWR-6 and does not utilize this system.* Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER including:	System lineup: BWR-2, 3
207000	A3.06	*Clinton is a BWR-6 and does not utilize this system.* Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER including:	Lights and alarms: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
207000	A3.07	Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER including:	Primary and shell side temperatures:
207000	A3.08	*Clinton is a BWR-6 and does not utilize this system.* Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER including:	System flow: BWR-2, 3
207000	A4.01	*Clinton is a BWR-6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Isolation condenser level: BWR-2, 3
207000	A4.02	*Clinton is a BWR-6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Steam line pressure: BWR-2, 3
207000	A4.03	*Clinton is a BWR-6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Primary and shell side temperatures:
207000	A4.04	*Clinton is a BWR-6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Vent line radiation levels: BWR-2, 3
207000	A4.05	*Clinton is a BWR-6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Major system valves: BWR-2, 3
207000	A4.06	*Clinton is a BWR-6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Shell side makeup valves: BWR-2, 3
207000	A4.07	*Clinton is a BWR-6 and does not utilize this system.* Ability to manually operate and/or monitor in the control room:	Manually initiate the isolation condenser: BWR-2, 3
207000	K1.01	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Reactor vessel: BWR-2, 3
207000	K1.02	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Reactor pressure: BWR-2, 3
207000	K1.03	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Reactor water level: BWR-2, 3
207000	K1.04	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Condensate transfer system: BWR-2, 3
207000	K1.05	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Demineralized water system: BWR-2,
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
207000	K1.06	Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Fire protection/service water: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K1.07	Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	LPCI: BWR-2, 3(P-Spec)
		Clinton is a BWR-6 and does not utilize this system.	
207000	K1.08	Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Recirculation system: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K1.09	Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Main steam system: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K1.10	Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Plant air systems: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K1.11	Knowledge of the physical connections and/or cause- effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:	Primary containment isolation system:
		Clinton is a BWR-6 and does not utilize this system.	
207000	K2.01	Knowledge of electrical power supplies to the following:	Motor operated valves: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K2.02	Knowledge of electrical power supplies to the following:	Initiation logic: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K3.01	Knowledge of the effect that a loss or malfunction of the ISOLATION (EMERGENCY) CONDENSER will have on following:	Reactor pressure control during conditions in which the reactor vessel is isolated:
		Clinton is a BWR-6 and does not utilize this system.	
207000	K3.02	Knowledge of the effect that a loss or malfunction of the ISOLATION (EMERGENCY) CONDENSER will have on following:	Reactor water level (EPG's address the isolation condenser as a water source):
		Clinton is a BWR-6 and does not utilize this system.	
207000	K4.01	Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:	Isolation of the system in the event of a line break: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K4.02	Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:	Automatic initiation: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K4.03	Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:	Filling of the system: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
207000	K4.04	Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:	Steam and condensate flow indication: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K4.05	Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:	Detection of a tube bundle leak: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K4.06	Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:	Throttling of system flow: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K4.07	Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:	Manual operation of the system: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K4.08	Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:	Protection against incomplete steam condensation (condensate outlet valve does not fully open): BWR-2,3,(P-Spec)
		Clinton is a BWR-6 and does not utilize this system.	
207000	K5.01	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Flow measurement across an elbow using differential pressure: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K5.02	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Heat exchanger operation: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K5.03	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Heat transfer: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K5.04	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Latent heat of vaporization: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K5.05	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Saturated steam: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K5.06	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Saturated liquid: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K5.07	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Temperature sensing: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
207000	K5.08	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Level indicator operation: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
207000	K5.09	Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	Cooldown rate: BWR-2, 3
207000	K5.10	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:	System venting: BWR-2, 3
207000	K6.01	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:	Demineralized water system: BWR-2,
207000	K6.02	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:	Fire protection/service water system:
207000	K6.03	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:	Condensate transfer system: BWR-2, 3
207000	K6.04	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:	Plant air systems: BWR-2, 3
207000	K6.05	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:	Primary containment isolation system:
207000	K6.06	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:	Recirculation system: BWR-2, 3
207000	K6.07	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:	A.C. power: BWR-2, 3
207000	K6.08	*Clinton is a BWR-6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:	D.C. power: BWR-2, 3
		Clinton is a BWR-6 and does not utilize this system.	
Low Pressure Core Spray System			
209001	A2.11	Ability to (a) predict the impacts of the following on the LOW PRESSURE CORE SPRAY SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or	Loss of fire protection: BWR-1
209001	A4.06	*Clinton is a BWR-6 and does not utilize this feature.* Ability to manually operate and/or monitor in the control room:	Testable check valves
		Clinton does not utilize this function.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
209001	A4.14	Ability to manually operate and/or monitor in the control room:	Containment level: BWR-1
		Clinton is a BWR-6 and does not utilize this feature.	
209001	K5.06	Knowledge of the operational implications of the following concepts as they apply to LOW PRESSURE CORE SPRAY SYSTEM:	Recirculation operation:
		Clinton is a BWR-6 and does not utilize this feature.	
209001	K6.09	Knowledge of the effect that a loss or malfunction of the following will have on the LOW PRESSURE CORE SPRAY SYSTEM:	Fire protection: BWR-1
		Clinton is a BWR-6 and does not utilize this feature.	
High Pressure Core Spray System (HPCS)			
209002	A4.06	Ability to manually operate and/or monitor in the control room:	Testable check valve: BWR-5, 6
		Clinton removed this function.	
209002	K1.13	Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE CORE SPRAY SYSTEM (HPCS)	Instrument nitrogen: BWR-5, 6
		Clinton is a BWR 6 and does not utilize this configuration.	
Standby Liquid Control System			
211000	A3.08	Ability to monitor automatic operations of the STANDBY LIQUID CONTROL SYSTEM	System initiation: Plant-Specific
		Clinton does not have automatic SLC system initiation; therefore, operators cannot be expected to monitor automatic system initiation.	
211000	K1.10	Knowledge of the physical connections and/or cause- effect relationships between STANDBY LIQUID CONTROL SYSTEM and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
211000	K4.09	Knowledge of STANDBY LIQUID CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Dampening of positive displacement pump discharge oscillations (accumulators): Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
211000	K5.05	Knowledge of the operational implications of the following concepts as they apply to STANDBY LIQUID CONTROL SYSTEM:	Accumulator operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
211000	K6.05	Knowledge of the effect that a loss or malfunction of the following will have on the STANDBY LIQUID CONTROL SYSTEM:	HPCI system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Reactor Protection System			
212000	A1.01	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR PROTECTION SYSTEM controls	RPS motor-generator output voltage
		Clinton is a BWR 6 and does not utilize this system.	
212000	A1.02	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR PROTECTION SYSTEM controls	RPS motor-generator output amps
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
212000	A1.03	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR PROTECTION SYSTEM controls	RPS motor-generator output frequency: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
212000	A1.09	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR PROTECTION SYSTEM controls	Individual relay status: Plant-Specific
		Clinton is a BWR 6 and uses a solid state system.	
212000	A2.01	Ability to (a) predict the impacts of the following on the REACTOR PROTECTION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	RPS motor-generator set failure
		Clinton is a BWR 6 and does not utilize this system.	
212000	A2.19	Ability to (a) predict the impacts of the following on the REACTOR PROTECTION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Partial system activation (half-SCRAM)
		Clinton is a BWR 6 and does not utilize this system.	
212000	A2.21	Ability to (a) predict the impacts of the following on the REACTOR PROTECTION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Failure of individual relays to reposition: Plant-Specific
		Clinton is a BWR 6 and utilizes a solid state system.	
212000	A3.02	Ability to monitor automatic operations of the REACTOR PROTECTION SYSTEM including:	Individual system relay status:
		Clinton is a BWR 6 and utilizes a solid state system.	
212000	A4.03	Ability to manually operate and/or monitor in the control room:	Provide manual select rod insertion: Plant-Specific
		Clinton is a BWR 6 and does not utilize this function.	
212000	A4.08	Ability to manually operate and/or monitor in the control room:	Individual system relay status:
		Clinton is a BWR 6 and utilizes a solid state system.	
212000	K2.01	Knowledge of electrical power supplies to the following:	RPS motor-generator sets
		Clinton is a BWR 6 and does not utilize this system.	
212000	K4.06	Knowledge of REACTOR PROTECTION SYSTEM design feature(s) and/or interlocks which provide for the following:	Select rod insertion: Plant-Specific
		Clinton is a BWR 6 and does not utiize this function.	
Rod Position Information System			
214000	K1.01	Knowledge of the physical connections and/or cause- effect relationships between ROD POSITION INFORMATION SYSTEM and the	RWM: Plant-Specific
		Clinton is a BWR-6 and does not utilize this system.	
214000	K1.02	Knowledge of the physical connections and/or cause- effect relationships between ROD POSITION INFORMATION SYSTEM and the	RSCS: Plant-Specific
		Clinton is a BWR-6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
214000	K1.04	Knowledge of the physical connections and/or cause- effect relationships between ROD POSITION INFORMATION SYSTEM and the	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this function.	
214000	K3.01	Knowledge of the effect that a loss or malfunction of the ROD POSITION INFORMATION SYSTEM will have on following:	RWM: Plant-Specific
		Clinton is a BWR-6 and does not utilize this system.	
214000	K3.02	Knowledge of the effect that a loss or malfunction of the ROD POSITION INFORMATION SYSTEM will have on following:	RSCS: Plant-Specific
		Clinton is a BWR-6 and does not utilize this system.	
214000	K3.03	Knowledge of the effect that a loss or malfunction of the ROD POSITION INFORMATION SYSTEM will have on following:	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utikize this functions.	
Traversing In-Core Probe			
215001	A1.03	Ability to predict and/or monitor changes in parameters associated with operating the TRAVERSING IN-CORE PROBE controls	Valve status: Mark-I&II(Not-BWR1)
		Clinton is a BWR 6 and does not utilize this configuration.	
215001	A2.01	Ability to (a) predict the impacts of the following on the TRAVERSING IN-CORE PROBE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Low reactor water level:
		Clinton is a BWR 6 and does not utilize this configuration.	
215001	A2.02	Ability to (a) predict the impacts of the following on the TRAVERSING IN-CORE PROBE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	High primary containment pressure: Mark-I&II(Not-BWR1)
		Clinton is a BWR 6 and does not utilize this configuration.	
215001	A2.06	Ability to (a) predict the impacts of the following on the TRAVERSING IN-CORE PROBE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Valve closures: Mark-I&II(Not-BWR1)
		Clinton is a BWR 6 and does not utilize this configuration.	
215001	A2.07	Ability to (a) predict the impacts of the following on the TRAVERSING IN-CORE PROBE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Failure to retract during accident conditions: Mark-I&II(Not-BWR1)
		Clinton is a BWR 6 and does not utilize this configuration.	
215001	A2.08	Ability to (a) predict the impacts of the following on the TRAVERSING IN-CORE PROBE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Failure to retract to shield: (Not-BWR1)
		At Clinton, Reactor Engineers, not operators, perform all TIP tasks. Detectors are stored in the RPV; there are no containment isolations associated with them.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
215001	A3.03	Ability to monitor automatic operations of the TRAVERSING IN-CORE PROBE including:	Valve operation: Not-BWR1
		Clinton is a BWR 6 and does not utilize isolation valves.	
215001	A4.03	Ability to manually operate and/or monitor in the control room:	Isolation valves: Mark-I&II(Not-BWR1)
		Clinton is a BWR 6 and does not utilize this configuration.	
215001	K1.03	Knowledge of the physical connections and/or cause- effect relationships between TRAVERSING IN-CORE PROBE and the	Nitrogen purge system: P-Spec(Not-BWR1)
		Clinton does utilize nitrogen purge for TIPS.	
215001	K2.01	Knowledge of electrical power supplies to the following:	Shear valves: Mark-I&II(Not-BWR1)
		Clinton is a BWR 6 and does not utilize this configuration.	
215001	K4.01	Knowledge of TRAVERSING IN-CORE PROBE design feature(s) and/or interlocks which provide for the following:	Primary containment isolation: Mark-I&II(Not-BWR1)
		Clinton is a BWR 6 and does not utilize this configuration.	
215001	K6.04	Knowledge of the effect that a loss or malfunction of the following will have on the TRAVERSING IN-CORE PROBE:	Primary containment isolation system: Mark-I&II(Not-BWR1)
		Clinton is a BWR 6 and does not utilize this configuration.	
Rod Block Monitor System			
215002	2.1.27	Conduct of Operations	Knowledge of system purpose and/or
		Clinton is a BWR 6 and does not utilize this system.	
215002	2.1.28	Conduct of Operations	Knowledge of the purpose and function of major system components and controls.
		Clinton is a BWR 6 and does not utilize this system.	
215002	2.1.32	Conduct of Operations	Ability to explain and apply system limits and precautions.
		Clinton is a BWR 6 and does not utilize this system.	
215002	2.1.33	Conduct of Operations	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.
		Clinton is a BWR 6 and does not utilize this system.	
215002	2.2.22	Equipment Control	Knowledge of limiting conditions for operations and safety limits.
		Clinton is a BWR 6 and does not utilize this system.	
215002	A1.01	Ability to predict and/or monitor changes in parameters associated with operating the ROD BLOCK MONITOR SYSTEM controls including:	Trip reference: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A2.01	Ability to (a) predict the impacts of the following on the ROD BLOCK MONITOR SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Withdrawal of control rod in high power region of core: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
215002	A2.02	Ability to (a) predict the impacts of the following on the ROD BLOCK MONITOR SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Loss or reduction in recirculation system flow (flow comparator): BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A2.03	Ability to (a) predict the impacts of the following on the ROD BLOCK MONITOR SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Loss of associated reference APRM channel: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A2.04	Ability to (a) predict the impacts of the following on the ROD BLOCK MONITOR SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Power supply losses: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A2.05	Ability to (a) predict the impacts of the following on the ROD BLOCK MONITOR SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	RBM high or inoperable: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A3.01	Ability to monitor automatic operations of the ROD BLOCK MONITOR SYSTEM including:	Four rod display: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A3.02	Ability to monitor automatic operations of the ROD BLOCK MONITOR SYSTEM including:	Meters and recorders: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A3.03	Ability to monitor automatic operations of the ROD BLOCK MONITOR SYSTEM including:	Alarm and indicating lights: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A3.04	Ability to monitor automatic operations of the ROD BLOCK MONITOR SYSTEM including:	Verification or proper functioning/ operability: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A3.05	Ability to monitor automatic operations of the ROD BLOCK MONITOR SYSTEM including:	Back panel meters and indicating lights: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A3.06	Ability to monitor automatic operations of the ROD BLOCK MONITOR SYSTEM including:	Transfer to alternate APRM when referenced APRM bypassed: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A4.01	Ability to manually operate and/or monitor in the control room:	IRM/RBM recorder/switch: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A4.02	Ability to manually operate and/or monitor in the control room:	RBM back panel switches, meters and indicating lights: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	A4.03	Ability to manually operate and/or monitor in the control room:	Trip bypasses: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
215002	A4.04	Ability to manually operate and/or monitor in the control room:	Push to Check pushbutton: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
215002	A4.05	Ability to manually operate and/or monitor in the control room:	Setup pushbutton: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
215002	A4.06	Ability to manually operate and/or monitor in the control room:	Surveillance testing: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K1.01	Knowledge of the physical connections and/or cause- effect relationships between ROD BLOCK MONITOR SYSTEM and the following:	APRM: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K1.02	Knowledge of the physical connections and/or cause- effect relationships between ROD BLOCK MONITOR SYSTEM and the following:	LPRM: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K1.03	Knowledge of the physical connections and/or cause- effect relationships between ROD BLOCK MONITOR SYSTEM and the following:	Reactor manual control: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K1.04	Knowledge of the physical connections and/or cause- effect relationships between ROD BLOCK MONITOR SYSTEM and the following:	Recirculation system: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K1.05	Knowledge of the physical connections and/or cause- effect relationships between ROD BLOCK MONITOR SYSTEM and the following:	Four rod display: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K1.06	Knowledge of the physical connections and/or cause- effect relationships between ROD BLOCK MONITOR SYSTEM and the following:	Control rod selection: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K1.07	Knowledge of the physical connections and/or cause- effect relationships between ROD BLOCK MONITOR SYSTEM and the following:	IRM: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K2.01	Knowledge of electrical power supplies to the following:	RBM channels: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K2.02	Knowledge of electrical power supplies to the following:	Recorders: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K2.03	Knowledge of electrical power supplies to the following:	APRM channels: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K3.01	Knowledge of the effect that a loss or malfunction of the ROD BLOCK MONITOR SYSTEM will have on following:	Reactor manual control system: BWR-3, 4,
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
215002	K3.02	Knowledge of the effect that a loss or malfunction of the ROD BLOCK MONITOR SYSTEM will have on following:	Limiting control rod pattern: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
215002	K4.01	Knowledge of ROD BLOCK MONITOR SYSTEM design feature(s) and/or interlocks which provide for the following:	Prevent control rod withdrawal: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K4.02	Knowledge of ROD BLOCK MONITOR SYSTEM design feature(s) and/or interlocks which provide for the following:	Allows stepping up of rod block setpoint: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K4.03	Knowledge of ROD BLOCK MONITOR SYSTEM design feature(s) and/or interlocks which provide for the following:	Initiation point (30%): BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K5.01	Knowledge of the operational implications of the following concepts as they apply to ROD BLOCK MONITOR SYSTEM:	Trip reference selection: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
215002	K5.02	Knowledge of the operational implications of the following concepts as they apply to ROD BLOCK MONITOR SYSTEM:	Null sequence control: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K6.01	Knowledge of the effect that a loss or malfunction of the following will have on the ROD BLOCK MONITOR SYSTEM:	RPS: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K6.02	Knowledge of the effect that a loss or malfunction of the following will have on the ROD BLOCK MONITOR SYSTEM:	Instrument power: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
215002	K6.03	Knowledge of the effect that a loss or malfunction of the following will have on the ROD BLOCK MONITOR SYSTEM:	Essential power: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
215002	K6.04	Knowledge of the effect that a loss or malfunction of the following will have on the ROD BLOCK MONITOR SYSTEM:	APRM reference channel: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	
215002	K6.05	Knowledge of the effect that a loss or malfunction of the following will have on the ROD BLOCK MONITOR SYSTEM:	LPRM detectors: BWR-3, 4, 5
		Clinton is a BWR 6 and does not utilize this system.	

Intermediate Range Monitor (IRM) System

215003	K1.02	Knowledge of the physical connections and/or cause- effect relationships between INTERMEDIATE RANGE MONITOR (IRM) SYSTEM and the following:	Reactor manual control
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
215003	K3.02	Knowledge of the effect that a loss or malfunction of the INTERMEDIATE RANGE MONITOR (IRM) SYSTEM will have on following:	Reactor manual control
		Clinton is a BWR 6 and does not utilize this system.	
Source Range Monitor (SRM) System			
215004	K1.02	Knowledge of the physical connections and/or cause- effect relationships between SOURCE RANGE MONITOR (SRM) SYSTEM and the	Reactor manual control
		Clinton is a BWR 6 and does not utilize this system.	
215004	K3.02	Knowledge of the effect that a loss or malfunction of the SOURCE RANGE MONITOR (SRM) SYSTEM will have on following:	Reactor manual control: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Average Power Range Monitor/Local Power			
215005	K1.03	Knowledge of the physical connections and/or cause- effect relationships between APRM/LPRM and the following:	RBM: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
215005	K1.10	Knowledge of the physical connections and/or cause- effect relationships between APRM/LPRM and the following:	Reactor manual control system:
		Clinton is a BWR 6 and does not utilize this system.	
215005	K3.03	Knowledge of the effect that a loss or malfunction of the APRM/LPRM will have on	Reactor manual control system:
		Clinton is a BWR 6 and does not utilize this system.	
Nuclear Boiler Instrumentation			
216000	K1.14	Knowledge of the physical connections and/or cause- effect relationships between NUCLEAR BOILER INSTRUMENTATION and the following:	High pressure coolant injection:
		Clinton is a BWR 6 and does not utilize this system.	
216000	K1.15	Knowledge of the physical connections and/or cause- effect relationships between NUCLEAR BOILER INSTRUMENTATION and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
216000	K3.14	Knowledge of the effect that a loss or malfunction of the NUCLEAR BOILER Instrumentation will have on following:	High pressure coolant injection:
		Clinton is a BWR 6 and does not utilize this system.	
216000	K3.15	Knowledge of the effect that a loss or malfunction of the NUCLEAR BOILER Instrumentation will have on following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
RHR/LPCI: Torus/Suppression Pool Cooling			
219000	A1.09	Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE controls including:	Suppression chamber air temperature: Plant-Specific
		Clinton is a BWR 6 and does not have a suppression chamber.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
219000	K1.07	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE and the following:	Condensate transfer
		This system does not serve any purpose for suppression pool cooling.	
219000	K6.07	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	Condensate transfer
		This system is not utilized for this function.	
Primary Containment System and Auxiliaries			
223001	A1.11	Ability to predict and/or monitor changes in parameters associated with operating the PRIMARY CONTAINMENT AND AUX.	Reactor building to suppression chamber differential pressure: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
223001	A1.12	Ability to predict and/or monitor changes in parameters associated with operating the PRIMARY CONTAINMENT AND AUX.	Moisture concentration
		Clinton is a BWR 6 and does not utilize this system.	
223001	A3.06	Ability to monitor automatic operations of the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES including:	Drywell/suppression chamber differential pressure: Mark-I,II
		Clinton is a BWR 6 and does not utilize this system.	
223001	A4.02	Ability to manually operate and/or monitor in the control room:	ACAD compressors: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
223001	A4.10	Ability to manually operate and/or monitor in the control room:	Drywell nitrogen makeup: Mark-I,II
		Clinton is a BWR 6 and does not utilize this system.	
223001	K1.15	Knowledge of the physical connections and/or cause- effect relationships between PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
223001	K1.17	Knowledge of the physical connections and/or cause- effect relationships between PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES and the following:	Reactor building HVAC: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
223001	K1.18	Knowledge of the physical connections and/or cause- effect relationships between PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES and the following:	Drywell pneumatic compressors:
		Clinton is a BWR 6 and does not utilize this system.	
223001	K2.02	Knowledge of electrical power supplies to the following:	Drywell compressors
		Clinton is a BWR 6 and does not utilize this system.	
223001	K2.03	Knowledge of electrical power supplies to the following:	Pumpback compressors: Plant-Specific
		Clinton is a BWR 6 and does not utilize this component.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
223001	K3.10	Knowledge of the effect that a loss or malfunction of the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES will have on	Containment/drywell moisture content
		Clinton is a BWR 6 and does not utilize this system.	
223001	K5.15	Knowledge of the operational implications of the following concepts as they apply to PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES:	Moisture content measurement:
		Clinton is a BWR 6 and does not utilize this system.	

Primary Containment Isolation System/Nuclear

223002	K1.04	Knowledge of the physical connections and/or cause- effect relationships between PCIS/NSSSS and the following:	High pressure coolant injection:
		Clinton is a BWR 6 and does not utilize this system.	
223002	K1.05	Knowledge of the physical connections and/or cause- effect relationships between PCIS/NSSSS and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
223002	K1.13	Knowledge of the physical connections and/or cause- effect relationships between PCIS/NSSSS and the following:	Traversing in-core probe system
		Clinton utilizes a TIP that is inside the primary containment.	
223002	K1.18	Knowledge of the physical connections and/or cause- effect relationships between PCIS/NSSSS and the following:	Reactor building drainage system:
		Clinton is a BWR 6 and does not utilize this system.	
223002	K1.22	Knowledge of the physical connections and/or cause- effect relationships between PCIS/NSSSS and the following:	Containment nitrogen inerting system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
223002	K3.04	Knowledge of the effect that a loss or malfunction of the PCIS/NSSSS will have on	Reactor building radiation level
		Clinton is a BWR 6 and does not utilize this system.	
223002	K3.12	Knowledge of the effect that a loss or malfunction of the PCIS/NSSSS will have on	High pressure coolant injection:
		Clinton is a BWR 6 and does not utilize this system.	
223002	K3.13	Knowledge of the effect that a loss or malfunction of the PCIS/NSSSS will have on	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
223002	K3.21	Knowledge of the effect that a loss or malfunction of the PCIS/NSSSS will have on	Traversing in-core probe system
		Clinton utilizes a TIP that is inside the primary containment.	
223002	K3.24	Knowledge of the effect that a loss or malfunction of the PCIS/NSSSS will have on	Reactor building drainage system
		Clinton is a BWR 6 and does not utilize this system.	
223002	K3.28	Knowledge of the effect that a loss or malfunction of the PCIS/NSSSS will have on	Containment nitrogen inerting system
		Clinton is a BWR 6 and does not utilize this system.	

RHR/LPCI: Containment Spray System Mode

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
226001	A1.03	Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE controls including:	Suppression chamber pressure: Mark-I-II
		Clinton is a BWR 6 and does not utilize this system.	
226001	A1.04	Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE controls including:	Suppression pool temperature: Mark-I-II
		Clinton is a BWR 6 and does not utilize this system.	
226001	A2.19	Ability to (a) predict the impacts of the following on the RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Low (or negative) suppression chamber pressure during system operation: Mark-I-II
		Clinton is a BWR 6 and does not utilize this system.	
226001	A4.15	Ability to manually operate and/or monitor in the control room:	Suppression chamber pressure: Mark-I-II
		Clinton is a BWR 6 and does not utilize this system.	
226001	K1.06	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE and the following:	Condensate transfer
		Clinton is a BWR 6 and does not utilize this system.	
226001	K1.09	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE and the following:	Drywell (spray penetration): Mark-I-II
		Clinton is a BWR 6 and does not utilize this system.	
226001	K1.12	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE and the following:	Suppression pool (spray penetration): Plant-Specific
		Clinton is a BWR 6 and does not utilize this suppression pool spray.	
226001	K6.06	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: CONTAINMENT SPRAY SYSTEM	Condensate transfer
		Clinton is a BWR 6 and does not utilize this system.	
226001	K6.09	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: CONTAINMENT SPRAY SYSTEM	Reactor building to suppression chamber vacuum breakers: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
226001	K6.10	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: CONTAINMENT SPRAY SYSTEM	Suppression chamber to drywell vacuum breakers: Mark-1-II
		Clinton is a BWR 6 and does not utilize this system.	

RHR/LPCI: Torus/Suppression Pool Spray

230000	2.1.14	Conduct of Operations	Knowledge of system status criteria which require the notification of plant personnel.
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	2.1.2	Conduct of Operations	Knowledge of operator responsibilities during all modes of plant operation.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.1.23	Conduct of Operations	Ability to perform specific system and integrated plant procedures during different modes of plant operation.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.1.27	Conduct of Operations	Knowledge of system purpose and/or
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.1.28	Conduct of Operations	Knowledge of the purpose and function of major system components and controls.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.1.30	Conduct of Operations	Ability to locate and operate components, including local controls.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.1.32	Conduct of Operations	Ability to explain and apply system limits and precautions.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.1.33	Conduct of Operations	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.2.22	Equipment Control	Knowledge of limiting conditions for operations and safety limits.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.2.25	Equipment Control	Knowledge of bases in technical specifications for limiting conditions for operations and safety limits.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.4.30	Emergency Procedures and Plan	Knowledge of which events related to system operations/status should be reported to outside agencies.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.4.31	Emergency Procedures and Plan	Knowledge of annunciators alarms and indications, and use of the response instructions.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.4.49	Emergency Procedures and Plan	Ability to perform without reference to procedures those actions that require immediate operation of system components
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.4.50	Emergency Procedures and Plan	Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.
		Clinton is a BWR 6 and does not utilize this system.	
230000	2.4.6	Emergency Procedures and Plan	Knowledge symptom based EOP mitigation strategies.
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	A1.01	Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	Suppression chamber pressure
230000	A1.02	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	Suppression pool temperature
230000	A1.03	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	Drywell pressure
230000	A1.04	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	System flow
230000	A1.05	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	System pressure
230000	A1.06	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	Suppression pool level
230000	A1.07	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	Condensate storage tank level
230000	A1.08	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	Motor amps
230000	A1.09	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	Emergency generator loading
230000	A1.10	*Clinton is a BWR 6 and does not utilize this system.* Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	System lineup
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	A1.11	Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:	Suppression chamber air temperature
		Clinton is a BWR 6 and does not utilize this system.	
230000	A2.01	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Inadequate net positive suction head
		Clinton is a BWR 6 and does not utilize this system.	
230000	A2.02	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Pump trips
		Clinton is a BWR 6 and does not utilize this system.	
230000	A2.03	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Valve closures
		Clinton is a BWR 6 and does not utilize this system.	
230000	A2.04	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Valve openings
		Clinton is a BWR 6 and does not utilize this system.	
230000	A2.05	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	A.C. electrical failures
		Clinton is a BWR 6 and does not utilize this system.	
230000	A2.06	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	D.C. electrical failures
		Clinton is a BWR 6 and does not utilize this system.	
230000	A2.07	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Emergency generator failure
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	A2.08	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR 6 and does not utilize this system.*	Pump seal failure
230000	A2.09	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR 6 and does not utilize this system.*	Inadequate room cooling
230000	A2.10	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR 6 and does not utilize this system.*	Nuclear boiler instrument failures
230000	A2.11	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR 6 and does not utilize this system.*	Motor operated valve failures
230000	A2.12	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR 6 and does not utilize this system.*	Valve logic failure
230000	A2.13	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR 6 and does not utilize this system.*	High suppression pool level
230000	A2.14	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR 6 and does not utilize this system.*	Low (or negative) suppression pool pressure during system operation
230000	A2.15	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: *Clinton is a BWR 6 and does not utilize this system.*	Loss of coolant accident

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	A2.16	Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Loss of, or inadequate, heat exchanger cooling flow
		Clinton is a BWR 6 and does not utilize this system.	
230000	A3.01	Ability to monitor automatic operations of the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE including:	Valve operation
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.01	Ability to manually operate and/or monitor in the control room:	Pumps
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.02	Ability to manually operate and/or monitor in the control room:	Spray valves
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.03	Ability to manually operate and/or monitor in the control room:	Keep fill system
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.04	Ability to manually operate and/or monitor in the control room:	Minimum flow valves
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.05	Ability to manually operate and/or monitor in the control room:	Heat exchanger cooling flow
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.06	Ability to manually operate and/or monitor in the control room:	Valve logic reset following automatic initiation of LPCI/RHR in injection mode
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.07	Ability to manually operate and/or monitor in the control room:	System flow
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.08	Ability to manually operate and/or monitor in the control room:	Pump/system discharge pressure
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.09	Ability to manually operate and/or monitor in the control room:	Indicating lights and alarms
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.10	Ability to manually operate and/or monitor in the control room:	Condensate storage tank level
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.11	Ability to manually operate and/or monitor in the control room:	System venting
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.12	Ability to manually operate and/or monitor in the control room:	Suppression pool level
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.13	Ability to manually operate and/or monitor in the control room:	Suppression chamber pressure
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	A4.14	Ability to manually operate and/or monitor in the control room:	Suppression pool temperature
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.15	Ability to manually operate and/or monitor in the control room:	Drywell pressure
		Clinton is a BWR 6 and does not utilize this system.	
230000	A4.16	Ability to manually operate and/or monitor in the control room:	The override for suppression pool spray valve logic
		Clinton is a BWR 6 and does not utilize this system.	
230000	K1.01	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	Suppression pool
		Clinton is a BWR 6 and does not utilize this system.	
230000	K1.02	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	Condensate storage and transfer system
		Clinton is a BWR 6 and does not utilize this system.	
230000	K1.03	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	LPCI/RHR piping
		Clinton is a BWR 6 and does not utilize this system.	
230000	K1.04	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	LPCI/RHR pumps
		Clinton is a BWR 6 and does not utilize this system.	
230000	K1.05	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	A.C. electrical
		Clinton is a BWR 6 and does not utilize this system.	
230000	K1.06	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	Keep fill system
		Clinton is a BWR 6 and does not utilize this system.	
230000	K1.07	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	D.C. electrical
		Clinton is a BWR 6 and does not utilize this system.	
230000	K1.08	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	Nuclear boiler instrumentation
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	K1.09	Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	Reactor building drain system
230000	K1.10	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the physical connections and/or cause- effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:	Component cooling water systems
230000	K2.01	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of electrical power supplies to the following:	Valves
230000	K2.02	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of electrical power supplies to the following:	Pumps
230000	K3.01	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE	Suppression chamber pressure
230000	K3.02	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE	Suppression pool temperature
230000	K3.03	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE	Drywell pressure
230000	K3.04	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the effect that a loss or malfunction of the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE	Suppression chamber air temperature
230000	K4.01	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Surveillance for all operable components
230000	K4.02	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Redundancy
230000	K4.03	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Unintentional reduction in vessel injection flow during accident conditions
230000	K4.04	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Prevention of piping overpressurization
230000	K4.05	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Pump minimum flow protection
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	K4.06	Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Pump motor cooling
		Clinton is a BWR 6 and does not utilize this system.	
230000	K4.07	Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Prevention of water hammer
		Clinton is a BWR 6 and does not utilize this system.	
230000	K4.08	Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Adequate pump net positive suction head
		Clinton is a BWR 6 and does not utilize this system.	
230000	K4.09	Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Spray flow cooling
		Clinton is a BWR 6 and does not utilize this system.	
230000	K4.10	Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide	Prevention of leakage to the environment through system heat exchanger
		Clinton is a BWR 6 and does not utilize this system.	
230000	K5.01	Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:	System venting
		Clinton is a BWR 6 and does not utilize this system.	
230000	K5.02	Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:	Pump cavitation
		Clinton is a BWR 6 and does not utilize this system.	
230000	K5.03	Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:	Pressure measurement
		Clinton is a BWR 6 and does not utilize this system.	
230000	K5.04	Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:	Evaporative cooling
		Clinton is a BWR 6 and does not utilize this system.	
230000	K5.05	Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:	Convective cooling
		Clinton is a BWR 6 and does not utilize this system.	
230000	K5.06	Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:	Heat exchanger operation
		Clinton is a BWR 6 and does not utilize this system.	
230000	K5.07	Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:	Vacuum breaker operation
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.01	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	A.C. electrical
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
230000	K6.02	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	D.C. electrical
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.03	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	Emergency generator
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.04	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	Keep fill system
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.05	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	Suppression pool
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.06	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	Condensate storage and transfer system
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.07	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	ECCS room cooling
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.08	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	Nuclear boiler instrumentation
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.09	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	Reactor building to suppression pool vacuum breakers
		Clinton is a BWR 6 and does not utilize this system.	
230000	K6.10	Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL	Component cooling water systems
		Clinton is a BWR 6 and does not utilize this system.	

Fuel Pool Cooling and Clean-up

233000	K1.07	Knowledge of the physical connections and/or cause- effect relationships between FUEL POOL COOLING AND CLEAN-UP and the following:	Condensate system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
233000	K1.11	Knowledge of the physical connections and/or cause- effect relationships between FUEL POOL COOLING AND CLEAN-UP and the following:	Reactor building drainage system:
		Clinton is a BWR 6 and does not utilize this configuration.	
233000	K6.05	Knowledge of the effect that a loss or malfunction of the following will have on the FUEL POOL COOLING AND CLEAN-UP:	Condensate system
		Clinton is a BWR 6 and does not utilize this configuration.	

Main and Reheat Steam System

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
239001	K1.18	Knowledge of the physical connections and/or cause- effect relationships between MAIN AND REHEAT STEAM SYSTEM and the following:	High pressure coolant injection:
		Clinton is a BWR 6 and does not utilize this system.	
239001	K1.21	Knowledge of the physical connections and/or cause- effect relationships between MAIN AND REHEAT STEAM SYSTEM and the following:	Isolation condenser system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
239001	K3.10	Knowledge of the effect that a loss or malfunction of the MAIN AND REHEAT STEAM SYSTEM will have on following:	High pressure coolant injection system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
239001	K3.12	Knowledge of the effect that a loss or malfunction of the MAIN AND REHEAT STEAM SYSTEM will have on following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
239001	K3.14	Knowledge of the effect that a loss or malfunction of the MAIN AND REHEAT STEAM SYSTEM will have on following:	Residual heat removal system:
		Clinton does not utilize this function.	
239001	K5.07	Knowledge of the operational implications of the following concepts as they apply to MAIN AND REHEAT STEAM SYSTEM:	Hydraulic operated MSIV's
		Clinton is a BWR 6 and does not utilize this system.	
Relief/Safety Valves			
239002	K1.06	Knowledge of the physical connections and/or cause- effect relationships between RELIEF/SAFETY VALVES and the following:	Drywell instrument air/ drywell pneumatics: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
239002	K1.09	Knowledge of the physical connections and/or cause- effect relationships between RELIEF/SAFETY VALVES and the following:	Drywell pressure (for safety valves which discharge to the drywell airspace):
		Clinton is a BWR 6 and does not utilize this system.	
MSIV Leakage Control System			
239003	2.1.14	Conduct of Operations	Knowledge of system status criteria which require the notification of plant personnel.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.1.2	Conduct of Operations	Knowledge of operator responsibilities during all modes of plant operation.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.1.23	Conduct of Operations	Ability to perform specific system and integrated plant procedures during different modes of plant operation.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.1.27	Conduct of Operations	Knowledge of system purpose and/or
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.1.28	Conduct of Operations	Knowledge of the purpose and function of major system components and controls.
		The MSIV Leakage Control System has been abandoned in place at CPS.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
239003	2.1.30	Conduct of Operations	Ability to locate and operate components, including local controls.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.1.32	Conduct of Operations	Ability to explain and apply system limits and precautions.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.1.33	Conduct of Operations	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.2.22	Equipment Control	Knowledge of limiting conditions for operations and safety limits.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.2.25	Equipment Control	Knowledge of bases in technical specifications for limiting conditions for operations and safety limits.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.4.30	Emergency Procedures and Plan	Knowledge of which events related to system operations/status should be reported to outside agencies.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.4.31	Emergency Procedures and Plan	Knowledge of annunciators alarms and indications, and use of the response instructions.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.4.49	Emergency Procedures and Plan	Ability to perform without reference to procedures those actions that require immediate operation of system components
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.4.50	Emergency Procedures and Plan	Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	2.4.6	Emergency Procedures and Plan	Knowledge symptom based EOP mitigation strategies.
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A1.01	Ability to predict and/or monitor changes in parameters associated with operating the MSIV LEAKAGE CONTROL SYSTEM controls	Main steam line pressure: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A1.02	Ability to predict and/or monitor changes in parameters associated with operating the MSIV LEAKAGE CONTROL SYSTEM controls	Heater operation: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A1.03	Ability to predict and/or monitor changes in parameters associated with operating the MSIV LEAKAGE CONTROL SYSTEM controls	Dilution air flow: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A1.04	Ability to predict and/or monitor changes in parameters associated with operating the MSIV LEAKAGE CONTROL SYSTEM controls	Status indicating lights and alarms: BWR-4,
		The MSIV Leakage Control System has been abandoned in place at CPS.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
239003	A1.05	Ability to predict and/or monitor changes in parameters associated with operating the MSIV LEAKAGE CONTROL SYSTEM controls	System lineup: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A1.06	Ability to predict and/or monitor changes in parameters associated with operating the MSIV LEAKAGE CONTROL SYSTEM controls	MSIV leakage flow: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A1.07	Ability to predict and/or monitor changes in parameters associated with operating the MSIV LEAKAGE CONTROL SYSTEM controls	Reactor building temperature: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.01	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Inboard MSIV valve leakage: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.02	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Outboard MSIV valves leakage: BWR-4, 5,
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.03	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Low dilution air flow (inboard and/or outboard): BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.04	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Outboard system logic failure: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.05	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Inboard system logic failure: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.06	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Blower failure: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.07	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Heater failure: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
239003	A2.08	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Motor operator valve failure(s): BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.09	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Outboard main steamline high pressure: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.10	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	A.C. distribution power failures: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.11	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	High reactor pressure: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A2.12	Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	MSIV valve failure to close: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A3.01	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	System logic initiation: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A3.02	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	Main steamline pressures: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A3.03	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	Dilution air flaws: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A3.04	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	MSIV leakage flows: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A3.05	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	Heater operation: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A3.06	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	System status lights and alarms: BWR-4, 5,
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A3.07	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	System lineups: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
239003	A3.08	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	Blower operation: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A3.09	Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:	Reactor building temperature: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.01	Ability to manually operate and/or monitor in the control room:	Manually initiate system operation: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.02	Ability to manually operate and/or monitor in the control room:	Surveillance testing: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.03	Ability to manually operate and/or monitor in the control room:	Main steamline pressures: BWR-4, 5, 6
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.04	Ability to manually operate and/or monitor in the control room:	Dilution air flows: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.05	Ability to manually operate and/or monitor in the control room:	MSIV leakage flows: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.06	Ability to manually operate and/or monitor in the control room:	Heater operation: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.07	Ability to manually operate and/or monitor in the control room:	System status lights and alarms: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.08	Ability to manually operate and/or monitor in the control room:	System lineups: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	A4.09	Ability to manually operate and/or monitor in the control room:	System reset: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K1.01	Knowledge of the physical connections and/or cause- effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:	Main steam system: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K1.02	Knowledge of the physical connections and/or cause- effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:	Standby gas treatment system: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K1.03	Knowledge of the physical connections and/or cause- effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:	Main steam line pressure instrumentation: BWR-4,5,6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K1.04	Knowledge of the physical connections and/or cause- effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:	A.C. electrical distribution: BWR-4, 5,
		The MSIV Leakage Control System has been abandoned in place at CPS.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
239003	K1.05	Knowledge of the physical connections and/or cause- effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:	Steam tunnel: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K1.06	Knowledge of the physical connections and/or cause- effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:	Radwaste system: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K1.07	Knowledge of the physical connections and/or cause- effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:	Floor drainage system: BWR-4, 5,
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K1.08	Knowledge of the physical connections and/or cause- effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:	Nuclear boiler instrumentation: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K2.01	Knowledge of electrical power supplies to the following:	Motor operated valves: BWR-4, 5,
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K2.02	Knowledge of electrical power supplies to the following:	Leakage control system blowers: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K2.03	Knowledge of electrical power supplies to the following:	Leakage control system heaters: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K3.01	Knowledge of the effect that a loss or malfunction of the MSIV LEAKAGE CONTROL SYSTEM will have on following:	Radiation release to the environment: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K4.01	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Performance of its safety function following a loss of offsite power: BWR-4, 5,
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K4.02	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Performance of intended safety function following any single active component failure: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K4.03	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	The prevention of inadvertent system operation: BWR-4,5,6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K4.04	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Surveillance for system operability: BWR-4,
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K4.05	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Assurance that any MSIV leakage will pass through the system and into standby gas treatment prior to release to the atmosphere: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
239003	K4.06	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	The depressurization of main steam piping prior to routing leakage through system:
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K4.07	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	The reduction of MSIV leakage temperature: BWR-4,5,6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K4.08	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Prevention of collected condensate in system bleed lines: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K4.09	Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	The dilution of MSIV leakage: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K6.01	Knowledge of the effect that a loss or malfunction of the following will have on the MSIV LEAKAGE CONTROL SYSTEM:	A.C. electrical distribution: BWR-4, 5,
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K6.02	Knowledge of the effect that a loss or malfunction of the following will have on the MSIV LEAKAGE CONTROL SYSTEM:	Standby gas treatment system: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
239003	K6.03	Knowledge of the effect that a loss or malfunction of the following will have on the MSIV LEAKAGE CONTROL SYSTEM:	Nuclear boiler instrumentation: BWR-4, 5, 6(P-Spec)
		The MSIV Leakage Control System has been abandoned in place at CPS.	
Reactor/Turbine Pressure Regulating System			
241000	A3.15	Ability to monitor automatic operations of the REACTOR/TURBINE PRESSURE REGULATING SYSTEM including:	Recirculation pump flow control:
		Clinton is a BWR 6 and there is no tie to this system.	
241000	K1.15	Knowledge of the physical connections and/or cause- effect relationships between REACTOR/TURBINE PRESSURE REGULATING SYSTEM and the following:	D.C. electrical power
		DC does not supply power to this system.	
241000	K1.23	Knowledge of the physical connections and/or cause- effect relationships between REACTOR/TURBINE PRESSURE REGULATING SYSTEM and the following:	Recirculation flow control system:
		Clinton is a BWR 6 and there is no tie to this system.	
241000	K1.37	Knowledge of the physical connections and/or cause- effect relationships between REACTOR/TURBINE PRESSURE REGULATING SYSTEM and the following:	Turbine stress evaluator: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
241000	K3.21	Knowledge of the effect that a loss or malfunction of the REACTOR/TURBINE PRESSURE REGULATING SYSTEM will have on	Recirculation flow control system:
		Clinton is a BWR 6 and there is no tie to this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
241000	K4.11	Knowledge of REACTOR/TURBINE PRESSURE REGULATING SYSTEM design feature(s) and/or interlocks which provide for the following:	Load following: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
241000	K4.12	Knowledge of REACTOR/TURBINE PRESSURE REGULATING SYSTEM design feature(s) and/or interlocks which provide for the following:	Recirculation flow control: Plant-Specific
		Clinton is a BWR 6 and there is no tie to this system.	
241000	K5.01	Knowledge of the operational Implications of the following concepts as they apply to REACTOR/TURBINE PRESSURE REGULATING SYSTEM:	Accumulator operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
241000	K5.07	Knowledge of the operational Implications of the following concepts as they apply to REACTOR/TURBINE PRESSURE REGULATING SYSTEM:	Unitized actuator operation: Fermi-Only
		Clinton is a BWR 6 and does not utilize this system.	
241000	K6.04	Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR/TURBINE PRESSURE REGULATING	Recirculation flow control system:
		Clinton is a BWR 6 and there is no tie to this system.	
Main Turbine Generator and Auxiliary Systems			
245000	K2.03	Knowledge of electrical power supplies to the following:	Amplidyne: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
245000	K2.05	Knowledge of electrical power supplies to the following:	Air seal oil pumps: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Reactor Condensate System			
256000	K1.03	Knowledge of the physical connections and/or cause- effect relationships between REACTOR CONDENSATE SYSTEM and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
256000	K1.12	Knowledge of the physical connections and/or cause- effect relationships between REACTOR CONDENSATE SYSTEM and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
256000	K1.14	Knowledge of the physical connections and/or cause- effect relationships between REACTOR CONDENSATE SYSTEM and the following:	RHR (LPCI): Plant-Specific
		Clinton does not utilize this function.	
256000	K1.17	Knowledge of the physical connections and/or cause- effect relationships between REACTOR CONDENSATE SYSTEM and the following:	ECCS keep fill system: Plant-Specific
		Clinton does not utilize this function from reactor condensate.	
256000	K3.05	Knowledge of the effect that a loss or malfunction of the REACTOR CONDENSATE SYSTEM will have on following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
256000	K3.07	Knowledge of the effect that a loss or malfunction of the REACTOR CONDENSATE SYSTEM will have on following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
256000	K4.01	Knowledge of REACTOR CONDENSATE SYSTEM design feature(s) and/or interlocks which provide for the following:	Condensate and/or booster pump auto start: Plant-Specific
		Clinton does not have an auto-start on the CD/CB pumps.	
256000	K4.08	Knowledge of REACTOR CONDENSATE SYSTEM design feature(s) and/or interlocks which provide for the following:	Dedicated ECCS water supply:
		Clinton does not utilize this function from reactor condensate.	
Reactor Feedwater System			
259001	A3.11	Ability to monitor automatic operations of the REACTOR FEEDWATER SYSTEM including:	Reactor feedpump runbacks: Plant-Specific
		Clinton does not utilize this function.	
259001	K1.02	Knowledge of the physical connections and/or cause- effect relationships between REACTOR FEEDWATER SYSTEM and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
259001	K1.18	Knowledge of the physical connections and/or cause- effect relationships between REACTOR FEEDWATER SYSTEM and the following:	Fire protection system (emergency cooling): Plant-Specific
		Clinton does not utilize this function through reactor feedwater.	
259001	K1.19	Knowledge of the physical connections and/or cause- effect relationships between REACTOR FEEDWATER SYSTEM and the following:	Redundant reactivity control system: Plant-Specific
		Clinton does not utilize this function.	
259001	K3.03	Knowledge of the effect that a loss or malfunction of the REACTOR FEEDWATER SYSTEM will have on following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
259001	K4.10	Knowledge of REACTOR FEEDWATER SYSTEM design feature(s) and/or interlocks which provide for the following:	Feedpump runbacks: Plant-Specific
		Clinton does not utilize feedpump runbacks.	
Reactor Water Level Control System			
259002	A1.06	Ability to predict and/or monitor changes in parameters associated with operating the REACTOR WATER LEVEL CONTROL SYSTEM controls including:	Feedwater string(s) selected for FWCI:
		Clinton is a BWR 6 and does not utilize this system.	
259002	A2.08	Ability to (a) predict the impacts of the following on the REACTOR WATER LEVEL CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Receipt of an ECCS initiation signal: FWCI
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
259002	A2.09	Ability to (a) predict the impacts of the following on the REACTOR WATER LEVEL CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	FWCI system failure alarm: FWCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	A3.08	Ability to monitor automatic operations of the REACTOR WATER LEVEL CONTROL SYSTEM including:	FWCI system initiation: FWCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	A4.08	Ability to manually operate and/or monitor in the control room:	Manually initiate FWCI: FWCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	K1.07	Knowledge of the physical connections and/or cause- effect relationships between REACTOR WATER LEVEL CONTROL SYSTEM and the	Rod worth minimizer: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
259002	K1.10	Knowledge of the physical connections and/or cause- effect relationships between REACTOR WATER LEVEL CONTROL SYSTEM and the	Emergency generator(s): FWCI/HPCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	K1.11	Knowledge of the physical connections and/or cause- effect relationships between REACTOR WATER LEVEL CONTROL SYSTEM and the	Drywell pressure: FWCI/HPCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	K1.12	Knowledge of the physical connections and/or cause- effect relationships between REACTOR WATER LEVEL CONTROL SYSTEM and the	Emergency condensate transfer pump: FWCI/HPCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	K1.16	Knowledge of the physical connections and/or cause- effect relationships between REACTOR WATER LEVEL CONTROL SYSTEM and the	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
259002	K2.02	Knowledge of electrical power supplies to the following:	Feedwater coolant injection (FWCI) initiation logic: FWCI/HPCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	K3.03	Knowledge of the effect that a loss or malfunction of the REACTOR WATER LEVEL CONTROL SYSTEM will have on following:	Rod worth minimizer: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
259002	K4.02	Knowledge of REACTOR WATER LEVEL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Bypassing of the RWM: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
259002	K4.07	Knowledge of REACTOR WATER LEVEL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	TDRFP 20% power interlock: TDRFP
		Clinton is a BWR 6 and does not utilize this system.	
259002	K4.15	Knowledge of REACTOR WATER LEVEL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Automatic initiation of the feedwater system upon receipt of an ECCS initiation signal: FWCI/HPCI
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
259002	K4.16	Knowledge of REACTOR WATER LEVEL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:	Dedication of feedwater string(s) to ECCS: FWCI/HPCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	K5.08	Knowledge of the operational implications of the following concepts as they apply to REACTOR WATER LEVEL CONTROL SYSTEM:	Heat removal mechanisms: FWCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	K5.09	Knowledge of the operational implications of the following concepts as they apply to REACTOR WATER LEVEL CONTROL SYSTEM:	Adequate core cooling: FWCI
		Clinton is a BWR 6 and does not utilize this system.	
259002	K6.07	Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR WATER LEVEL CONTROL SYSTEM:	Drywell pressure input: FWCI
		Clinton is a BWR 6 and does not utilize this system.	
Standby Gas Treatment System			
261000	A1.05	Ability to predict and/or monitor changes in parameters associated with operating the STANDBY GAS TREATMENT SYSTEM controls	Primary containment oxygen level:
		Clinton is a BWR 6 and does not utilize this configuration.	
261000	A1.06	Ability to predict and/or monitor changes in parameters associated with operating the STANDBY GAS TREATMENT SYSTEM controls	Drywell and suppression chamber differential pressure: Mark-I
		Clinton is a BWR 6 and does not utilize this configuration.	
261000	A4.05	Ability to manually operate and/or monitor in the control room:	Drywell to suppression chamber/torus differential pressure: Mark-I,II
		Clinton is a BWR 6 and does not utilize this configuration.	
261000	K1.06	Knowledge of the physical connections and/or cause- effect relationships between STANDBY GAS TREATMENT SYSTEM and the following:	High pressure coolant injection system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
261000	K3.03	Knowledge of the effect that a loss or malfunction of the STANDBY GAS TREATMENT SYSTEM will have on following:	Primary containment pressure: Mark-I&II
		Clinton is a BWR 6 and does not utilize this configuration.	
261000	K3.04	Knowledge of the effect that a loss or malfunction of the STANDBY GAS TREATMENT SYSTEM will have on following:	High pressure coolant injection system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
261000	K3.06	Knowledge of the effect that a loss or malfunction of the STANDBY GAS TREATMENT SYSTEM will have on following:	Primary containment oxygen content:
		Clinton is a BWR 6 and does not utilize this configuration.	
Uninterruptable Power Supply (A.C./D.C.)			
262002	A1.02	Ability to predict and/or monitor changes in parameters associated with operating the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) controls including:	Motor generator outputs
		Clinton is a BWR 6 and does not utilize this configuration.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
262002	A2.04	Ability to (a) predict the impacts of the following on the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Abnormal battery operation: BWR-1
		Clinton is a BWR-6 and does not utilize this feature.	
262002	K1.04	Knowledge of the physical connections and/or cause- effect relationships between UNINTERRUPTABLE POWER SUPPLY	Reactor manual control: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K1.07	Knowledge of the physical connections and/or cause- effect relationships between UNINTERRUPTABLE POWER SUPPLY	Rod worth minimizer: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K1.09	Knowledge of the physical connections and/or cause- effect relationships between UNINTERRUPTABLE POWER SUPPLY	Drywell ventilation control: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K1.10	Knowledge of the physical connections and/or cause- effect relationships between UNINTERRUPTABLE POWER SUPPLY	Fire protection system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K1.13	Knowledge of the physical connections and/or cause- effect relationships between UNINTERRUPTABLE POWER SUPPLY	Recirculation pump speed control:
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K1.15	Knowledge of the physical connections and/or cause- effect relationships between UNINTERRUPTABLE POWER SUPPLY	Stack gas monitors: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K1.20	Knowledge of the physical connections and/or cause- effect relationships between UNINTERRUPTABLE POWER SUPPLY	Plant communications equipment:
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K3.02	Knowledge of the effect that a loss or malfunction of the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) will have on	Recirculation pump speed: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K3.04	Knowledge of the effect that a loss or malfunction of the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) will have on	Fire protection system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K3.05	Knowledge of the effect that a loss or malfunction of the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) will have on	Rod worth minimizer: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K3.09	Knowledge of the effect that a loss or malfunction of the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) will have on	Drywell ventilation control: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
262002	K5.02	Knowledge of the operational implications of the following concepts as they apply to UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.):	General principles of motor generator operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
262002	K5.03	Knowledge of the operational implications of the following concepts as they apply to UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.):	General principles of inertia fly wheel operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
Emergency Generators (Diesel/Jet)			
264000	A1.07	Ability to predict and/or monitor changes in parameters associated with operating the EMERGENCY GENERATORS (DIESEL/JET) controls including:	Gas generator temperature: Plant-Specific
		Clinton does not use jet engines to power emergency generators.	
264000	A1.08	Ability to predict and/or monitor changes in parameters associated with operating the EMERGENCY GENERATORS (DIESEL/JET) controls including:	Gas generator speed: Plant-Specific
		Clinton does not use jet engines to power emergency generators.	
264000	K2.03	Knowledge of electrical power supplies to the following:	Turning gear (jet engine): Plant-Specific
		Clinton does not use jet engines to power emergency generators.	
264000	K2.04	Knowledge of electrical power supplies to the following:	Ignition system (jet engine): Plant-Specific
		Clinton does not use jet engines to power emergency generators.	
264000	K6.04	Knowledge of the effect that a loss or malfunction of the following will have on the EMERGENCY GENERATORS (DIESEL/JET):	Turning gear (jet engine): Plant-Specific
		Clinton does not use jet engines to power emergency generators.	
264000	K6.05	Knowledge of the effect that a loss or malfunction of the following will have on the EMERGENCY GENERATORS (DIESEL/JET):	Ignition system (jet engine): Plant-Specific
		Clinton does not use jet engines to power emergency generators.	
Offgas System			
271000	A2.07	Ability to (a) predict the impacts of the following on the OFFGAS SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Low oxygen injection flow: Plant-Specific
		Clinton does not utilize this function.	
271000	K1.08	Knowledge of the physical connections and/or cause- effect relationships between OFFGAS SYSTEM and the following:	Oxygen injection system: Plant-Specific
		Clinton does not utilize this function.	
271000	K5.05	Knowledge of the operational implications of the following concepts as they apply to OFFGAS SYSTEM:	Oxygen concentration measurement
		Clinton does not utilize this function.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
271000	K6.06	Knowledge of the effect that a loss or malfunction of the following will have on the	Oxygen injection system: Plant-Specific
		Clinton does not utilize this function.	
271000	K6.13	Knowledge of the effect that a loss or malfunction of the following will have on the	Plant exhaust: BWR-1
		Clinton is a BWR-6 and does not utilize this feature.	
Radiation Monitoring System			
272000	A2.07	Ability to (d) predict the impacts of the following on the RADIATION MONITORING SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Hydrogen injection operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
272000	A3.11	Ability to monitor automatic operations of the RADIATION MONITORING SYSTEM including:	Circulating water system blowdown isolations: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
272000	K1.06	Knowledge of the physical connections and/or cause- effect relationships between RADIATION MONITORING SYSTEM and the following:	Reactor building ventilation system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
272000	K1.07	Knowledge of the physical connections and/or cause- effect relationships between RADIATION MONITORING SYSTEM and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
272000	K1.12	Knowledge of the physical connections and/or cause- effect relationships between RADIATION MONITORING SYSTEM and the following:	Reactor building
		Clinton is a BWR 6 and does not utilize this system.	
272000	K1.15	Knowledge of the physical connections and/or cause- effect relationships between RADIATION MONITORING SYSTEM and the following:	Filter building: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Fire Protection System			
286000	A1.06	Ability to predict and/or monitor changes in parameters associated with operating the FIRE PROTECTION SYSTEM controls including:	Tank pressure: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
286000	A2.04	Ability to (a) predict the impacts of the following on the FIRE PROTECTION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:	Applicable component cooling water system failure: Plant-Specific
		The only component cooled is the Fire Pumps which are self cooled.	
286000	A4.02	Ability to manually operate and/or monitor in the control room:	Applicable component cooling water system: Plant-Specific
		The only component cooled is the Fire Pumps which are self cooled.	
286000	A4.03	Ability to manually operate and/or monitor in the control room:	Applicable component cooling water pressure
		The only component cooled is the Fire Pumps which are self cooled.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
286000	K1.01	Knowledge of the physical connections and/or cause- effect relationships between FIRE PROTECTION SYSTEM and the following:	Component cooling water systems
		The only component cooled is the Fire Pumps which are self cooled.	
286000	K1.02	Knowledge of the physical connections and/or cause- effect relationships between FIRE PROTECTION SYSTEM and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
286000	K1.06	Knowledge of the physical connections and/or cause- effect relationships between FIRE PROTECTION SYSTEM and the following:	Auxiliary (boiler) steam system:
		Clinton is a BWR 6 and does not utilize this system.	
286000	K1.08	Knowledge of the physical connections and/or cause- effect relationships between FIRE PROTECTION SYSTEM and the following:	Intake canals: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
286000	K5.08	Knowledge of the operational implications of the following concepts as they apply to FIRE PROTECTION SYSTEM:	Gas refrigeration: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
286000	K6.03	Knowledge of the effect that a loss or malfunction of the following will have on the FIRE PROTECTION SYSTEM	Applicable component cooling water system: Plant-Specific
		The only component cooled is the Fire Pumps which are self cooled.	
Plant Ventilation Systems			
288000	K3.02	Knowledge of the effect that a loss or malfunction of the PLANT VENTILATION SYSTEMS will have on following:	Reactor building temperature: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
288000	K3.05	Knowledge of the effect that a loss or malfunction of the PLANT VENTILATION SYSTEMS will have on following:	Reactor building pressure: Plant-Specific
		Clinton is a BWR 6 and does not utilize this configuration.	
Secondary Containment			
290001	K1.01	Knowledge of the physical connections and/or cause- effect relationships between SECONDARY CONTAINMENT and the following:	Reactor building ventilation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
290001	K1.08	Knowledge of the physical connections and/or cause- effect relationships between SECONDARY CONTAINMENT and the following:	Exhaust stack: BWR-2, 3, 4
		Clinton is a BWR 6 and does not utilize this system.	
290001	K1.10	Knowledge of the physical connections and/or cause- effect relationships between SECONDARY CONTAINMENT and the following:	Auxiliary boiler system: BWR-2, 3, 4
		Clinton is a BWR 6 and does not utilize this system.	
290001	K5.01	Knowledge of the operational implications of the following concepts as they apply to SECONDARY CONTAINMENT:	Vacuum breaker operation: BWR-4
		Clinton is a BWR 6 and does not utilize this configuration.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
290001	K5.02	Knowledge of the operational implications of the following concepts as they apply to SECONDARY CONTAINMENT:	Flow measurement: BWR-3
		Clinton is a BWR 6 and does not utilize this configuration.	
290001	K6.01	Knowledge of the effect that a loss or malfunction of the following will have on the SECONDARY CONTAINMENT:	Reactor building ventilation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
290001	K6.07	Knowledge of the effect that a loss or malfunction of the following will have on the SECONDARY CONTAINMENT:	Auxiliary boiler system: BWR-3, 4
		Clinton is a BWR 6 and does not have this configuration	
Reactor Vessel Internals			
290002	K1.04	Knowledge of the physical connections and/or cause- effect relationships between REACTOR VESSEL INTERNALS and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
290002	K1.07	Knowledge of the physical connections and/or cause- effect relationships between REACTOR VESSEL INTERNALS and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
290002	K6.10	Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR VESSEL INTERNALS:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
290002	K6.12	Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR VESSEL INTERNALS:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Control Room HVAC			
290003	K1.02	Knowledge of the physical connections and/or cause- effect relationships between CONTROL ROOM HVAC and the following:	Chlorine amonia detectors: Plant-Specific
		Clinton does not utilize this component.	
294001	2.2.3	Equipment Control	(multi-unit) Knowledge of the design, procedural, and operational differences between units.
		Clinton is a single-unit facility.	
294001	2.2.4	Equipment Control	(multi-unit) Ability to explain the variations in control board layouts, systems, instrumentation and procedural actions between units at a facility.
		Clinton is a single-unit facility.	
Partial or Complete Loss of Forced Core Flow			
295001	AA1.03	Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION:	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
295001	AA1.08	Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION:	Standby liquid control: BWR-1
		Clinton is a BWR-6 and does not utilize this feature.	
295001	AK2.05	Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION and the following:	LPCI loop select logic: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295001	AK2.08	Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION and the following:	Standby liquid control: BWR-1
		Clinton is a BWR-6 and does not utilize this feature.	
Loss of Main Condenser Vacuum			
295002	AK3.08	Knowledge of the reasons for the following responses as they apply to LOSS OF MAIN CONDENSER VACUUM:	Recirculation system run-backs:
		Clinton does not utilize this function.	
Partial or Complete Loss of A.C. Power			
295003	AK2.05	Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF A.C. POWER and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295003	AK3.07	Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER:	Initiation of isolation condenser:
		Clinton is a BWR 6 and does not utilize this system.	
Partial or Complete Loss of D.C. Power			
295004	AK1.01	Knowledge of the operational implications of the following concepts as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER:	Automatic load shedding: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Main Turbine Generator Trip			
295005	AK2.09	Knowledge of the interrelations between MAIN TURBINE GENERATOR TRIP and the following:	Feedwater-HPCI: BWR-2
		Clinton is a BWR 6 and does not utilize this system.	
295005	AK3.08	Knowledge of the reasons for the following responses as they apply to MAIN TURBINE GENERATOR TRIP:	Feedwater-HPCI actuation: BWR-2
		Clinton is a BWR 6 and does not utilize this system.	
High Reactor Pressure			
295007	AA1.01	Ability to operate and/or monitor the following as they apply to HIGH REACTOR PRESSURE:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295007	AA1.02	Ability to operate and/or monitor the following as they apply to HIGH REACTOR PRESSURE:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
295007	AK3.01	Knowledge of the reasons for the following responses as they apply to HIGH REACTOR	Isolation condenser operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295007	AK3.02	Knowledge of the reasons for the following responses as they apply to HIGH REACTOR	HPCI operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
High Reactor Water Level			
295008	AA1.04	Ability to operate and/or monitor the following as they apply to HIGH REACTOR WATER LEVEL:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295008	AK1.04	Knowledge of the operational implications of the following concepts as they apply to HIGH REACTOR WATER LEVEL:	Containment integrity: Alis-Chalmers
		Clinton is a BWR 6 and does not utilize this system.	
295008	AK2.05	Knowledge of the interrelations between HIGH REACTOR WATER LEVEL and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295008	AK3.05	Knowledge of the reasons for the following responses as they apply to HIGH REACTOR	HPCI turbine trip: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
High Drywell Pressure			
295010	AA1.03	Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE:	Nitrogen makeup: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295010	AA2.04	Ability to determine and/or interpret the following as they apply to HIGH DRYWELL PRESSURE:	Drywell humidity: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295010	AK1.01	Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL PRESSURE:	Downcomer submergence: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	
295010	AK2.02	Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following:	Drywell/suppression chamber differential pressure: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	
295010	AK2.04	Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following:	Nitrogen makeup system: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295010	AK3.06	Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL	Termination of drywell inerting:
		Clinton is a BWR 6 and does not utilize this system.	
High Containment Temperature (Mark III			
295011	AA2.03	Ability to determine and/or interpret the following as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):	Containment humidity: Mark-III
		Clinton does not utilize humidity monitoring instrumentation for containment.	
High Drywell Temperature			

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
295012	AA2.03	Ability to determine and/or interpret the following as they apply to HIGH DRYWELL	Drywell humidity: Plant-Specific
		Clinton does not utilize humidity monitoring instrumentation for containment.	
Inadvertent Reactivity Addition			
295014	AA1.03	Ability to operate and/or monitor the following as they apply to INADVERTENT REACTIVITY	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295014	AK1.01	Knowledge of the operational implications of the following concepts as they apply to INADVERTENT REACTIVITY ADDITION:	Prompt critical
		Prompt criticality is addressed during fundamental training only.	
295014	AK2.08	Knowledge of the interrelations between INADVERTENT REACTIVITY ADDITION and the following:	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Incomplete SCRAM			
295015	AA1.03	Ability to operate and/or monitor the following as they apply to INCOMPLETE SCRAM:	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295015	AA1.05	Ability to operate and/or monitor the following as they apply to INCOMPLETE SCRAM:	Rod worth minimizer: Plant-Specific
		Clinton is a BWR 6 and does not utilize this component.	
295015	AA1.06	Ability to operate and/or monitor the following as they apply to INCOMPLETE SCRAM:	RSCS: Plant-Specific
		Clinton is a BWR-6 and does not utilize this system.	
295015	AK2.02	Knowledge of the interrelations between INCOMPLETE SCRAM and the following:	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295015	AK2.05	Knowledge of the interrelations between INCOMPLETE SCRAM and the following:	Rod worth minimizer: Plant-Specific
		Clinton is a BWR 6 and does not utilize this component.	
295015	AK2.06	Knowledge of the interrelations between INCOMPLETE SCRAM and the following:	RSCS: Plant-Specific
		Clinton is a BWR-6 and does not utilize this system.	
295015	AK2.09	Knowledge of the interrelations between INCOMPLETE SCRAM and the following:	RPIS
		Clinton is a BWR 6 and does not utilize this system.	
Control Room Abandonment			
295016	AA1.02	Ability to operate and/or monitor the following as they apply to CONTROL ROOM	Reactor/turbine pressure regulating system
		At Clinton, no part of the reactor/turbine pressure regulating system is operated during Control Room Abandonment.	
295016	AA1.09	Ability to operate and/or monitor the following as they apply to CONTROL ROOM	Isolation/emergency condenser(s): Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295016	AA2.05	Ability to determine and/or interpret the following as they apply to CONTROL ROOM	Drywell pressure
		It has been determined that no psychometrically viable question (i.e., that satisfies all of the NUREG-1021, Rev.9 requirements and expectations) can be developed in accordance with	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
295016	AA2.07	Ability to determine and/or interpret the following as they apply to CONTROL ROOM	Suppression chamber pressure
		Clinton is a BWR 6 and does not utilize this system.	
Partial or Complete Loss of Instrument Air			
295019	AK2.13	Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295019	AK2.17	Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR and the following:	High pressure coolant injection:
		Clinton is a BWR 6 and does not utilize this system.	
Inadvertent Containment Isolation			
295020	AK2.05	Knowledge of the interrelations between INADVERTENT CONTAINMENT ISOLATION and the following:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295020	AK2.06	Knowledge of the interrelations between INADVERTENT CONTAINMENT ISOLATION and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295020	AK2.08	Knowledge of the interrelations between INADVERTENT CONTAINMENT ISOLATION and the following:	Traversing in-core probes: Plant-Specific
		Clinton is a BWR 6 and Traversing In-Core Probes are inside Primary Containment.	
295020	AK3.08	Knowledge of the reasons for the following responses as they apply to INADVERTENT CONTAINMENT ISOLATION:	Suppression chamber pressure response
		Clinton is a BWR 6 and does not utilize this system.	
Loss of CRD Pumps			
295022	AK2.06	Knowledge of the interrelations between LOSS OF CRD PUMPS and the following:	Shared components with other units: Plant-Specific
		Clinton is a single-unit facility.	
Refueling Accidents			
295023	AK3.05	Knowledge of the reasons for the following responses as they apply to REFUELING	Initiation of SLC/shut-down cooling: Plant-Specific(BWR-1)
		Clinton is a BWR-6 and does not utilize this feature.	
High Drywell Pressure			
295024	EA1.01	Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE:	HPCI (FWCI): Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295024	EA1.11	Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE:	Drywell spray: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	
295024	EA1.12	Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE:	Suppression pool spray: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
295024	EA1.21	Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE:	Recirculation system (LPCI loop select logic): Plant-Specific
295024	EA2.04	*Clinton is a BWR 6 and does not utilize this system.* Ability to determine and/or interpret the following as they apply to HIGH DRYWELL PRESSURE:	Suppression chamber pressure:
295024	EA2.05	*Clinton is a BWR 6 and does not utilize this system.* Ability to determine and/or interpret the following as they apply to HIGH DRYWELL PRESSURE:	Suppression chamber air-space temperature: Plant-Specific
295024	EK2.01	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following:	HPCI (FWCI): Plant-Specific
295024	EK2.11	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following:	Drywell spray (RHR) logic: Mark-I&II
295024	EK2.13	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following:	Suppression pool spray: Plant-Specific
295024	EK2.17	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following:	Auxiliary building isolation logic:
295024	EK3.01	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL	Drywell spray operation: Mark-I&II
295024	EK3.02	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL	Suppression pool spray operation:
295024	EK3.09	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL	Auxiliary building isolation: Plant-Specific.
		Clinton is a BWR 6 and does not utilize this system.	
High Reactor Pressure			
295025	EA1.04	Ability to operate and/or monitor the following as they apply to HIGH REACTOR PRESSURE:	HPCI: Plant-Specific
295025	EA1.06	*Clinton is a BWR 6 and does not utilize this system.* Ability to operate and/or monitor the following as they apply to HIGH REACTOR PRESSURE:	Isolation condenser: Plant-Specific
295025	EA1.08	*Clinton is a BWR 6 and does not utilize this system.* Ability to operate and/or monitor the following as they apply to HIGH REACTOR PRESSURE:	RRCS: Plant-Specific
295025	EK2.02	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the interrelations between HIGH REACTOR PRESSURE and the following:	Isolation condenser: Plant-Specific
295025	EK2.03	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the interrelations between HIGH REACTOR PRESSURE and the following:	RRCS: Plant-Specific
295025	EK2.06	*Clinton is a BWR 6 and does not utilize this system.* Knowledge of the interrelations between HIGH REACTOR PRESSURE and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
295025	EK3.03	Knowledge of the reasons for the following responses as they apply to HIGH REACTOR	HPCI operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295025	EK3.04	Knowledge of the reasons for the following responses as they apply to HIGH REACTOR	Isolation condenser initiation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295025	EK3.07	Knowledge of the reasons for the following responses as they apply to HIGH REACTOR	RRCs initiation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Suppression Pool High Water Temperature			
295026	EA1.02	Ability to operate and/or monitor the following as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE:	Suppression pool spray: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295026	EK2.02	Knowledge of the interrelations between SUPPRESSION POOL HIGH WATER TEMPERATURE and the following:	Suppression pool spray: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295026	EK2.03	Knowledge of the interrelations between SUPPRESSION POOL HIGH WATER TEMPERATURE and the following:	Suppression chamber pressure: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	
295026	EK3.03	Knowledge of the reasons for the following responses as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE:	Suppression pool spray: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
High Drywell Temperature			
295028	EA1.01	Ability to operate and/or monitor the following as they apply to HIGH DRYWELL TEMPERATURE:	Drywell spray: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	
295028	EA2.05	Ability to determine and/or interpret the following as they apply to HIGH DRYWELL	Torus/suppression chamber pressure: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295028	EA2.06	Ability to determine and/or interpret the following as they apply to HIGH DRYWELL	Torus/suppression chamber air space temperature: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295028	EK2.01	Knowledge of the interrelations between HIGH DRYWELL TEMPERATURE and the following:	Drywell spray: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	
295028	EK3.03	Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL	Drywell spray operation: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	
High Suppression Pool Water Level			
295029	EA1.01	Ability to operate and/or monitor the following as they apply to HIGH SUPPRESSION POOL WATER LEVEL:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
295029	EK2.02	Knowledge of the interrelations between HIGH SUPPRESSION POOL WATER LEVEL and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Low Suppression Pool Water Level			
295030	EA1.05	Ability to operate and/or monitor the following as they apply to LOW SUPPRESSION POOL WATER LEVEL:	HPCI
		Clinton is a BWR 6 and does not utilize this system.	
295030	EA2.04	Ability to determine and/or interpret the following as they apply to LOW SUPPRESSION POOL WATER LEVEL:	Drywell/ suppression chamber differential pressure: Mark-I&II
		Clinton is a BWR 6 and does not utilize this system.	
295030	EK2.01	Knowledge of the interrelations between LOW SUPPRESSION POOL WATER LEVEL and the following:	HPCI: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295030	EK3.02	Knowledge of the reasons for the following responses as they apply to LOW SUPPRESSION POOL WATER LEVEL:	HPCI operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Reactor Low Water Level			
295031	EA1.02	Ability to operate and/or monitor the following as they apply to REACTOR LOW WATER LEVEL:	High pressure (feedwater) coolant injection: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295031	EA1.09	Ability to operate and/or monitor the following as they apply to REACTOR LOW WATER LEVEL:	Isolation condenser: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295031	EK2.06	Knowledge of the interrelations between REACTOR LOW WATER LEVEL and the	High pressure (feedwater) coolant injection (FWCI/HPCI): Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Secondary Containment High Differential			
295035	EK2.04	Knowledge of the interrelations between SECONDARY CONTAINMENT HIGH DIFFERENTIAL PRESSURE and the following:	Blow-out panels: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295035	EK3.01	Knowledge of the reasons for the following responses as they apply to SECONDARY CONTAINMENT HIGH DIFFERENTIAL	Blow-out panel operation: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
Secondary Containment High Sump/Area Water			
295036	EK2.02	Knowledge of the interrelations between SECONDARY CONTAINMENT HIGH SUMP/AREA WATER LEVEL and the following:	Post-accident sampling system:
		The PASS system is not operated, nor is it monitored by licensed operators at Clinton.	
SCRAM Condition Present and Reactor Power			

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
295037	EA1.02	Ability to operate and/or monitor the following as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN:	RRCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295037	EA1.07	Ability to operate and/or monitor the following as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN:	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295037	EK2.02	Knowledge of the interrelations between SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN and the following:	RRCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295037	EK2.11	Knowledge of the interrelations between SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN and the following:	RMCS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
295037	EK2.14	Knowledge of the interrelations between SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN and the following:	RPIS: Plant-Specific
		Clinton is a BWR 6 and does not utilize this system.	
High Off-Site Release Rate			
295038	EA1.05	Ability to operate and/or monitor the following as they apply to HIGH OFF-SITE RELEASE RATE:	Post accident sample system (PASS): Plant-Specific
		The PASS system is not operated, nor is it monitored by licensed operators at Clinton.	
Instrument Air System (IAS)			
300000	K1.01	Knowledge of the connections and/or cause effect relationships between INSTRUMENT AIR SYSTEM and the following:	Sensor air
		Clinton is a single-unit facility which does use this configuration.	
300000	K2.02	Knowledge of electrical power supplies to the following:	Emergency air compressor
		Clinton does not utilize this component.	
300000	K3.03	Knowledge of the effect that a loss or malfunction of the (INSTRUMENT AIR SYSTEM) will have on the following:	Cross-tied units
		Clinton is a single-unit facility which does use this configuration.	
300000	K4.01	Knowledge of (INSTRUMENT AIR SYSTEM) design feature(s) and or interlocks which provide for the following:	Manual/automatic transfers of control
		It has been determined that no psychometrically viable question (i.e., one that meets all of the NUREG-1021, Rev. 9 requirements and expectations) can be developed in accordance with	
300000	K5.04	Knowledge of the operational implications of the following concepts as they apply to the INSTRUMENT AIR SYSTEM:	Service air refusal valve
		Clinton does not utilize this component.	

System No.	K/A No.	Category	K/A Statement
		Suppression Basis	
300000	K6.04	Knowledge of the effect that a loss or malfunction of the following will have on the INSTRUMENT AIR SYSTEM:	Service air refusal valve
		Clinton is a single-unit facility which does use this configuration.	
High Containment Hydrogen Concentration			
500000	EA1.05	Ability to operate and monitor the following as they apply to HIGH CONTAINMENT HYDROGEN CONTROL:	Wetwell sprays
		Clinton is a BWR 6 and does not utilize this system.	
500000	EA1.06	Ability to operate and monitor the following as they apply to HIGH CONTAINMENT HYDROGEN CONTROL:	Drywell sprays
		Clinton is a BWR 6 and does not utilize this system.	
500000	EA1.07	Ability to operate and monitor the following as they apply to HIGH CONTAINMENT HYDROGEN CONTROL:	Nitrogen purge system
		Clinton is a BWR 6 and does not utilize this system.	
500000	EA2.04	Ability to determine and/or interpret the following as they apply to HIGH PRIMARY CONTAINMENT HYDROGEN	Combustible limits for wetwell
		Clinton is a BWR 6 and does not utilize this system.	
500000	EK2.06	Knowledge of the interrelations between HIGH CONTAINMENT HYDROGEN CONCENTRATIONS the following:	Wetwell Spray system
		Clinton is a BWR 6 and does not utilize this system.	
500000	EK2.08	Knowledge of the interrelations between HIGH CONTAINMENT HYDROGEN CONCENTRATIONS the following:	Wet Well vent system
		Clinton is a BWR 6 and does not utilize this system.	
500000	EK2.09	Knowledge of the interrelations between HIGH CONTAINMENT HYDROGEN CONCENTRATIONS the following:	Drywell nitrogen purge system
		Clinton is a BWR 6 and does not utilize this system.	
500000	EK3.04	Knowledge of the reasons for the following responses as they apply to HIGH PRIMARY CONTAINMENT HYDROGEN	Emergency depressurization
		Clinton does not perform an Emergency Depressurization due to a high containment hydrogen condition.	
500000	EK3.05	Knowledge of the reasons for the following responses as they apply to HIGH PRIMARY CONTAINMENT HYDROGEN	Operation of wet well (suppression pool)
		Clinton is a BWR 6 and does not utilize this system.	
500000	EK3.06	Knowledge of the reasons for the following responses as they apply to HIGH PRIMARY CONTAINMENT HYDROGEN	Operation of wet well vent
		Clinton is a BWR 6 and does not utilize this system.	
500000	EK3.08	Knowledge of the reasons for the following responses as they apply to HIGH PRIMARY CONTAINMENT HYDROGEN	Operation of drywell nitrogen purge system
		Clinton is a BWR 6 and does not utilize this system.	

11/15/06 — Training Department — Date

11/15/06 — Operating Department — Date

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: **201001**

System Name: **Control Rod Drive Hydraulic System**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between CONTROL ROD DRIVE HYDRAULIC SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Condensate system		3.1	3.1	3.1	3.1
K1.02	Condensate storage tanks		3.0	3.0	3.0	3.0
K1.03	Recirculation pumps (seal purge): Plant-Specific		3.1	3.1	3.1	3.1
K1.04	~~Head spray: BWR-3~~		2.7	2.7	2.7	2.7
K1.05	~~Feedwater (or reactor water cleanup) CRD return to vessel: Plant-Specific~~		2.7	2.7	2.7	2.7
K1.06	Component cooling water systems: Plant-Specific		2.8	2.8	2.8	2.8
K1.07	Reactor protection system		3.4	3.4	3.4	3.4
K1.08	Reactor manual control system		3.4	3.4	3.4	3.4
K1.09	Plant air systems		3.1	3.2	3.1	3.2
K1.10	Control rod drive mechanisms		2.8	2.8	2.8	2.8
K1.11	Reactor water cleanup pumps: Plant-Specific		2.8	2.8	2.8	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Pumps		2.9	3.1	2.9	3.1
K2.02	Scram valve solenoids		3.6*	3.7	3.6	3.7
K2.03	Backup SCRAM valve solenoids		3.5*	3.6*	3.5	3.6
K2.04	Scram discharge volume vent and drain valve solenoids		3.2	3.3	3.2	3.3
K2.05	*Alternate rod insertion valve solenoids: Plant-Specific*		4.5*	4.5*	4.5	4.5
K2.06	Motor operated valves		2.1*	2.3*	2.1	2.3
K2.07	Breaker control		2.0*	2.1*	2.0	2.1
K3	**Knowledge of the effect that a loss or malfunction of the CONTROL ROD DRIVE HYDRAULIC SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Recirculation pumps: Plant-Specific		3.0	3.1	3.0	3.1
K3.02	Reactor water level		2.6	2.6	2.6	2.6
K3.03	Control rod drive mechanisms		3.1	3.2	3.1	3.2
K3.04	~~Head spray: BWR-3~~		2.7	2.7	2.7	2.7
K3.05	Reactor water cleanup pumps: Plant-Specific		2.3	2.3	2.3	2.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201001

System Name: Control Rod Drive Hydraulic System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
K4	**Knowledge of CONTROL ROD DRIVE HYDRAULIC SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Protection against pump runout during SCRAM conditions (location of the CRD system flow element and a restricting orifice in the accumulator charging water line)		2.5	2.6	2.5	2.6
K4.02	Stable system flow when moving control rods (stabilizing valves)		2.6	2.6	2.6	2.6
K4.03	Control rod drive mechanism cooling water flow		2.7	2.7	2.7	2.7
K4.04	Scramming control rods with inoperative SCRAM solenoid valves (back-up SCRAM valves)		3.6	3.6	3.6	3.6
K4.05	Control rod SCRAM		3.8	3.8	3.8	3.8
K4.06	Isolation of the SCRAM discharge volumes during SCRAM conditions		3.8	3.9	3.8	3.9
K4.07	Testing SCRAM discharge volume isolation valves		2.8	2.8	2.8	2.8
K4.08	Controlling control rod drive header pressure		3.1	3.0	3.1	3.0
K4.09	Controlling control rod drive cooling header pressure		2.9	2.8	2.9	2.8
K4.10	Control of rod movement (HCU directional control valves)		3.1	3.0	3.1	3.0
K4.11	Protection against filling the SDV during non-SCRAM conditions		3.6	3.6	3.6	3.6
K4.12	Controlling CRD system flow		2.9	2.9	2.9	2.9
K4.13	Motor cooling		2.3	2.3	2.3	2.3
K5	**Knowledge of the operational implications of the following concepts as they apply to CONTROL ROD DRIVE HYDRAULIC SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Pump operation		2.4	2.4	2.4	2.4
K5.02	Flow indication		2.6	2.6	2.6	2.6
K5.03	Pressure indication		2.7	2.7	2.7	2.7
K5.04	Indications of pump cavitation		2.4	2.4	2.4	2.4
K5.05	Indications of pump runout: Plant-Specific		2.7	2.7	2.7	2.7
K5.06	Differential pressure indication		2.5	2.6	2.5	2.6
K5.07	Air operated control valves		2.3	2.4	2.3	2.4
K5.08	Solenoid operated valves		2.5	2.6	2.5	2.6
K5.09	System venting		2.2*	2.2*	2.2	2.2
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the CONTROL ROD DRIVE HYDRAULIC System:**	**(41.7 / 45.7)**				
K6.01	Condensate system		2.8	2.8	2.8	2.8
K6.02	Condensate storage tanks		3.0	3.1	3.0	3.1
K6.03	Plant air systems		3.0	2.9	3.0	2.9
K6.04	RPS		3.6	3.7	3.6	3.7
K6.05	A.C. power		3.3	3.3	3.3	3.3
K6.06	Component cooling water systems: Plant-Specific		2.8	2.8	2.8	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201001

System Name: Control Rod Drive Hydraulic System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the CONTROL ROD DRIVE HYDRAULIC SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	CRD drive water header pressure		3.1	2.9	3.1	2.9
A1.02	CRD cooling water header pressure		2.9	2.9	2.9	2.9
A1.03	CRD system flow		2.9	2.8	2.9	2.8
A1.04	~~Head spray flow: BWR-3~~		2.7	2.7	2.7	2.7
A1.05	SDV isolation valve position		3.5	3.4	3.5	3.4
A1.06	HCU pressure/level		3.4	3.4	3.4	3.4
A1.07	Reactor water level		3.3	3.2	3.3	3.2
A1.08	Pump amps		2.3	2.2	2.3	2.2
A1.09	CRD drive water flow		2.9	2.8	2.9	2.8
A1.10	CRD cooling water flow		2.8	2.6	2.8	2.6
A2	**Ability to (a) predict the impacts of the following on the CONTROL ROD DRIVE HYDRAULIC SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Pumps trips		3.2	3.3	3.2	3.3
A2.02	Valve closures		3.2	3.3	3.2	3.3
A2.03	Power supply failures		3.0	3.1	3.0	3.1
A2.04	†Scram conditions		3.8	3.9*	3.8	3.9
A2.05	Discharge strainer(s) becoming plugged		2.9	2.9	2.9	2.9
A2.06	Suction strainer(s) becoming plugged		2.9	2.9	2.9	2.9
A2.07	Flow control valve failure		3.2	3.1	3.2	3.1
A2.08	Inadequate system flow		2.8	2.8	2.8	2.8
A2.09	Loss of applicable plant air systems		3.2	3.1	3.2	3.1
A2.10	†Low HCU accumulator pressure/high level		3.5	3.6	3.5	3.6
A2.11	Valve openings		2.6	2.7	2.6	2.7
A2.12	High cooling water flow		2.8	2.9	2.8	2.9
A2.13	Low cooling water flow		2.7	2.8	2.7	2.8
A2.14	Low drive header pressure		2.8	2.8	2.8	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201001

System Name: Control Rod Drive Hydraulic System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I SRO
A3	**Ability to monitor automatic operations of the CONTROL ROD DRIVE HYDRAULIC SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		3.0	3.0	3.0	3.0
A3.02	Pump start: Plant-Specific		2.8	2.8	2.8	2.8
A3.03	System pressure		2.7	2.7	2.7	2.7
A3.04	System flow		2.8	2.7	2.8	2.7
A3.05	Reactor water level		2.8	2.8	2.8	2.8
A3.06	Reactor power		2.8	2.8	2.8	2.8
A3.07	HCU accumulator pressure/level		3.3	3.3	3.3	3.3
A3.08	Drive water flow		3.0	2.9	3.0	2.9
A3.09	Cooling water flow		2.8	2.8	2.8	2.8
A3.10	Lights and alarms		3.0	2.9	3.0	2.9
A3.11	SDV level		3.5	3.5	3.5	3.5
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	CRD pumps		3.1	3.1	3.1	3.1
A4.02	CRD pump discharge valve		2.6	2.6	2.6	2.6
A4.03	CRD system flow control valve		2.9	2.8	2.9	2.8
A4.04	Drive water header pressure control valve		3.1	3.0	3.1	3.0
A4.05	Cooling water header pressure control valve		2.7	2.8	2.7	
A4.06	SDV isolation valve test switch		2.8	2.7	2.8	2.7
A4.07	Stabilizing valve selector switch		2.2*	2.1*	2.2	2.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 201002

System Name: Reactor Manual Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR MANUAL CONTROL SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Control rod drive hydraulic system~~		3.2	3.2	3.2	3.2
K1.02	~~Control rod and drive mechanism~~		3.0	2.9	3.0	2.9
K1.03	~~Control rod block interlocks/power operation refueling~~		3.4	3.6	3.4	3.6
K1.04	~~Rod block monitor: Plant-Specific~~		3.5	3.6	3.5	3.6
K1.05	~~Rod worth minimizer: Plant-Specific~~		3.4	3.5	3.4	3.5
K1.06	~~Rod sequence control system: Plant-Specific~~		3.2	3.3	3.2	3.3
K1.07	~~Process computer: Plant-Specific~~		2.8	2.9	2.8	2.9
K1.08	~~†Refueling interlocks: Plant-Specific~~		3.2	3.6	3.2	3.6
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~Select matrix~~		2.1*	2.3*	2.1	2.3
K2.02	~~CRD HCU directional control valves~~		2.1*	2.3*	2.1	2.3
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR MANUAL CONTROL SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	~~Ability to move control rods~~		3.4	3.4	3.4	3.4
K3.02	~~†Rod block monitor: Plant-Specific~~		2.9	3.2	2.9	3.2
K3.03	~~Ability to process rod block signals~~		2.9	3.0	2.9	3.0
K4	**Knowledge of REACTOR MANUAL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Detection of sequence timer malfunction~~		2.7	2.7	2.7	2.7
K4.02	~~Control rod blocks~~		3.5	3.5	3.5	3.5
K4.03	~~Detection of drifting control rods~~		3.6	3.6	3.6	3.6
K4.04	~~Single notch rod withdrawal and insertion~~		3.3	3.3	3.3	3.3
K4.05	~~Notch override rod withdrawal~~		3.3	3.3	3.3	3.3
K4.06	~~Emergency In rod insertion~~		3.5	3.5	3.5	3.5
K4.07	~~Timing of rod insert and withdrawal cycles (rod movement sequence timer)~~		2.5*	2.5*	2.5	2.5
K4.08	~~Continuous In rod insertion~~		3.2	3.2	3.2	3.2
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR MANUAL CONTROL SYSTEM:**	**(41.7 / 45.7)**				
K6.01	~~Select matrix power~~		2.5	2.6	2.5	2.6

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 201002

System Name: Reactor Manual Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the REACTOR MANUAL CONTROL SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	~~CRD drive water flow~~		2.8	2.8	2.8	2.8
A1.02	~~Control rod position~~		3.4	3.3	3.4	3.3
A1.03	~~Rod movement sequence lights~~		3.0	2.9	3.0	2.9
A1.04	~~Overall reactor power~~		3.6	3.5	3.6	3.5
A1.05	~~Local reactor power~~		3.4	3.6	3.4	3.6
A2	**Ability to (a) predict the impacts of the following on the REACTOR MANUAL CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Rod movement sequence timer malfunctions~~		2.7	2.8	2.7	2.8
A2.02	~~Rod drift alarm~~		3.2	3.3	3.2	3.3
A2.03	~~Select block~~		2.9	2.8	2.9	2.8
A2.04	~~Control rod block~~		3.2	3.1	3.2	3.1
A3	**Ability to monitor automatic operations of the REACTOR MANUAL CONTROL SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	~~Control rod block actuation~~		3.2	3.1	3.2	
A3.02	~~Rod movement sequence lights~~		2.8	2.7	2.8	2.7
A3.03	~~Rod drift alarm~~		3.2	3.2	3.2	3.2
A3.04	~~Rod movement sequence timer malfunction alarm: Plant-Specific~~		2.8	2.8	2.8	2.8
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	~~Rod movement control switch~~		3.5	3.4	3.5	3.4
A4.02	~~Emergency in/notch override switch~~		3.5	3.5	3.5	3.5
A4.03	~~Rod drift test switch~~		2.8	2.8	2.8	2.8
A4.04	~~Timer malfunction test switch: Plant-Specific~~		2.8	2.8	2.8	2.8
A4.05	~~Rod select matrix~~		3.1	3.0	3.1	3.0
A4.06	~~Rod select matrix power switch~~		2.8	2.8	2.8	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201003

System Name: Control Rod and Drive Mechanism

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between CONTROL ROD AND DRIVE MECHANISM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Control rod drive hydraulic system		3.2	3.3	3.2	3.3
K1.02	Reactor water		2.9	3.0	2.9	3.0
K1.03	RPIS		3.1	3.1	3.1	3.1
K1.04	Reactor vessel		2.9	2.9	2.9	2.9
K1.05	CRD mechanism temperature monitor		2.6	2.6	2.6	2.6
K3	**Knowledge of the effect that a loss or malfunction of the CONTROL ROD AND DRIVE MECHANISM will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor power		3.2	3.4	3.2	3.4
K3.02	†Flux shaping		2.8	3.1	2.8	3.1
K3.03	†Shutdown margin		3.2	3.8	3.2	3.8
K4	**Knowledge of CONTROL ROD AND DRIVE MECHANISM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Limiting control rod speed in the event of a rod drop		2.9*	3.0*	2.9	3.0
K4.02	Detection of an uncoupled rod		3.8	3.9	3.8	3.9
K4.03	Slowing the drive mechanism near the end of its travel following a SCRAM: Plant-Specific		2.1*	2.1*	2.1	2.1
K4.04	The use of either accumulator or reactor water to SCRAM the control rod		3.6	3.7	3.6	3.7
K4.05	Rod position indication		3.2	3.3	3.2	3.3
K4.06	Uncoupling the control rod from the drive mechanism		2.4	2.6	2.4	2.6
K4.07	Maintaining the control rod at a given location		3.2	3.2	3.2	3.2
K4.08	Monitoring CRD mechanism temperature		2.6	2.7	2.6	2.7
K5	**Knowledge of the operational implications of the following concepts as they apply to CONTROL ROD AND DRIVE MECHANISM:**	**(41.5 / 45.3)**				
K5.01	Hydraulics		2.6	2.7	2.6	2.7
K5.02	†Flux shaping		2.8	3.3	2.8	3.3
K5.03	Reactor power control		3.3	3.4	3.3	3.4
K5.04	†Rod sequence patterns		3.1	3.4	3.1	3.4
K5.05	Reverse power effect		3.0	3.1	3.0	3.1
K5.06	How control rod worth varies with moderator temperature and voids		2.7	2.9	2.7	2.9
K5.07	How control rod movements affect core reactivity		3.3	3.6	3.3	3.6
K5.08	†How control rods affect shutdown margin		3.1	3.5	3.1	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201003

System Name: Control Rod and Drive Mechanism

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I SRO
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the CONTROL ROD AND DRIVE MECHANISM:**	**(41.7 / 45.7)**				
K6.01	Control rod drive hydraulic system		3.3	3.3	3.3	3.3
K6.02	Reactor pressure		3.0	3.0	3.0	3.0
A1	**Ability to predict and/or monitor changes in parameters associated with operating the CONTROL ROD AND DRIVE MECHANISM controls including:**	**(41.5 / 45.5)**				
A1.01	Reactor power		3.7	3.8	3.7	3.8
A1.02	CRD drive pressure		2.8	2.8	2.8	2.8
A1.03	CRD drive water flow		2.9	2.9	2.9	2.9
A2	**Ability to (a) predict the impacts of the following on the CONTROL ROD AND DRIVE MECHANISM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	†Stuck rod		3.4	3.6	3.4	3.6
A2.02	Uncoupled rod		3.7	3.8	3.7	3.8
A2.03	†Drifting rod		3.4	3.7	3.4	^
A2.04	Single control rod SCRAM		3.5	3.6	3.5	- -
A2.05	Reactor scram		4.1*	4.1	4.1	4.1
A2.06	Loss of CRD cooling water flow		3.0	3.1	3.0	3.1
A2.07	Loss of CRD drive water flow		3.1	3.2	3.1	3.2
A2.08	Low HCU accumulator pressure/high level		3.8	3.7	3.8	3.7
A2.09	Low reactor pressure		3.2	3.4	3.2	3.4
A2.10	†Excessive SCRAM time for a given drive mechanism		3.0	3.4	3.0	3.4
A3	**Ability to monitor automatic operations of the CONTROL ROD AND DRIVE MECHANISM including:**	**(41.7 / 45.7)**				
A3.01	Control rod position		3.7	3.6	3.7	3.6
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	CRD mechanism temperature		2.6*	2.6*	2.6	2.6
A4.02	CRD mechanism position: Plant-Specific		3.5	3.5	3.5	3.5

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 201004

System Name: Rod Sequence Control System (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between ROD SEQUENCE CONTROL SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Reactor manual control system: BWR-4, 5~~		3.2	3.3	3.2	3.3
K1.02	~~Turbine generator system: BWR-4, 5~~		3.1	3.1	3.1	3.1
K1.03	~~Rod position information system: BWR-4, 5~~		3.1	3.1	3.1	3.1
K1.04	~~Rod worth minimizer: BWR-4, 5~~		2.8	3.0	2.8	3.0
K3	**Knowledge of the effect that a loss or malfunction of the ROD SEQUENCE CONTROL SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	~~Reactor manual control: BWR-4, 5~~		3.3	3.4	3.3	3.4
K4	**Knowledge of ROD SEQUENCE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Select blocks: BWR-4, 5~~		3.0	3.1	3.0	3.1
K4.02	~~Insert rod blocks: BWR-4, 5~~		3.1	3.2	3.1	3.2
K4.03	~~Withdraw rod blocks: BWR-4, 5~~		3.3	3.4	3.3	3.4
K4.04	~~RSCS bypass as reactor power increases: BWR-4, 5~~		3.3	3.3	3.3	3.3
K4.05	~~Rod movement, direction, and selection information: BWR-4, 5~~		3.2	3.2	3.2	3.2
K4.06	~~Group notch control: BWR-4, 5~~		3.3	3.4	3.3	3.4
K4.07	~~Minimizing rod worth: BWR-4, 5~~		3.6	3.7	3.6	3.7
K5	**Knowledge of the operational implications of the following concepts as they apply to ROD SEQUENCE CONTROL SYSTEM:**	**(41.5 / 45.3)**				
K5.01	~~Prevention of clad damage if a control rod drop accident (CRDA) occurs: BWR-4, 5~~		3.6	4.0	3.6	4.0
K5.02	~~Sequences and groups: BWR-4, 5~~		3.1	3.3	3.1	3.3
K5.03	~~Group notch control limits and rod density: BWR-4, 5~~		3.3	3.5	3.3	3.5
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the ROD SEQUENCE CONTROL SYSTEM:**	**(41.7 / 45.7)**				
K6.01	~~Rod position information: BWR-4, 5~~		3.3	3.3	3.3	3.3
K6.02	~~Rod direction information: BWR-4, 5~~		3.1	3.2	3.1	3.2
K6.03	~~Rod movement information: BWR-4, 5~~		3.2	3.2	3.2	3.2
K6.04	~~Turbine generator (1st stage shell pressure): BWR-4, 5~~		3.3	3.4	3.3	3.4
A1	**Ability to predict and/or monitor changes in parameters, associated with operating the ROD SEQUENCE CONTROL SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	~~Reactor manual control system: BWR-4, 5~~		3.3	3.3	3.3	3.3

Facility: CPS Printed: 08/11/2006

System Number: 201004

System Name: Rod Sequence Control System (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility 1 RO	Facility 1
A2	**Ability to (a) predict the impacts of the following on the ROD SEQUENCE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Loss of rod position information: BWR-4, 5~~		3.3	3.6	3.3	3.6
A2.02	~~Attempting to move a stuck control rod: BWR-4, 5~~		3.4	3.6	3.4	3.6
A2.03	~~Turbine trip: BWR-4, 5~~		3.2	3.2	3.2	3.2
A3	**Ability to monitor automatic operations of the ROD SEQUENCE CONTROL SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	~~Rod select switch light: BWR-4, 5~~		3.2	3.1	3.2	3.1
A3.02	~~Rod select bottom lamp dimmer logic: BWR-4, 5~~		3.1*	3.1*	3.1	3.1
A3.03	~~Back panel indicators: BWR-4, 5~~		2.8	3.0	2.8	3.0
A3.04	~~Annunciator and alarm signals: BWR-4, 5~~		3.2	3.2	3.2	3.2
A3.05	~~†Verification of proper function/ operability: BWR-4, 5~~		3.5	3.7	3.5	3.7
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	~~†System bypass switches: BWR-4, 5~~		3.4	35	3.4	35
A4.02	~~RSCS console switches and indicators: BWR-4, 5~~		3.5	3.2	3.5	3
A4.03	~~†RSCS back panel switches and indicators: BWR-4, 5~~		3.0	3.1	3.0	3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201005

System Name: Rod Control and Information System (RCIS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between ROD CONTROL AND INFORMATION SYSTEM (RCIS) and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Neutron monitoring system: BWR-6		3.3	3.3	3.3	3.3
K1.02	Reactor/turbine pressure control system: BWR-6		3.3	3.5	3.3	3.5
K1.03	Control rod drive system: BWR-6		3.7	3.7	3.7	3.7
K1.04	Rod position information system: BWR-6		3.7	3.7	3.7	3.7
K1.05	Rod action control system: BWR-6		3.5	3.5	3.5	3.5
K1.06	Rod gang drive system: BWR-6		3.3	3.3	3.3	3.3
K1.07	Rod interface system: BWR-6		3.3	3.3	3.3	3.3
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	A.C. electrical power: BWR-6		2.4	2.6	2.4	2.6
K3	**Knowledge of the effect that a loss or malfunction of the ROD CONTROL AND INFORMATION SYSTEM (RCIS) will have on following:**	**(41.7 / 45.4)**				
K3.01	Control rod drive system: BWR-6		3.3	3.7	3.3	3.7
K3.02	Reactor startup: BWR-6		3.5	3.5	3.5	3.5
K3.03	Reactor shutdown: BWR-6		3.0	3.2	3.0	3.2
K3.04	†Flux shaping: BWR-6		3.0	3.3	3.0	3.3
K4	**Knowledge of ROD CONTROL AND INFORMATION SYSTEM (RCIS) design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Limiting the effects of a control Rod accident: BWR-6		3.2	3.2	3.2	3.2
K4.02	Bank position withdrawal sequence (BPWS): BWR-6		3.3	3.3	3.3	3.3
K4.03	Rod withdrawal block signals: BWR-6		3.5	3.5	3.5	3.5
K4.04	Rod insertion block signals: BWR-6		3.5	3.5	3.5	3.5
K4.05	Rod withdrawal limiter: BWR-6		3.5	3.5	3.5	3.5
K4.06	Rod pattern controller rod blocks: BWR-6		3.5	3.5	3.5	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201005

System Name: Rod Control and Information System (RCIS)

			CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S
K5	**Knowledge of the operational implications of the following concepts as they apply to ROD CONTROL AND INFORMATION SYSTEM (RCIS):**	**(41.5 / 45.3)**					
K5.01	†Rod pattern and program development: BWR-6			2.3	3.2	2.3	3.2
K5.02	Rod pattern controller (RPC): BWR-6			2.8	3.3	2.8	3.3
K5.03	Rod groups: BWR-6			2.3	2.7	2.3	2.7
K5.04	†Rod sequences: BWR-6			2.7	3.0	2.7	3.0
K5.05	Rod density: BWR-6			2.7	3.0	2.7	3.0
K5.06	Target rod pattern: BWR-6			2.8	2.8	2.8	2.8
K5.07	Low power alarm point: BWR-6			3.5	3.5	3.5	3.5
K5.08	Transition zone: BWR-6			3.2	3.5	3.2	3.5
K5.09	High power setpoints BWR-6			3.5	3.0	3.5	3.0
K5.10	Rod withdrawal limiter: BWR-6			3.2	3.3	3.2	3.3
K5.11	Control rod motion: BWR-6			3.3	3.3	3.3	3.3
K5.12	Command word generation and sequencing (operator follow, scan and test): BWR-6			1.8*	2.2*	1.8	2.2
K5.13	Position indication probes: BWR-6			2.5	2.7	2.5	2.7
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the ROD CONTROL AND INFORMATION SYSTEM (RCIS):**	**(41.7 / 45.7)**					
K6.01	First stage shell pressure or opening of a bypass valve(s): BWR-6			3.2	3.2	3.2	?
K6.02	Rod position signal: BWR-6			3.2	3.3	3.2	3.3
K6.03	A.C. electrical power: BWR-6			2.5	2.8	2.5	2.8
K6.04	IRM channel: BWR-6			3.0	3.2	3.0	3.2
K6.05	SRM channel: BWR-6			3.0	3.2	3.0	3.2
K6.06	APRM channel: BWR-6			3.0	3.0	3.0	3.0
A1	**Ability to predict and/or monitor changes in parameters associated with operating the ROD CONTROL AND INFORMATION SYSTEM (RCIS) controls including:**	**(41.5 / 45.5)**					
A1.01	First stage shell pressure/turbine load: BWR-6			3.2	3.3	3.2	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201005

System Name: Rod Control and Information System (RCIS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the ROD CONTROL AND INFORMATION SYSTEM (RCIS); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	High flux (SRM, IRM, APRM): BWR-6		3.5	3.5	3.5	3.5
A2.02	Position indication probe failure: BWR-6		2.8	3.2	2.8	3.2
A2.03	Insert block: BWR-6		3.2	3.2	3.2	3.2
A2.04	Withdraw block: BWR-6		3.2	3.2	3.2	3.2
A2.05	Insert required: BWR-6		3.2	3.2	3.2	3.2
A2.06	Insert inhibit: BWR-6		3.2	3.2	3.2	3.2
A2.07	Withdraw inhibit: BWR-6		3.2	3.2	3.2	3.2
A2.08	LPRM upscale/down scale: BWR-6		3.2	3.2	3.2	3.2
A2.09	Test display blinking: BWR-6		2.8	3.0	2.8	3.0
A2.10	Data fault: BWR-6		2.8	3.0	2.8	3.0
A2.11	Accumulator fault: BWR-6		3.3	3.5	3.3	3.5
A2.12	Rod uncoupled: BWR-6		3.7	3.8	3.7	3.8
A2.13	Rod drift: BWR-6		3.8	3.8	3.8	3.8
A3	**Ability to monitor automatic operations of the ROD CONTROL AND INFORMATION System (RCIS) including:**	**(41.7 / 45.7)**				
A3.01	Operator control module lights: BWR-6		3.5	3.5	3.5	3.5
A3.02	Rod display module lights: BWR-6		3.5	3.5	3.5	3.5
A3.03	†Verification of proper functioning/ operability: BWR-6		3.4	3.3	3.4	3.3
A3.04	Annunciator and alarm signals: BWR-6		3.3	3.3	3.3	3.3
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Operator control module (lights and push buttons): BWR-6		3.7	3.7	3.7	3.7
A4.02	Rod display module (lights and push buttons): BWR-6		3.7	3.7	3.7	3.7
A4.03	Back panel indicating lights: BWR-6		3.4	3.3	3.4	3.3

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 201006

System Name: Rod Worth Minimizer System (RWM) (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between ROD WORTH MINIMIZER SYSTEM (RWM) and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Reactor manual control: P-Spec(Not-BWR6)~~		3.4	3.4	3.4	3.4
K1.02	~~Rod position indication system: P-Spec(Not-BWR6)~~		3.4	3.4	3.4	3.4
K1.03	~~Reactor water level control (feed flow): P-Spec(Not-BWR6)~~		3.1	3.2	3.1	3.2
K1.04	~~Steam flow/reactor power: P-Spec(Not-BWR6)~~		3.1	3.2	3.1	3.2
K1.05	~~Control rod drop accident: P-Spec(Not-BWR6)~~		3.5	3.8	3.5	3.8
K1.06	~~Rod sequence control system: P-Spec(Not-BWR6)~~		3.3	3.4	3.3	3.4
K1.07	~~Process computer: P-Spec(Not-BWR6)~~		2.8	2.9	2.8	2.9
K1.08	~~Reactor power (turbine first-stage pressure): P-Spec(Not-BWR6)~~		3.2	3.3	3.2	3.3
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~Rod worth minimizer: P-Spec(Not-BWR6)~~		2.2*	2.5*	2.2	2.5
K3	**Knowledge of the effect that a loss or malfunction of the ROD WORTH MINIMIZER SYSTEM (RWM) will have on following:**	**(41.7 / 45.4)**				
K3.01	~~Reactor manual control system: P-Spec(Not-BWR6)~~		3.2	3.5	3.2	3.5
K4	**Knowledge of ROD WORTH MINIMIZER SYSTEM (RWM) design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Insert blocks/errors: P-Spec(Not-BWR6)~~		3.4	3.5	3.4	3.5
K4.02	~~Withdraw blocks/errors: P-Spec(Not-BWR6)~~		3.5	3.5	3.5	3.5
K4.03	~~Select blocks/errors: P-Spec(Not-BWR6)~~		3.3	3.4	3.3	3.4
K4.04	~~System bypass: P-Spec(Not-BWR6)~~		3.4	3.5	3.4	3.5
K4.05	~~Substitute rod position data: P-Spec(Not-BWR6)~~		2.8	3.0	2.8	3.0
K4.06	~~Correction of out of sequence rod positions: P-Spec(Not-BWR6)~~		3.2	3.4	3.2	3.4
K4.07	~~Display of out of position control rods without rod blocks (transition zone): P-Spec(Not-BWR6)~~		3.1	3.2	3.1	3.2
K4.08	~~System testing: P-Spec(Not-BWR6)~~		2.8	2.8	2.8	2.8
K4.09	~~System initialization: P-Spec(Not-BWR6)~~		3.2	3.2	3.2	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201006

System Name: Rod Worth Minimizer System (RWM) (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K5	**Knowledge of the operational implications of the following concepts as they apply to ROD WORTH MINIMIZER SYSTEM (RWM):**	**(41.5 / 45.3)**				
K5.01	~~Minimize clad damage if a control rod drop accident (CRDA) occurs: P-Spec(Not-BWR6)~~		3.3	3.7	3.3	3.7
K5.02	~~Low power set point: P-Spec(Not-BWR6)~~		2.9	3.0	2.9	3.0
K5.03	~~Low power alarm point (LPAP): P-Spec(Not-BWR6)~~		2.8	2.9	2.8	2.9
K5.04	~~Transition zone: P-Spec(Not-BWR6)~~		2.8	2.8	2.8	2.8
K5.05	~~High power set point: P-Spec(Not-BWR6)~~		2.9	3.0	2.9	3.0
K5.06	~~Rod groups and steps: P-Spec(Not-BWR6)~~		2.8	3.0	2.8	3.0
K5.07	~~Latch groups: P-Spec(Not-BWR6)~~		2.8	2.9	2.8	2.9
K5.08	~~Operating sequence: P-Spec(Not-BWR6)~~		2.9	2.9	2.9	2.9
K5.09	~~Select error: P-Spec(Not-BWR6)~~		3.2	3.2	3.2	3.2
K5.10	~~Withdraw error: P-Spec(Not-BWR6)~~		3.2	3.3	3.2	3.3
K5.11	~~Insert error: P-Spec(Not-BWR6)~~		3.2	3.3	3.2	3.3
K5.12	~~Withdraw block: P-Spec(Not-BWR6)~~		3.5	3.5	3.5	3.5
K5.13	~~Insert block: P-Spec(Not-BWR6)~~		3.5	3.5	3.5	3.5
K5.14	~~Alternate withdraw and insert limits: P-Spec(Not-BWR6)~~		3.0	3.0	3.0	3.0
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the ROD WORTH MINIMIZER SYSTEM (RWM):**	**(41.7 / 45.7)**				
K6.01	~~RWM power supply: P-Spec(Not-BWR6)~~		2.8	3.2	2.8	3...
K6.02	~~Reactor water level control input: P-Spec(Not-BWR6)~~		2.9	2.9	2.9	2.9
K6.03	~~Rod position indication: P-Spec(Not-BWR6)~~		2.9	2.9	2.9	2.9
K6.04	~~Process computer: P-Spec(Not-BWR6)~~		2.7	2.8	2.7	2.8
K6.05	~~Steam flow input: P-Spec(Not-BWR6)~~		2.7	2.7	2.7	2.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the ROD WORTH MINIMIZER SYSTEM (RWM) controls including:**	**(41.5 / 45.5)**				
A1.01	~~Rod position: P-Spec(Not-BWR6)~~		3.2	3.3	3.2	3.3
A1.02	~~Status of control rod movement blocks; P-Spec(Not-BWR6)~~		3.4	3.5	3.4	3.5
A1.03	~~Latched group indication: P-Spec(Not-BWR6)~~		2.9	3.0	2.9	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 201006

System Name: Rod Worth Minimizer System (RWM) (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the ROD WORTH MINIMIZER SYSTEM (RWM); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Power supply loss: P-Spec(Not-BWR6)~~		2.5	2.8	2.5	2.8
A2.02	~~Loss of steam flow input: P-Spec(Not-BWR6)~~		2.6	2.9	2.6	2.9
A2.03	~~Rod drift: P-Spec(Not-BWR6)~~		3.0	3.2	3.0	3.2
A2.04	~~Stuck rod: P-Spec(Not-BWR6)~~		3.0	3.3	3.0	3.3
A2.05	~~Out-of-sequence rod movement; P-Spec(Not-BWR6)~~		3.1	3.5	3.1	3.5
A2.06	~~Loss of reactor water level control input: P-Spec(Not-BWR6)~~		2.9	3.3	2.9	3.3
A2.07	~~RWM hardware/software failure: P-Spec(Not-BWR6)~~		2.5	2.8	2.5	2.8
A3	**Ability to monitor automatic operations of the ROD WORTH MINIMIZER SYSTEM (RWM) including:**	**(41.7 / 45.7)**				
A3.01	~~System window and light indication: P-Spec(Not-BWR6)~~		3.2	3.1	3.2	3.1
A3.02	~~Verification of proper functioning/ operability: P-Spec(Not-BWR6)~~		3.5	3.4	3.5	3.4
A3.03	~~Annunciator and alarm signals: P-Spec(Not-BWR6)~~		3.1	3.0	3.1	3.0
A3.04	~~Control rod movement blocks: P-Spec(Not-BWR6)~~		3.5	3.4	3.5	3.4
A3.05	~~Latched group indication: P-Spec(Not-BWR6)~~		3.0	3.1	3.0	3.1
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	~~System bypass switch: P-Spec(Not-BWR6)~~		3.2	3.4	3.2	3.4
A4.02	~~Pushbutton indicating switches: P-Spec(Not-BWR6)~~		2.9	2.9	2.9	2.9
A4.03	~~Latched group indication: P-Spec(Not-BWR6)~~		3.0	3.0	3.0	3.0
A4.04	~~Rod withdrawal error indication: P-Spec(Not-BWR6)~~		3.3	3.2	3.3	3.2
A4.05	~~Rod insert error indication: P-Spec(Not-BWR6)~~		3.2	3.2	3.2	3.2
A4.06	~~Selected rod position indication: P-Spec(Not-BWR6)~~		3.2	3.2	3.2	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202001

System Name: Recirculation System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RECIRCULATION SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Core flow		3.6	3.7	3.6	3.7
K1.02	Reactor power		4.1	4.1	4.1	4.1
K1.03	Reactor moderator temperature		3.2	3.3	3.2	3.3
K1.04	Reactor/turbine pressure regulating system: Plant-Specific		3.3	3.3	3.3	3.3
K1.05	Reactor pressure		3.4	3.4	3.4	3.4
K1.06	Jet pumps		3.6	3.6	3.6	3.6
K1.07	Component cooling water systems		3.1	3.2	3.1	3.2
K1.08	A.C. electrical		3.1	3.2	3.1	3.2
K1.09	D.C. electrical		2.7	2.9	2.7	2.9
K1.10	Control rod drive system: Plant-Specific		2.8	2.8	2.8	2.8
K1.11	Drywell equipment drain sump		2.7	2.8	2.7	2.8
K1.12	Recirculation system motor-generator sets: Plant-Specific		3.6	3.6	3.6	3.6
K1.13	Jet pump ring header and risers: Plant-Specific		3.1	3.2	3.1	3.2
K1.14	~~Rod block monitor: Plant-Specific~~		3.0	3.2	3.0	3.2
K1.15	Nuclear boiler instrumentation (reactor water level/pressure)		3.2	3.2	3.2	3.2
K1.16	Low pressure coolant injection logic: Plant-Specific		3.9	3.9	3.9	3.9
K1.17	Vessel bottom head drain temperature		3.1	3.3	3.1	3.3
K1.18	RHR shutdown cooling mode		3.3	3.3	3.3	3.3
K1.19	Feedwater flow		3.2	3.2	3.2	3.2
K1.20	Plant air systems: Plant-Specific		2.4	2.5	2.4	2.5
K1.21	Reactor water cleanup system		2.6	2.6	2.6	2.6
K1.22	Reactor water level		3.5	3.6	3.5	3.6
K1.23	Average power range monitor flow converters: Plant-Specific		3.4	3.5	3.4	3.5
K1.24	~~Isolation condenser: Plant-Specific~~		3.4	3.4	3.4	3.4
K1.25	Reactor water sampling system		2.6	2.7	2.6	2.7
K1.26	Recirculation flow control system: Plant-Specific		3.6	3.6	3.6	3.6
K1.27	ATWS circuitry: Plant-Specific		4.1	4.3*	4.1	4.3
K1.28	End-of-cycle recirculation pump trip circuitry: Plant-Specific		3.9	4.1	3.9	4.1
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Recirculation pumps: Plant-Specific		3.2*	3.2	3.2	3.2
K2.02	MG sets: Plant-Specific		3.2	3.3	3.2	3.3
K2.03	Recirculation system valves		2.7*	2.8*	2.7	2.8
K2.04	Hydraulic power unit oil pumps: Plant-Specific		2.5*	2.5*	2.5	2.5
K2.05	~~MG set oil pumps: Plant-Specific~~		2.3*	2.3*	2.3	2.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202001

System Name: Recirculation System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
K3	**Knowledge of the effect that a loss or malfunction of the RECIRCULATION SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Core flow		3.6	3.6	3.6	3.6
K3.02	~~Load following capabilities: Plant-Specific~~		2.8	2.8	2.8	2.8
K3.03	Reactor power		3.9	3.9	3.9	3.9
K3.04	Reactor water level		3.7	3.7	3.7	3.7
K3.05	Recirculation system MG sets: Plant-Specific		3.3	3.3	3.3	3.3
K3.06	Low pressure coolant injection logic: Plant-Specific		3.7	3.9	3.7	3.9
K3.07	Vessel bottom head drain temperature		2.9	2.9	2.9	2.9
K3.08	Shutdown cooling system		2.8	2.9	2.8	2.9
K3.09	Reactor water cleanup system		2.4*	2.5*	2.4	2.5
K3.10	Average power range monitor flow converters		3.3	3.4	3.3	3.4
K3.11	Component cooling water systems		2.3*	2.3*	2.3	2.3
K3.12	~~Isolation condenser: Plant-Specific~~		3.0	3.0	3.0	3.0
K3.13	Reactor water sampling system		2.5	2.5	2.5	2.5
K3.14	Primary containment integrity: Plant-Specific		3.5	3.5	3.5	3.5
K4	**Knowledge of RECIRCULATION System design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	2/3 core coverage: Plant-Specific		3.9	3.9	3.9	3.9
K4.02	Adequate recirculation pump NPSH		3.1	3.2	3.1	
K4.03	Recirculation pump motor cooling		2.8	2.8	2.8	2.8
K4.04	Controlled seal flow		3.0	3.1	3.0	3.1
K4.05	Seal cooling		2.9	2.9	2.9	2.9
K4.06	Automatic voltage/frequency regulation: Plant-Specific		2.6*	2.7	2.6	2.7
K4.07	Motor generator set trips: Plant-Specific		2.8	2.9	2.8	2.9
K4.08	~~Oil pump automatic starts: Plant-Specific~~		2.8	2.9	2.8	2.9
K4.09	Pump minimum flow limit: Plant-Specific		2.7	2.9	2.7	2.9
K4.10	Pump start permissives: Plant-Specific		3.3	3.4	3.3	3.4
K4.11	Limitation of recirculation pumps flow mismatch: Plant-Specific		3.1	3.5	3.1	3.5
K4.12	Minimization of reactor vessel bottom head temperature gradients: Plant-Specific		3.2	3.5	3.2	3.5
K4.13	†End-of-cycle recirculation pump trip circuitry: Plant-Specific		3.7	4.0*	3.7	4.0
K4.14	ATWS: Plant-Specific		4.0	4.1*	4.0	4.1
K4.15	Slow speed pump start: Plant-Specific		3.1	3.4	3.1	3.4
K4.16	Recirculation pump downshift/runback: Plant-Specific		3.3	3.6	3.3	3.6
K4.17	Fast speed pump start: Plant-Specific		3.3	3.5	3.3	3.5
K4.18	Automatic MG set start sequencing: Plant-Specific		2.8	3.0	2.8	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202001

System Name: Recirculation System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K5	**Knowledge of the operational implications of the following concepts as they apply to RECIRCULATION SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Indications of pump cavitation		2.7	2.8	2.7	2.8
K5.02	Jet pump operation: BWR-3, 4, 5, 6		3.1	3.2	3.1	3.2
K5.03	Pump/motor cooling: Plant-Specific		2.7	2.7	2.7	2.7
K5.04	System venting		2.4	2.6	2.4	2.6
K5.05	End of cycle recirculation pump trip: Plant-Specific		3.5	3.6	3.5	3.6
K5.06	ATWS RPT: Plant-Specific		3.6	3.7	3.6	3.7
K5.07	Natural circulation		3.3	3.4	3.3	3.4
K5.08	E/P converters: Plant-Specific		2.0*	2.1*	2.0	2.1
K5.09	Hydraulically operated valves: Plant-Specific		2.6*	2.6*	2.6	2.6
K5.10	Motor generator set operation: Plant-Specific		2.8*	2.8*	2.8	2.8
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RECIRCULATION SYSTEM:**	**(41.7 / 45.7)**				
K6.01	Jet pumps: Plant-Specific		3.5	3.7	3.5	3.7
K6.02	Component cooling water systems		3.1	3.2	3.1	3.2
K6.03	A.C. power: Plant-Specific		2.9	3.0	2.9	3.0
K6.04	D.C. power: Plant-Specific		2.7	2.8	2.7	2.8
K6.05	Control rod drive system: Plant-Specific		2.7	2.8	2.7	2.8
K6.06	Recirculation system motor-generator sets: Plant-Specific		3.1	3.1	3.1	3.1
K6.07	Feedwater flow		3.3	3.3	3.3	3.3
K6.08	Reactor water cleanup system		2.3	2.3	2.3	2.3
K6.09	Reactor water level		3.4	3.4	3.4	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202001

System Name: Recirculation System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RECIRCULATION SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Recirculation pump flow: Plant-Specific		3.6	3.5	3.6	3.5
A1.02	Jet pump flow		3.4	3.4	3.4	3.4
A1.03	Core flow		3.6	3.6	3.6	3.6
A1.04	Reactor water level		3.3	3.3	3.3	3.3
A1.05	Reactor power		3.9	3.9	3.9	3.9
A1.06	Recirculation pump motor amps		2.5	2.6	2.5	2.6
A1.07	Recirculation pump speed		2.7	2.8	2.7	2.8
A1.08	Recirculation FCV position: BWR-5, 6		3.7	3.7	3.7	3.7
A1.09	Recirculation pump seal pressures		3.3	3.3	3.3	3.3
A1.10	Recirculation seal purge flows		2.6	2.7	2.6	2.7
A1.11	Vessel bottom head drain temperature		2.8	2.9	2.8	2.9
A1.12	Recirculation pump differential pressure: Plant-Specific		2.6	2.6	2.6	2.6
A1.13	Recirculation loop temperatures: Plant-Specific		3.1	3.2	3.1	3.2
A1.14	Recirculation drive motor temperature: Plant-Specific		2.4	2.4	2.4	2.4
A1.15	Recirculation MG set temperatures: Plant-Specific		2.4	2.4	2.4	2.4
A1.16	Recirculation MG drive motor amps: Plant-Specific		2.3*	2.3	2.3	2.3
A1.17	Recirculation MG set generator current, power, voltage: Plant-Specific		2.3*	2.3	2.3	2.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202001

System Name: Recirculation System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the RECIRCULATION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	†Jet pump failure: Plant-Specific		3.4	3.9	3.4	3.9
A2.02	Recirculation system leak		3.7	3.9	3.7	3.9
A2.03	Single recirculation pump trip		3.6	3.7	3.6	3.7
A2.04	Multiple recirculation pump trip		3.7	3.8	3.7	3.8
A2.05	Inadvertent recirculation flow increase		3.8	4.0	3.8	4.0
A2.06	Inadvertent recirculation flow decrease		3.6	3.8	3.6	3.8
A2.07	Recirculation pump speed mismatch: Plant-Specific		3.1	3.3	3.1	3.3
A2.08	Recirculation flow mismatch: Plant-Specific		3.1	3.4	3.1	3.4
A2.09	~~Recirculation scoop tube lockup: Plant-Specific~~		3.2	3.4	3.2	3.4
A2.10	†Recirculation pump seal failure		3.5	3.9	3.5	3.9
A2.11	Low reactor water level		3.7	3.9	3.7	3.9
A2.12	Loss of reactor feedwater		3.6	3.8	3.6	3.8
A2.13	Carryunder		2.6	2.8	2.6	2.8
A2.14	High reactor pressure (ATWS circuitry initiation): Plant-Specific		3.9	4.2*	3.9	4.2
A2.15	End of cycle trip circuitry: Plant-Specific		3.7	3.9	3.7	3.9
A2.16	Loss of seal purge flow (CRD)		2.9	3.1	2.9	3.1
A2.17	Loss of seal cooling water		3.1	3.2	3.1	3.2
A2.18	Loss of motor cooling		2.9	3.1	2.9	3.1
A2.19	Loss of A.C. power: Plant-Specific		3.1	3.2	3.1	3.2
A2.20	Loss of D.C. power: Plant-Specific		2.8	2.9	2.8	2.9
A2.21	†Recirculation loop temperature out of spec: Plant-Specific		3.3	3.7	3.3	3.7
A2.22	Loss of component cooling water		3.1	3.2	3.1	3.2
A2.23	Suppression pool level: BWR-2, 3, 4		3.2	3.2	3.2	3.2
A2.24	Valve opening		3.1	3.1	3.1	3.1
A2.25	Recirculation flow control valve lockup: Plant-Specific		3.3	3.3	3.3	3.3
A2.26	Incomplete start sequence: Plant-Specific		2.9	3.1	2.9	3.1
A3	**Ability to monitor automatic operations of the RECIRCULATION SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		3.1	3.1	3.1	3.1
A3.02	Pump/MG set start sequence: Plant-Specific		3.1	3.0	3.1	3.0
A3.03	System flow		3.2	3.2	3.2	3.2
A3.04	Lights and alarms		3.2	3.1	3.2	3.1
A3.05	Pump speed: Plant-Specific		2.9	2.9	2.9	2.9
A3.06	Flow control valve position: BWR-5, 6		3.6	3.6	3.6	3.6
A3.07	Pump trips: Plant-Specific		3.3	3.3	3.3	3.3
A3.08	Pump downshift: BWR-5, 6		3.4	3.3	3.4	3.3
A3.09	MG set trip: Plant-Specific		3.3	3.3	3.3	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202001

System Name: Recirculation System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Recirculation pumps		3.7	3.7	3.7	3.7
A4.02	System valves		3.5	3.4	3.5	3.4
A4.03	Reactor power		4.1	4.1	4.1	4.1
A4.04	System flow		3.7	3.7	3.7	3.7
A4.05	Lights and alarms		3.3	3.3	3.3	3.3
A4.06	Oil pumps		2.7	2.7	2.7	2.7
A4.07	Vent fans: Plant-Specific		2.4	2.3	2.4	2.3
A4.08	Motor-generator sets: Plant-Specific		3.2	3.1	3.2	3.1
A4.09	Reactor water level		3.7	3.7	3.7	3.7
A4.10	Seal flow: Plant-Specific		2.8	2.8	2.8	2.8
A4.11	Seal pressures: Plant-Specific		3.2	3.3	3.2	3.3
A4.12	Core flow		3.9	3.8	3.9	3.8
A4.13	Core differential pressure		3.1	3.3	3.1	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202002

System Name: Recirculation Flow Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RECIRCULATION FLOW CONTROL SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Recirculation system		3.5	3.6	3.5	3.6
K1.02	Reactor power		4.2*	4.2	4.2	4.2
K1.03	Reactor core flow		3.7	3.7	3.7	3.7
K1.04	Reactor/turbine pressure regulating system: Plant-Specific		3.1	3.1	3.1	3.1
K1.05	Recirculation MG set: Plant-Specific		3.5	3.5	3.5	3.5
K1.06	A.C. electrical		2.9	3.0	2.9	3.0
K1.07	D.C. electrical		2.6	2.8	2.6	2.8
K1.08	Feedwater flow		3.1	3.2	3.1	3.2
K1.09	Reactor water level		3.1	3.2	3.1	3.2
K1.10	Rod pattern		2.5	2.6	2.5	2.6
K1.11	APRM system		3.4	3.4	3.4	3.4
K1.12	Recirculation flow control valves: Plant-Specific		3.7	3.9	3.7	3.9
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Recirculation flow control system		2.4*	2.4*	2.4	2.4
K2.02	Hydraulic power unit: Plant-Specific		2.6	2.6	2.6	2.6
K3	**Knowledge of the effect that a loss or malfunction of the RECIRCULATION FLOW CONTROL SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Core flow		3.5	3.5	3.5	3.5
K3.02	Reactor power		4.0	4.0	4.0	4.0
K3.03	Reactor water level		3.3	3.4	3.3	3.4
K3.04	Reactor/turbine pressure regulation system		2.9	3.1	2.9	3.1
K3.05	Recirculation pump speed: Plant-Specific		3.2	3.3	3.2	3.3
K3.06	Recirculation flow control valve position: Plant-Specific		3.7	3.7	3.7	3.7
K4	**Knowledge of RECIRCULATION FLOW CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Scoop tube break: Plant-Specific~~		3.1	3.1	3.1	3.1
K4.02	Recirculation pump speed control: Plant-Specific		3.0	3.0	3.0	3.0
K4.03	Signal failure detection: Plant-Specific		3.0	3.0	3.0	3.0
K4.04	~~Automatic load following: Plant-Specific~~		2.4*	2.4*	2.4	2.4
K4.05	Limiting recirculation pump speed mismatch: Plant-Specific		3.1	3.4	3.1	3.4
K4.06	Recirculation pump adequate NPSH: Plant-Specific		3.1	3.1	3.1	3.1
K4.07	Minimum and maximum pump speed setpoints		2.9	2.9	2.9	2.9
K4.08	~~Automatic flow control valve positioning: BWR-5, 6~~		3.3	3.4	3.3	3.4
K4.09	Minimum and maximum flow control valve position setpoints: BWR-5, 6		3.3	3.4	3.3	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202002

System Name: Recirculation Flow Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
K5	**Knowledge of the operational implications of the following concepts as they apply to RECIRCULATION FLOW CONTROL SYSTEM:**	**(41.5 / 45.3)**				
K5.01	~~Fluid coupling: BWR-3, 4~~		2.8	2.8	2.8	2.8
K5.02	Feedback signals		2.6	2.6	2.6	2.6
K5.03	Error signals		2.4	2.4	2.4	2.4
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RECIRCULATION FLOW CONTROL SYSTEM:**	**(41.7 / 45.7)**				
K6.01	A.C. power		2.8	2.9	2.8	2.9
K6.02	D.C. power		2.6	2.6	2.6	2.6
K6.03	Recirculation system		2.8	2.8	2.8	2.8
K6.04	Feedwater flow inputs: BWR-3, 4, 5,6		3.5	3.5	3.5	3.5
K6.05	Reactor water level		3.1	3.1	3.1	3.1
K6.06	Reactor/turbine pressure regulating system: Plant-Specific		2.9	2.9	2.9	2.9
K6.07	APRM signal input: BWR-5, 6		3.6	3.7	3.6	3.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RECIRCULATION FLOW CONTROL SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Recirculation pump speed: BWR-2, 3, 4, 5, 6		3.2	3.2	3.2	
A1.02	MG set drive motor amps: Plant-Specific		2.7	2.7	2.7	2.7
A1.03	MG set generator current, power, voltage		2.5*	2.4*	2.5	2.4
A1.04	Reactor water level		2.9	2.9	2.9	2.9
A1.05	Reactor power		3.6	3.6	3.6	3.6
A1.06	Reactor core flow		3.4	3.3	3.4	3.3
A1.07	Recirculation loop flow: Plant-Specific		3.1	3.1	3.1	3.1
A1.08	Recirculation FCV position: BWR-5, 6		3.4	3.4	3.4	3.4
A2	**Ability to (a) predict the impacts of the following on the RECIRCULATION FLOW CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Recirculation pump trip		3.4	3.4	3.4	3.4
A2.02	Loss of A.C.		2.9	3.0	2.9	3.0
A2.03	Loss of D.C.		2.6	2.7	2.6	2.7
A2.04	Recirculation pump speed mismatch between loops: Plant-Specific		3.0	3.2	3.0	3.2
A2.05	~~Scoop tube lockup: BWR-2, 3, 4~~		3.1	3.1	3.1	3.1
A2.06	Low reactor water level: Plant-Specific		3.3	3.3	3.3	3.3
A2.07	Loss of feedwater signal inputs: Plant-Specific		3.3	3.3	3.3	3.3
A2.08	FCV lockup: BWR-5, 6		3.3	3.3	3.3	3.3
A2.09	†Recirculation flow mismatch: Plant-Specific		3.1	3.3	3.1	

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 202002

System Name: Recirculation Flow Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A3	**Ability to monitor automatic operations of the RECIRCULATION FLOW CONTROL SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Flow control valve operation: BWR-5, 6		3.6	3.4	3.6	3.4
A3.02	Lights and alarms		3.4	3.4	3.4	3.4
A3.03	~~Scoop tube operation: BWR-2, 3, 4~~		3.1	3.0	3.1	3.0
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	MG sets		3.3	3.1	3.3	3.1
A4.02	Hydraulic power unit: BWR-5, 6		2.8	2.8	2.8	2.8
A4.03	Lights and alarms		3.1	3.1	3.1	3.1
A4.04	Reactor power		3.8	3.8	3.8	3.8
A4.05	Reactor level		3.4	3.4	3.4	3.4
A4.06	~~Scoop tube power: BWR-2, 3, 4~~		2.7	2.6	2.7	2.6
A4.07	Recirculation pump speed: BWR-2, 3, 4, 5, 6		3.3	3.2	3.3	3.2
A4.08	Recirculation system flow		3.3	3.3	3.3	3.3
A4.09	Core flow		3.2	3.3	3.2	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 203000

System Name: RHR/LPCI: Injection Mode (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RHR/LPCI: INJECTION MODE and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Condensate storage and transfer system: Plant-Specific		2.8	2.8	2.8	2.8
K1.02	Suppression pool		3.9	3.9	3.9	3.9
K1.03	Condensate transfer		2.5*	2.6*	2.5	2.6
K1.04	Keep fill system		3.3	3.4	3.3	3.4
K1.05	~~Recirculation system: BWR-3, 4~~		3.8	3.8	3.8	3.8
K1.06	Automatic depressurization		3.9	3.9	3.9	3.9
K1.07	D.C. electrical power		3.1	3.3	3.1	3.3
K1.08	A.C. electrical power		3.5	3.5	3.5	3.5
K1.09	Emergency generators		3.8	3.8	3.8	3.8
K1.10	ECCS room coolers		3.2	3.2	3.2	3.2
K1.11	Nuclear boiler instrumentation		3.7	3.7	3.7	3.7
K1.12	Plant air systems: Plant-Specific		2.6*	2.7*	2.6	2.7
K1.13	Drywell pressure		3.9	4.0	3.9	4.0
K1.14	Shutdown cooling system: Plant-Specific		3.6	3.7	3.6	3.7
K1.15	Reactor building drain system: Plant-Specific		2.4*	2.6*	2.4	2.6
K1.16	Component cooling water systems		3.1	3.2	3.1	3.2
K1.17	Reactor pressure		4.0	4.0	4.0	4.0
K1.18	Reactor vessel: Plant-Specific		3.8	3.8	3.8	3.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Pumps		3.5*	3.5*	3.5	3.5
K2.02	Valves		2.5*	2.7*	2.5	2.7
K2.03	Initiation logic		2.7*	2.9*	2.7	2.9
K3	**Knowledge of the effect that a loss or malfunction of the *RHR/LPCI: INJECTION MODE will have on following:***	**(41.7 / 45.4)**				
K3.01	Reactor water level		4.3*	4.4*	4.3	4.4
K3.02	Suppression pool level		3.5	3.5	3.5	3.5
K3.03	Automatic depressurization logic		4.2*	4.3*	4.2	4.3
K3.04	Adequate core cooling		4.6*	4.6*	4.6	4.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 203000

System Name: RHR/LPCI: Injection Mode (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K4	**Knowledge of RHR/LPCI: INJECTION MODE design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Automatic system initiation/ injection		4.2*	4.2	4.2	4.2
K4.02	Prevention of piping overpressurization		3.3	3.4	3.3	3.4
K4.03	Pump minimum flow protection		3.2	3.3	3.2	3.3
K4.04	Pump motor cooling: Plant-Specific		2.6	2.7	2.6	2.7
K4.05	Prevention of water hammer		3.2	3.3	3.2	3.3
K4.06	Adequate pump net positive suction head (interlock suction valve open): Plant-Specific		3.5	3.5	3.5	3.5
K4.07	Emergency generator load sequencing		3.7	3.9	3.7	3.9
K4.08	Pump operability testing		3.3	3.4*	3.3	3.4
K4.09	Surveillance for all operable components		3.1	3.4	3.1	3.4
K4.10	Dedicated injection system during automatic system initiation (injection valve interlocks)		3.9	4.1	3.9	4.1
K4.11	~~Loop selection logic: Plant-Specific~~		4.0	4.0	4.0	4.0
K4.12	System redundancy		3.5	3.6	3.5	3.6
K4.13	The prevention of leakage to the environment through LPCI/RHR heat exchanger: Plant-Specific		3.4	3.7	3.4	3.7
K4.14	†Operation from remote shutdown panel		3.6	3.7	3.6	3.7
K4.15	Pump runout protection: Plant-Specific		2.5*	2.5	2.5	2.5
K5	**Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: INJECTION MODE:**	**(41.5 / 45.3)**				
K5.01	~~Testable check valve operation~~		2.7*	2.9	2.7	2.9
K5.02	†Core cooling methods		3.5	3.7	3.5	3.7
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: INJECTION MODE:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical power		3.6	3.7	3.6	3.7
K6.02	D.C. electrical power		2.8*	3.0*	2.8	3.0
K6.03	Emergency generator		3.7	3.9	3.7	3.9
K6.04	Keep fill system		3.3	3.5	3.3	3.5
K6.05	Condensate storage and transfer system: Plant-Specific		2.5	2.5	2.5	2.5
K6.06	Suppression pool		3.8	3.9	3.8	3.9
K6.07	Plant air systems: Plant-Specific		2.7	2.7	2.7	2.7
K6.08	ECCS room cooling		2.9	3.1	2.9	3.1
K6.09	Nuclear boiler instrumentation		3.4	3.4	3.4	3.4
K6.10	Component cooling water systems		3.0	3.1	3.0	3.1
K6.11	ADS		4.1*	4.1	4.1	4.1
K6.12	†ECCS room integrity		2.7	2.9	2.7	2.9

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 203000

System Name: RHR/LPCI: Injection Mode (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: INJECTION MODE controls including:**	**(41.5 / 45.5)**				
A1.01	Reactor water level		4.2*	4.3*	4.2	4.3
A1.02	Reactor pressure		3.9*	4.0*	3.9	4.0
A1.03	System flow		3.8	3.7	3.8	3.7
A1.04	System pressure		3.6	3.6	3.6	3.6
A1.05	Suppression pool level		3.8	3.7	3.8	3.7
A1.06	Condensate storage tank level: Plant-Specific		2.4*	2.5*	2.4	2.5
A1.07	Motor amps: Plant-Specific		2.4*	2.5*	2.4	2.5
A1.08	Emergency generator loading		3.7	3.8	3.7	3.8
A1.09	Component cooling water systems		2.9	2.9	2.9	2.9
A2	**Ability to (a) predict the impacts of the following on the RHR/LPCI: INJECTION MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Inadequate net positive suction head		3.2	3.4	3.2	3.4
A2.02	Pump trips		3.5	3.5	3.5	3.5
A2.03	Valve closures		3.2	3.3	3.2	3.3
\2.04	A.C. failures		3.5	3.6	3.5	3.6
A2.05	D.C. failures		3.0	3.2	3.0	3.2
A2.06	Emergency generator failure		3.8	3.9	3.8	3.9
A2.07	Pump seal failure		3.0	3.1	3.0	3.1
A2.08	Inadequate room cooling		2.9	3.0	2.9	3.0
A2.09	Inadequate system flow		3.3	3.4	3.3	3.4
A2.10	Nuclear boiler instrument failures		3.3	3.5	3.3	3.5
A2.11	Motor operated valve failures		3.4	3.6	3.4	3.6
A2.12	Pump runout		2.6*	2.7*	2.6	2.7
A2.13	Valve openings		3.2	3.3	3.2	3.3
A2.14	Initiating logic failure		3.8	3.9*	3.8	3.9
A2.15	~~Loop selection logic failure: Plant-Specific~~		4.2*	4.2*	4.2	4.2
A2.16	Loss of coolant accident		4.4*	4.5*	4.4	4.5
A2.17	Keep fill system failure		3.3	3.5	3.3	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 203000

System Name: RHR/LPCI: Injection Mode (Plant Specific)

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I SRO
A3	**Ability to monitor automatic operations of the RHR/LPCI: INJECTION MODE including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		3.8*	3.7	3.8	3.7
A3.02	Pump start		4.0	3.9	4.0	3.9
A3.03	Pump discharge pressure		3.7	3.6	3.7	3.6
A3.04	System flow		3.8	3.7	3.8	3.7
A3.05	Reactor water level		4.4*	4.4*	4.4	4.4
A3.06	Indicating lights and alarms		3.7*	3.6	3.7	3.6
A3.07	~~Loop selection: Plant-Specific~~		4.2*	4.6*	4.2	4.6
A3.08	System initiation sequence		4.1	4.1	4.1	4.1
A3.09	Emergency generator load sequencing		3.6	3.9*	3.6	3.9
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Pumps		4.3*	4.1*	4.3	4.1
A4.02	System valves		4.1*	4.1*	4.1	4.1
A4.03	Keep fill system		3.4	3.4	3.4	3.4
A4.04	Heat exchanger cooling flow		3.6	3.6	3.6	3.6
A4.05	Manual initiation controls		4.3*	4.1*	4.3	4.1
A4.06	System reset following automatic initiation: Plant-Specific		3.9	3.9	3.9	3.9
A4.07	Reactor water level		4.5*	4.5*	4.5	4.5
A4.08	Reactor pressure		4.3*	4.3	4.3	4
A4.09	System flow		4.1	4.0	4.1	4.0
A4.10	Pump/system discharge pressure: Plant-Specific		3.7	3.6	3.7	3.6
A4.11	Indicating lights and alarms		3.7*	3.5*	3.7	3.5
A4.12	~~Condensate storage tank level: Plant-Specific~~		2.5	2.6	2.5	2.6
A4.13	Suppression pool level/temperature		3.9	3.9	3.9	3.9
A4.14	~~Testable check valves~~		2.7*	2.7*	2.7	2.7

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 204000

System Name: Reactor Water Cleanup System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR WATER CLEANUP SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Reactor vessel		3.1	3.3	3.1	3.3
K1.02	Recirculation system: Plant-Specific		2.9	3.0	2.9	3.0
K1.03	Reactor feedwater system		3.1	3.1	3.1	3.1
K1.04	Component cooling water systems		2.9	2.9	2.9	2.9
K1.05	Plant air systems		2.7	2.7	2.7	2.7
K1.06	Main condenser		2.8	2.8	2.8	2.8
K1.07	Radwaste		2.6	2.7	2.6	2.7
K1.08	SBLC		3.7	3.8	3.7	3.8
K1.09	Reactor water level		3.2	3.3	3.2	3.3
K1.10	Reactor water quality		3.3	3.5	3.3	3.5
K1.11	PCIS/NSSSS		3.5	3.7	3.5	3.7
K1.12	Condensate storage and transfer system: Plant-Specific		2.6	2.6	2.6	2.6
K1.13	RHR system: Plant-Specific		2.5	2.5	2.5	2.5
K1.14	Process sample system		2.5	2.5	2.5	2.5
K1.15	Leak detection: Plant-Specific		3.1	3.2	3.1	3.2
K1.16	CRD system: Plant-Specific		2.8	2.8	2.8	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Pumps		2.3*	2.4*	2.3	2.4
K2.02	Valve motors		2.0*	2.1*	2.0	2.1
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR WATER CLEANUP SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor water quality		3.2	3.6	3.2	3.6
K3.02	Reactor water level		3.1	3.1	3.1	3.1
K3.03	Component cooling water systems		2.4	2.4	2.4	2.4
K3.04	Reactor water temperature		2.6	2.6	2.6	2.6
K3.05	Area temperature		2.4*	2.4*	2.4	2.4
K3.06	Area radiation levels		2.6	2.7	2.6	2.7
K3.07	Drywell temperature		2.1*	2.2*	2.1	2.2
K3.08	Drywell pressure		2.2*	2.3*	2.2	2.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 204000

System Name: Reactor Water Cleanup System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K4	**Knowledge of REACTOR WATER CLEANUP SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Pump protection		2.5	2.5	2.5	2.5
K4.02	Piping over-pressurization protection: Plant-Specific		2.7	2.9	2.7	2.9
K4.03	Over temperature protection for system components		2.9	2.9	2.9	2.9
K4.04	System isolation upon-receipt of isolation signals		3.5	3.6	3.5	3.6
K4.05	Double valve isolation from the reactor		3.0	3.1	3.0	3.1
K4.06	Maximize plant efficiency (use of regenerative heat exchanger)		2.6	2.8	2.6	2.8
K4.07	Draining of reactor water to various locations		2.9	2.9	2.9	2.9
K4.08	~~Reducing reactor pressure upstream of low pressure piping: LP-RWCU~~		3.3	3.4	3.3	3.4
K5	**Knowledge of the operational implications of the following concepts as they apply to REACTOR WATER CLEANUP SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Electro/ Pneumatic converter operation		1.9*	2.1*	1.9	2.1
K5.02	Control device operation		2.2*	2.2*	2.2	2.2
K5.03	Demineralizer operation		2.4	2.5	2.4	2.5
K5.04	Heat exchanger operation		2.7	2.7	2.7	2.7
K5.05	Flow controllers		2.6	2.6	2.6	2
K5.06	Pressure controllers		2.6*	2.6	2.6	2.
K5.07	Conductivity measurement		2.5	2.6	2.5	2.6
K5.08	Temperature measurement		2.6	2.6	2.6	2.6
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR WATER CLEANUP SYSTEM:**	**(41.7 / 45.7)**				
K6.01	Component cooling water systems		3.1	3.3	3.1	3.3
K6.02	Main condenser		2.4	2.4	2.4	2.4
K6.03	Radwaste		2.4	2.4	2.4	2.4
K6.04	Plant air systems		2.7	2.8	2.7	2.8
K6.05	A.C. power		2.6	2.6	2.6	2.6
K6.06	Reactor feedwater system		2.4*	2.4*	2.4	2.4
K6.07	SBLC logic		3.3	3.5	3.3	3.5
K6.08	PCIS/NSSSS		3.5	3.5	3.5	3.5
K6.09	CRD hydraulics: Plant-Specific		2.7	2.9	2.7	2.9

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 204000

System Name: Reactor Water Cleanup System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the REACTOR WATER CLEANUP SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Reactor water level		3.1	3.2	3.1	3.2
A1.02	Component cooling water temperature		2.9	2.9	2.9	2.9
A1.03	Reactor water temperature		2.8	2.9	2.8	2.9
A1.04	System flow		2.8	2.8	2.8	2.8
A1.05	System pressure		2.6	2.6	2.6	2.6
A1.06	System temperature		2.8	2.8	2.8	2.8
A1.07	RWCU drain flow		2.9	2.9	2.9	2.9
A1.08	Main condenser hotwell level		2.3	2.3	2.3	2.3
A1.09	Reactor water conductivity		3.0	3.2	3.0	3.2
A2	**Ability to (a) predict the impacts of the following on the REACTOR WATER CLEANUP SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Loss of component cooling water		3.2	3.4	3.2	3.4
A2.02	~~Pressure control valve failure: LP-RWCU~~		3.2	3.2	3.2	3.2
A2.03	Flow control valve failure		2.9	2.9	2.9	2.9
A2.04	Pump trips		2.7	2.9	2.7	2.9
A2.05	Valve openings		2.7	2.8	2.7	2.8
A2.06	A.C. failure		2.5	2.6	2.5	2.6
A2.07	Loss of plant air systems		2.5	2.6	2.5	2.6
A2.08	RWCU pump seal failure		2.9	3.1	2.9	3.1
A2.09	Loss of room coolers		2.8	2.8	2.8	2.8
A2.10	Valve closures		2.7	2.8	2.7	2.8
A2.11	Inadequate system flow: Plant-Specific		2.6	2.7	2.6	2.7
A2.12	Excessive drain flow rates		2.7	2.8	2.7	2.8
A2.13	Signal received which results in a system isolation		3.4	3.4	3.4	3.4
A2.14	System high temperature		3.2	3.2	3.2	3.2
A2.15	Cleanup demineralizer high differential pressure		2.8	2.9	2.8	2.9
A3	**Ability to monitor *automatic operations of the* REACTOR WATER CLEANUP SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	~~System pressure downstream of the pressure regulating valve: LP-RWCU~~		3.3	3.3	3.3	3.3
A3.02	Reactor water quality		2.9	3.2	2.9	3.2
A3.03	Response to system isolations		3.6	3.6	3.6	3.6
A3.04	Response to interlocks and trips designed to protect system components		3.4	3.5	3.4	3.5
A3.05	Reactor water temperature		2.8	2.8	2.8	2.8
A3.06	Lights and alarms		3.1	3.1	3.1	3.1

System Number: 204000

System Name: Reactor Water Cleanup System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Im RO	Facility Im SI
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	System pumps		3.1	3.0	3.1	3.0
A4.02	Valve controllers		2.9	2.9	2.9	2.9
A4.03	RWCU drain flow regulator		3.2	3.1	3.2	3.1
A4.04	Heat exchanger operation		2.8	2.8	2.8	2.8
A4.05	System pressure		2.9	2.8	2.9	2.8
A4.06	System flow		3.0	2.9	3.0	2.9
A4.07	System temperature		3.1	3.1	3.1	3.1
A4.08	Reactor water level		3.4	3.4	3.4	3.4
A4.09	Reactor water temperature		2.9	2.9	2.9	2.9

Facility: CPS Printed: 08/11/2006

System Number: 205000

	System Name: Shutdown Cooling System (RHR Shutdown Cooling Mode)	CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between SHUTDOWN COOLING SYSTEM (RHR SHUTDOWN COOLING MODE) and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Reactor pressure		3.6	3.6	3.6	3.6
K1.02	Reactor water level		3.6	3.6	3.6	3.6
K1.03	Recirculation loop temperature		3.4	3.5	3.4	3.5
K1.04	Fuel pool cooling assist: Plant-Specific		2.7	2.7	2.7	2.7
K1.05	Component cooling water systems		3.1	3.1	3.1	3.1
K1.06	A.C. electrical power		3.2	3.3	3.2	3.3
K1.07	D.C. electrical power		2.4	2.6	2.4	2.6
K1.08	LPCI		3.9	3.9	3.9	3.9
K1.09	~~Auxiliary steam supply: Plant-Specific~~		2.2*	2.2*	2.2	2.2
K1.10	RWCU		2.3*	2.3*	2.3	2.3
K1.11	~~Nitrogen: Plant-Specific~~		1.8*	1.8*	1.8	1.8
K1.12	~~Isolation condenser: Plant-Specific~~		2.4*	2.4*	2.4	2.4
K1.13	Floor drain system: Plant-Specific		1.7*	1.7*	1.7	1.7
K1.14	Reactor temperatures (moderator, vessel, flange)		3.6	3.6	3.6	3.6
K1.15	*RHR service water:* Plant-Specific		3.5	3.6	3.5	3.6
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Pump motors		3.1*	3.1*	3.1	3.1
K2.02	Motor operated valves		2.5*	2.7*	2.5	2.7
K3	**Knowledge of the effect that a loss or malfunction of the SHUTDOWN COOLING SYSTEM/MODE will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor pressure		3.3	3.3	3.3	3.3
K3.02	Reactor water level: Plant-Specific		3.2	3.3	3.2	3.3
K3.03	Reactor temperatures (moderator, vessel, flange)		3.8	3.9	3.8	3.9
K3.04	Recirculation loop temperatures		3.7	3.7	3.7	3.7
K3.05	Fuel pool cooling assist: Plant-Specific		2.6	2.7	2.6	2.7
K4	**Knowledge of SHUTDOWN COOLING SYSTEM/MODE design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	High temperature isolation: Plant-Specific		3.4	3.4	3.4	3.4
K4.02	High pressure isolation: Plant-Specific		3.7	3.8	3.7	3.8
K4.03	Low reactor water level: Plant-Specific		3.8	3.8	3.8	3.8
K4.04	Adequate pump NPSH		2.6	2.6	2.6	2.6
K4.05	Reactor cooldown rate		3.6	3.7	3.6	3.7
K4.06	Motor cooling: Plant-Specific		2.3*	2.3	2.3	2.3
K4.07	Pump minimum flow		2.7	2.8	2.7	2.8

Facility: CPS Printed: 08/11/2006

System Number: 205000

System Name: Shutdown Cooling System (RHR Shutdown Cooling Mode)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Im RO	Facility Im S.
K5	**Knowledge of the operational implications of the following concepts as they apply to SHUTDOWN COOLING SYSTEM/MODE:**	**(41.5 / 45.3)**				
K5.01	NPSH		2.2*	2.4	2.2	2.4
K5.02	Valve operation		2.8	2.9	2.8	2.9
K5.03	Heat removal mechanisms		2.8	3.1	2.8	3.1
K5.04	System venting		2.4	2.4	2.4	2.4
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the SHUTDOWN COOLING SYSTEM/MODE:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical power		3.3	3.4	3.3	3.4
K6.02	D.C. electrical power		2.7	2.9	2.7	2.9
K6.03	Recirculation system		3.1	3.2	3.1	3.2
K6.04	Reactor water level		3.6	3.6	3.6	3.6
K6.05	Component cooling water systems		3.2	3.3	3.2	3.3
K6.06	Auxiliary steam supply: Plant-Specific		2.0*	2.0*	2.0	2.0
K6.07	~~Nitrogen: Plant-Specific~~		1.8*	1.8*	1.8	1.8
K6.08	RHR service water: Plant-Specific		3.5	3.7	3.5	3.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the SHUTDOWN COOLING SYSTEM/MODE controls including:**	**(41.5 / 45.5)**				
A1.01	Heat exchanger cooling flow		3.3	3.2	3.3	3.2
A1.02	SDC/RHR pump flow		3.3	3.2	3.3	3.2
A1.03	Recirculation loop temperatures		3.3	3.3	3.3	3.3
A1.04	SDC/RHR pump suction pressure		2.7	2.7	2.7	2.7
A1.05	Reactor water level		3.4	3.4	3.4	3.4
A1.06	Reactor temperatures (moderator, vessel, flange)		3.7	3.7	3.7	3.7
A1.07	Motor amps: Plant--Specific		2.2*	2.1*	2.2	2.1
A1.08	Heat exchanger temperatures		3.1	2.9	3.1	2.9
A1.09	SDC/RHR pump/system discharge pressure		2.8	2.8	2.8	2.8
A1.10	Throttle valve position		3.0	2.9	3.0	2.9

Facility: CPS Printed: 08/11/2006

System Number: 205000

System Name: Shutdown Cooling System (RHR Shutdown Cooling Mode)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the SHUTDOWN COOLING SYSTEM/MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal *conditions or operations:***	**(41.5 / 45.6)**				
A2.01	Recirculation loop high temperature: Plant-Specific		3.1	3.3	3.1	3.3
A2.02	Low shutdown cooling suction pressure: Plant-Specific		2.6	2.7	2.6	2.7
A2.03	A.C. failure		3.2	3.2	3.2	3.2
A2.04	D.C. failure		2.5	2.6	2.5	2.6
A2.05	System isolation		3.5	3.7	3.5	3.7
A2.06	SDC/RHR pump trips		3.4	3.5	3.4	3.5
A2.07	Loss of motor cooling: Plant-Specific		2.7	2.7	2.7	2.7
A2.08	Loss of heat exchanger cooling		3.3	3.5	3.3	3.5
A2.09	Reactor low water level		3.6	3.8	3.6	3.8
A2.10	Valve operation		2.9	2.9	2.9	2.9
A2.11	~~Recirculation pump trips: Plant-Specific~~		2.5	2.7	2.5	2.7
A2.12	Inadequate system flow		2.9	3.0	2.9	3.0
A3	**Ability to monitor automatic *operations* of the SHUTDOWN COOLING SYSTEM/MODE including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		3.2	3.1	3.2	3.1
A3.02	Pump trips		3.2	3.2	3.2	3.2
A3.03	Lights and alarms		3.5	3.3	3.5	3.3
A4	**Ability to manually operate and/or monitor *in the control* room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	SDC/RHR pumps		3.7	3.7	3.7	3.7
A4.02	SDC/RHR suction valves		3.6	3.5	3.6	3.5
A4.03	SDC/RHR discharge valves		3.6	3.5	3.6	3.5
A4.04	Heat exchanger cooling water valves		3.4	3.3	3.4	3.3
A4.05	Minimum flow valves		3.2	3.2	3.2	3.2
A4.06	Reactor water level		3.8	3.7	3.8	3.7
A4.07	Reactor temperatures (moderator, vessel, flange)		3.7	3.7	3.7	3.7
A4.08	Reactor power: Plant-Specific		3.1	3.2	3.1	3.2
A4.09	System flow		3.1	3.1	3.1	3.1
A4.10	System pressures		2.9	3.0	2.9	3.0
A4.11	Heat exchanger cooling flow		3.2	3.2	3.2	3.2
A4.12	Recirculation loop temperatures		3.4	3.4	3.4	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 206000

System Name: High Pressure Coolant Injection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Reactor vessel: BWR-2, 3, 4~~		3.8	3.9	3.8	3.9
K1.02	~~Reactor water level: BWR-2, 3, 4~~		4.0	4.1*	4.0	4.1
K1.03	~~Reactor pressure: BWR-2, 3, 4~~		3.8	3.8	3.8	3.8
K1.04	~~Reactor feedwater system: BWR-2, 3, 4~~		3.6	3.6	3.6	3.6
K1.05	~~Condensate storage system: BWR-2, 3, 4~~		3.7	3.7	3.7	3.7
K1.06	~~Suppression chamber: BWR-2, 3, 4~~		3.7	3.7	3.7	3.7
K1.07	~~D.C. power: BWR-2, 3, 4~~		3.7*	3.8*	3.7	3.8
K1.08	~~A.C. power: BWR-2, 3, 4~~		3.0*	3.1*	3.0	3.1
K1.09	~~ECCS keep fill system: BWR-2, 3, 4(P-Spec)~~		4.0*	4.0	4.0	4.0
K1.10	~~Condensate storage and transfer system: BWR-2, 3, 4~~		3.4	3.4	3.4	3.4
K1.11	~~PCIS: BWR-2, 3, 4~~		3.5	3.5	3.5	3.5
K1.12	~~Nuclear boiler instrumentation: BWR-2, 3, 4~~		3.4	3.4	3.4	3.4
K1.13	~~Main condenser: BWR-2, 3, 4(P-Spec)~~		2.8*	2.9*	2.8	2.9
K1.14	~~SBGT: BWR-2, 3, 4(P-Spec)~~		2.9	3.1	2.9	3.1
K1.15	~~Plant air systems: BWR-2, 3, 4~~		2.3*	2.3*	2.3	2.3
K1.16	~~Containment/Torus pressure: BWR-2, 3, 4~~		3.5	3.5	3.5	3.5
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~System valves: BWR-2, 3, 4~~		3.2*	3.3*	3.2	3.3
K2.02	~~System pumps: BWR-2, 3, 4~~		2.8*	3.1*	2.8	3.1
K2.03	~~Initiation logic: BWR-2, 3, 4~~		2.8*	2.9*	2.8	2.9
K2.04	~~Turbine control circuits: BWR-2, 3, 4~~		2.5*	2.7*	2.5	2.7
K3	**Knowledge of the effect that a loss or malfunction of the HIGH PRESSURE COOLANT INJECTION SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	~~Reactor water level control: BWR-2, 3, 4~~		4.0	4.0	4.0	4.0
K3.02	~~Reactor pressure control: BWR-2, 3, 4~~		3.8*	3.8*	3.8	3.8
K3.03	~~Suppression pool level control: BWR-2, 3, 4~~		3.4*	3.5*	3.4	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 206000

System Name: High Pressure Coolant Injection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
K4	**Knowledge of HIGH PRESSURE COOLANT INJECTION SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Turbine trips: BWR-2, 3, 4~~		3.8	3.9	3.8	3.9
K4.02	~~System isolation: BWR-2, 3, 4~~		3.9	4.0	3.9	4.0
K4.03	~~Resetting turbine trips: BWR-2, 3, 4~~		4.2	4.1	4.2	4.1
K4.04	~~Resetting system isolations: BWR-2, 3, 4~~		4.0	3.9	4.0	3.9
K4.05	~~Preventing water hammer in turbine exhaust line (procedural control): BWR-2, 3, 4~~		3.1	3.4	3.1	3.4
K4.06	~~Preventing water hammer in pump discharge line (procedural control): BWR-2, 3, 4~~		3.2	3.4	3.2	3.4
K4.07	~~Automatic system initiation: BWR-2, 3, 4~~		4.3*	4.3	4.3	4.3
K4.08	~~Manual system initiation: BWR-2, 3, 4~~		4.2*	4.3	4.2	4.3
K4.09	~~Automatic flow control: BWR-2, 3, 4~~		3.8	3.9	3.8	3.9
K4.10	~~Surveillance for all operable components: BWR-2, 3, 4~~		3.7	3.8	3.7	3.8
K4.11	~~Turbine speed control: BWR-2, 3, 4~~		3.4	3.5	3.4	3.5
K4.12	~~Condensation of shaft sealing steam: BWR-2, 3, 4~~		2.9	3.0	2.9	3.0
K4.13	~~Turbine and pump lubrication: BWR-2, 3, 4~~		3.0	3.1	3.0	3.1
K4.14	~~Control oil to turbine speed controls: BWR-2, 3, 4~~		3.4	3.4	3.4	3.4
K4.15	~~Low speed turning of the turbine rotor: BWR-2, 3, 4(P-Spec)~~		3.2	3.2	3.2	3.2
K4.16	~~Minimizing fission product concentration in the condensate storage tank (valve closures on system initiation): BWR-2, 3, 4(P-Spec)~~		3.1	3.3	3.1	3.3
K4.17	~~Protection against draining the condensate storage tank to the suppression pool: BWR-2, 3, 4~~		3.4	3.4	3.4	3.4
K4.18	~~Pump minimum flow: BWR-2, 3, 4~~		3.2	3.3	3.2	3.3
K4.19	~~Automatic transfer of HPCI pump suction: BWR-2, 3, 4~~		3.7	3.8	3.7	3.8
K5	**Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM:**	**(41.5 / 45.3)**				
K5.01	~~Turbine operation: BWR-2, 3, 4~~		3.3	3.4	3.3	3.4
K5.02	~~Turbine shaft sealing: BWR-2, 3, 4~~		2.8	2.9	2.8	2.9
K5.03	~~GEMAC controllers: BWR-2, 3, 4(P-Spec)~~		3.1	3.1	3.1	3.1
K5.04	~~Indications of pump cavitation: BWR-2, 3, 4~~		2.6*	2.7*	2.6	2.7
K5.05	~~Turbine speed control: BWR-2, 3, 4~~		3.3	3.3	3.3	3.3
K5.06	~~Turbine speed measurement: BWR-2, 3, 4~~		2.6*	2.6	2.6	2.6
K5.07	~~System venting: BWR-2, 3, 4~~		2.8	2.8	2.8	2.8
K5.08	~~Vacuum breaker operation: BWR-2, 3, 4~~		3.0	3.2	3.0	3.2
K5.09	~~Testable check valve operation: BWR-2, 3, 4~~		2.7*	2.8	2.7	2.8
K5.10	~~Assist core cooling: BWR-2, 3, 4~~		3.5	3.5	3.5	3.5

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 206000

System Name: High Pressure Coolant Injection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI):**	**(41.7 / 45.7)**				
K6.01	~~Plant air systems: BWR-2, 3, 4~~		2.3*	2.3*	2.3	2.3
K6.02	~~D.C. power: BWR-2, 3, 4~~		3.3	3.7*	3.3	3.7
K6.03	~~A.C. power: BWR-2, 3, 4~~		2.9	3.1*	2.9	3.1
K6.04	~~Condensate storage tank level: BWR-2, 3, 4~~		3.5	3.7	3.5	3.7
K6.05	~~Suppression pool level: BWR-2, 3, 4~~		3.5	3.7	3.5	3.7
K6.06	~~SBGTS: BWR-2, 3, 4(P-Spec)~~		3.1	3.2	3.1	3.2
K6.07	~~ECCS keep fill system: BWR-2, 3, 4(P-Spec)~~		3.4	3.4	3.4	3.4
K6.08	~~Reactor pressure: BWR-2, 3, 4~~		3.8	3.8	3.8	3.8
K6.09	~~Condensate storage and transfer system: BWR-2, 3, 4~~		3.5	3.5	3.5	3.5
K6.10	~~PCIS: BWR-2, 3, 4~~		3.8	4.0	3.8	4.0
K6.11	~~Nuclear boiler instrumentation: BWR-2, 3, 4~~		3.6	3.7	3.6	3.7
K6.12	~~Reactor water level: BWR-2, 3, 4~~		4.2*	4.3*	4.2	4.3
A1	**Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) controls including:**	**(41.5 / 45.5)**				
A1.01	~~Reactor water level: BWR-2, 3, 4~~		4.3*	4.4*	4.3	4.4
A1.02	~~Reactor pressure: BWR-2, 3, 4~~		4.2*	4.2	4.2	4.2
A1.03	~~Condensate storage tank level: BWR-2, 3, 4~~		3.5	3.6	3.5	3.6
A1.04	~~Suppression pool level: BWR-2, 3, 4~~		3.7	3.8	3.7	3.8
A1.05	~~Suppression pool temperature: BWR-2, 3, 4~~		4.1	4.2	4.1	4.2
A1.06	~~System flow: BWR-2, 3, 4~~		3.8	3.7	3.8	3.7
A1.07	~~System discharge pressure: BWR-2, 3, 4~~		3.7	3.6	3.7	3.6
A1.08	~~System lineup: BWR-2, 3, 4~~		4.1*	4.0	4.1	4.0
A1.09	~~Turbine speed: BWR-2, 3, 4~~		3.5	3.4	3.5	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 206000

System Name: High Pressure Coolant Injection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
A2	**Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Turbine trips: BWR-2, 3, 4~~		4.0	4.0	4.0	4.0
A2.02	~~Valve closures: BWR-2, 3, 4~~		3.5	3.5	3.5	3.5
A2.03	~~Valve openings: BWR-2, 3, 4~~		3.5	3.5	3.5	3.5
A2.04	~~A.C. failures: BWR-2, 3, 4~~		2.7*	3.0*	2.7	3.0
A2.05	~~D.C. failures: BWR-2, 3, 4~~		3.5	3.8*	3.5	3.8
A2.06	~~Inadequate system flow: BWR-2, 3, 4~~		3.3	3.5	3.3	3.5
A2.07	~~Low suppression pool level: BWR-2, 3, 4~~		3.4	3.6	3.4	3.6
A2.08	~~†High suppression pool temperature: BWR-2, 3, 4~~		3.9	4.2	3.9	4.2
A2.09	~~†Low condensate storage tank level: BWR-2, 3, 4~~		3.5	3.7	3.5	3.7
A2.10	~~System isolation: BWR-2, 3, 4~~		4.0	4.1	4.0	4.1
A2.11	~~Low reactor water level: BWR-2, 3, 4~~		4.1	4.2	4.1	4.2
A2.12	~~Loss of room cooling: BWR-2, 3, 4~~		3.4	3.5	3.4	3.5
A2.13	~~Loss of applicable plant air systems: BWR-2, 3, 4~~		2.4*	2.4*	2.4	2.4
A2.14	~~Flow controller failure: BWR-2, 3, 4~~		3.3	3.4	3.3	3.4
A2.15	~~Loss of control oil pressure: BWR-2, 3, 4~~		3.4	3.5	3.4	3.5
A2.16	~~High drywell pressure: BWR-2, 3, 4~~		4.0	4.1	4.0	4.1
A2.17	~~HPCI inadvertent initiation: BWR-2, 3, 4~~		3.9	4.3*	3.9	4.3
A3	**Ability to monitor automatic operations of the HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) including:**	**(41.7 / 45.7)**				
A3.01	~~Turbine speed: BWR-2, 3, 4~~		3.6	3.5	3.6	3.5
A3.02	~~System Flow: BWR-2, 3, 4~~		3.8	3.8	3.8	3.8
A3.03	~~System lineup: BWR-2, 3, 4~~		3.9	3.8	3.9	3.8
A3.04	~~Reactor pressure: BWR-2, 3, 4~~		4.2*	4.1	4.2	4.1
A3.05	~~Reactor water level: BWR-2, 3, 4~~		4.3*	4.3*	4.3	4.3
A3.06	~~System discharge pressure: BWR-2, 3, 4~~		3.8	3.8	3.8	3.8
A3.07	~~Lights and alarms: BWR-2, 3, 4~~		3.9	3.8	3.9	3.8
A3.08	~~Condensate storage tank level: BWR-2, 3, 4~~		3.7	3.6	3.7	3.6
A3.09	~~Response to system isolation: BWR-2, 3, 4~~		4.2*	4.1	4.2	4.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 206000

System Name: High Pressure Coolant Injection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	~~Turbine speed controls: BWR-2, 3, 4~~		3.8	3.7	3.8	3.7
A4.02	~~Flow controller: BWR-2, 3, 4~~		4.0*	3.8	4.0	3.8
A4.03	~~Turbine temperatures: BWR-2, 3, 4~~		3.1	3.0	3.1	3.0
A4.04	~~Major system valves: BWR-2, 3, 4~~		3.7	3.7	3.7	3.7
A4.05	~~Reactor water level: BWR-2, 3, 4~~		4.4*	4.4*	4.4	4.4
A4.06	~~Reactor pressure: BWR-2, 3, 4~~		4.3*	4.3*	4.3	4.3
A4.07	~~Condensate storage tank level: BWR-2, 3, 4~~		3.5	3.5	3.5	3.5
A4.08	~~Suppression pool temperature: BWR-2, 3, 4~~		4.1*	4.1	4.1	4.1
A4.09	~~Suppression pool level: BWR-2, 3, 4~~		3.8	3.7	3.8	3.7
A4.10	~~System pumps: BWR-2, 3, 4~~		3.7	3.5	3.7	3.5
A4.11	~~Turning gear: BWR-2, 3, 4(P-Spec)~~		3.0	3.0	3.0	3.0
A4.12	~~Turbine trip controls: BWR-2, 3, 4~~		4.0	3.9	4.0	3.9
A4.13	~~Turbine reset control: BWR-2, 3, 4~~		4.1*	4.0	4.1	4.0
A4.14	~~System auto start control: BWR-2, 3, 4(P-Spec)~~		4.2*	4.1	4.2	4.1

KA Catalog

Facility: CPS *Printed:* 08/11/2006

System Number: 207000

System Name: Isolation (Emergency) Condenser

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between ISOLATION (EMERGENCY) CONDENSER and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Reactor vessel: BWR-2, 3~~		3.8	4.0	3.8	4.0
K1.02	~~Reactor pressure: BWR-2, 3~~		4.0	4.2	4.0	4.2
K1.03	~~Reactor water level: BWR-2, 3~~		3.7	3.8	3.7	3.8
K1.04	~~Condensate transfer system: BWR-2, 3~~		3.5	3.8	3.5	3.8
K1.05	~~Demineralized water system: BWR-2, 3(P-Spec)~~		3.4	3.6	3.4	3.6
K1.06	~~Fire protection/service water: BWR-2, 3~~		3.3*	3.7	3.3	3.7
K1.07	~~LPCI: BWR-2, 3(P-Spec)~~		3.0*	3.3*	3.0	3.3
K1.08	~~Recirculation system: BWR-2, 3~~		3.0	3.2	3.0	3.2
K1.09	~~Main steam system: BWR-2, 3~~		3.0	3.2	3.0	3.2
K1.10	~~*Plant air systems:* BWR-2, 3~~		3.0*	3.2	3.0	3.2
K1.11	~~Primary containment isolation system: BWR-2, 3~~		3.4	3.6	3.4	3.6
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~Motor operated valves: BWR-2, 3~~		3.6	3.8	3.6	3.8
K2.02	~~Initiation logic: BWR-2, 3~~		3.5	3.7	3.5	3.7
K3	**Knowledge of the effect that a loss or malfunction of the ISOLATION (EMERGENCY) CONDENSER will have on following:**	**(41.7 / 45.4)**				
K3.01	~~Reactor pressure control during conditions in which the reactor vessel is isolated: BWR-2, 3~~		4.2*	4.3*	4.2	4.3
K3.02	~~†Reactor water level (EPG's address the isolation condenser as a water source): BWR-2, 3~~		3.8*	4.0*	3.8	4.0
K4	**Knowledge of ISOLATION (EMERGENCY) CONDENSER design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Isolation of the system in the event of a line break: BWR-2, 3~~		4.3*	4.5*	4.3	4.5
K4.02	~~Automatic initiation: BWR-2, 3~~		4.2*	4.2	4.2	4.2
K4.03	~~Filling of the system: BWR-2, 3~~		3.3	3.5	3.3	3.5
K4.04	~~Steam and condensate flow indication: BWR-2, 3~~		3.2	3.4	3.2	3.4
K4.05	~~Detection of a tube bundle leak: BWR-2, 3~~		4.0*	4.2	4.0	4.2
K4.06	~~Throttling of system flow: BWR-2, 3~~		3.8*	4.0	3.8	4.0
K4.07	~~Manual operation of the system: BWR-2, 3~~		3.8*	4.0	3.8	4.0
K4.08	~~Protection against incomplete steam condensation (condensate outlet valve does not fully open): BWR-2,3,(P-Spec)~~		3.4	3.6	3.4	3.6

Facility: CPS Printed: 08/11/2006

System Number: 207000

System Name: Isolation (Emergency) Condenser

			CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir
K5	**Knowledge of the operational implications of the following concepts as they apply to ISOLATION (EMERGENCY) CONDENSER:**	**(41.5 / 45.3)**					
K5.01	~~Flow measurement across an elbow using differential pressure: BWR-2, 3~~			2.6*	3.0*	2.6	3.0
K5.02	~~Heat exchanger operation: BWR-2, 3~~			3.0	3.3	3.0	3.3
K5.03	~~Heat transfer: BWR-2, 3~~			2.7	3.0	2.7	3.0
K5.04	~~Latent heat of vaporization: BWR-2, 3~~			2.3*	2.7*	2.3	2.7
K5.05	~~Saturated steam: BWR-2, 3~~			2.5*	2.7	2.5	2.7
K5.06	~~Saturated liquid: BWR-2, 3~~			2.5*	2.7	2.5	2.7
K5.07	~~Temperature sensing: BWR-2, 3~~			2.7*	2.8	2.7	2.8
K5.08	~~Level indicator operation: BWR-2, 3~~			2.8*	3.0	2.8	3.0
K5.09	~~Cooldown rate: BWR-2, 3~~			3.7	4.0	3.7	4.0
K5.10	~~System venting: BWR-2, 3~~			3.0	3.2	3.0	3.2
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the ISOLATION (EMERGENCY) CONDENSER:**	**(41.7 / 45.7)**					
K6.01	~~Demineralized water system: BWR-2, 3(P-Spec)~~			3.3	3.7	3.3	3.7
K6.02	~~Fire protection/service water system: BWR-2, 3~~			3.5	3.7	3.5	3.7
K6.03	~~Condensate transfer system: BWR-2, 3~~			3.5	3.8	3.5	3.8
K6.04	~~Plant air systems: BWR-2, 3~~			3.2*	3.3*	3.2	3.
K6.05	~~Primary containment isolation system: BWR-2, 3~~			3.6	3.8	3.6	3.
K6.06	~~Recirculation system: BWR-2, 3~~			3.0	3.2	3.0	3.2
K6.07	~~A.C. power: BWR-2, 3~~			3.0*	3.2	3.0	3.2
K6.08	~~D.C. power: BWR-2, 3~~			3.5	3.7	3.5	3.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the ISOLATION (EMERGENCY) CONDENSER controls including:**	**(41.5 / 45.5)**					
A1.01	~~Isolation condenser level: BWR-2, 3~~			3.7	3.8	3.7	3.8
A1.02	~~Shell side water temperature: BWR-2, 3~~			3.2	3.4	3.2	3.4
A1.03	~~Steam flow: BWR-2, 3~~			3.3*	3.0	3.3	3.0
A1.04	~~Condensate flow: BWR-2, 3(P-Spec)~~			3.3	3.5	3.3	3.5
A1.05	~~Reactor pressure: BWR-2, 3~~			4.0*	4.2	4.0	4.2
A1.06	~~Reactor water level: BWR-2, 3~~			3.5	3.7	3.5	3.7
A1.07	~~Vent radiation level: BWR-2, 3~~			3.5	3.7	3.5	3.7
A1.08	~~Cooldown rate: BWR-2, 3~~			3.7	4.0	3.7	4.0
A1.09	~~Valve operations: BWR-2, 3~~			3.7	3.7	3.7	3.7
A1.10	~~Primary side temperature: BWR-2, 3~~			3.2	3.4	3.2	3.4

Facility: CPS Printed: 08/11/2006

System Number: 207000

System Name: Isolation (Emergency) Condenser

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the ISOLATION (EMERGENCY) CONDENSER; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Tube bundle leak: BWR-2, 3~~		4.2*	4.5*	4.2	4.5
A2.02	~~†High vent radiation: BWR-2, 3~~		4.3*	4.7*	4.3	4.7
A2.03	~~PCIS signal resulting in system isolation: BWR-2, 3~~		3.8*	4.0	3.8	4.0
A2.04	~~Inadequate system flow: BWR-2, 3~~		3.8	4.0	3.8	4.0
A2.05	~~Insufficient shell side makeup flow: BWR-2, 3~~		4.0	4.0	4.0	4.0
A2.06	~~Valve openings: BWR-2, 3~~		3.3	3.3	3.3	3.3
A2.07	~~Valve closures: BWR-2, 3~~		3.5	3.5	3.5	3.5
A2.08	~~System initiation: BWR-2, 3~~		3.8*	3.8*	3.8	3.8
A3	**Ability to monitor automatic operations of the ISOLATION (EMERGENCY) CONDENSER *including*:**	**(41.7 / 45.7)**				
A3.01	~~Isolation condenser level: BWR-2, 3~~		3.5	3.7	3.5	3.7
A3.02	~~Reactor pressure: BWR-2, 3~~		3.8	4.0	3.8	4.0
A3.03	~~Reactor water level: BWR-2, 3~~		3.5	3.7	3.5	3.7
A3.04	~~Vent radiation levels: BWR-2, 3~~		4.0*	4.2	4.0	4.2
A3.05	~~System lineup: BWR-2, 3~~		3.6	3.8	3.6	3.8
A3.06	~~Lights and alarms: BWR-2, 3~~		3.3	3.3	3.3	3.3
A3.07	~~Primary and shell side temperatures: BWR-2, 3~~		3.2	3.3	3.2	3.3
A3.08	~~System flow: BWR-2, 3~~		3.2	3.4	3.2	3.4
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	~~Isolation condenser level: BWR-2, 3~~		3.7	3.8	3.7	3.8
A4.02	~~Steam line pressure: BWR-2, 3~~		3.2	3.3	3.2	3.3
A4.03	~~Primary and shell side temperatures: BWR-2, 3~~		3.0	3.2	3.0	3.2
A4.04	~~Vent line radiation levels: BWR-2, 3~~		3.8*	4.0	3.8	4.0
A4.05	~~Major system valves: BWR-2, 3~~		3.5	3.7	3.5	3.7
A4.06	~~Shell side makeup valves: BWR-2, 3~~		3.8*	4.0	3.8	4.0
A4.07	~~Manually initiate the isolation condenser: BWR-2, 3~~		4.2*	4.3*	4.2	4.3

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 209001

System Name: *Low Pressure Core Spray System*

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between LOW PRESSURE CORE SPRAY SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Condensate storage tank: Plant-Specific		3.1	3.1	3.1	3.1
K1.02	Torus/suppression pool		3.4	3.4	3.4	3.4
K1.03	Keep fill system		2.9	3.0	2.9	3.0
K1.04	condensate transfer system		2.3	2.4	2.3	2.4
K1.05	Automatic depressurization system		3.7	3.7	3.7	3.7
K1.06	Plant air systems		2.0*	2.1*	2.0	2.1
K1.07	D.C. electrical power		2.5	2.7	2.5	2.7
K1.08	A.C. electrical power		3.2	3.3	3.2	3.3
K1.09	Nuclear boiler instrumentation		3.2	3.4	3.2	3.4
K1.10	Emergency generator		3.7	3.8	3.7	3.8
K1.11	Drywell coolers: Plant-Specific		2.4	2.6	2.4	2.6
K1.12	ECCS room coolers		2.9	3.1	2.9	3.1
K1.13	Leak detection		2.8	3.0	2.8	3.0
K1.14	Reactor vessel		3.7	3.8	3.7	3.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Pump power		3.0*	3.1*	3.0	3.1
K2.02	Valve power		2.5*	2.7*	2.5	2.7
K2.03	Initiation logic		2.9*	3.1*	2.9	3.1
K3	**Knowledge of the effect that a loss or malfunction of the LOW PRESSURE CORE SPRAY SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor water level		3.8	3.9	3.8	3.9
K3.02	ADS logic		3.8	3.9	3.8	3.9
K3.03	Emergency generators		2.9	3.0	2.9	3.0
K3.04	Component cooling water systems		2.1*	2.2	2.1	2.2
K3.05	Drywell cooling: Plant-Specific		2.3	2.7	2.3	2.7
K4	**Knowledge of LOW PRESSURE CORE SPRAY SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Prevention of overpressurization of core spray piping		3.2	3.4	3.2	3.4
K4.02	Prevents water hammer		3.0	3.2	3.0	3.2
K4.03	Motor cooling		2.4	2.5	2.4	2.5
K4.04	Line break detection		3.0	3.2	3.0	3.2
K4.05	Pump minimum flow		2.6	2.6	2.6	2.6
K4.06	Adequate pump net positive suction head		2.6	2.9	2.6	2.9
K4.07	Pump operability testing		2.8	3.0	2.8	3.0
K4.08	Automatic system initiation		3.8	4.0	3.8	4.0
K4.09	Load sequencing		3.3	3.5	3.3	3.5
K4.10	Testability of all operable components		2.8	2.9	2.8	2.9

Facility: CPS Printed: 08/11/2006

System Number: 209001

System Name: Low Pressure Core Spray System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
K5	**Knowledge of the operational implications of the following concepts as they apply to LOW PRESSURE CORE SPRAY SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Indications of pump cavitation		2.6	2.7	2.6	2.7
K5.02	Differential pressure indication		2.4	2.6	2.4	2.6
K5.03	Testable check valve operation		2.3	2.4	2.3	2.4
K5.04	Heat removal (transfer) mechanisms		2.8	2.9	2.8	2.9
K5.05	System venting		2.5	2.5	2.5	2.5
K5.06	~~Recirculation operation: Plant-Specific(BWR-1)~~		3.7	4.0	3.7	4.0
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the LOW PRESSURE CORE SPRAY SYSTEM:**	**(41.7 / 45.7)**				
K6.01	A.C. power		3.4	3.4	3.4	3.4
K6.02	Emergency generators		3.8	3.9	3.8	3.9
K6.03	Torus/suppression pool water level		3.3	3.4	3.3	3.4
K6.04	D.C. power		2.8	2.9	2.8	2.9
K6.05	ECCS room cooler(s)		2.8	2.9	2.8	2.9
K6.06	Pump motor cooler(s)		2.4	2.4	2.4	2.4
K6.07	Pump seal cooler(s)		2.3	2.3	2.3	2.3
K6.08	Keep fill system		2.9	3.0	2.9	3.0
K6.09	~~Fire protection: BWR-1~~		3.0*	3.3	3.0	3.
K6.10	ECCS room integrity: Plant-Specific		2.3	2.5	2.3	2.
K6.11	ADS		3.6	3.7	3.6	3.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the LOW PRESSURE CORE SPRAY SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Core spray flow		3.4	3.6	3.4	3.6
A1.02	Core spray pressure		3.2	3.4	3.2	3.4
A1.03	Reactor water level		3.8	3.9	3.8	3.9
A1.04	Reactor pressure		3.7	3.7	3.7	3.7
A1.05	Torus/suppression pool water level		3.5	3.6	3.5	3.6
A1.06	Motor amps		2.3*	2.4	2.3	2.4
A1.07	Emergency generator loading		3.0	3.1	3.0	3.1
A1.08	System lineup		3.3	3.2	3.3	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: **209001**

System Name: **Low Pressure Core Spray System**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the LOW PRESSURE CORE SPRAY SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Pump trips		3.4	3.4	3.4	3.4
A2.02	Valve closures		3.2	3.2	3.2	3.2
A2.03	A.C. failures		3.4	3.6	3.4	3.6
A2.04	D.C. failures		2.9	3.0	2.9	3.0
A2.05	Core spray line break		3.3	3.6	3.3	3.6
A2.06	Inadequate system flow		3.2	3.2	3.2	3.2
A2.07	Loss of room cooling		2.6	2.8	2.6	2.8
A2.08	Valve openings		3.1	3.1	3.1	3.1
A2.09	Low suppression pool level		3.1	3.3	3.1	3.3
A2.10	†High suppression pool temperature		3.1	3.4	3.1	3.4
A2.11	~~‡Loss of fire protection: BWR-1~~		2.8*	3.3	2.8	3.3
A3	**Ability to monitor automatic operations of the LOW PRESSURE CORE SPRAY SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		3.6	3.6	3.6	3.6
A3.02	Pump start		3.8	3.7	3.8	3.7
3.03	System pressure		3.5	3.5	3.5	3.5
3.04	System flow		3.7	3.6	3.7	3.6
A3.05	Reactor water level		3.9	3.9	3.9	3.9
A3.06	Lights and alarms		3.6	3.5	3.6	3.5
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Core spray pump		3.8	3.6	3.8	3.6
A4.02	Suction valves		3.5	3.4	3.5	3.4
A4.03	Injection valves		3.7	3.6	3.7	3.6
A4.04	Minimal flow valves		2.9	2.9	2.9	2.9
A4.05	Manual initiation controls		3.8	3.6	3.8	3.6
A4.06	~~Testable check valves~~		2.4	2.4	2.4	2.4
A4.07	Fill pump		2.7	2.8	2.7	2.8
A4.08	Reactor water level		3.9	3.9	3.9	3.9
A4.09	Suppression pool level		3.6	3.5	3.6	3.5
A4.10	Reactor pressure		3.9	3.8	3.9	3.8
A4.11	System flow		3.7	3.6	3.7	3.6
A4.12	System pressure		3.6	3.5	3.6	3.5
A4.13	Lights and alarms		3.4	3.4	3.4	3.4
A4.14	~~Containment level: BWR-1~~		3.6	3.6	3.6	3.6

KA Catalog

Facility: CPS *Printed:* 08/11/2006

System Number: 209002

System Name: High Pressure Core Spray System (HPCS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between HIGH PRESSURE CORE SPRAY SYSTEM (HPCS) and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Condensate transfer and storage system: BWR-5, 6		3.4	3.4	3.4	3.4
K1.02	Suppression Pool: BWR-5, 6		3.5	3.5	3.5	3.5
K1.03	Water leg (jockey) pump: BWR-5, 6		3.0	3.0	3.0	3.0
K1.04	HPCS diesel generator: BWR-5, 6		3.8	3.8	3.8	3.8
K1.05	Standby liquid control system: Plant-Specific		2.8	2.8	2.8	2.8
K1.06	Suppression pool cleanup system: BWR-5, 6		2.0*	2	2.0	2
K1.07	ECCS room coolers: BWR-5, 6		2.4	2.4	2.4	2.4
K1.08	*Component cooling water systems*: BWR-5, 6		2.4	2.6	2.4	2.6
K1.09	Leak detection: BWR-5, 6		2.5	2.5	2.5	2.5
K1.10	Suppression pool suction strainers: BWR-5, 6		2.1*	2.1	2.1	2.1
K1.11	Adequate core cooling: BWR-5, 6		3.8	4.0	3.8	4.0
K1.12	Reactor vessel: BWR-5, 6		3.4	3.6	3.4	3.6
K1.13	~~Instrument nitrogen: BWR-5, 6~~		2.7	2.7	2.7	2.7
K1.14	Plant air systems: BWR-5, 6		2.6	2.6	2.6	2.6
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Pump electrical power: BWR-5, 6		3.2	3.3	3.2	3.3
K2.02	Valve electrical power: BWR-5, 6		2.8	2.9	2.8	2.9
2.03	Initiation logic: BWR-5, 6		2.8	2.9	2.8	2.9
K3	**Knowledge of the effect that a loss or malfunction of the HIGH PRESSURE CORE SPRAY SYSTEM (HPCS) will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor water level: BWR-5, 6		3.9	3.9	3.9	3.9
K3.02	Standby liquid control system: Plant-Specific		3.3	3.3	3.3	3.3
K3.03	Adequate core cooling: BWR-5, 6		3.9	4.1	3.9	4.1
K4	**Knowledge of HIGH PRESSURE CORE SPRAY SYSTEM (HPCS) design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Prevents water hammer: BWR-5, 6		2.9	3.0	2.9	3.0
K4.02	Prevents over filling reactor vessel: Plant-Specific		3.4	3.5	3.4	3.5
K4.03	Prevents pump over heating: BWR-5, 6		2.3*	2.4	2.3	2.4
K4.04	*Testable check valve operation*: BWR-5, 6		2.3*	2.3*	2.3	2.3
K4.05	Motor operated valve operation: BWR-5, 6		2.4	2.4	2.4	2.4
K4.06	Centrifugal pump operation: BWR-5, 6		2.3*	2.4*	2.3	2.4
K4.07	Override of reactor water level interlock: Plant-Specific		3.5	3.0	3.5	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 209002

System Name: High Pressure Core Spray System (HPCS)

			CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
K5	**Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE CORE SPRAY SYSTEM (HPCS):**	**(41.5 / 45.3)**					
K5.01	Indications of pump cavitation: BWR-5, 6			2.5	2.8	2.5	2.8
K5.02	Heat removal (transfer) mechanism: BWR-5, 6			2.6	2.7	2.6	2.7
K5.03	System venting: BWR-5, 6			2.4	2.4	2.4	2.4
K5.04	Adequate core cooling: BWR-5, 6			3.8	4.0	3.8	4.0
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the HIGH PRESSURE CORE SPRAY SYSTEM (HPCS):**	**(41.7 / 45.7)**					
K6.01	Electrical power: BWR-5, 6			3.6	3.6	3.6	3.6
K6.02	Condensate storage tank water level: BWR-5, 6			3.4	3.4	3.4	3.4
K6.03	Component cooling water systems: BWR-5, 6			2.5	2.6	2.5	2.6
K6.04	Suppression pool suction strainer: BWR-5, 6			2.5	2.5	2.5	2.5
A1	**Ability to predict and/or monitor changes in parameters associated with operating the HIGH PRESSURE CORE SPRAY SYSTEM (HPCS) controls including:**	**(41.5 / 45.5)**					
A1.01	HPCS flow: BWR-5, 6			3.6	3.7	3.6	3.7
A1.02	HPCS pressure: BWR-5, 6			3.4	3.6	3.4	3.6
A1.03	Reactor water level: BWR-5, 6			3.7	3.7	3.7	3.
A1.04	Reactor pressure: BWR-5, 6			3.3	3.3	3.3	3.3
A1.05	Suppression pool water level: BWR-5, 6			3.3	3.4	3.3	3.4
A1.06	Motor amps: BWR-5, 6			1.9*	2.3*	1.9	2.3
A1.07	Diesel loading: BWR-5, 6			2.5*	2.8	2.5	2.8
A1.08	System lineup: BWR-5, 6			3.1	3.3	3.1	3.3
A1.09	Condensate storage tank level: BWR-5, 6			2.6	2.8	2.6	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 209002

System Name: High Pressure Core Spray System (HPCS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the HIGH PRESSURE CORE SPRAY SYSTEM (HPCS); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	System initiation: BWR-5, 6		3.8	3.8	3.8	3.8
A2.02	Pump trips: BWR-5, 6		3.6	3.7	3.6	3.7
A2.03	Valve closures: BWR-5, 6		3.2	3.4	3.2	3.4
A2.04	A.C. electrical failure: BWR-5, 6		3.1	3.2	3.1	3.2
A2.05	D.C. electrical failure: BWR-5, 6		2.8	2.9	2.8	2.9
A2.06	Core spray line break: BWR-5, 6		3.4	3.6	3.4	3.6
A2.07	Pump seal failure: BWR-5, 6		2.6	3.0	2.6	3.0
A2.08	Inadequate system flow: BWR-5, 6		3.1	3.2	3.1	3.2
A2.09	Loss of room cooling: BWR-5, 6		2.4	2.6	2.4	2.6
A2.10	Valve openings: BWR-5, 6		2.7	3.0	2.7	3.0
A2.11	Low suppression pool level: BWR-5, 6		3.3	3.5	3.3	3.5
A2.12	High suppression pool leve: BWR-5, 6		3.3	3.5	3.3	3.5
A2.13	Low condensate storage tank level BWR-5, 6		3.4	3.5	3.4	3.5
A2.14	High suppression pool temperature: BWR-5, 6		3.0	3.3	3.0	3.3
A2.15	Clogged suppression pool suction strainers: BWR-5, 6		2.4	2.6	2.4	2.6
A3	**Ability to monitor automatic operations of the HIGH PRESSURE CORE SPRAY SYSTEM (HPCS) including:**	**(41.7 / 45.7)**				
A3.01	Valve operation: BWR-5, 6		3.3	3.3	3.3	3.3
A3.02	Pump start: BWR-5, 6		3.8	3.8	3.8	3.8
A3.03	System pressure: BWR-5, 6		3.6	3.6	3.6	3.6
A3.04	System flow: BWR-5, 6		3.7	3.7	3.7	3.7
A3.05	Reactor water level BWR-5, 6		3.7	3.7	3.7	3.7
A3.06	Lights and alarms: BWR-5, 6		2.8	2.8	2.8	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 209002

System Name: High Pressure Core Spray System (HPCS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	HPCS pump: BWR-5, 6		3.7	3.7	3.7	3.7
A4.02	Suction valves: BWR-5, 6		3.6	3.6	3.6	3.6
A4.03	Injection valve: BWR-5, 6		3.8	3.8	3.8	3.8
A4.04	Minimum flow valve: BWR-5, 6		3.1	3.1	3.1	3.1
A4.05	Manual Initiation controls: BWR-5, 6		3.8	3.8	3.8	3.8
A4.06	~~Testable check valve: BWR-5, 6~~		2.6	2.6	2.6	2.6
A4.07	Line fill pump: BWR-5, 6		2.8	2.8	2.8	2.8
A4.08	Reactor water level: BWR-5, 6		3.6	3.7	3.6	3.7
A4.09	Suppression pool level: BWR-5, 6		3.4	3.5	3.4	3.5
A4.10	Reactor pressure: BWR-5, 6		3.3	3.3	3.3	3.3
A4.11	System flow: BWR-5, 6		3.8	3.8	3.8	3.8
A4.12	System pressure: BWR-5, 6		3.4	3.4	3.4	3.4
A4.13	Lights and alarms: BWR-5, 6		2.6	2.6	2.6	2.6
A4.14	Test return valve: BWR-5, 6		3.0	3.0	3.0	3.0
A4.15	Initiation reset: BWR-5, 6		3.6	3.6	3.6	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: **211000**

System Name: **Standby Liquid Control System**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between STANDBY LIQUID CONTROL SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Core spray line break detection: Plant-Specific		3.0*	3.3*	3.0	3.3
K1.02	Core plate differential pressure indication		2.7	2.7	2.7	2.7
K1.03	Plant air systems: Plant-Specific		2.5	2.6	2.5	2.6
K1.04	Demineralized water/ condensate storage system		2.2*	2.3*	2.2	2.3
K1.05	RWCU		3.4	3.6	3.4	3.6
K1.06	Reactor vessel		3.7	3.7	3.7	3.7
K1.07	Jet pump differential pressure indication: Plant-Specific		2.6	2.6	2.6	2.6
K1.08	CRD drive and cooling water differential pressure indication		2.3*	2.4*	2.3	2.4
K1.09	Core spray system: Plant-Specific		3.2*	3.4*	3.2	3.4
K1.10	~~HPCI: Plant-Specific~~		2.8*	3.0*	2.8	3.0
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	SBLC pumps		2.9*	3.1*	2.9	3.1
K2.02	Explosive valves		3.1*	3.2*	3.1	3.2
K2.03	Heater power		2.2*	2.4*	2.2	2.4
K3	**Knowledge of the effect that a loss or malfunction of the STANDBY LIQUID CONTROL SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	†Ability to shutdown the reactor in certain conditions		4.3*	4.4*	4.3	4.4
K3.02	Core spray line break detection system: Plant-Specific		3.0*	3.2*	3.0	3.2
K3.03	Core plate differential pressure indication		2.6*	2.7*	2.6	2.7
K3.04	Jet pump differential pressure indication: BWR-3,4,5,6		2.4*	2.5*	2.4	2.5
K3.05	CRD drive and cooling water differential pressure indication		2.3*	2.3*	2.3	2.3
K4	**Knowledge of STANDBY LIQUID CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Zero leakage to the reactor (squib valves)		3.1	3.2	3.1	3.2
K4.02	Component and system testing		3.0	3.2	3.0	3.2
K4.03	Keeping sodium pentaborate in solution		3.8	3.9	3.8	3.9
K4.04	Indication of fault in explosive valve firing circuits		3.8	3.9	3.8	3.9
K4.05	Dispersal of boron upon injection into the vessel		3.4	3.6	3.4	3.6
K4.06	Core plate differential pressure indication		2.6	2.7	2.6	2.7
K4.07	RWCU isolation		3.8*	3.9*	3.8	3.9
K4.08	System initiation upon operation of SBLC control switch		4.2*	4.2*	4.2	4.2
K4.09	~~Dampening of positive displacement pump discharge oscillations (accumulators): Plant-Specific~~		2.5*	2.5*	2.5	2.5
K4.10	Over pressure protection		2.8	3.1	2.8	3.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 211000

System Name: Standby Liquid Control System

			CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
K5	**Knowledge of the operational implications of the following concepts as they apply to STANDBY LIQUID CONTROL SYSTEM:**	**(41.5 / 45.3)**					
K5.01	Effects of the moderator temperature coefficient of reactivity on the boron			2.7	2.9	2.7	2.9
K5.02	Chugging (as it pertains to boron mixing)			2.8	3.0	2.8	3.0
K5.03	Shutdown margin			3.2	3.5	3.2	3.5
K5.04	Explosive valve operation			3.1	3.2	3.1	3.2
K5.05	~~Accumulator operation: Plant-Specific~~			2.5*	2.5	2.5	2.5
K5.06	Tank level measurement			3.0	3.2	3.0	3.2
K5.07	Tank heater operation			2.7	2.9	2.7	2.9
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the STANDBY LIQUID CONTROL SYSTEM:**	**(41.7 / 45.7)**					
K6.01	Plant air systems: Plant-Specific			2.4*	2.4*	2.4	2.4
K6.02	Demineralized water storage system			1.9*	2.0*	1.9	2.0
K6.03	A.C. power			3.2	3.3	3.2	3.3
K6.04	Core spray system: Plant-Specific			2.7*	2.8*	2.7	2.8
K6.05	~~HPCI system: Plant-Specific~~			2.7*	2.7*	2.7	2.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the STANDBY LIQUID CONTROL SYSTEM controls including:**	**(41.5 / 45.5)**					
A1.01	Tank level			3.6	3.7	3.6	3.7
A1.02	Explosive valve indication			3.8	3.9	3.8	3.9
A1.03	Pump discharge pressure			3.6	3.6	3.6	3.6
A1.04	Valve operations			3.6	3.7	3.6	3.7
A1.05	Pump amps: Plant-Specific			2.7*	2.8*	2.7	2.8
A1.06	Flow indication: Plant-Specific			3.8	3.9	3.8	3.9
A1.07	Reactor power			4.3*	4.4*	4.3	4.4
A1.08	RWCU system lineup			3.7	3.8	3.7	3.8
A1.09	SBLC system lineup			4.0*	4.1	4.0	4.1
A1.10	Lights and alarms			3.7	3.7	3.7	3.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 211000

System Name: Standby Liquid Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the *STANDBY LIQUID CONTROL SYSTEM*; *and* (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Pump trip		3.5	3.8*	3.5	3.8
A2.02	Failure of explosive valve to fire		3.6	3.9*	3.6	3.9
A2.03	A.C. power failures		3.2	3.4*	3.2	3.4
A2.04	Inadequate system flow		3.1*	3.4*	3.1	3.4
A2.05	Loss of SBLC tank heaters		3.1	3.4	3.1	3.4
A2.06	Valve openings		3.1	3.3	3.1	3.3
A2.07	Valve closures		2.9	3.2	2.9	3.2
A2.08	Failure to SCRAM		4.1*	4.2*	4.1	4.2
A3	**Ability to monitor automatic operations of the STANDBY LIQUID CONTROL SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Pump discharge pressure: Plant-Specific		3.5	3.5	3.5	3.5
A3.02	Tank level: Plant-Specific		3.9	3.9	3.9	3.9
A3.03	*Explosive valves indicating lights:* Plant-Specific		3.8	3.8	3.8	3.8
A3.04	Reactor power: Plant-Specific		4.3*	4.4*	4.3	4.4
A3.05	Flow indication: Plant-Specific		4.1*	4.2*	4.1	4.2
\3.06	RWCU system isolation: Plant-Specific		4.0*	4.1*	4.0	4.1
A3.07	Lights and alarms: Plant-Specific		3.7	3.6	3.7	3.6
A3.08	~~System initiation: Plant-Specific~~		4.2*	4.2*	4.2	4.2
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Tank level		3.9	3.9	3.9	3.9
A4.02	SBLC control switch		4.2*	4.2*	4.2	4.2
A4.03	Explosive valves firing circuit status		4.1	4.1	4.1	4.1
A4.04	Reactor power		4.5*	4.6*	4.5	4.6
A4.05	Flow indication: Plant-Specific		4.1*	4.0	4.1	4.0
A4.06	RWCU system isolation		3.9*	3.9	3.9	3.9
A4.07	Lights and alarms		3.6	3.6	3.6	3.6
A4.08	System initiation: Plant-Specific		4.2*	4.2*	4.2	4.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 212000

System Name: Reactor Protection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR PROTECTION SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Nuclear instrumentation		3.7	3.9	3.7	3.9
K1.02	Nuclear boiler instrumentation		3.7	3.9	3.7	3.9
K1.03	Recirculation system		3.4	3.6	3.4	3.6
K1.04	A.C. electrical distribution		3.4	3.6	3.4	3.6
K1.05	Process radiation monitoring system		3.3	3.6	3.3	3.6
K1.06	Control rod drive hydraulic system		3.5	3.6	3.5	3.6
K1.07	Relief/safety valves (low-low-set logic): Plant-Specific		3.3	3.5	3.3	3.5
K1.08	Control rod and drive mechanism		3.0	3.1	3.0	3.1
K1.09	Process computer		2.3	2.4	2.3	2.4
K1.10	Main turbine		3.2	3.4	3.2	3.4
K1.11	Condenser vacuum		3.3	3.5	3.3	3.5
K1.12	Reactor/turbine pressure control system: Plant-Specific		3.4	3.6	3.4	3.6
K1.13	Containment pressure		3.5	3.6	3.5	3.6
K1.14	Main steam system		3.6	3.7	3.6	3.7
K1.15	SCRAM air header pressure		3.8	3.9	3.8	3.9
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~RPS motor-generator sets~~		3.2	3.3	3.2	3.3
K2.02	Analog trip system logic cabinets		2.7	2.9	2.7	2.9
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR PROTECTION SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Process radiation monitoring		3.0	3.2	3.0	3.2
K3.02	Primary containment isolation system/nuclear steam supply shut-off: Plant-Specific		3.7	3.9	3.7	3.9
K3.03	Local power range monitoring system: Plant-Specific		3.3	3.4	3.3	3.4
K3.04	Average power range monitoring system: Plant-Specific		3.5	3.6	3.5	3.6
K3.05	RPS logic channels		3.7	3.8	3.7	3.8
K3.06	Scram air header solenoid operated valves		4.0	4.1	4.0	4.1
K3.07	Reactor power (thermal heat flux)		3.8	3.9	3.8	3.9
K3.08	Reactor coolant primary system integrity		3.6	3.8	3.6	3.8
K3.09	The magnitude of heat energy that must be absorbed by the containment during accident/transient conditions		3.2	3.6	3.2	3.6
K3.10	The ability of the core cooling systems to provide adequate core cooling during loss of coolant accidents		3.5	3.9	3.5	3.9
K3.11	Recirculation system		3.0	3.3	3.0	3.3
K3.12	Secondary containment integrity		3.2	3.3	3.2	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 212000

System Name: Reactor Protection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K4	**Knowledge of REACTOR PROTECTION SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	System redundancy and reliability		3.4	3.6	3.4	3.6
K4.02	The prevention of a reactor SCRAM following a single component failure		3.5	3.7	3.5	3.7
K4.03	The prevention of supplying power to a given RPS bus from multiple sources simultaneously		3.0*	3.1	3.0	3.1
K4.04	The prevention of supplying both RPS buses simultaneously from the alternate power source: Plant-Specific		3.1	3.1	3.1	3.1
K4.05	Functional testing of the system while maintaining power operation		3.4	3.6	3.4	3.6
K4.06	~~Select rod insertion: Plant-Specific~~		3.0	3.0	3.0	3.0
K4.07	Manual system activation (trip)		4.1*	4.1	4.1	4.1
K4.08	Complete control rod insertion following SCRAM signal generation		4.2*	4.2	4.2	4.2
K4.09	Control rod insertion following RPS system electrical failure		3.8*	3.9	3.8	3.9
K4.10	Individual rod SCRAM testing		3.3	3.6	3.3	3.6
K4.11	Operation with shorting links removed: Plant-Specific		3.3	3.5	3.3	3.5
K4.12	Bypassing of selected SCRAM signals (manually and automatically): Plant-Specific		3.9	4.1	3.9	4.1
K5	**Knowledge of the operational implications of the following concepts as they apply to REACTOR PROTECTION SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Fuel thermal time constant		2.7	2.9	2.7	2.9
K5.02	Specific logic arrangements		3.3	3.4	3.3	3.4
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR PROTECTION SYSTEM:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical distribution		3.6	3.8	3.6	3.8
K6.02	Nuclear instrumentation		3.7	3.9	3.7	3.9
K6.03	Nuclear boiler instrumentation		3.5	3.7	3.5	3.7
K6.04	D.C. electrical distribution		2.8	3.1	2.8	3.1
K6.05	RPS sensor inputs		3.5	3.8	3.5	3.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 212000

System Name: Reactor Protection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the REACTOR PROTECTION SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	~~RPS motor-generator output voltage~~		2.8*	2.9	2.8	2.9
A1.02	~~RPS motor-generator output amps~~		2.8*	2.9	2.8	2.9
A1.03	~~RPS motor-generator output frequency: Plant-Specific~~		2.4*	2.5	2.4	2.5
A1.04	RPS bus voltage: Plant-Specific		2.8*	3.0	2.8	3.0
A1.05	RPS bus frequency: Plant-Specific		2.6*	2.7	2.6	2.7
A1.06	Reactor power		4.2*	4.2*	4.2	4.2
A1.07	Rod position information		3.4	3.4	3.4	3.4
A1.08	Valve position		3.4	3.4	3.4	3.4
A1.09	~~Individual relay status: Plant-Specific~~		2.7	3.0	2.7	3.0
A1.10	Process computer		2.2*	2.4*	2.2	2.4
A1.11	System status lights and alarms		3.4	3.3	3.4	3.3
A2	**Ability to (a) predict the impacts of the following on the REACTOR PROTECTION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~RPS motor-generator set failure~~		3.7	3.9	3.7	3.9
A2.02	RPS bus power supply failure		3.7	3.9	3.7	3.9
A2.03	Surveillance testing		3.3	3.5	3.3	3.5
A2.04	Nuclear instrument system failure		3.5	3.7	3.5	3.7
A2.05	Nuclear boiler instrument system failure		3.4	3.7	3.4	3.7
A2.06	High reactor power		4.1*	4.2*	4.1	4.2
A2.07	High reactor pressure		4.1*	4.2*	4.1	4.2
A2.08	Low reactor level		4.1*	4.2*	4.1	4.2
A2.09	High containment/drywell pressure		4.1*	4.3*	4.1	4.3
A2.10	Reactor/turbine pressure control system low pressure: Plant-Specific		3.6	3.8	3.6	3.8
A2.11	Main steamline isolation valve closure		4.0	4.1	4.0	4.1
A2.12	Main turbine stop control valve closure		4.0	4.1*	4.0	4.1
A2.13	Low condenser vacuum: Plant-Specific		3.8	3.9*	3.8	3.9
A2.14	High SCRAM instrument volume water level		3.9	4.0	3.9	4.0
A2.15	Load rejection		3.7	3.8	3.7	3.8
A2.16	Changing mode switch position		4.0	4.1	4.0	4.1
A2.17	Main steamline high radiation		4.0	4.2*	4.0	4.2
A2.18	†SCRAM air header low pressure		3.8	3.9	3.8	3.9
A2.19	~~Partial system activation (half-SCRAM)~~		3.8	3.9	3.8	3.9
A2.20	Full system activation (full-SCRAM)		4.1*	4.2*	4.1	4.2
A2.21	~~†Failure of individual relays to reposition: Plant-Specific~~		3.6	3.9	3.6	3.9

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 212000

System Name: Reactor Protection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
A3	**Ability to monitor automatic operations of the REACTOR PROTECTION SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Reactor power		4.4*	4.4*	4.4	4.4
A3.02	~~Individual system relay status: Plant-Specific~~		3.2	3.5	3.2	3.5
A3.03	Rod position		4.2*	4.2*	4.2	4.2
A3.04	System status lights and alarms		3.9*	3.8	3.9	3.8
A3.05	SCRAM instrument volume level		3.9	3.9	3.9	3.9
A3.06	Main turbine trip: Plant-Specific		4.2*	4.2*	4.2	4.2
A3.07	SCRAM air header pressure		3.6	3.6	3.6	3.6
A3.08	Recirculation pump trip		3.7	3.7	3.7	3.7
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Provide manual SCRAM signal(s)		4.6*	4.6*	4.6	4.6
A4.02	Perform system functional test(s)		3.6	3.7	3.6	3.7
A4.03	~~Provide manual select rod insertion: Plant-Specific~~		3.9	3.9	3.9	3.9
A4.04	Bypass SCRAM instrument volume high level SCRAM signal		3.9	3.9	3.9	3.9
A4.05	Reactor power		4.3*	4.3*	4.3	4.3
A4.06	Control rod position		4.2*	4.1*	4.2	4.1
A4.07	System status lights and alarms		4.0*	3.9*	4.0	3.9
A4.08	~~Individual system relay status: Plant-Specific~~		3.4	3.4	3.4	?
A4.09	SCRAM instrument volume level		3.9	3.8	3.9	3.6
A4.10	Main turbine trip: Plant-Specific		4.1	4.0	4.1	4.0
A4.11	SCRAM air header pressure		3.7	3.7	3.7	3.7
A4.12	Close/open SCRAM instrument volume vent and/or drain valves		3.9	3.9	3.9	3.9
A4.13	†Perform individual control rod SCRAM testing		3.4	3.6	3.4	3.6
A4.14	Reset system following system activation		3.8	3.8	3.8	3.8
A4.15	Recirculation pump trip/EOC RPT		3.9	3.8	3.9	3.8
A4.16	Manually activate anticipated transient without SCRAM circuitry/RRCS: Plant-Specific		4.4*	4.4*	4.4	4.4
A4.17	Perform alternate reactivity/ shutdown operations		4.1	4.1	4.1	4.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: **214000**

System Name: **Rod Position Information System**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between ROD POSITION INFORMATION SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~RWM: Plant-Specific~~		3.0	3.2	3.0	3.2
K1.02	~~RSCS: Plant-Specific~~		2.9	3.1	2.9	3.1
K1.03	CRDM		3.0	3.1	3.0	3.1
K1.04	~~RMCS: Plant-Specific~~		3.2	3.2	3.2	3.2
K1.05	Full core display: Plant-Specific		3.3	3.3	3.3	3.3
K1.06	RCIS: Plant-Specific		3.4	3.4	3.4	3.4
K1.07	Process computer		2.7	3.0	2.7	3.0
K1.08	CRIDS/ERIS/SPDS/GDS: Plant-Specific		2.4	2.6	2.4	2.6
K3	**Knowledge of the effect that a loss or malfunction of the ROD POSITION INFORMATION SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	~~RWM: Plant-Specific~~		3.0	3.2	3.0	3.2
K3.02	~~RSCS: Plant-Specific~~		3.0	3.1	3.0	3.1
K3.03	~~RMCS: Plant-Specific~~		3.1	3.2	3.1	3.2
K3.04	RCIS: Plant-Specific		3.3	3.3	3.3	3.3
K3.05	Process computer		2.3	2.6	2.3	2.6
K3.06	CRIDS/ERIS/SPDS/GDS: Plant-Specific		2.3*	2.5	2.3	2.5
K4	**Knowledge of ROD POSITION INFORMATION SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Reed switch locations		3.0	3.1	3.0	3.1
K4.02	Thermocouple		2.5*	2.5	2.5	2.5
K5	**Knowledge of the operational implications of the following concepts as they apply to ROD POSITION INFORMATION SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Reed switches		2.7	2.8	2.7	2.8
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the ROD POSITION INFORMATION SYSTEM:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical power		2.5	2.6	2.5	2.6
K6.02	Position indication probe		2.7	2.7	2.7	2.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 214000

System Name: Rod Position Information System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
A2	**Ability to (a) predict the impacts of the following on the ROD POSITION INFORMATION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Failed reed switches		3.1	3.3	3.1	3.3
A2.02	Reactor scram		3.6	3.7	3.6	3.7
A2.03	Overtravel/in-out		3.6	3.9	3.6	3.9
A3	**Ability to monitor automatic operations of the ROD POSITION INFORMATION SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Full core display		3.4	3.3	3.4	3.3
A3.02	Alarm and indicating lights		3.2	3.1	3.2	3.1
A3.03	Verification of proper functioning/ operability		3.5	3.7	3.5	3.7
A3.04	RCIS: Plant-Specific		3.5	3.8	3.5	3.8
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	RCIS rod action control bypass switches		3.2	3.3	3.2	3.3
A4.02	Control rod position		3.8*	3.8*	3.8	3.8
A4.03	Control rod drive temperature		2.8	2.7	2.8	

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215001

System Name: Traversing In-Core Probe

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between TRAVERSING IN-CORE PROBE and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Local power range monitors: (Not-BWR1)		2.5	2.8	2.5	2.8
K1.02	Process computer: (Not-BWR1)		2.5	3.1	2.5	3.1
K1.03	~~Nitrogen purge system: P-Spec(Not-BWR1)~~		2.1*	2.3	2.1	2.3
K1.04	Plant air systems: P-Spec(Not-BWR1)		1.8*	2.0	1.8	2.0
K1.05	Primary containment isolation system: (Not-BWR1)		3.3	3.4	3.3	3.4
K1.06	D.C. electrical: (Not-BWR1)		1.8*	1.9*	1.8	1.9
K1.07	A.C. electrical: (Not-BWR1)		1.8*	1.9*	1.8	1.9
K1.08	Reactor pressure vessel: (Not-BWR1)		2.5	2.6	2.5	2.6
K1.09	Primary containment: (Not-BWR1)		2.6	2.7	2.6	2.7
K1.10	Area radiation monitoring system: (Not-BWR1)		2.6	2.8	2.6	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~Shear valves: Mark-I&II(Not-BWR1)~~		1.9*	2.1*	1.9	2.1
K3	**Knowledge of the effect that a loss or malfunction of the TRAVERSING IN-CORE PROBE will have on following:**	**(41.7 / 45.4)**				
K3.01	Local power range monitor's calibration: (Not-BWR1)		2.3	2.8	2.3	2.8
K4	**Knowledge of TRAVERSING IN-CORE PROBE design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Primary containment isolation: Mark-I&II(Not-BWR1)~~		3.4	3.5	3.4	3.5
K4.02	Corrosion prevention: (Not-BWR1)		1.9*	2.1*	1.9	2.1
K4.03	Radiation shielding: (Not-BWR1)		2.4	2.7	2.4	2.7
K5	**Knowledge of the operational implications of the following concepts as they apply to TRAVERSING IN-CORE PROBE:**	**(41.5 / 45.3)**				
K5.01	Neutron flux detection: (Not-BWR1)		2.2*	2.5	2.2	2.5
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the TRAVERSING IN-CORE PROBE:**	**(41.7 / 45.7)**				
K6.01	D.C. electrical power: (Not-BWR1)		1.9*	2.1*	1.9	2.1
K6.02	A.C. electrical power: (Not-BWR1)		1.9*	2.1*	1.9	2.1
K6.03	Process computer: BWR-2, 3, 4, 5, 6		2.3	2.7	2.3	2.7
K6.04	~~Primary containment isolation system: Mark-I&II(Not-BWR1)~~		3.1	3.4	3.1	3.4
K6.05	Plant air systems: P-Spec(Not-BWR1)		1.8*	2.1*	1.8	2.1
K6.06	Nitrogen purge: P-Spec(Not-BWR1)		2.0*	2.2	2.0	2.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215001

System Name: Traversing In-Core Probe

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
A1	**Ability to predict and/or monitor changes in parameters associated with operating the TRAVERSING IN-CORE PROBE controls including:**	**(41.5 / 45.5)**				
A1.01	Radiation levels: (Not-BWR1)		2.8	2.9	2.8	2.9
A1.02	Detector position: (Not-BWR1)		2.5	2.4	2.5	2.4
A1.03	~~Valve status: Mark-I&II(Not-BWR1)~~		2.6*	2.8	2.6	2.8
A1.04	Drive speed: (Not-BWR1)		2.0*	2.2*	2.0	2.2
A1.05	Detector output: (Not-BWR1)		2.0*	2.3*	2.0	2.3
A1.06	Radiation alarms: (Not-BWR1)		2.9	2.8	2.9	2.8
A2	**Ability to (a) predict the impacts of the following on the TRAVERSING IN-CORE PROBE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Low reactor water level: Mark-I&II(Not-BWR1)~~		2.7	2.9	2.7	2.9
A2.02	~~High primary containment pressure: Mark-I&II(Not-BWR1)~~		2.9	3.0	2.9	3.0
A2.03	Drive mechanism failure: (Not-BWR1)		2.1*	2.4	2.1	2.4
A2.04	A.C. failure: (Not-BWR1)		1.8*	2.0*	1.8	2.0
A2.05	D.C. failure: (Not-BWR1)		1.8*	2.0*	1.8	2.0
A2.06	~~Valve closures: Mark-I&II(Not-BWR1)~~		2.4*	2.6	2.4	2.6
A2.07	~~†Failure to retract during accident conditions: Mark-I&II(Not-BWR1)~~		3.4	3.7	3.4	
A2.08	~~Failure to retract to shield: (Not-BWR1)~~		2.7*	2.9	2.7	2.9
A3	**Ability to monitor automatic operations of the TRAVERSING IN-CORE PROBE including:**	**(41.7 / 45.7)**				
A3.01	Detector position: P-Spec(Not-BWR1)		2.3*	2.5*	2.3	2.5
A3.02	Detector drive speed: P-Spec(Not-BWR1)		1.9*	2.2*	1.9	2.2
A3.03	~~Valve operation: Not-BWR1~~		2.5*	2.6*	2.5	2.6
A3.04	Indicating lights: P-Spec(Not-BWR1)		2.2*	2.4*	2.2	2.4
A3.05	Detector output: P-Spec(Not-BWR1)		1.9*	2.2*	1.9	2.2
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Detector drive speed: P-Spec(Not-BWR1)		2.0*	2.2*	2.0	2.2
A4.02	Detector position: P-Spec(Not-BWR1)		2.4*	2.6*	2.4	2.6
A4.03	~~Isolation valves: Mark-I&II(Not-BWR1)~~		3.0	3.1	3.0	3.1
A4.04	Detector output: P-Spec(Not-BWR1)		2.0*	2.3*	2.0	2.3
A4.05	Indicating lights: P-Spec(Not-BWR1)		2.3*	2.5	2.3	2.5

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 215002

System Name: Rod Block Monitor System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between ROD BLOCK MONITOR SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~APRM: BWR-3, 4, 5~~		2.9	3.0	2.9	3.0
K1.02	~~LPRM: BWR-3, 4, 5~~		3.2	3.1	3.2	3.1
K1.03	~~Reactor manual control: BWR-3, 4, 5~~		3.2	3.2	3.2	3.2
K1.04	~~Recirculation system: BWR-3, 4, 5~~		3.1	3.1	3.1	3.1
K1.05	~~Four rod display: BWR-3, 4, 5~~		3.0	3.0	3.0	3.0
K1.06	~~Control rod selection: BWR-3, 4, 5~~		3.0	3.1	3.0	3.1
K1.07	~~IRM: BWR-3, 4, 5~~		2.0*	2.0*	2.0	2.0
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~RBM channels: BWR-3, 4, 5~~		2.5*	2.8*	2.5	2.8
K2.02	~~Recorders: BWR-3, 4, 5~~		2.1*	2.1*	2.1	2.1
K2.03	~~APRM channels: BWR-3, 4, 5~~		2.8	2.9	2.8	2.9
K3	**Knowledge of the effect that a loss or malfunction of the ROD BLOCK MONITOR SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	~~Reactor manual control system: BWR-3, 4, 5~~		3.3	3.5	3.3	3.5
K3.02	~~†Limiting control rod pattern: Plant-Specific~~		3.1	3.6	3.1	3.6
K4	**Knowledge of ROD BLOCK MONITOR SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Prevent control rod withdrawal: BWR-3, 4, 5~~		3.4	3.5	3.4	3.5
K4.02	~~Allows stepping up of rod block setpoint: BWR-3, 4, 5~~		2.9	3.0	2.9	3.0
K4.03	~~Initiation point (30%): BWR-3, 4, 5~~		2.9	3.0	2.9	3.0
K5	**Knowledge of the operational implications of the following concepts as they apply to ROD BLOCK MONITOR SYSTEM:**	**(41.5 / 45.3)**				
K5.01	~~Trip reference selection: Plant-Specific~~		2.6	2.8	2.6	2.8
K5.02	~~Null sequence control: BWR-3, 4, 5~~		2.4	2.5	2.4	2.5
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the ROD BLOCK MONITOR SYSTEM:**	**(41.7 / 45.7)**				
K6.01	~~RPS: BWR-3, 4, 5~~		3.0	3.2	3.0	3.2
K6.02	~~Instrument power: Plant-Specific~~		2.4	2.5	2.4	2.5
K6.03	~~Essential power: Plant-Specific~~		2.5	2.5	2.5	2.5
K6.04	~~APRM reference channel: BWR-3, 4, 5~~		2.8	3.0	2.8	3.0
K6.05	~~LPRM detectors: BWR-3, 4, 5~~		2.8	3.1	2.8	3.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215002

System Name: Rod Block Monitor System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A1	**Ability to predict and/or monitor changes in parameters associated with operating the ROD BLOCK MONITOR SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	~~Trip reference: BWR-3, 4, 5~~		2.7	2.8	2.7	2.8
A2	**Ability to (a) predict the impacts of the following on the ROD BLOCK MONITOR SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Withdrawal of control rod in high power region of core: BWR-3, 4, 5~~		3.3	3.5	3.3	3.5
A2.02	~~Loss or reduction in recirculation system flow (flow comparator): BWR-3, 4, 5~~		3.0	3.0	3.0	3.0
A2.03	~~Loss of associated reference APRM channel: BWR-3, 4, 5~~		3.1	3.3	3.1	3.3
A2.04	~~Power supply losses: BWR-3, 4, 5~~		2.7	2.8	2.7	2.8
A2.05	~~RBM high or inoperable: BWR-3, 4, 5~~		3.2	3.3	3.2	3.3
A3	**Ability to monitor automatic operations of the ROD BLOCK MONITOR SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	~~Four rod display: BWR-3, 4, 5~~		3.1	3.1	3.1	3.1
A3.02	~~Meters and recorders: BWR-3, 4, 5~~		3.1	3.0	3.1	[illegible]
A3.03	~~Alarm and indicating lights: BWR-3, 4, 5~~		3.1	3.1	3.1	3.1
A3.04	~~Verification or proper functioning/ operability: BWR-3, 4, 5~~		3.6	3.5	3.6	3.5
A3.05	~~Back-panel meters and indicating lights: BWR-3, 4, 5~~		3.2	3.2	3.2	3.2
A3.06	~~Transfer to alternate APRM when referenced APRM bypassed: BWR-3, 4, 5~~		2.6	2.6	2.6	2.6
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	~~IRM/RBM recorder/switch: BWR-3, 4, 5~~		2.8	2.7	2.8	2.7
A4.02	~~RBM back-panel switches, meters and indicating lights: BWR-3, 4, 5~~		2.9	2.9	2.9	2.9
A4.03	~~Trip bypasses: BWR-3, 4, 5~~		2.8	2.8	2.8	2.8
A4.04	~~Push-to-Check pushbutton: Plant-Specific~~		2.6	2.5	2.6	2.5
A4.05	~~Setup pushbutton: Plant-Specific~~		2.5	2.5	2.5	2.5
A4.06	~~Surveillance testing: BWR-3, 4, 5~~		2.7*	2.7	2.7	2.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215003

System Name: Intermediate Range Monitor (IRM) System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between INTERMEDIATE RANGE MONITOR (IRM) SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	RPS		3.9	3.9	3.9	3.9
K1.02	~~Reactor manual control~~		3.6	3.6	3.6	3.6
K1.03	Rod control and information system: Plant-Specific		3.1	3.1	3.1	3.1
K1.04	Process computer/ performance monitoring system (SPDS/ERIS/CRIDS/GDS): Plant-Specific		2.5	2.8	2.5	2.8
K1.05	Display control system: Plant-Specific		3.3	3.3	3.3	3.3
K1.06	APRM SCRAM signals: Plant-Specific		3.9	4.0	3.9	4.0
K1.07	Reactor vessel		3.0	3.0	3.0	3.0
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	IRM channels/detectors		2.5*	2.7	2.5	2.7
K3	**Knowledge of the effect that a loss or malfunction of the INTERMEDIATE RANGE MONITOR (IRM) SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	RPS		3.9	4.0	3.9	4.0
K3.02	~~Reactor manual control~~		3.6	3.6	3.6	3.6
K3.03	Rod control and information system: Plant-Specific		3.7	3.7	3.7	3.7
K3.04	Reactor power indication		3.6	3.6	3.6	3.6
K3.05	APRM: Plant-Specific		3.7	3.8	3.7	3.8
K4	**Knowledge of INTERMEDIATE RANGE MONITOR (IRM) SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Rod withdrawal blocks		3.7	3.7	3.7	3.7
K4.02	Reactor SCRAM signals		4.0	4.0	4.0	4.0
K4.03	Gamma compensation		2.4	2.4	2.4	2.4
K4.04	Varying system sensitivity levels using range switches		2.9	2.9	2.9	2.9
K4.05	Changing detector position		2.9	3.0	2.9	3.0
K4.06	Alarm seal-in		2.6	2.6	2.6	2.6
K5	**Knowledge of the operational implications of the following concepts as they apply to INTERMEDIATE RANGE MONITOR (IRM) SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Detector operation		2.6	2.7	2.6	2.7
K5.02	Gamma discrimination		2.2*	2.3	2.2	2.3
K5.03	Changing detector position		3.0	3.1	3.0	3.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215003

System Name: Intermediate Range Monitor (IRM) System

			CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the INTERMEDIATE RANGE MONITOR (IRM) SYSTEM:**	**(41.7 / 45.7)**					
K6.01	Reactor protection system (power supply): Plant-Specific			3.8	3.8	3.8	3.8
K6.02	24/48 volt D.C. power: Plant-Specific			3.6	3.8	3.6	3.8
K6.03	Detector drive motor			2.8	2.9	2.8	2.9
K6.04	Detectors			3.0	3.0	3.0	3.0
K6.05	Trip units			3.1	3.2	3.1	3.2
K6.06	APRM			3.2	3.4	3.2	3.4
K6.07	Recorder			2.3	2.3	2.3	2.3
A1	**Ability to predict and/or monitor changes in parameters associated with operating the INTERMEDIATE RANGE MONITOR (IRM) SYSTEM controls including:**	**(41.5 / 45.5)**					
A1.01	Detector position			3.4	3.3	3.4	3.3
A1.02	Reactor power indication response to rod position changes			3.7	3.7	3.7	3.7
A1.03	RPS status			3.6	3.7	3.6	3.7
A1.04	Control rod block status			3.4	3.4	3.4	3.4
A1.05	SCRAM and rod block trip setpoints			3.9	3.9	3.9	3.9
A1.06	Lights and alarms			3.3	3.2	3.3	3.2
A2	**Ability to (a) predict the impacts of the following on the INTERMEDIATE RANGE MONITOR (IRM) SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**					
A2.01	Power supply degraded			2.8	3.2	2.8	3.2
A2.02	IRM inop condition			3.5	3.7	3.5	3.7
A2.03	Stuck detector			2.9	3.1	2.9	3.1
A2.04	Up scale or down scale trips			3.7	3.8	3.7	3.8
A2.05	Faulty or erratic operation of detectors/system			3.3	3.5	3.3	3.5
A2.06	Faulty range switch			3.0	3.2	3.0	3.2
A2.07	Failed recorder			2.5	2.7	2.5	2.7
A3	**Ability to monitor automatic operations of the INTERMEDIATE RANGE MONITOR (IRM) SYSTEM including:**	**(41.7 / 45.7)**					
A3.01	Meters and recorders			3.3	3.3	3.3	3.3
A3.02	Annunciator and alarm signals			3.3	3.3	3.3	3.3
A3.03	RPS status			3.7	3.6	3.7	3.6
A3.04	Control rod block status			3.5	3.5	3.5	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215003

System Name: Intermediate Range Monitor (IRM) System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	IRM recorder indication		3.3	3.3	3.3	3.3
A4.02	CRT display indications: Plant-Specific		2.9	2.8	2.9	2.8
A4.03	IRM range switches		3.6	3.4	3.6	3.4
A4.04	IRM back panel switches, meters, and indicating lights		3.1	3.3	3.1	3.3
A4.05	Trip bypasses		3.4	3.4	3.4	3.4
A4.06	Detector drives		3.0	2.9	3.0	2.9
A4.07	Verification of proper functioning/ operability		3.6	3.6	3.6	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215004

System Name: Source Range Monitor (SRM) System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between SOURCE RANGE MONITOR (SRM) SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Reactor protection system		3.6	3.7	3.6	3.7
K1.02	~~Reactor manual control~~		3.4	3.4	3.4	3.4
K1.03	Rod control and information system: Plant-Specific		3.0	3.0	3.0	3.0
K1.04	Process computer/ performance monitoring system (SPDS/ERIS/CRIDS/GDS): Plant-Specific		2.4	2.6*	2.4	2.6
K1.05	Display control system: Plant-Specific		2.8	3.0	2.8	3.0
K1.06	Reactor vessel		2.8	2.8	2.8	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	SRM channels/detectors		2.6	2.8	2.6	2.8
K2.02	Detector drive modules		2.1*	2.3*	2.1	2.3
K2.03	Detector drive module control		2.0*	2.1*	2.0	2.1
K3	**Knowledge of the effect that a loss or malfunction of the SOURCE RANGE MONITOR (SRM) SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	RPS		3.4	3.4	3.4	3.4
K3.02	~~Reactor manual control: Plant-Specific~~		3.4	3.4	3.4	3.4
K3.03	Rod control and information system: Plant-Specific		3.3	3.3	3.3	3.3
K3.04	Reactor power and indication		3.7	3.7	3.7	3.7
K4	**Knowledge of SOURCE RANGE MONITOR (SRM) SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Rod withdrawal blocks		3.7	3.7	3.7	3.7
K4.02	Reactor SCRAM signals		3.4	3.5	3.4	3.5
K4.03	Gamma compensation		2.4	2.4	2.4	2.4
K4.04	Changing detector position		2.8	2.9	2.8	2.9
K4.05	*Alarm seal-in*		2.5	2.5	2.5	2.5
K4.06	IRM/SRM interlock		3.2	3.2	3.2	3.2
K5	**Knowledge of the operational implications of the following concepts as they apply to SOURCE RANGE MONITOR (SRM) SYSTEM:**	**(41.5 / 45.3)**				
K5.01	*Detector operation*		2.6	2.6	2.6	2.6
K5.02	Gamma discrimination		2.2*	2.2*	2.2	2.2
K5.03	Changing detector position		2.8	2.8	2.8	2.8

Facility: CPS Printed: 08/11/2006

System Number: 215004

System Name: Source Range Monitor (SRM) System

			CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the SOURCE RANGE MONITOR (SRM) SYSTEM:**	**(41.7 / 45.7)**					
K6.01	RPS			3.2	3.3	3.2	3.3
K6.02	24/48 volt D.C. power			3.1	3.3	3.1	3.3
K6.03	Detector drive motor			2.4	2.4	2.4	2.4
K6.04	Detectors			2.9	2.9	2.9	2.9
K6.05	Trip units			2.6	2.8	2.6	2.8
K6.06	Recorder			2.2*	2.2*	2.2	2.2
A1	**Ability to predict and/or monitor changes in parameters associated with operating the SOURCE RANGE MONITOR (SRM) SYSTEM controls including:**	**(41.5 / 45.5)**					
A1.01	Detector position			3.0	3.1	3.0	3.1
A1.02	Reactor power indication			3.6	3.7	3.6	3.7
A1.03	RPS status			3.4	35	3.4	35
A1.04	Control rod block status			3.5	3.5	3.5	3.5
A1.05	SCRAM, rod block, and period alarm trip setpoints			3.6	3.8	3.6	3.8
A1.06	Lights and alarms			3.1	3.1	3.1	3.1
A2	**Ability to (a) predict the impacts of the following on the SOURCE RANGE MONITOR (SRM) SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**					
A2.01	Power supply degraded			2.7	2.9	2.7	2.9
A2.02	SRM inop condition			3.4	3.7	3.4	3.7
A2.03	Stuck detector			3.0	3.3	3.0	3.3
A2.04	Up scale and downscale trips			3.5	3.7	3.5	3.7
A2.05	Faulty or erratic operation of detectors/system			3.3	3.5	3.3	3.5
A2.06	Failed recorder			2.4*	2.5	2.4	2.5
A3	**Ability to monitor automatic operations of the SOURCE RANGE MONITOR (SRM) SYSTEM including:**	**(41.7 / 45.7)**					
A3.01	Meters and recorders			3.2	3.2	3.2	3.2
A3.02	Annunciator and alarm signals			3.4	3.3	3.4	3.3
A3.03	RPS status			3.6	3.5	3.6	3.5
A3.04	Control rod block status			3.6	3.6	3.6	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215004

System Name: Source Range Monitor (SRM) System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	SRM count rate and period		3.9	3.8	3.9	3.8
A4.02	SRM recorder		3.0	3.1	3.0	3.1
A4.03	CRT displays: Plant-Specific		2.9	2.7	2.9	2.7
A4.04	SRM drive control switches		3.2	3.2	3.2	3.2
A4.05	SRM back panel switches, meters, and indicating lights		3.1	3.2	3.1	3.2
A4.06	Alarms and lights		3.2	3.1	3.2	3.1
A4.07	Verification of proper functioning/ operability		3.4	3.6	3.4	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215005

System Name: Average Power Range Monitor/Local Power Range Monitor System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between APRM/LPRM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	RPS		4.0	4.0	4.0	4.0
K1.02	IRM		3.7	3.7	3.7	3.7
K1.03	~~RBM: Plant-Specific~~		3.4	3.5	3.4	3.5
K1.04	LPRM channels		3.6	3.6	3.6	3.6
K1.05	Four rod display: Plant-Specific		3.2	3.2	3.2	3.2
K1.06	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.5	2.8*	2.5	2.8
K1.07	Process computer, performance monitoring system		2.6	2.9	2.6	2.9
K1.08	Display control system: Plant-Specific		3.0	3.0	3.0	3.0
K1.09	Reactor recirculation system: BWR-5, 6		3.6	3.6	3.6	3.6
K1.10	~~Reactor manual control system: Plant-Specific~~		3.3	3.3	3.3	3.3
K1.11	Rod control and information system: Plant-Specific		3.4	3.4	3.4	3.4
K1.12	Full core display		3.2	3.2	3.2	3.2
K1.13	Traversing incore probe system		2.6	3.0	2.6	3.0
K1.14	Reactor vessel		2.8	2.9	2.8	2.9
K1.15	Redundant reactivity control system: Plant-Specific		3.7	4.0	3.7	4.0
K1.16	Flow converter/comparator network: Plant-Specific		3.3	3.4	3.3	3.4
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
2.01	LPRM channels		2.4	2.6	2.4	2.6
K2.02	APRM channels		2.6	2.8	2.6	2.8
K3	**Knowledge of the effect that a loss or malfunction of the APRM/LPRM will have on following:**	**(41.7 / 45.4)**				
K3.01	RPS		4.0	4.0	4.0	4.0
K3.02	Reactor recirculation system: BWR-5, 6		3.5	3.5	3.5	3.5
K3.03	~~Reactor manual control system: Plant-Specific~~		3.3	3.3	3.3	3.3
K3.04	Rod control and information system: Plant-Specific		3.4	3.4	3.4	3.4
K3.05	Reactor power indication		3.8	3.8	3.8	3.8
K3.06	IRM: Plant-Specific		3.5	3.6	3.5	3.6
K3.07	Rod block monitor: Plant-Specific		3.2	3.3	3.2	3.3
K3.08	†core thermal calculations		3.0	3.4	3.0	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215005

System Name: Average Power Range Monitor/Local Power Range Monitor System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S
K4	**Knowledge of APRM/LPRM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Rod withdrawal blocks		3.7	3.7	3.7	3.7
K4.02	Reactor SCRAM signals		4.1*	4.2	4.1	4.2
K4.03	Eliminates the necessity of running signal leads from each LPRM channel to the operator's console full core display		2.1*	2.2*	2.1	2.2
K4.04	Individual LPRM detector replacement		2.0*	2.4*	2.0	2.4
K4.05	Alarm seal-in		2.4	2.4	2.4	2.4
K4.06	Effects of detector aging on LPRM/APRM readings		2.6	2.8	2.6	2.8
K4.07	Flow biased trip setpoints		3.7	3.7	3.7	3.7
K4.08	Sampling of overall core power in each APRM (accomplished through LPRM assignments and symmetrical rod patterns)		2.7	3.1	2.7	3.1
K5	**Knowledge of the operational implications of the following concepts as they apply to APRM/LPRM:**	**(41.5 / 45.3)**				
K5.01	LPRM detector operation		2.8	2.9	2.8	2.9
K5.02	Effects of voids on LPRM indication		2.7	2.8	2.7	2.8
K5.03	Control rod symmetrical patterns		2.9	3.3	2.9	3.3
K5.04	LPRM detector location and core symmetry		2.9	3.2	2.9	3.2
K5.05	Core flow effects on APRM trip setpoints		3.6	3.6	3.6	3.6
K5.06	Assignment of LPRM's to specific APRM channels		2.5*	2.6*	2.5	2
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the APRM/LPRM:**	**(41.7 / 45.7)**				
K6.01	RPS		3.7	3.8	3.7	3.8
K6.02	†Traversing incore probe system		2.4	2.8	2.4	2.8
K6.03	Detectors		3.1	3.3	3.1	3.3
K6.04	Trip units		3.1	3.2	3.1	3.2
K6.05	IRM: Plant-Specific		2.9	3.1	2.9	3.1
K6.06	Recorder		2.3	2.4	2.3	2.4
K6.07	Flow converter/comparator network: Plant-Specific		3.2	3.3	3.2	3.3
A1	**Ability to predict and/or monitor changes in parameters associated with operating the APRM/LPRM controls including:**	**(41.5 / 45.5)**				
A1.01	Reactor power indication		4.0	4.0	4.0	4.0
A1.02	RPS status		3.9	4.0	3.9	4.0
A1.03	Control rod block status		3.6	3.6	3.6	3.6
A1.04	SCRAM and rod block trip setpoints		4.1	4.1	4.1	4.1
A1.05	Lights and alarms		3.3	3.2	3.3	3.2
A1.06	Recirculation flow control valve position: Plant-Specific		3.1	3.3	3.1	3.3
A1.07	APRM (gain adjustment factor)		3.0	3.4	3.0	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 215005

System Name: Average Power Range Monitor/Local Power Range Monitor System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the APRM/LPRM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Power supply degraded		2.7	3.1	2.7	3.1
A2.02	Upscale or downscale trips		3.6	3.7	3.6	3.7
A2.03	Inoperative trip (all causes)		3.6	3.8	3.6	3.8
A2.04	SCRAM trip signals		3.8	3.9	3.8	3.9
A2.05	Loss of recirculation flow signal		3.5	3.6	3.5	3.6
A2.06	Recirculation flow channels upscale		3.4	3.5	3.4	3.5
A2.07	Recirculation flow channels flow mismatch		3.2	3.4	3.2	3.4
A2.08	Faulty or erratic operation of detectors/systems		3.2	3.4	3.2	3.4
A2.09	Failed recorder		2.4*	2.4	2.4	2.4
A2.10	Changes in void concentration		2.8	2.9	2.8	2.9
A3	**Ability to monitor automatic operations of the APRM/LPRM including:**	**(41.7 / 45.7)**				
A3.01	Four rod display: Plant-Specific		3.5	3.5	3.5	3.5
A3.02	Full core display		3.5	3.5	3.5	3.5
A3.03	Meters and recorders		3.3	3.3	3.3	3.3
A3.04	Annunciator and alarm signals		3.2	3.2	3.2	3.2
A3.05	Flow converter/comparator alarms		3.3	3.3	3.3	3.3
A3.06	Maximum disagreement between flow comparator channels: Plant-Specific		3.0	3.1	3.0	3.1
A3.07	RPS status		3.8	3.8	3.8	3.8
A3.08	Control rod block status		3.7	3.6	3.7	3.6
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	IRM/APRM recorder		3.2	3.1	3.2	3.1
A4.02	CRT display indicators: Plant-Specific		2.8	2.8	2.8	2.8
A4.03	APRM back panel switches, meters and indicating lights		3.2	3.3	3.2	3.3
A4.04	LPRM back panel switches, meters and indicating lights		3.2	3.2	3.2	3.2
A4.05	Trip bypasses		3.4	3.4	3.4	3.4
A4.06	Verification of proper functioning/ operability		3.6	3.8	3.6	3.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 216000

System Name: Nuclear Boiler Instrumentation

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between NUCLEAR BOILER INSTRUMENTATION and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Reactor protection system		3.9	4.1	3.9	4.1
K1.02	PCIS/NSSSS		3.8	4.0	3.8	4.0
K1.03	Reactor core isolation cooling system: Plant-Specific		3.4	3.6	3.4	3.6
K1.04	High pressure core spray system: Plant-Specific		3.9	4.0	3.9	4.0
K1.05	Residual heat removal: Plant-Specific		3.7	3.9	3.7	3.9
K1.06	Low pressure core spray		3.9	3.9	3.9	3.9
K1.07	Automatic depressurization system		3.9	4.1	3.9	4.1
K1.08	Relief/safety valves		3.7	3.9	3.7	3.9
K1.09	Redundant reactivity control/ alternate rod insertion; Plant-Specific		3.7	4.0	3.7	4.0
K1.10	Recirculation flow control system		3.2	3.4	3.2	3.4
K1.11	MSIV leakage control system: Plant-Specific		2.7	2.8	2.7	2.8
K1.12	Reactor water level control system		3.6	3.7	3.6	3.7
K1.13	Feedwater system		3.4	3.5	3.4	3.5
K1.14	~~High pressure coolant injection: Plant-Specific~~		3.8	4.1*	3.8	4.1
K1.15	~~Isolation condenser: Plant-Specific~~		3.9*	4.1*	3.9	4.1
K1.16	Main turbine		3.0	3.1	3.0	3.1
K1.17	Emergency generators		3.5	3.7	3.5	3.7
K1.18	Analog trip system: Plant-Specific		3.0*	3.1	3.0	3.1
K1.19	Anticipated transient without scram system: Plant-Specific		3.8	3.9	3.8	3.9
K1.20	Process computer: Plant-Specific		2.6	2.8	2.6	2.8
K1.21	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.6*	2.9*	2.6	2.9
K1.22	Reactor vessel		3.6	3.8	3.6	3.8
K1.23	Recirculation system		3.3	3.4	3.3	3.4
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Analog trip system: Plant-Specific		2.8	2.8	2.8	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 216000

System Name: Nuclear Boiler Instrumentation

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir
K3	**Knowledge of the effect that a loss or malfunction of the NUCLEAR BOILER Instrumentation will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor protection system		4.0	4.3*	4.0	4.3
K3.02	PCIS/NSSSS		4.0	4.3*	4.0	4.3
K3.03	Reactor core isolation cooling system: Plant-Specific		3.5	3.8	3.5	3.8
K3.04	High pressure core spray system: Plant-Specific		3.8	4.0	3.8	4.0
K3.05	Residual heat removal: Plant-Specific		3.8	3.9	3.8	3.9
K3.06	Low pressure core spray		3.8	3.9	3.8	3.9
K3.07	Automatic depressurization system		3.9	4.1	3.9	4.1
K3.08	Relief/safety valves		3.6	3.7	3.6	3.7
K3.09	Redundant reactivity control/ alternate rod insertion: Plant-Specific		3.7	4.0	3.7	4.0
K3.10	Recirculation flow control system		3.2	3.3	3.2	3.3
K3.11	MSIV leakage control system: Plant-Specific		2.8	2.8	2.8	2.8
K3.12	Reactor water level control system		3.7	3.8	3.7	3.8
K3.13	Feedwater system		3.4	3.5	3.4	3.5
K3.14	~~High pressure coolant injection: Plant-Specific~~		3.8	4.2*	3.8	4.2
K3.15	~~Isolation condenser: Plant-Specific~~		3.8	4.2*	3.8	4.2
K3.16	Main turbine		3.0	3.1	3.0	3.1
K3.17	Emergency generators		3.5	3.7	3.5	3.7
K3.18	Analog trip system: Plant-Specific		2.9*	3.0	2.9	3.0
K3.19	Anticipated transient without scram system: Plant-Specific		3.7	4.0*	3.7	
K3.20	Process computer: Plant-Specific		2.5	2.6	2.5	2.6
K3.21	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.6*	2.8*	2.6	2.8
K3.22	Reactor vessel		3.2	3.3	3.2	3.3
K3.23	Vessel temperature monitoring		3.1	3.3	3.1	3.3
K3.24	Vessel level monitoring		3.9	4.1	3.9	4.1
K3.25	Vessel pressure monitoring		3.9	4.1	3.9	4.1
K3.26	Core flow monitoring		3.6	3.7	3.6	3.7
K3.27	Core differential pressure monitoring		2.9	3.1	2.9	3.1
K3.28	Loose parts detection in the primary system: Plant-Specific		2.0*	2.1*	2.0	2.1
K3.29	Jet pump flow monitoring: Plant-Specific		3.1	3.2	3.1	3.2
K3.30	Recirculation system		3.2	3.3	3.2	3.3

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 216000

System Name: Nuclear Boiler Instrumentation

			CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K4	**Knowledge of NUCLEAR BOILER INSTRUMENTATION design feature(s) and/or interlocks which provide for the following:**	**(41.7)**					
K4.01	Reading of nuclear boiler parameters outside the control room			3.6	3.6	3.6	3.6
K4.02	Physical separation of sensors			3.0	3.2	3.0	3.2
K4.03	Redundancy of sensors			3.4	3.6	3.4	3.6
K4.04	Inputs to the reactor protection system			3.7	3.8	3.7	3.8
K4.05	Initiation of the emergency core cooling systems			3.9	4.1	3.9	4.1
K4.06	Initiation of the PCIS/NSSSS			3.8	4.0	3.8	4.0
K4.07	Recirculation pump protection: Plant-Specific			2.9	3.0	2.9	3.0
K4.08	Protection for the main turbine from high moisture carryover			2.9	3.0	2.9	3.0
K4.09	Protection against filling the main steam lines from the feed system			3.3	3.3	3.3	3.3
K4.10	Automatic recirculation pump speed control: Plant-Specific			2.9	3.1	2.9	3.1
K4.11	Inputs to the redundant reactivity control system/alternate rod insertion: Plant-Specific			4.0	4.0	4.0	4.0
K4.12	Reactor vessel overpressure protection			3.7	3.9	3.7	3.9
K4.13	Overpressure protection for various low-pressure systems			3.4	3.4	3.4	3.4
K4.14	Temperature compensation for reactor water level indication: Plant-Specific			3.3	3.4	3.3	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 216000

System Name: Nuclear Boiler Instrumentation

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K5	**Knowledge of the operational implications of the following concepts as they apply to NUCLEAR BOILER INSTRUMENTATION:**	**(41.5 / 45.3)**				
K5.01	Vessel level measurement		3.1	3.2	3.1	3.2
K5.02	Vessel pressure measurement		3.1	3.2	3.1	3.2
K5.03	Vessel temperature measurement		3.0	3.2	3.0	3.2
K5.04	Vessel differential pressure measurement		2.8	2.9	2.8	2.9
K5.05	Vessel vibration measurement (loose parts monitor)		2.3	2.3	2.3	2.3
K5.06	Rapid vessel depressurization effects on vessel level indications		3.4	3.6	3.4	3.6
K5.07	Elevated containment temperature effects on level indication		3.6	3.8	3.6	3.8
K5.08	Steam flow effect on reactor water level		3.1	3.2	3.1	3.2
K5.09	Recirculation flow effects on level indications: Design-Specific		2.9	2.9	2.9	2.9
K5.10	Indicated level versus actual vessel level during vessel heatups or cooldowns		3.1	3.3	3.1	3.3
K5.11	Indicated vessel temperature response during rapid heatups or cooldowns		3.2	3.3	3.2	3.3
K5.12	Effects on level indication due to rapid changes in void fraction		3.2	3.3	3.2	3.3
K5.13	Reference leg flashing: Design-Specific		3.5	3.6	3.5	3.6
K5.14	Density		2.6	2.6	2.6	
K5.15	Static pressure		2.2*	2.2*	2.2	2.2
K5.16	Dynamic pressure		2.1*	2.1*	2.1	2.1
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the NUCLEAR BOILER INSTRUMENTATION:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical distribution		3.1	3.3	3.1	3.3
K6.02	D.C. electrical distribution		2.8	3.0	2.8	3.0
K6.03	Temperature compensation: Plant-Specific		2.8	2.8	2.8	2.8
A1	**Ability to predict and/or monitor changes in parameters associated with operating the NUCLEAR BOILER INSTRUMENTATION controls including:**	**(41.5 / 45.5)**				
A1.01	Recorders and meters		3.4	3.3	3.4	3.3
A1.02	Removing or returning a sensor (transmitter) to service		2.9*	3.1*	2.9	3.1
A1.03	Surveillance testing		2.9*	3.2*	2.9	3.2
A1.04	System venting		2.6*	2.8*	2.6	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 216000

System Name: Nuclear Boiler Instrumentation

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the NUCLEAR BOILER INSTRUMENTATION; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Detector equalizing valve leaks		2.9	3.2	2.9	3.2
A2.02	Instrument line plugging		2.9	3.0	2.9	3.0
A2.03	Instrument line leakage		3.0	3.1	3.0	3.1
A2.04	Detector diaphragm failure or leakage		2.9	3.0	2.9	3.0
A2.05	Surveillance testing		2.8	3.1	2.8	3.1
A2.06	Loss of power supply		2.9	3.1	2.9	3.1
A2.07	Reference leg flashing		3.4	3.5	3.4	3.5
A2.08	Elevated containment temperature		3.2	3.4	3.2	3.4
A2.09	Jet pump flow: Design-Specific		3.1	3.2	3.1	3.2
A2.10	Rapid vessel depressurizations		3.3	3.5	3.3	3.5
A2.11	Heatup or cooldown of the reactor vessel		3.2	3.3	3.2	3.3
A2.12	Instrument isolation valve closures		2.8	2.9	2.8	2.9
A2.13	Instrument isolation valve openings		2.8	3.0	2.8	3.0
A2.14	Recirculation flow: Design-Specific		2.9	2.9	2.9	2.9
A3	**Ability to monitor automatic operations of the NUCLEAR BOILER Instrumentation including:**	**(41.7 / 45.7)**				
A3.01	Relationship between meter/recorder readings and actual parameter values: Plant-Specific		3.4	3.4	3.4	3.4
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Recorders		3.3	3.1	3.3	3.1
A4.02	Channel select controls		3.3	3.1	3.3	3.1
A4.03	Process computer: Design-Specific		3.0	3.1	3.0	3.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 217000

System Name: Reactor Core Isolation Cooling System (RCIC)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR CORE ISOLATION COOLING SYSTEM (RCIC) and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Condensate storage and transfer system		3.5	3.5	3.5	3.5
K1.02	Nuclear boiler system		3.5	3.5	3.5	3.5
K1.03	Suppression pool		3.6	2.6	3.6	2.6
K1.04	Main condenser		2.6	2.6	2.6	2.6
K1.05	Residual heat removal system		2.6	2.6	2.6	2.6
K1.06	Plant air systems: Plant-Specific		2.3*	2.3*	2.3	2.3
K1.07	Leak detection		3.1	3.2	3.1	3.2
K1.08	Line fill pump: Plant-Specific		3.3	3.4	3.3	3.4
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Motor operated valves		2.8*	2.8*	2.8	2.8
K2.02	RCIC initiation signals (logic)		2.8*	2.9*	2.8	2.9
K2.03	RCIC flow controller		2.7*	2.8*	2.7	2.8
K2.04	Gland seal compressor (vacuum pump)		2.6*	2.6*	2.6	2.6
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC) will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor water level		3.7	3.7	3.7	3.7
K3.02	Reactor vessel pressure		3.6	3.6	3.6	3.6
K3.03	Decay heat removal		3.5	3.5	3.5	3.5
K3.04	Adequate core cooling		3.6	3.6	3.6	3.6
K4	**Knowledge of REACTOR CORE ISOLATION COOLING SYSTEM (RCIC) design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Prevent water hammer: Plant-Specific		2.8	2.8	2.8	2.8
K4.02	Prevent over filling reactor vessel		3.3	3.3	3.3	3.3
K4.03	Prevents pump over heating		2.9	3.0	2.9	3.0
K4.04	Prevents turbine damage: Plant-Specific		3.0	3.1	3.0	3.1
K4.05	Prevents radioactivity release to auxiliary/reactor building		3.2	3.5	3.2	3.5
K4.06	Manual initiation		3.5	3.5	3.5	3.5
K4.07	Alternate supplies of water		3.6	3.6	3.6	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 217000

System Name: Reactor Core Isolation Cooling System (RCIC)

			CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
K5	**Knowledge of the operational implications of the following concepts as they apply to REACTOR CORE ISOLATION COOLING SYSTEM (RCIC):**	**(41.5 / 45.3)**					
K5.01	Indications of pump cavitation			2.6*	2.6*	2.6	2.6
K5.02	Flow indication			3.1	3.1	3.1	3.1
K5.03	Differential pressure indication			2.6	2.6	2.6	2.6
K5.04	Testable check valve operation			2.6	2.7	2.6	2.7
K5.05	Centrifugal pump operation			2.4*	2.4*	2.4	2.4
K5.06	Turbine operation			2.7*	2.7	2.7	2.7
K5.07	Assist core cooling			3.1	3.1	3.1	3.1
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC):**	**(41.7 / 45.7)**					
K6.01	Electrical power			3.4	3.5	3.4	3.5
K6.02	Plant air systems			2.2*	2.2*	2.2	2.2
K6.03	Suppression pool water supply			3.5	3.5	3.5	3.5
K6.04	Condensate storage and transfer system			3.5	3.5	3.5	3.5
A1	**Ability to predict and/or monitor changes in parameters associated with operating the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC) controls including:**	**(41.5 / 45.5)**					
A1.01	RCIC flow			3.7	3.7	3.7	3.7
A1.02	RCIC pressure			3.3	3.3	3.3	3.3
A1.03	Reactor water level			4.0	4.0	4.0	4.0
A1.04	Reactor pressure			3.6*	3.6*	3.6	3.6
A1.05	RCIC turbine speed			3.7	3.7	3.7	3.7
A1.06	Condensate storage tank level			3.2	3.3	3.2	3.3
A1.07	Suppression pool level			3.3	3.5	3.3	3.5
A1.08	Suppression pool temperature			3.5	3.6	3.5	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 217000

System Name: Reactor Core Isolation Cooling System (RCIC)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	System initiation signal		3.8	3.7	3.8	3.7
A2.02	Turbine trips		3.8	3.7	3.8	3.7
A2.03	Valve closures		3.4	3.3	3.4	3.3
A2.04	A.C. power loss		2.3*	2.3*	2.3	2.3
A2.05	D.C. power loss		3.3	3.3	3.3	3.3
A2.06	Loss of applicable plant air systems		2.2*	2.2*	2.2	2.2
A2.07	Loss of lube oil		3.1	3.1	3.1	3.1
A2.08	Loss of lube oil cooling		3.0	3.1	3.0	3.1
A2.09	Loss of vacuum pump		2.9	3.0	2.9	3.0
A2.10	Turbine control system failures		3.1	3.1	3.1	3.1
A2.11	Inadequate system flow		3.1	3.2	3.1	3.2
A2.12	Valve openings		3.0	3.0	3.0	3.0
A2.13	Loss of room cooling		2.9	3.0	2.9	3.0
A2.14	Rupture disc failure: Exhaust-Diaphragm		3.3	3.4	3.3	3.4
A2.15	Steam line break		3.8	3.8	3.8	3.8
A2.16	Low Condensate storage tank level		3.5	3.4	3.5	3.4
A2.17	High suppression pool level		3.2	3.4	3.2	3.4
A2.18	Low suppression pool level		3.1	3.2	3.1	3.2
A2.19	High suppression pool temperature		3.5	3.6	3.5	3.6
A3	**Ability to monitor automatic operations of the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC) including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		3.5	3.5	3.5	3.5
A3.02	Turbine startup		3.6	3.5	3.6	3.5
A3.03	System pressure		3.7	3.6	3.7	3.6
A3.04	System flow		3.6	3.5	3.6	3.5
A3.05	Reactor water level		3.9	3.9	3.9	3.9
A3.06	Lights and alarms		3.5	3.4	3.5	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 217000

System Name: Reactor Core Isolation Cooling System (RCIC)

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	RCIC turbine speed		3.7	3.7	3.7	3.7
A4.02	Turbine trip throttle valve reset		3.9	3.9	3.9	3.9
A4.03	System valves		3.4	3.3	3.4	3.3
A4.04	Manually initiated controls		3.6	3.6	3.6	3.6
A4.05	Reactor water level		4.1	4.1	4.1	4.1
A4.06	Suppression pool level		3.6	3.7	3.6	3.7
A4.07	Reactor pressure		3.9	3.8	3.9	3.8
A4.08	System flow		3.7	3.6	3.7	3.6
A4.09	System pressure		3.7	3.6	3.7	3.6
A4.10	Lights and alarms		3.6	3.5	3.6	3.5
A4.11	Condensate storage tank level		3.5	3.5	3.5	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 218000

System Name: Automatic Depressurization System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between AUTOMATIC DEPRESSURIZATION SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	RHR/LPCI: Plant-Specific		4.0	4.1	4.0	4.1
K1.02	Low pressure core spray: Plant-Specific		4.0	4.1	4.0	4.1
K1.03	Nuclear boiler instrument system		3.7	3.8	3.7	3.8
K1.04	Drywell/containment pressure: Plant-Specific		3.9	4.2	3.9	4.2
K1.05	Remote shutdown system: Plant-Specific		3.9	3.9	3.9	3.9
K1.06	Safety/relief valves		3.9*	3.9*	3.9	3.9
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	ADS logic		3.1*	3.3*	3.1	3.3
K3	**Knowledge of the effect that a loss or malfunction of the AUTOMATIC DEPRESSURIZATION SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Restoration of reactor water level after a break that does not depressurize the reactor when required		4.4*	4.4*	4.4	4.4
K3.02	Ability to rapidly depressurize the reactor		4.5*	4.6*	4.5	4.6
K4	**Knowledge of AUTOMATIC DEPRESSURIZATION SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Prevent inadvertent initiatior of ADS logic		3.7	3.9	3.7	3.9
K4.02	Allows manual initiation of ADS logic		3.8	4.0	3.8	4.0
K4.03	ADS logic control		3.8	4.0	3.8	4.0
K4.04	Insures adequate air supply to ADS valves: Plant-Specific		3.5	3.6	3.5	3.6
K5	**Knowledge of the operational implications of the following concepts as they apply to AUTOMATIC DEPRESSURIZATION SYSTEM:**	**(41.5 / 45.3)**				
K5.01	ADS logic operation		3.8	3.8	3.8	3.8
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the AUTOMATIC DEPRESSURIZATION SYSTEM:**	**(41.7 / 45.7)**				
K6.01	RHR/LPCI system pressure: Plant-Specific		3.9	4.1	3.9	4.1
K6.02	Low pressure core spray system pressure: Plant-Specific		4.1	4.1	4.1	4.1
K6.03	Nuclear boiler instrument system (level indication)		3.8	3.9	3.8	3.9
K6.04	Air supply to ADS valves: Plant-Specific		3.6	3.7	3.6	3.7
K6.05	A.C. power: Plant-Specific		3.0*	3.1*	3.0	3.1
K6.06	D.C. power: Plant-Specific		3.4*	3.6	3.4	3.6
K6.07	Primary containment instrumentation		3.4	3.5	3.4	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 218000

System Name: Automatic Depressurization System

			CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
A1	**Ability to predict and/or monitor changes in parameters associated with operating the AUTOMATIC DEPRESSURIZATION SYSTEM controls including:**	**(41.5 / 45.5)**					
A1.01	ADS valve tail pipe temperatures			3.4	3.6	3.4	3.6
A1.02	ADS valve acoustical monitor noise: Plant-Specific			3.7	4.0	3.7	4.0
A1.03	ADS valve air supply pressure: Plant-Specific			3.2	3.4*	3.2	3.4
A1.04	Reactor pressure			4.1*	4.2*	4.1	4.2
A1.05	Reactor water level			4.1*	4.1	4.1	4.1
A1.06	Suppression pool temperature			4.1	4.3*	4.1	4.3
A2	**Ability to (a) predict the impacts of the following on the AUTOMATIC DEPRESSURIZATION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**					
A2.01	Small steam line break LOCA			4.1	4.3*	4.1	4.3
A2.02	Large break LOCA			3.5	3.6*	3.5	3.6
A2.03	Loss of air supply to ADS valves: Plant-Specific			3.4	3.6	3.4	3.6
A2.04	ADS failure to initiate			4.1	4.2*	4.1	4.2
A2.05	Loss of A.C. or D.C. power to ADS valves			3.4*	3.6*	3.4	3.6
A2.06	ADS initiation signals present			4.2	4.3*	4.2	4.3
A3	**Ability to monitor automatic operations of the AUTOMATIC DEPRESSURIZATION SYSTEM including:**	**(41.7 / 45.7)**					
A3.01	ADS valve operation			4.2*	4.3	4.2	4.3
A3.02	ADS valve tail pipe temperatures			3.6	3.7	3.6	3.7
A3.03	ADS valve acoustical monitor noise: Plant-Specific			3.7	3.8	3.7	3.8
A3.04	Primary containment pressure			3.7	3.8	3.7	3.8
A3.05	Suppression pool level			3.6	3.7	3.6	3.7
A3.06	Suppression pool temperature			3.9	3.9	3.9	3.9
A3.07	Lights and alarms			3.7	3.6	3.7	3.6
A3.08	Reactor pressure			4.2*	4.3*	4.2	4.3
A3.09	Reactor vessel water level			4.1*	4.2*	4.1	4.2

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 218000

System Name: *Automatic Depressurization System*

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	ADS valves		4.4*	4.4*	4.4	4.4
A4.02	ADS logic initiation		4.2*	4.2*	4.2	4.2
A4.03	ADS logic reset		4.2*	4.2*	4.2	4.2
A4.04	ADS inhibit: Plant-Specific		4.1	4.1*	4.1	4.1
A4.05	ADS timer reset		4.2*	4.2*	4.2	4.2
A4.06	ADS valve tail pipe temperature		3.5	3.6	3.5	3.6
A4.07	ADS valve acoustical monitor noise: Plant-Specific		3.5	3.8	3.5	3.8
A4.08	Suppression pool level		3.7	3.8	3.7	3.8
A4.09	Suppression pool temperature		3.9	3.9	3.9	3.9
A4.10	Lights and alarms		3.8	3.8	3.8	3.8
A4.11	Reactor pressure		4.3*	4.3*	4.3	4.3
A4.12	Reactor vessel water level		4.2*	4.3*	4.2	4.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 219000

System Name: RHR/LPCI: Torus/Suppression Pool Cooling Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Suppression pool		3.8	3.9	3.8	3.9
K1.02	Condensate storage tank		2.4*	2.4	2.4	2.4
K1.03	LPCI/RHR piping		3.7	3.8	3.7	3.8
K1.04	LPCI/RHR pumps		3.9	3.9	3.9	3.9
K1.05	A.C. electrical power		3.5	3.6	3.5	3.6
K1.06	Keep fill system		3.2	3.3	3.2	3.3
K1.07	~~Condensate transfer~~		2.5	2.6	2.5	2.6
K1.08	D.C. electrical power		2.6	2.8	2.6	2.8
K1.09	Nuclear boiler instrumentation		3.3	3.4	3.3	3.4
K1.10	Reactor building drain system: Plant-Specific		1.9*	1.9*	1.9	1.9
K1.11	Component cooling water systems		3.0	3.0	3.0	3.0
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	†Valves		2.5*	2.9*	2.5	2.9
K2.02	Pumps		3.1*	3.3*	3.1	3.3
K2.03	Valve control logic: Plant-Specific		2.2*	2.6*	2.2	2.6
K3	**Knowledge of the effect that a loss or malfunction of the RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE will have on following:**	**(41.7 / 45.4)**				
K3.01	Suppression pool temperature control		3.9	4.1	3.9	4.1
K4	**Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Surveillance for all operable components		3.1	3.4	3.1	3.4
K4.02	Redundancy		3.5	3.5	3.5	3.5
K4.03	Unintentional reduction in vessel injection flow during accident conditions: Plant-Specific		3.8	3.8	3.8	3.8
K4.04	Prevention of piping overpressurization: Plant-Specific		3.3	3.4	3.3	3.4
K4.05	Pump minimum flow protection		3.0	3.2	3.0	3.2
K4.06	Pump motor cooling: Plant-Specific		2.7	2.7	2.7	2.7
K4.07	Prevention of water hammer: Plant-Specific		3.0	3.1	3.0	3.1
K4.08	Adequate pump net positive suction head		2.9	3.0	2.9	3.0
K4.09	Heat exchanger cooling		3.3	3.4	3.3	3.4
K4.10	Prevention of leakage to the environment through system heat exchanger: Plant-Specific		3.3	3.6	3.3	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: **219000**

System Name: **RHR/LPCI: Torus/Suppression Pool Cooling Mode**

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K5	**Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE:**	**(41.5 / 45.3)**				
K5.01	System venting		2.6	2.7	2.6	2.7
K5.02	Pump cavitation		2.7	2.8	2.7	2.8
K5.03	Pressure measurement		2.7*	2.8*	2.7	2.8
K5.04	Heat exchanger operation		2.9	2.9	2.9	2.9
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical power		3.2	3.3	3.2	3.3
K6.02	D.C. electrical power		2.5*	2.8*	2.5	2.8
K6.03	Emergency generator		3.5	3.5	3.5	3.5
K6.04	Keep fill system		2.9*	3.0	2.9	3.0
K6.05	Condensate storage tank: Plant-Specific		2.2*	2.2*	2.2	2.2
K6.06	Suppression pool		3.7	3.7	3.7	3.7
K6.07	~~Condensate transfer~~		2.2*	2.2*	2.2	2.2
K6.08	ECCS room cooling		2.7*	2.7	2.7	2.7
K6.09	Nuclear boiler instrumentation		3.0	3.1	3.0	3.1
K6.10	Component cooling water systems		2.9	2.9	2.9	2.9
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE controls including:**	**(41.5 / 45.5)**				
A1.01	Suppression pool temperature		4.0	4.0	4.0	4.0
A1.02	System flow		3.5	3.5	3.5	3.5
A1.03	System pressure		2.9	2.9	2.9	2.9
A1.04	Suppression pool level		3.2	3.2	3.2	3.2
A1.05	Condensate storage tank level: Plant-Specific		2.1*	2.1*	2.1	2.1
A1.06	Motor amps: Plant-Specific		2.3*	2.4*	2.3	2.4
A1.07	Emergency generator loading		3.2	3.3	3.2	3.3
A1.08	System lineup		3.7	3.6	3.7	3.6
A1.09	~~Suppression chamber air temperature: Plant-Specific~~		3.2	3.3	3.2	3.3
A1.10	Containment air temperature: Mark-III		3.1	3.1	3.1	3.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 219000

System Name: RHR/LPCI: Torus/Suppression Pool Cooling Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	***Ability to (a) predict the impacts of the following on the* RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Inadequate net positive suction head		3.0	3.1	3.0	3.1
A2.02	Pumps trips		3.3	3.3	3.3	3.3
A2.03	Valve closures		3.1	3.2	3.1	3.2
A2.04	Valve openings		3.1	3.2	3.1	3.2
A2.05	A.C. electrical failures		3.3	3.5	3.3	3.5
A2.06	D.C. electrical failures		2.7*	2.9*	2.7	2.9
A2.07	Emergency generator failure		3.5	3.7	3.5	3.7
A2.08	Pump seal failure		2.8	3.1	2.8	3.1
A2.09	Inadequate room cooling		2.7	2.9	2.7	2.9
A2.10	Nuclear boiler instrument failures		3.1	3.2	3.1	3.2
A2.11	Motor operated valve failures		3.1	3.3	3.1	3.3
A2.12	Valve logic failure: Plant-Specific		3.0	3.1*	3.0	3.1
A2.13	High suppression pool temperature		3.5	3.7	3.5	3.7
A2.14	Loss of coolant accident		4.1	4.3*	4.1	4.3
A2.15	Loss of, or inadequate, heat exchanger cooling flow		3.3	3.4	3.3	3.4
A2.16	High suppression pool level		2.9	3.2	2.9	3.2
A3	**Ability to monitor automatic operations of the RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		3.3	3.3	3.3	3.3
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Pumps		3.8*	3.7*	3.8	3.7
A4.02	Valve lineup		3.7*	3.5	3.7	3.5
A4.03	Keep fill system		2.9	2.9	2.9	2.9
A4.04	Minimum flow valves		3.0	2.9	3.0	2.9
A4.05	Heat exchanger cooling flow		3.4	3.4	3.4	3.4
A4.06	Valve logic reset following automatic initiation of LPCI/RHR in injection mode		3.9	3.7	3.9	3.7
A4.07	System flow		3.5	3.4	3.5	3.4
A4.08	Pump/system discharge pressure: Plant-Specific		2.9	2.9	2.9	2.9
A4.09	Indicating lights and alarms		3.4	3.3	3.4	3.3
A4.10	Condensate storage tank level: Plant-Specific		2.2*	2.2*	2.2	2.2
A4.11	System venting		2.4	2.4	2.4	2.4
A4.12	Suppression pool temperature		4.1	4.1	4.1	4.1
A4.13	Suppression pool level		3.9	3.8	3.9	3.8
A4.14	The overrides for suppression pool cooling valve logic: Plant-Specific		3.7	3.5	3.7	3.5

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 219000

System Name:	**RHR/LPCI: Torus/Suppression Pool Cooling Mode**	**CFR**	**NRC Imp**		**Facility I**	
			RO	SRO	RO	S.

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 223001

System Name: Primary Containment System and Auxiliaries

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Containment isolation/integrity: Plant-Specific		3.7	3.9	3.7	3.9
K1.02	Drywell isolation/integrity: Plant-Specific		3.6	3.8	3.6	3.8
K1.03	Containment/drywell atmosphere control		3.2	3.3	3.2	3.3
K1.04	Drywell floor and equipment floor drain system		3.1	3.2	3.1	3.2
K1.05	Suppression pool makeup system: Mark-III		3.3	3.5	3.3	3.5
K1.06	RHR/LPCI		3.4	3.6	3.4	3.6
K1.07	Suppression pool cleanup system: Plant-Specific		2.4	2.5	2.4	2.5
K1.08	Relief/safety valves		3.6	3.8	3.6	3.8
K1.09	SBGT/FRVS: Plant-Specific		3.4	3.6	3.4	3.6
K1.10	Plant air systems		3.0	3.1	3.0	3.1
K1.11	Post accident sampling system		2.7	2.9	2.7	2.9
K1.12	LPCS: Plant-Specific		3.5	3.6	3.5	3.6
K1.13	HPCS: Plant-Specific		3.4	3.5	3.4	3.5
K1.14	RCIC: Plant-Specific		3.3	3.6	3.3	3.6
K1.15	~~HPCI: Plant-Specific~~		3.5	3.9	3.5	3.9
K1.16	Containment and drywell atmosphere monitoring: Plant-Specific		3.3	3.4	3.3	3.4
K1.17	~~Reactor building HVAC: Plant-Specific~~		3.1	3.1	3.1	3.1
K1.18	~~Drywell pneumatic compressors: Plant-Specific~~		2.6	2.8	2.6	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Atmosphere containment/ atmospheric dilution compressors: Plant-Specific		2.0*	2.1	2.0	2.1
K2.02	~~Drywell compressors~~		1.8*	1.9*	1.8	1.9
K2.03	~~Pumpback compressors: Plant-Specific~~		1.8*	1.8*	1.8	1.8
K2.04	Combustible gas mixing compressors: Mark-III		2.0*	2.4	2.0	2.4
K2.05	Hydrogen recombiners: Plant-Specific		2.2*	2.4*	2.2	2.4
K2.06	Hydrogen igniters: Plant-Specific		2.0*	2.4	2.0	2.4
K2.07	Containment atmosphere monitoring system		2.3*	2.6*	2.3	2.6
K2.08	Containment cooling air handling units: Plant-Specific		2.7	3.0*	2.7	3.0
K2.09	Drywell cooling fans: Plant-Specific		2.7	2.9*	2.7	2.9
K2.10	Drywell chillers: Plant-Specific		2.7	2.9*	2.7	2.9

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 223001

System Name: Primary Containment System and Auxiliaries

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K3	**Knowledge of the effect that a loss or malfunction of the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES will have on following:**	**(41.7 / 45.4)**				
K3.01	Secondary containment		3.6	3.8	3.6	3.8
K3.02	Containment/drywell temperature: Plant-Specific		3.3	3.4	3.3	3.4
K3.03	Containment/drywell pressure: Plant-Specific		3.4	3.5	3.4	3.5
K3.04	Containment/drywell hydrogen gas concentration		3.3	3.5	3.3	3.5
K3.05	Containment/drywell oxygen gas concentration		3.1	3.2	3.1	3.2
K3.06	Differential pressure between secondary and primary containment		3.3	3.6	3.3	3.6
K3.07	Differential pressure between suppression pool and drywell/containment		3.1	3.2	3.1	3.2
K3.08	Pneumatically operated valves internal to containment/drywell: Plant-Specific		2.7	2.9	2.7	2.9
K3.09	Nuclear boiler instrumentation		2.8	3.1	2.8	3.1
K3.10	~~Containment/drywell moisture content~~		2.4	2.6	2.4	2.6
K4	**Knowledge of PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Allows for absorption of the energy released during a LOCA		3.7	3.8	3.7	3.8
K4.02	Contains fission products after a LOCA		3.6	3.7	3.6	3.
K4.03	Containment/drywell isolation		3.7	3.8	3.7	3.8
K4.04	Prevents hydrogen from reaching an explosive mixture		3.5	3.8	3.5	3.8
K4.05	Maintains proper suppression pool to drywell differential pressure		2.9	3.1	2.9	3.1
K4.06	Maintains proper containment/secondary containment to drywell differential pressure		3.1	3.3	3.1	3.3
K4.07	Prevents localized heating of suppression pool (SRV steam quenchers)		3.1	3.3	3.1	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 223001

System Name: ***Primary Containment* System and Auxiliaries**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K5	**Knowledge of the operational implications of the following concepts as they apply to PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES:**	**(41.5 / 45.3)**				
K5.01	Vacuum breaker/relief operation		3.1	3.3	3.1	3.3
K5.02	Guard pipe operation: Mark-III		2.3	2.4	2.3	2.4
K5.03	Down comer operation		2.8	2.9	2.8	2.9
K5.04	Horizontal vent operation: Mark-III		2.4	2.6	2.4	2.6
K5.05	Hydrogen recombiner operation: Plant-Specific		2.8	3.0	2.8	3.0
K5.06	Hydrogen igniter operation: Plant-Specific		2.8	2.9	2.8	2.9
K5.07	Suppression pool clean-up: Plant-Specific		2.3	2.4	2.3	2.4
K5.08	Pressure measurement		2.7	2.8	2.7	2.8
K5.09	Hydrogen production mechanisms		2.6	2.9*	2.6	2.9
K5.10	Hydrogen combustibility versus hydrogen concentration and oxygen concentration		2.9	3.1	2.9	3.1
K5.11	Temperature measurement		2.7	2.7	2.7	2.7
K5.12	Hydrogen concentration measurement		2.7	2.8	2.7	2.8
K5.13	Oxygen concentration measurement: Plant-Specific		2.7	2.8	2.7	2.8
K5.14	Differential pressure measurement		2.6	2.8*	2.6	2.8
K5.15	~~Moisture content measurement: Plant-Specific~~		2.4	2.6	2.4	2.6
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES:**	**(41.7 / 45.7)**				
K6.01	Drywell cooling		3.6	3.8	3.6	3.8
K6.02	Containment cooling: Mark-III		3.5	3.6	3.5	3.6
K6.03	Suppression pool makeup: Plant-Specific		2.8	3.1	2.8	3.1
K6.04	Combustible gas mixing: Plant-Specific		2.8	2.8	2.8	2.8
K6.05	Hydrogen recombiner: Plant-Specific		3.1	3.3	3.1	3.3
K6.06	Backup hydrogen purge: Plant-Specific		2.8	3.0	2.8	3.0
K6.07	Hydrogen igniter system: Plant-Specific		3.0	3.1	3.0	3.1
K6.08	Containment atmospheric control		3.3	3.4	3.3	3.4
K6.09	Drywell vacuum relief system		3.4	3.6	3.4	3.6
K6.10	Containment vacuum relief system: Mark-III		3.0	3.2	3.0	3.2
K6.11	A.C. electrical distribution		3.0	3.2	3.0	3.2
K6.12	D.C. electrical distribution		2.7	3.0	2.7	3.0
K6.13	Applicable plant air system/ nitrogen make-up system.		3.2	3.4	3.2	3.4
K6.14	RHR/LPCI		3.6	3.8	3.6	3.8

Facility: CPS Printed: 08/11/2006

System Number: 223001

System Name: Primary Containment System and Auxiliaries

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
A1	**Ability to predict and/or monitor changes in parameters associated with operating the PRIMARY CONTAINMENT AND AUX. CONTROLS including:**	**(41.5 / 45.5)**				
A1.01	Drywell temperature		3.5	3.6	3.5	3.6
A1.02	Drywell pressure		3.6	3.7	3.6	3.7
A1.03	Containment pressure: Mark-III		3.6	3.8	3.6	3.8
A1.04	Containment temperature: Mark-III		3.3	3.4	3.3	3.4
A1.05	Hydrogen concentration		3.1	3.3	3.1	3.3
A1.06	Oxygen concentration		3.1	3.3	3.1	3.3
A1.07	Drywell/suppression chamber differential pressure (drywell to containment building): Plant-Specific		3.2	3.4	3.2	3.4
A1.08	Suppression pool level		3.5	3.6	3.5	3.6
A1.09	Suppression pool temperature		3.5	3.6	3.5	3.6
A1.10	Drywell leak detection system: Plant-Specific		3.4	3.6	3.4	3.6
A1.11	~~Reactor building to suppression chamber differential pressure: Plant-Specific~~		3.1	3.2	3.1	3.2
A1.12	~~Moisture concentration~~		2.5	2.6	2.5	2.6
A2	**Ability to (a) predict the impacts of the following on the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Loss of coolant accident		4.3*	4.4*	4.3	4.4
A2.02	Steam bypass of suppression pool		3.9	4.1	3.9	4.1
A2.03	Safety/relief valve leaking or stuck open		4.0	4.2*	4.0	4.2
A2.04	High containment/drywell hydrogen concentration		3.7	3.8	3.7	3.8
A2.05	High containment/drywell oxygen concentration		3.5	3.6	3.5	3.6
A2.06	High containment pressure: Mark-III		4.1*	4.1*	4.1	4.1
A2.07	High drywell pressure		4.2*	4.3*	4.2	4.3
A2.08	Compressor trips (loss of air): Plant-Specific		3.1	3.1	3.1	3.1
A2.09	Vacuum breaker malfunction		3.4	3.6	3.4	3.6
A2.10	High drywell temperature		3.6	3.8	3.6	3.8
A2.11	Abnormal suppression pool level		3.6	3.8	3.6	3.8
A2.12	Abnormal suppression pool temperature		3.7	3.8	3.7	3.8
A2.13	High containment temperature: Mark-III		3.3	3.4	3.3	3.4
A2.14	Low containment to annulus pressure: Mark-III		3.4	3.4	3.4	3.4

Facility: CPS Printed: 08/11/2006

System Number: 223001

System Name: Primary Containment System and Auxiliaries

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A3	**Ability to monitor automatic operations of the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES including:**	**(41.7 / 45.7)**				
A3.01	*Suppression pool level*		3.4	3.5	3.4	3.5
A3.02	Vacuum breaker/relief valve operation		3.4	3.4	3.4	3.4
A3.03	System indicating light and alarms		3.4	3.3	3.4	3.3
A3.04	Containment/drywell response during LOCA		4.2*	4.3*	4.2	4.3
A3.05	Drywell pressure		4.3*	4.3*	4.3	4.3
A3.06	~~Drywell/suppression chamber differential pressure: Mark-I,II~~		3.4	3.3	3.4	3.3
A3.07	Containment/drywell differential pressure: Mark-III		3.6	3.6	3.6	3.6
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Containment relief valves: Mark-III		3.5	3.5	3.5	3.5
A4.02	~~ACAD compressors: Plant-Specific~~		2.6	2.6	2.6	2.6
A4.03	Air dilution valves to drywell and suppression pool: Plant-Specific		3.4	3.4	3.4	3.4
A4.04	Containment/drywell hydrogen concentration		3.5	3.6	3.5	3.6
A4.05	Containment/drywell oxygen concentration		3.6	3.6	3.6	3.6
A4.06	Containment pressure: Mark-III		4.0*	4.0*	4.0	4.0
A4.07	Drywell pressure		4.2*	4.1	4.2	4.1
A4.08	System indicating lights and alarms		3.4	3.3	3.4	3.3
A4.09	SPDS/CRIDS/ERIS/GDS: Plant-Specific		2.5	2.9*	2.5	2.9
A4.10	~~Drywell nitrogen makeup: Mark-I,II~~		3.2	3.2	3.2	3.2
A4.11	Drywell pneumatics		3.1	3.0	3.1	3.0
A4.12	Drywell coolers/chillers		3.5	3.6	3.5	3.6
A4.13	Hydrogen recombiners: Plant-Specific		3.4	3.4	3.4	3.4
A4.14	Hydrogen igniters: Plant-Specific		3.7	3.7	3.7	3.7

KA Catalog

Facility: CPS *Printed:* 08/11/2006

System Number: 223002

System Name: Primary Containment Isolation System/Nuclear Steam Supply Shut-Off

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between PCIS/NSSSS and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Main steam system		3.8	3.9	3.8	3.9
K1.02	Reactor water cleanup		3.3	3.5	3.3	3.5
K1.03	Plant ventilation		3.0	3.2	3.0	3.2
K1.04	~~High pressure coolant injection: Plant-Specific~~		3.5	3.8	3.5	3.8
K1.05	~~Isolation condenser: Plant-Specific~~		3.8	3.8	3.8	3.8
K1.06	Recirculation system		2.9	3.2	2.9	3.2
K1.07	Reactor core isolation cooling; Plant-Specific		3.4	3.6	3.4	3.6
K1.08	Shutdown cooling system/RHR		3.4	3.5	3.4	3.5
K1.09	Reactor vessel head spray: Plant-Specific		3.0	3.2	3.0	3.2
K1.10	Containment ventilation		3.1	3.2	3.1	3.2
K1.11	Containment atmosphere sampling		2.9	3.2	2.9	3.2
K1.12	Standby gas treatment system		3.1	3.3	3.1	3.3
K1.13	~~Traversing in-core probe system~~		2.7	2.9	2.7	2.9
K1.14	Containment drainage system		2.8	3.1	2.8	3.1
K1.15	High pressure core spray : Plant-Specific		3.4	3.4	3.4	3.4
K1.16	Process computer		2.1*	2.2*	2.1	2.2
K1.17	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.4	2.6	2.4	2.6
K1.18	~~Reactor building drainage system: Plant-Specific~~		2.5	2.6	2.5	2.6
K1.19	Component cooling water systems		2.7	2.9	2.7	2.9
1.20	A.C. distribution: Plant-Specific		2.8	3.0	2.8	3.0
K1.21	Circulating water: Plant-Specific		2.2*	2.3*	2.2	2.3
K1.22	~~Containment nitrogen inerting system: Plant-Specific~~		2.8	3.0	2.8	3.0
K1.23	River water makeup: Plant-Specific		2.0*	2.1	2.0	2.1
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Logic power supplies		2.4*	2.7*	2.4	2.7

Facility: CPS **Printed:** 08/11/2006

System Number: 223002

System Name: Primary Containment Isolation System/Nuclear Steam Supply Shut-Off

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
K3	**Knowledge of the effect that a loss or malfunction of the PCIS/NSSSS will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor water level		3.7	3.7	3.7	3.7
K3.02	Fuel cladding temperature		3.6	3.7	3.6	3.7
K3.03	†Off-site radioactive release rates		3.6	3.8	3.6	3.8
K3.04	~~Reactor building radiation level~~		3.4	3.6	3.4	3.6
K3.05	Drainage sump levels		2.7	2.8	2.7	2.8
K3.06	Turbine building radiation		2.8	2.9	2.8	2.9
K3.07	Reactor pressure		3.7	3.8	3.7	3.8
K3.08	Reactor vessel temperature		3.4	3.5	3.4	3.5
K3.09	Main steam system		3.4	3.6	3.4	3.6
K3.10	Reactor water cleanup		2.9	3.1	2.9	3.1
K3.11	Plant ventilation		2.8	2.9	2.8	2.9
K3.12	~~High pressure coolant injection: Plant-Specific~~		3.6	3.6	3.6	3.6
K3.13	~~Isolation condenser: Plant-Specific~~		3.7	3.7	3.7	3.7
K3.14	Recirculation system: Plant-Specific		3.0	3.0	3.0	3.0
K3.15	Reactor core isolation cooling: Plant-Specific		3.4	3.5	3.4	3.5
K3.16	Shutdown cooling system/RHR		3.2	3.3	3.2	3.3
K3.17	Reactor vessel head spray: Plant-Specific		2.8	2.9	2.8	2.9
K3.18	Containment ventilation		3.0	3.1	3.0	3.1
K3.19	Containment atmosphere sampling		2.8	3.0	2.8	3.0
K3.20	Standby gas treatment system		3.3	3.4	3.3	3.4
K3.21	~~Traversing in-core probe system~~		2.6	2.7	2.6	2
K3.22	Containment drainage system		2.5	2.6	2.5	2.6
K3.23	High pressure core spray : Plant-Specific		3.6	3.6	3.6	3.6
K3.24	~~Reactor building drainage system~~		2.4	2.6	2.4	2.6
K3.25	Component cooling water systems		2.3	2.3	2.3	2.3
K3.26	A.C. distribution		2.1*	2.2*	2.1	2.2
K3.27	Circulating water		1.8*	1.8*	1.8	1.8
K3.28	~~Containment nitrogen inerting system~~		2.7	2.8	2.7	2.8
K3.29	River water makeup		1.8*	1.8*	1.8	1.8

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 223002

System Name: Primary Containment Isolation System/Nuclear Steam Supply Shut-Off

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K4	**Knowledge of PCIS/NSSSS design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Redundancy		3.0	3.2	3.0	3.2
K4.02	Testability		2.7	2.9	2.7	2.9
K4.03	Manual initiation capability: Plant-Specific		3.5	3.6	3.5	3.6
K4.04	†Automatic bypassing of selected isolations during specified plant conditions		3.2	3.6	3.2	3.6
K4.05	Single failures will not impair the function ability of the system		2.9	3.1	2.9	3.1
K4.06	Once initiated, system reset requires deliberate operator action		3.4	3.5	3.4	3.5
K4.07	Physical separation of system components (to prevent localized environmental factors, electrical faults, and physical events from impairing system response)		2.8	2.9	2.8	2.9
K4.08	†Manual defeating of selected isolations during specified emergency conditions		3.3	3.7	3.3	3.7
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the PCIS/NSSSS:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical distribution		3.1	3.3	3.1	3.3
K6.02	D.C. electrical distribution		3.0	3.2	3.0	3.2
K6.03	Process radiation monitoring system		2.9	3.1	2.9	3.1
K6.04	Nuclear boiler instrumentation		3.3	3.5	3.3	3.5
K6.05	Containment instrumentation		3.0	3.3	3.0	3.3
K6.06	Various process instrumentation		2.8	2.9	2.8	2.9
K6.07	Essential A.C. power		3.2	3.3	3.2	3.3
K6.08	Reactor protection system		3.5	3.7	3.5	3.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the PCIS/NSSSS controls including:**	**(41.5 / 45.5)**				
A1.01	System indicating lights and alarms		3.5	3.5	3.5	3.5
A1.02	Valve closures		3.7	3.7	3.7	3.7
A1.03	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.5*	2.8*	2.5	2.8
A1.04	Individual system relay status		2.6	2.8	2.6	2.8

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 223002

System Name: Primary Containment Isolation System/Nuclear Steam Supply Shut-Off

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S...
A2	**Ability to (a) predict the impacts of the following on the PCIS/NSSSS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	A.C. electrical distribution failures		3.2	3.5	3.2	3.5
A2.02	D.C. electrical distribution failures		2.9	3.2	2.9	3.2
A2.03	System logic failures		3.0	3.3	3.0	3.3
A2.04	Process radiation monitoring system failures		2.9	3.2	2.9	3.2
A2.05	Nuclear boiler instrumentation failures		3.3	3.6	3.3	3.6
A2.06	Containment instrumentation failures		3.0	3.2	3.0	3.2
A2.07	Various process instrumentation failures		2.7	2.9	2.7	2.9
A2.08	†Surveillance testing		2.7	3.1	2.7	3.1
A2.09	System initiation		3.6	3.7	3.6	3.7
A2.10	Loss of coolant accidents		3.9	4.2	3.9	4.2
A2.11	†Standby liquid initiation		3.8	3.9	3.8	3.9
A3	**Ability to monitor automatic operations of the PCIS/NSSSS including:**	**(41.7 / 45.7)**				
A3.01	System indicating lights and alarms		3.4	3.4	3.4	3.4
A3.02	Valve closures		3.5	3.5	3.5	3.5
A3.03	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.5*	2.8*	2.5	2.8
A3.04	Verification of relay operation: Plant-Specific		2.3*	2.5	2.3	
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Valve closures		3.6	3.5	3.6	3.5
A4.02	Manually initiate the system		3.9	3.8	3.9	3.8
A4.03	Reset system isolations		3.6	3.5	3.6	3.5
A4.04	System indicating lights and alarms		3.5	3.6	3.5	3.6
A4.05	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.5*	2.8*	2.5	2.8
A4.06	Confirm initiation to completion		3.6*	3.7	3.6	3.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 226001

System Name: RHR/LPCI: Containment Spray System Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Suppression pool		3.4	3.6	3.4	3.6
K1.02	LPCI/RHR piping		3.5	3.7	3.5	3.7
K1.03	LPCI/RHR pumps		3.5	3.6	3.5	3.6
K1.04	A.C. electrical power		3.1	3.3	3.1	3.3
K1.05	Keep fill system		2.9	2.9	2.9	2.9
K1.06	~~Condensate transfer~~		2.5	2.5	2.5	2.5
K1.07	D.C. electrical power		2.4	2.5	2.4	2.5
K1.08	Nuclear boiler instrumentation		3.2	3.4	3.2	3.4
K1.09	~~Drywell (spray penetration): Mark-I-II~~		3.0	3.1	3.0	3.1
K1.10	Containment (spray penetration): Mark-III		3.0	3.0	3.0	3.0
K1.11	Component cooling water systems		2.8	3.0	2.8	3.0
K1.12	~~Suppression pool (spray penetration): Plant-Specific~~		3.0	3.0	3.0	3.0
K1.13	Containment instrumentation		3.1	3.2	3.1	3.2
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Valves		2.1*	2.3*	2.1	2.3
K2.02	Pumps		2.9*	2.9*	2.9	2.9
K2.03	Valve control logic		2.4*	2.5*	2.4	2.5
K3	**Knowledge of the effect that a loss or malfunction of the RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE will have on following:**	**(41.7 / 45.4)**				
K3.01	Containment/drywell/suppression chamber pressure		3.6	3.7	3.6	3.7
K3.02	Containment/drywell/suppression chamber temperature		3.5	3.5	3.5	3.5
K3.03	Containment/drywell/suppression chamber components, continued operation with elevated pressure and/or temperature and/or level		2.9	3.2	2.9	3.2

Facility: CPS Printed: 08/11/2006

System Number: 226001

System Name: RHR/LPCI: Containment Spray System Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp St.
K4	**Knowledge of RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Testability of all operable components		2.6	2.8	2.6	2.8
K4.02	Redundancy		2.8	2.9	2.8	2.9
K4.03	Reduction in vessel injection flow during accident conditions		2.9	3.1	2.9	3.1
K4.04	Prevention of piping overpressurization		2.4	2.6	2.4	2.6
K4.05	Pump minimum flow protection		2.5	2.5	2.5	2.5
K4.06	Pump motor cooling		2.2*	2.3	2.2	2.3
K4.07	Prevention of water hammer		2.6	2.8	2.6	2.8
K4.08	Adequate pump net positive suction head		2.3	2.4	2.3	2.4
K4.09	Automatic containment spray initiation: BWR-6		3.2	3.4	3.2	3.4
K4.10	Spray flow cooling		2.9	3.0	2.9	3.0
K4.11	Prevention of leakage to the environment through system heat exchanger		2.7	2.9	2.7	2.9
K4.12	Prevention of inadvertent containment spray activation		2.9	2.9	2.9	2.9
K5	**Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE:**	**(41.5 / 45.3)**				
K5.01	System venting		2.2*	2.2*	2.2	2.2
K5.02	Water hammer		2.6	2.7	2.6	2.7
K5.03	Pump cavitation		2.3	2.5	2.3	2.5
K5.04	Evaporative cooling		2.1*	2.1*	2.1	2.1
K5.05	Convective cooling		2.1*	2.1*	2.1	2.1
K5.06	Vacuum breaker operation		2.6	2.8	2.6	2.8
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical power		3.0	3.3	3.0	3.3
K6.02	D.C. electrical power		2.6	2.9	2.6	2.9
K6.03	Emergency generator		3.4	3.6	3.4	3.6
K6.04	Keep fill system		2.7	2.7	2.7	2.7
K6.05	Suppression pool (temperature level and pressure)		3.4	3.6	3.4	3.6
K6.06	~~Condensate transfer~~		2.4	2.4	2.4	2.4
K6.07	ECCS room cooling		2.4	2.6	2.4	2.6
K6.08	Nuclear boiler instrumentation		2.7	2.8	2.7	2.8
K6.09	~~†Reactor building to suppression chamber vacuum breakers: Plant-Specific~~		3.3	3.5	3.3	3.5
K6.10	~~†Suppression chamber to drywell vacuum breakers: Mark-1-II~~		3.3	3.5	3.3	3.5
K6.11	Component cooling water systems		2.8	2.8	2.8	2.8
K6.12	Containment integrity		3.4	3.5	3.4	3.5
K6.13	Suction flow path		3.2	3.2	3.2	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 226001

System Name: RHR/LPCI: Containment Spray System Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE controls including:**	**(41.5 / 45.5)**				
A1.01	Containment/drywell pressure		3.6	3.8	3.6	3.8
A1.02	Containment/drywell temperature		3.4	3.5	3.4	3.5
A1.03	~~†Suppression chamber pressure: Mark I-II~~		3.5	3.8	3.5	3.8
A1.04	~~Suppression pool temperature: Mark I-II~~		3.3	3.6	3.3	3.6
A1.05	System lineup		3.1	3.4	3.1	3.4
A1.06	System flow		3.2	3.2	3.2	3.2
A1.07	System pressure		3.1	3.1	3.1	3.1
A1.08	Suppression pool level		3.1	3.4*	3.1	3.4
A1.09	Motor amps		2.1*	2.1	2.1	2.1
A1.10	Emergency generator loading		3.0	3.2	3.0	3.2
A2	**Ability to (a) predict the impacts of the following on the RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Inadequate net positive suction head		2.4	2.6	2.4	2.6
2.02	Pumps trips		3.1	3.2	3.1	3.2
A2.03	Valve closures		3.1	3.1	3.1	3.1
A2.04	Valve openings		3.0	3.0	3.0	3.0
A2.05	A.C. electrical failures		3.3	3.4	3.3	3.4
A2.06	D.C. electrical failures		2.8	2.9	2.8	2.9
A2.07	Emergency generator failure		3.4	3.6	3.4	3.6
A2.08	Pump seal failure		2.4	2.5	2.4	2.5
A2.09	Inadequate room cooling		2.6	2.7	2.6	2.7
A2.10	Nuclear boiler instrument failures		3.0	3.1	3.0	3.1
A2.11	Motor operated valve failures		3.0	3.0	3.0	3.0
A2.12	Pump runout		2.5	2.6	2.5	2.6
A2.13	Valve logic failure		2.8	2.9	2.8	2.9
A2.14	High suppression pool level		2.9	3.1	2.9	3.1
A2.15	High containment / drywell pressure		3.6	3.8	3.6	3.8
A2.16	Loss of, or inadequate heat exchanger cooling flow		3.1	3.1	3.1	3.1
A2.17	†High containment / drywell temperature		3.2	3.2	3.2	3.2
A2.18	†Loss of motor cooling		2.9	3.1	2.9	3.1
A2.19	~~†Low (or negative) suppression chamber pressure during system operation: Mark I-II~~		3.5	3.8	3.5	3.8
A2.20	†Loss of coolant accident		3.7	4.1	3.7	4.1
A2.21	Loss of containment/drywell cooling system(s)		3.3	3.4	3.3	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 226001

System Name: RHR/LPCI: Containment Spray System Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
A3	**Ability to monitor automatic operations of the RHR/LPCI: CONTAINMENT SPRAY SYSTEM MODE including:**	**(41.7 / 45.7)**				
A3.01	Breaker tripping		3.1	3.2	3.1	3.2
A3.02	System pressure		2.8	2.8	2.8	2.8
A3.03	Load shedding		3.4	3.5	3.4	3.5
A3.04	Lights and alarms		3.1	3.1	3.1	3.1
A3.05	Containment pressure		4.0*	4.0*	4.0	4.0
A3.06	Containment temperature		3.5	3.5	3.5	3.5
A3.07	Pump start		3.5	3.5	3.5	3.5
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Pumps		3.5	3.4	3.5	3.4
A4.02	Suction valves		3.1	3.1	3.1	3.1
A4.03	Spray valves		3.5	3.4	3.5	3.4
A4.04	Keep fill system		2.8	2.7	2.8	2.7
A4.05	Minimum flow valves		2.9	2.8	2.9	2.8
A4.06	Heat exchanger cooling flow		2.9	2.8	2.9	2.8
A4.07	Valve logic reset/ bypass/ override		3.5	3.5	3.5	3.5
A4.08	System flow		3.2	3.1	3.2	3.1
A4.09	Pump discharge pressure		2.8	2.7	2.8	2
A4.10	Indicating lights and alarms		3.3	3.2	3.3	3.2
A4.11	System venting		2.2*	2.2*	2.2	2.2
A4.12	Containment/drywell pressure		3.8	3.8	3.8	3.8
A4.13	Containment/drywell temperature		3.3	3.3	3.3	3.3
A4.14	Suppression pool temperature		3.3	3.6	3.3	3.6
A4.15	~~Suppression chamber pressure: Mark I-II~~		3.6	3.6	3.6	3.6
A4.16	The override for suppression pool spray valve logic		3.5	3.5	3.5	3.5
A4.17	Manual initiation controls: BWR-6		3.8	3.8	3.8	3.8
A4.18	Automatic system initiation reset: BWR-6		3.8	3.8	3.8	3.8
A4.19	Drywell temperature		3.4	3.4	3.4	3.4
A4.20	Drywell pressure		3.9	3.8	3.9	3.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 230000

System Name: RHR/LPCI: Torus/Suppression Pool Spray Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Suppression pool~~		3.6	3.7	3.6	3.7
K1.02	~~Condensate storage and transfer system~~		2.4*	2.5	2.4	2.5
K1.03	~~LPCI/RHR piping~~		3.3	3.5	3.3	3.5
K1.04	~~LPCI/RHR pumps~~		3.4	3.6	3.4	3.6
K1.05	~~A.C. electrical~~		3.2	3.3	3.2	3.3
K1.06	~~Keep fill system~~		3.0	3.1	3.0	3.1
K1.07	~~D.C. electrical~~		2.4*	2.6*	2.4	2.6
K1.08	~~Nuclear boiler instrumentation~~		3.1	3.2	3.1	3.2
K1.09	~~Reactor building drain system~~		2.3*	2.4*	2.3	2.4
K1.10	~~Component cooling water systems~~		2.8	2.8	2.8	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~Valves~~		2.1*	2.3*	2.1	2.3
K2.02	~~Pumps~~		2.8*	2.9*	2.8	2.9
K3	**Knowledge of the effect that a loss or malfunction of the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE will have on following:**	**(41.7 / 45.4)**				
K3.01	~~Suppression chamber pressure~~		3.7	3.9	3.7	3.9
K3.02	~~Suppression pool temperature~~		3.3	3.5	3.3	3.5
K3.03	~~Drywell pressure~~		3.4	3.6	3.4	3.6
K3.04	~~Suppression chamber air temperature~~		3.7	3.8	3.7	3.8
K4	**Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Surveillance for all operable components~~		3.1	3.3	3.1	3.3
K4.02	~~Redundancy~~		3.1*	3.2	3.1	3.2
K4.03	~~Unintentional reduction in vessel injection flow during accident conditions~~		3.5	3.6	3.5	3.6
K4.04	~~Prevention of piping overpressurization~~		3.0	3.2	3.0	3.2
K4.05	~~Pump minimum flow protection~~		2.8	3.1	2.8	3.1
K4.06	~~Pump motor cooling~~		2.7	2.8	2.7	2.8
K4.07	~~Prevention of water hammer~~		3.1	3.2	3.1	3.2
K4.08	~~Adequate pump net positive suction head~~		2.9	3.2	2.9	3.2
K4.09	~~Spray flow cooling~~		3.0	3.1	3.0	3.1
K4.10	~~Prevention of leakage to the environment through system heat exchanger~~		3.2	3.5	3.2	3.5

Facility: CPS Printed: 08/11/2006

System Number: 230000

System Name: RHR/LPCI: Torus/Suppression Pool Spray Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
K5	**Knowledge of the operational implications of the following concepts as they apply to RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:**	**(41.5 / 45.3)**				
K5.01	~~System venting~~		2.6	2.7	2.6	2.7
K5.02	~~Pump cavitation~~		2.4*	2.5	2.4	2.5
K5.03	~~Pressure measurement~~		2.5*	2.6*	2.5	2.6
K5.04	~~Evaporative cooling~~		2.5*	2.5*	2.5	2.5
K5.05	~~Convective cooling~~		2.6*	2.6*	2.6	2.6
K5.06	~~Heat exchanger operation~~		2.5*	2.6	2.5	2.6
K5.07	~~Vacuum breaker operation~~		2.9*	3.1	2.9	3.1
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE:**	**(41.7 / 45.7)**				
K6.01	~~A.C. electrical~~		3.3	3.4	3.3	3.4
K6.02	~~D.C. electrical~~		2.5*	2.8*	2.5	2.8
K6.03	~~Emergency generator~~		3.5	3.6	3.5	3.6
K6.04	~~Keep fill system~~		2.8	2.8	2.8	2.8
K6.05	~~Suppression pool~~		3.3	3.4	3.3	3.4
K6.06	~~Condensate storage and transfer system~~		2.3*	2.4*	2.3	2.4
K6.07	~~ECCS room cooling~~		2.8	3.0	2.8	3.0
K6.08	~~Nuclear boiler instrumentation~~		2.9	3.1	2.9	3.1
K6.09	~~Reactor building to suppression pool vacuum breakers~~		3.5	3.8	3.5	3.8
K6.10	~~Component cooling water systems~~		2.5	2.6	2.5	2.6
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE controls including:**	**(41.5 / 45.5)**				
A1.01	~~Suppression chamber pressure~~		3.8	3.9	3.8	3.9
A1.02	~~Suppression pool temperature~~		3.7	3.8	3.7	3.8
A1.03	~~Drywell pressure~~		3.6	3.8	3.6	3.8
A1.04	~~System flow~~		3.2*	3.3	3.2	3.3
A1.05	~~System pressure~~		2.8*	2.9	2.8	2.9
A1.06	~~Suppression pool level~~		3.3	3.3	3.3	3.3
A1.07	~~Condensate storage tank level~~		2.2*	2.3*	2.2	2.3
A1.08	~~Motor amps~~		2.4*	2.5*	2.4	2.5
A1.09	~~Emergency generator loading~~		3.3	3.5	3.3	3.5
A1.10	~~System lineup~~		3.7	3.7	3.7	3.7
A1.11	~~Suppression chamber air temperature~~		3.6	3.6	3.6	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 230000

System Name: RHR/LPCI: Torus/Suppression Pool Spray Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY MODE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Inadequate net positive suction head~~		3.0	3.2	3.0	3.2
A2.02	~~Pump trips~~		3.3	3.4	3.3	3.4
A2.03	~~Valve closures~~		2.9	3.2	2.9	3.2
A2.04	~~Valve openings~~		2.8	3.1	2.8	3.1
A2.05	~~A.C. electrical failures~~		3.3	3.6	3.3	3.6
A2.06	~~D.C. electrical failures~~		2.6*	2.9*	2.6	2.9
A2.07	~~Emergency generator failure~~		3.5	3.8	3.5	3.8
A2.08	~~Pump seal failure~~		2.8	3.2	2.8	3.2
A2.09	~~Inadequate room cooling~~		2.8	3.0	2.8	3.0
A2.10	~~Nuclear boiler instrument failures~~		2.8	3.0	2.8	3.0
A2.11	~~Motor operated valve failures~~		3.1	3.3	3.1	3.3
A2.12	~~Valve logic failure~~		3.2	3.3	3.2	3.3
A2.13	~~High suppression pool level~~		2.9	3.2	2.9	3.2
A2.14	~~Low (or negative) suppression pool pressure during system operation~~		3.2	3.5	3.2	3.5
A2.15	~~Loss of coolant accident~~		4.0	4.1	4.0	4.1
A2.16	~~Loss of, or inadequate, heat exchanger cooling flow~~		3.1	3.3	3.1	3.3
A3	**Ability to monitor automatic operations of the RHR/LPCI: TORUS/SUPPRESSION POOL SPRAY *MODE including:***	**(41.7 / 45.7)**				
A3.01	~~Valve operation~~		3.4	3.3	3.4	3.3

Facility: CPS Printed: 08/11/2006

System Number: 230000

System Name: RHR/LPCI: Torus/Suppression Pool Spray Mode

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	~~Pumps~~		3.7*	3.5	3.7	3.5
A4.02	~~Spray valves~~		3.8	3.6	3.8	3.6
A4.03	~~Keep fill system~~		3.1	3.0	3.1	3.0
A4.04	~~Minimum flow valves~~		3.1	2.9	3.1	2.9
A4.05	~~Heat exchanger cooling flow~~		3.2	3.1	3.2	3.1
A4.06	~~Valve logic reset following automatic initiation of LPCI/RHR in injection mode~~		4.0	3.9	4.0	3.9
A4.07	~~System flow~~		3.6	3.4	3.6	3.4
A4.08	~~Pump/system discharge pressure~~		3.0	2.9	3.0	2.9
A4.09	~~Indicating lights and alarms~~		3.6	3.3	3.6	3.3
A4.10	~~Condensate storage tank level~~		2.4*	2.4	2.4	2.4
A4.11	~~System venting~~		2.5	2.4	2.5	2.4
A4.12	~~Suppression pool level~~		3.8	3.8	3.8	3.8
A4.13	~~Suppression chamber pressure~~		4.0	3.9	4.0	3.9
A4.14	~~Suppression pool temperature~~		3.8	3.8	3.8	3.8
A4.15	~~Drywell pressure~~		3.9	4.0	3.9	4.0
A4.16	~~The override for suppression pool spray valve logic~~		3.8	3.8	3.8	3.8

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 233000

System Name: Fuel Pool Cooling and Clean-up

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between FUEL POOL COOLING AND CLEAN-UP and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Shutdown cooling system: Plant-Specific		2.6	2.9	2.6	2.9
K1.02	Residual heat removal system: Plant-Specific		2.9	3.0	2.9	3.0
K1.03	Condensate storage tank		2.3	2.3	2.3	2.3
K1.04	Process sampling system		2.0*	2.0*	2.0	2.0
K1.05	Plant air systems		2.2	2.2	2.2	2.2
K1.06	A.C. electrical power		2.3	2.3	2.3	2.3
K1.07	~~Condensate system: Plant-Specific~~		2.1	2.2	2.1	2.2
K1.08	Condensate transfer		2.5	2.5	2.5	2.5
K1.09	Component cooling water systems		2.6	2.6	2.6	2.6
K1.10	Containment drainage system: Plant-Specific		2.2	2.3	2.2	2.3
K1.11	~~Reactor building drainage system: Plant-Specific~~		2.3	2.4	2.3	2.4
K1.12	Radwaste system		2.5	2.6	2.5	2.6
K1.13	Suppression pool cleanup system: Plant-Specific		2.3	2.6	2.3	2.6
K1.14	Reactor building ventilation		2.5	2.5	2.5	2.5
K1.15	Storage pools		2.9	2.9	2.9	2.9
K1.16	Emergency cooling water systems: Plant-Specific		2.7	2.8	2.7	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Fuel pool cooling pumps		2.1*	2.2*	2.1	2.2
K2.02	RHR pumps		2.8*	2.9*	2.8	2.9
K3	**Knowledge of the effect that a loss or malfunction of the FUEL POOL COOLING AND CLEAN-UP will have on following:**	**(41.7 / 45.4)**				
K3.01	Fuel pool temperature		3.2	3.4	3.2	3.4
K3.02	Fuel pool water level		3.1	3.2	3.1	3.2
K3.03	Fuel pool water clarity		2.6	2.8	2.6	2.8
K3.04	Fuel pool water chemistry		2.4*	2.6	2.4	2.6
K3.05	Fuel pool water fission product concentration		2.6	2.8	2.6	2.8
K3.06	Area radiation levels		2.9	3.2	2.9	3.2
K3.07	Suppression pool chemistry: Plant-Specific		2.3	2.9	2.3	2.9
K3.08	†Refueling operations		2.9	3.5	2.9	3.5

System Number: 233000

System Name: Fuel Pool Cooling and Clean-up

		CFR	NRC Imp RO	NRC Imp SRO	Facility In RO	Facility In S
K4	**Knowledge of FUEL POOL COOLING AND CLEAN-UP design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Redundancy		2.4	2.6	2.4	2.6
K4.02	Pool clarity		2.4	2.6	2.4	2.6
K4.03	Maintenance of adequate pool temperature		2.8	3.1	2.8	3.1
K4.04	Overpressure protection for fuel pool cooling system filter		2.0*	2.1	2.0	2.1
K4.05	Net positive suction head requirements for fuel pool cooling pumps		2.1*	2.2*	2.1	2.2
K4.06	Maintenance of adequate pool level		2.9	3.2	2.9	3.2
K4.07	Supplemental heat removal capability		2.7	2.9	2.7	2.9
K4.08	Pool cooling during loss of coolant accident: BWR-6		2.6*	2.8	2.6	2.8
K4.09	Maintenance of filter/demineralizer precoat during low flow conditions		2.1	2.4*	2.1	2.4
K5	**Knowledge of the operational implications of the following concepts as they apply to FUEL POOL COOLING AND CLEAN-UP:**	**(41.5 / 45.3)**				
K5.01	Heat removal mechanisms		2.5	2.7	2.5	2.7
K5.02	Pump cavitation		2.1*	2.2*	2.1	2.2
K5.03	Spent fuel decay heat generation		2.6	2.8*	2.6	2.8
K5.04	Demineralizer ion exchange		2.0*	2.2*	2.0	2.2
K5.05	Mechanical filtration operation		2.1*	2.2	2.1	2.2
K5.06	Maximum normal heat load		2.5	2.7	2.5	2.7
K5.07	Maximum (abnormal) heat 102d		2.5	2.8	2.5	2.8
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the FUEL POOL COOLING AND CLEAN-UP:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical power		2.5	2.7	2.5	2.7
K6.02	Shutdown cooling system: Plant-Specific		2.2*	2.5*	2.2	2.5
K6.03	Residual heat removal: Plant-Specific		2.3	2.7	2.3	2.7
K6.04	Condensate transfer		2.5	2.6	2.5	2.6
K6.05	~~Condensate system~~		2.1*	2.3	2.1	2.3
K6.06	Condensate storage tanks		2.4	2.4	2.4	2.4
K6.07	Component cooling water systems		2.7	2.8	2.7	2.8
K6.08	Plant air systems		2.4	2.4	2.4	2.4
K6.09	Radwaste system		2.2	2.4	2.2	2.4
K6.10	Reactor cavity seal failure		2.9	3.3	2.9	3.3
K6.11	NSSSS/PCIS: Plant-Specific		2.3	2.6*	2.3	2.6

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 233000

System Name: Fuel Pool Cooling and Clean-up

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the FUEL POOL COOLING AND CLEAN-UP controls including:**	**(41.5 / 45.5)**				
A1.01	Surge tank level		2.6	2.9	2.6	2.9
A1.02	Pool level		2.9	3.1	2.9	3.1
A1.03	Pool temperature		3.1	3.3	3.1	3.3
A1.04	Pump discharge pressure		2.4	2.5	2.4	2.5
A1.05	Filter/ demineralize differential pressure		2.4*	2.3*	2.4	2.3
A1.06	System flow		2.5	2.4	2.5	2.4
A1.07	System temperature		2.7	2.8	2.7	2.8
A1.08	Pool chemistry		2.1*	2.4*	2.1	2.4
A1.09	Pool clarity		2.4	2.6	2.4	2.6
A1.10	Pool activity levels		2.4	2.6*	2.4	2.6
A1.11	Suppression pool chemistry: BWR-6		2.4	2.9	2.4	2.9
A2	**Ability to (a) predict the impacts of the following on the FUEL POOL COOLING AND CLEAN-UP; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	High pool level		2.7	2.9	2.7	2.9
A2.02	Low pool level		3.1	3.3	3.1	3.3
A2.03	Low surge tank level/high level		2.8	3.0	2.8	3.0
A2.04	Pump trip		2.6	2.7	2.6	2.7
A2.05	Valve closures		2.5	2.5	2.5	2.5
A2.06	Valve openings		2.5	2.5	2.5	2.5
A2.07	High fuel pool temperature		3.0	3.2	3.0	3.2
A2.08	Closed cooling water failure		2.9	3.1	2.9	3.1
A2.09	A.C. electrical power failures		2.7	2.9	2.7	2.9
A2.10	Refueling bellows seal high flow		2.9	3.3	2.9	3.3
A2.11	Fuel pool gate seal high flow		2.9	3.2	2.9	3.2
A2.12	High filter/ demineralizer differential pressure		2.4*	2.4	2.4	2.4
A2.13	Low filter/ demineralizer differential pressure		2.2*	2.2*	2.2	2.2
A2.14	Low system flow		2.3	2.3	2.3	2.3
A2.15	High system temperature		2.8	2.9	2.8	2.9
A2.16	Loss of coolant accident signal		2.5	2.8	2.5	2.8
A2.17	Fuel transfer tube drain tank high level/low level: BWR-6		2.0	2.8	2.0	2.8
A2.18	Low pool clarity		2.5	2.7	2.5	2.7
A2.19	Inadequate system/pool chemistry		2.5	2.8	2.5	2.8
A3	**Ability to monitor automatic operations of the FUEL POOL COOLING AND CLEAN-UP including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		2.4	2.5	2.4	2.5
A3.02	Pump trip(s)		2.6	2.6	2.6	2.6
A3.03	System indicating lights and alarms		2.6	2.6	2.6	2.6

Facility: CPS Printed: 08/11/2006

System Number: 233000

System Name: Fuel Pool Cooling and Clean-up

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	System lights and alarms		2.6	2.6	2.6	2.6
A4.02	System lineups		2.3	2.5	2.3	2.5
A4.03	System flow		2.2	2.4	2.2	2.4
A4.04	Pool level		2.9*	3.1	2.9	3.1
A4.05	Pool temperature		2.7*	3.1*	2.7	3.1
A4.06	System temperature		2.5*	2.6*	2.5	2.6
A4.07	System pressures		2.2*	2.3*	2.2	2.3
A4.08	System differential pressures		2.2*	2.2*	2.2	2.2
A4.09	Pump operation		2.2*	2.2*	2.2	2.2
A4.10	Tank levels		2.5	2.6	2.5	2.6
A4.11	Closed cooling water temperature		2.5	2.5	2.5	2.5

Facility: CPS Printed: 08/11/2006

System Number: 234000

System Name: Fuel Handling Equipment

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between FUEL HANDLING EQUIPMENT and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	†Fuel		3.2	3.7	3.2	3.7
K1.02	†Core components		2.9	3.3	2.9	3.3
K1.03	Spent fuel cask		2.7*	2.9	2.7	2.9
K1.04	†Reactor manual control system: Plant-Specific		3.3	3.6	3.3	3.6
K1.05	Reactor vessel components: Plant-Specific		2.9	3.3	2.9	3.3
K1.06	RC & IS: Plant-Specific		3.0	3.2	3.0	3.2
K1.07	Fuel transfer tube system: Mark-III		3.0	3.4	3.0	3.4
K1.08	Fuel pools configuration: Mark-III		2.7	3.0	2.7	3.0
K1.09	Fuel pool ventilation: Plant-Specific		2.8	2.9	2.8	2.9
K3	**Knowledge of the effect that a loss or malfunction of the FUEL HANDLING EQUIPMENT will have on following:**	**(41.7 / 45.4)**				
K3.01	†Reactor manual control system: Plant-Specific		2.9	3.3	2.9	3.3
K3.02	RC & IS: Plant-Specific		3.0	3.0	3.0	3.0
K3.03	†Fuel handling operations		3.1	3.8	3.1	3.8
K3.04	†core modifications/alterations		2.9	3.8	2.9	3.8
K4	**Knowledge of FUEL HANDLING EQUIPMENT design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	†Prevention of core alterations during control rod movements		3.3	4.1	3.3	4.1
K4.02	†Prevention of control rod movement during core alterations		3.3	4.1	3.3	4.1
K4.03	†Protection against inadvertently lifting radioactive components out of the water		3.4	4.2	3.4	4.2
K4.04	†Movement of the spent fuel cask only over designated areas: Plant-Specific		2.8	3.3	2.8	3.3
K4.05	†Movement of fuel via fuel transfer tube: Mark-III		3.0	3.8	3.0	3.8
K5	**Knowledge of the operational implications of the following concepts as they apply to FUEL HANDLING EQUIPMENT:**	**(41.5 / 45.3)**				
K5.01	†Crane/hoist operation		2.9	3.4	2.9	3.4
K5.02	†Fuel handling equipment interlocks		3.1	3.7	3.1	3.7
K5.03	†Water as a shield against radiation		2.9	3.4	2.9	3.4
K5.04	Spent fuel pool design		2.6	3.1	2.6	3.1
K5.05	†Fuel orientation		3.0	3.7	3.0	3.7

Facility: CPS Printed: 08/11/2006

System Number: 234000

System Name: Fuel Handling Equipment

			NRC Imp		Facility Im	
		CFR	RO	SRO	RO	SI
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the FUEL HANDLING EQUIPMENT**	**(41.7 / 45.7)**				
K6.01	†Electrical power		2.7	3.2	2.7	3.2
K6.02	Reactor manual control system: Plant-Specific		2.8	3.5	2.8	3.5
K6.03	RC & IS: Plant-Specific		3.0	3.6	3.0	3.6
K6.04	†Refueling platform air system: Plant-Specific		2.9	3.7	2.9	3.7
K6.05	Upper fuel pool water inventory: Mark-III		3.2	3.3	3.2	3.3
K6.06	Fuel transfer tube interlocks: Mark-III		3.0	3.6	3.0	3.6
K6.07	†Fuel pool ventilation: Plant-Specific		2.9	3.4	2.9	3.4
A1	**Ability to predict and/or monitor changes in parameters associated with operating the FUEL HANDLING EQUIPMENT controls including:**	**(41.5 / 45.5)**				
A1.01	Spent fuel pool level		3.1	3.4	3.1	3.4
A1.02	†Refuel floor radiation levels/ airborne levels		3.3	3.8	3.3	3.8
A1.03	†core reactivity level		3.4	3.9	3.4	3.9
A2	**Ability to (a) predict the impacts of the following on the FUEL HANDLING EQUIPMENT; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	†Interlock failure		3.3	3.7	3.3	3.7
A2.02	†Loss of refueling platform air system		3.1	3.6	3.1	3.6
A2.03	†Loss of electrical power		2.8	3.1	2.8	3.1
A3	**Ability to monitor automatic operations of the FUEL HANDLING EQUIPMENT including:**	**(41.7 / 45.7)**				
A3.01	†Crane/refuel bridge movement: Plant-Specific		2.6	3.6	2.6	3.6
A3.02	†Interlock operation		3.1	3.7	3.1	3.7
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	†Neutron monitoring system		3.7	3.9	3.7	3.9
A4.02	Control rod drive system		3.4	3.7	3.4	3.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 239001

System Name: Main and Reheat Steam System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between MAIN AND REHEAT STEAM SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Reactor vessel		3.4	3.4	3.4	3.4
K1.02	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.6	2.8	2.6	2.8
K1.03	Main turbine		3.4	3.4	3.4	3.4
K1.04	Moisture separators		2.8	2.9	2.8	2.9
K1.05	Moisture separator reheaters: Plant-Specific		2.8	2.8	2.8	2.8
K1.06	Steam bypass system		3.4	3.4	3.4	3.4
K1.07	Offgas system		3.1	3.0	3.1	3.0
K1.08	Condenser air removal system		2.9	3.1	2.9	3.1
K1.09	Steam seal/gland seal system		2.7	2.8	2.7	2.8
K1.10	Extraction steam system		2.7	2.8	2.7	2.8
K1.11	High pressure heater drains and vents		2.5	2.5	2.5	2.5
K1.12	Plant air systems		2.5	2.6	2.5	2.6
K1.13	Main steam isolation valve leakage control: Plant-Specific		2.6	2.8	2.6	2.8
K1.14	Positive leakage control system: Plant Specific		2.8	3.1	2.8	3.1
K1.15	Process computer/ performance monitor system		2.1*	2.1	2.1	2.1
K1.16	Process radiation monitoring system		3.2	3.4	3.2	3.4
K1.17	Containment system		3.1	3.2	3.1	3.2
K1.18	~~High pressure coolant injection: Plant-Specific~~		3.5	3.6	3.5	3.6
K1.19	Reactor core isolation cooling system: Plant-Specific		3.1	3.2	3.1	3.2
1.20	Residual heat removal system: Plant-Specific		2.9	2.9	2.9	2.9
K1.21	~~Isolation condenser system: Plant-Specific~~		3.7	3.7	3.7	3.7
K1.22	Feedwater system		3.1	3.2	3.1	3.2
K1.23	Reactor water level control system		3.2	3.3	3.2	3.3
K1.24	Head vent		2.7	2.7	2.7	2.7
K1.25	Relief valves		3.5	3.5	3.5	3.5
K1.26	Safety valves		3.7	3.8	3.7	3.8
K1.27	Reactor protection system		4.0	4.1	4.0	4.1
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Main steam isolation valve solenoids		3.2*	3.3*	3.2	3.3
K2.02	Main steam line shutoff valves (guard valves): Plant-Specific		2.3*	2.3*	2.3	2.3

Facility: CPS Printed: 08/11/2006

System Number: 239001

System Name: Main and Reheat Steam System

			CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir
K3	**Knowledge of the effect that a loss or malfunction of the MAIN AND REHEAT STEAM SYSTEM will have on following:**	**(41.7 / 45.4)**					
K3.01	Turbine generator			3.2	3.3	3.2	3.3
K3.02	Condenser			3.1	3.2	3.1	3.2
K3.03	Reactor feedpump turbines: Plant-Specific			3.2	3.3	3.2	3.3
K3.04	Offgas system			2.8	2.8	2.8	2.8
K3.05	Condenser air removal			2.8	2.9	2.8	2.9
K3.06	Seal steam/gland and seal system			2.6	2.7	2.6	2.7
K3.07	Containment			3.1	3.3	3.1	3.3
K3.08	Decay heat removal			3.4	3.5	3.4	3.5
K3.09	Steam bypass capability			3.6	3.7	3.6	3.7
K3.10	~~High pressure coolant injection system: Plant-Specific~~			3.5	3.5	3.5	3.5
K3.11	Reactor core isolation cooling system: Plant-Specific			3.2	3.3	3.2	3.3
K3.12	~~Isolation condenser: Plant-Specific~~			3.7	3.7	3.7	3.7
K3.13	Moisture separator reheaters: Plant-Specific			2.4	2.4	2.4	2.4
K3.14	~~Residual heat removal system: Plant-Specific~~			2.7	2.7	2.7	2.7
K3.15	Reactor water level control			3.5	3.5	3.5	3.5
K3.16	Relief/safety valves			3.6	3.6	3.6	3.6
K3.17	Reactor vessel and internals			3.2	3.3	3.2	3.3
K4	**Knowledge of MAIN AND REHEAT STEAM SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**					
K4.01	Automatic isolation of steam lines			3.8	3.8	3.8	3.8
K4.02	Automatic isolation and opening of drain valves: Plant-Specific			3.1	3.2	3.1	3.2
K4.03	Insures that steam released from a steam line break will not bypass suppression pool; BWR-6			3.2	3.3	3.2	3.3
K4.04	Limits steam flow during a steam line rupture to 200%			3.4	3.5	3.4	3.5
K4.05	Steam flow measurement			3.1	3.2	3.1	3.2
K4.06	Allows for removal or prevents escape of radioactive steam from systems that have leaky MSIV's			3.1	3.2	3.1	3.2
K4.07	Over pressure control			3.7	3.7	3.7	3.7
K4.08	Removal of non condensable gases from reactor head area			2.5	2.6	2.5	2.6
K4.09	Equalization of pressure across the MSIV's before opening			3.3	3.3	3.3	3.3
K4.10	Moisture removal from steam lines prior to admitting steam			2.9	3.0	2.9	3.0
K4.11	Positive sealing of the MSIV's when shutdown: Plant-Specific			2.9	3.1	2.9	3.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 239001

System Name: Main and Reheat Steam System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K5	**Knowledge of the operational implications of the following concepts as they apply to MAIN AND REHEAT STEAM SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Constant enthalpy expansion through a valve		2.0*	2.2*	2.0	2.2
K5.02	Definition and causes of steam/water hammer		2.9	3.1	2.9	3.1
K5.03	Definition and causes of thermal stress		2.7	2.9	2.7	2.9
K5.04	Definition and reason for steam blanketing of moisture separator reheater: Plant-Specific		1.8*	1.9*	1.8	1.9
K5.05	Flow indication		2.8	2.8	2.8	2.8
K5.06	Air operated MSIV's		2.8	2.9	2.8	2.9
K5.07	~~Hydraulic operated MSIV's~~		2.6	2.7	2.6	2.7
K5.08	Solenoid operated valves		2.6	2.7	2.6	2.7
K5.09	Decay heat removal		3.4	3.5	3.4	3.5
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the MAIN AND REHEAT STEAM SYSTEM:**	**(41.7 / 45.7)**				
K6.01	Electrical power		3.1	3.3	3.1	3.3
K6.02	Plant air systems		3.2	3.2	3.2	3.2
K6.03	Safety valve operability		3.6	3.6	3.6	3.6
K6.04	Relief valve operability: Plant-Specific		3.4	3.5	3.4	3.5
K6.05	Steam line integrity		3.6	3.7	3.6	3.7
K6.06	MSIV isolation signal		3.8	3.9	3.8	3.9
K6.07	MSIV leakage control		2.8	3.2	2.8	3.2
K6.08	Main condenser vacuum		3.3	3.4	3.3	3.4
K6.09	PCIS/NSSSS		3.9	4.1*	3.9	4.1
K6.10	ADS/low low set: Plant-Specific		3.6	3.7	3.6	3.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the MAIN AND REHEAT STEAM SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Main steam pressure		3.6	3.6	3.6	3.6
A1.02	Main steam temperature		2.6	2.6	2.6	2.6
A1.03	Reheat steam pressure: Plant-Specific		2.2*	2.2*	2.2	2.2
A1.04	Reheater temperature: Plant-Specific		2.3*	2.3	2.3	2.3
A1.05	Main steam line radiation monitors		3.6	3.6	3.6	3.6
A1.06	Air ejector process radiation monitor		3.4	3.4	3.4	3.4
A1.07	Reactor water level		3.7	3.7	3.7	3.7
A1.08	Reactor pressure		3.8	3.8	3.8	3.8
A1.09	Main steam flow		3.5	3.4	3.5	3.4
A1.10	Reactor power		3.8	3.8	3.8	3.8

Facility: CPS Printed: 08/11/2006

System Number: 239001

System Name: Main and Reheat Steam System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
A2	**Ability to (a) predict the impacts of the following on the MAIN AND REHEAT STEAM SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Malfunction of reactor turbine pressure regulating system		3.8	3.9	3.8	3.9
A2.02	Change in steam demand and its effect on reactor pressure and power		3.6	3.8	3.6	3.8
A2.03	MSIV closure		4.0	4.2	4.0	4.2
A2.04	Main steam line low pressure		3.5	3.6	3.5	3.6
A2.05	†Main steam line high radiation		3.9	4.2*	3.9	4.2
A2.06	Turbine trip without bypass valves		4.1	4.3*	4.1	4.3
A2.07	Main steam area high temperature or differential temperature high		3.8	3.9	3.8	3.9
A2.08	Low condenser vacuum		3.6	3.6	3.6	3.6
A2.09	Opening of head vent to drywell equipment sump with pressure in the reactor vessel		3.4	3.7	3.4	3.7
A2.10	Closure of one or more MSIV's at power		3.8	3.9	3.8	3.9
A2.11	Steam line break		4.1	4.3*	4.1	4.3
A2.12	PCIS/NSSSS actuation		4.2*	4.3*	4.2	4.3
A2.13	High reactor water level		3.5	3.7	3.5	3.7
A2.14	Inadvertent initiation of HPCI/HPCS/RCIC (steam quality and steam flow): Plant-Specific		3.4	3.5	3.4	3.5
A3	**Ability to monitor automatic operations of the MAIN AND REHEAT STEAM SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Isolation of main steam system		4.2*	4.1*	4.2	4.1
A3.02	Opening and closing of drain valves as turbine load changes: Plant-Specific		2.9	2.9	2.9	2.9
A3.03	Moisture separator reheat steam supply: Plant-Specific		2.8	2.8	2.8	2.8
A3.04	Isolation of moisture separator reheater: Plant-Specific		2.7	2.7	2.7	2.7
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	MSIV's		4.2*	4.0	4.2	4.0
A4.02	Maim steam line drain valves		3.2	3.2	3.2	3.2
A4.03	System flow		3.5	3.5	3.5	3.5
A4.04	System pressure		3.8	3.7	3.8	3.7
A4.05	System temperature		2.7	2.7	2.7	2.7
A4.06	System radiation levels		3.6	3.8	3.6	3.8
A4.07	Lights and alarms		3.3	3.3	3.3	3.3
A4.08	Reactor water level		3.7	3.7	3.7	3.7
A4.09	Reactor pressure		3.9	3.9	3.9	3.9
A4.10	Reactor power		3.8	3.8	3.8	3.8
A4.11	Alternate methods of verifying valve positions		3.1	3.3	3.1	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 239002

System Name: Relief/Safety Valves

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RELIEF/SAFETY VALVES and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Nuclear boiler		3.8	3.9	3.8	3.9
K1.02	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.5*	2.9	2.5	2.9
K1.03	Nuclear boiler instrument system		3.5	3.0	3.5	3.0
K1.04	Main steam		3.6	3.7	3.6	3.7
K1.05	Plant air systems: Plant-Specific		3.1	3.3	3.1	3.3
K1.06	~~Drywell instrument air/ drywell pneumatics: Plant-Specific~~		3.4	3.6	3.4	3.6
K1.07	Suppression pool		3.6	3.8	3.6	3.8
K1.08	Automatic depressurization system		4.0*	4.1	4.0	4.1
K1.09	~~Drywell pressure (for safety valves which discharge to the drywell airspace): Plant-Specific~~		4.0	4.0	4.0	4.0
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	SRV solenoids		2.8*	3.2*	2.8	3.2
K3	**Knowledge of the effect that a loss or malfunction of the RELIEF/SAFETY VALVES will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor pressure control		3.9	4.0	3.9	4.0
K3.02	Reactor over pressurization		4.2*	4.4*	4.2	4.4
K3.03	Ability to rapidly depressurize the reactor		4.3*	4.4*	4.3	4.4
K4	**Knowledge of RELIEF/SAFETY VALVES design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Insures that only one or two safety/relief valves reopen following the initial portion of a reactor isolation event (LLS logic): Plant-Specific		3.9	4.0	3.9	4.0
K4.02	Minimizes containment fatigue duty cycles resulting from relief valve cycling during decay-heat-dominant period late in an isolation transient (LLS logic): Plant-Specific		3.4	3.6	3.4	3.6
K4.03	Prevents siphoning of water into SRV discharge piping and limits loads on subsequent actuation of SRV's		3.1	3.3	3.1	3.3
K4.04	Ensures even distribution of heat load to suppression pool, and adequate steam condensing		3.4	3.6	3.4	3.6
K4.05	Allows for SRV operation from more than one location: Plant-Specific		3.6	3.7	3.6	3.7
K4.06	Detection of valve leakage		3.5	3.7	3.5	3.7
K4.07	Minimum steam pressure required to keep SRV open or to open SRV		3.1	3.2	3.1	3.2
K4.08	Opening of the SRV from either an electrical or mechanical signal		3.6	3.7	3.6	3.7
K4.09	Manual opening of the SRV		3.7	3.6	3.7	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 239002

System Name: Relief/Safety Valves

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
K5	**Knowledge of the operational implications of the following concepts as they apply to RELIEF/SAFETY VALVES:**	**(41.5 / 45.3)**				
K5.01	Relief function of SRV operation		3.4	3.5	3.4	3.5
K5.02	Safety function of SRV operation		3.7	3.8	3.7	3.8
K5.03	Acoustical monitoring: Plant-Specific		3.7	3.8	3.7	3.8
K5.04	Tail pipe temperature monitoring		3.3	3.5	3.3	3.5
K5.05	Discharge line quencher operation		2.6	2.9	2.6	2.9
K5.06	Vacuum breaker operation		2.7	3.0	2.7	3.0
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RELIEF/SAFETY VALVES:**	**(41.7 / 45.7)**				
K6.01	Nuclear boiler instrument system (pressure indication)		3.2	3.4	3.2	3.4
K6.02	Air (Nitrogen) supply: Plant-Specific		3.4	3.5	3.4	3.5
K6.03	A.C. power: Plant-Specific		2.7*	2.9*	2.7	2.9
K6.04	D.C. power: Plant-Specific		3.0	3.2	3.0	3.2
K6.05	Discharge line vacuum breaker		3.0	3.2	3.0	3.2
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RELIEF/SAFETY VALVES controls including:**	**(41.5 / 45.5)**				
A1.01	Tail pipe temperature		3.3	3.4	3.3	3.
A1.02	Acoustical monitor noise: Plant-Specific		3.7	3.8	3.7	3.8
A1.03	Air supply: Plant-Specific		2.8	2.9	2.8	2.9
A1.04	Reactor pressure		3.8	3.8	3.8	3.8
A1.05	Reactor water level		3.7	3.8	3.7	3.8
A1.06	Reactor power		3.7	3.8	3.7	3.8
A1.07	Turbine load		2.9	3.0	2.9	3.0
A1.08	Suppression pool water temperature		3.8	4.1	3.8	4.1
A1.09	Indicated vs. actual steam flow: Plant-Specific		3.1	3.3	3.1	3.3
A2	**Ability to (a) predict the impacts of the following on the RELIEF/SAFETY VALVES; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Stuck open vacuum breakers		3.0	3.3	3.0	3.3
A2.02	Leaky SRV		3.1	3.2	3.1	3.2
A2.03	Stuck open SRV		4.1	4.2*	4.1	4.2
A2.04	ADS actuation		4.1*	4.2*	4.1	4.2
A2.05	Low reactor pressure		3.2	3.4	3.2	3.4
A2.06	Reactor high pressure		4.1	4.3*	4.1	4.3

Facility: CPS Printed: 08/11/2006

System Number: 239002

System Name: Relief/Safety Valves

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A3	**Ability to monitor automatic operations of the RELIEF/SAFETY VALVES including:**	**(41.7 / 45.7)**				
A3.01	SRV operation after ADS actuation		3.8*	3.9*	3.8	3.9
A3.02	SRV operation on high reactor pressure		4.3*	4.3*	4.3	4.3
A3.03	Tail pipe temperatures		3.6	3.6	3.6	3.6
A3.04	Acoustical monitor noise: Plant-Specific		3.6	3.7	3.6	3.7
A3.05	Suppression pool temperature		4.1*	4.2*	4.1	4.2
A3.06	Reactor pressure		4.1*	4.1*	4.1	4.1
A3.07	Reactor water level		3.8	3.9	3.8	3.9
A3.08	Lights and alarms		3.6	3.6	3.6	3.6
A3.09	Low low set logic: Plant-Specific		3.9	3.9	3.9	3.9
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	SRV's		4.4*	4.4*	4.4	4.4
A4.02	Tail pipe temperatures		3.6	3.7	3.6	3.7
A4.03	Acoustical monitor noise: Plant-Specific		3.8	3.9	3.8	3.9
A4.04	Suppression pool temperature		4.3*	4.3*	4.3	4.3
A4.05	Reactor pressure		4.3*	4.3*	4.3	4.3
A4.06	Reactor water level		3.9	4.1	3.9	4.1
A4.07	Lights and alarms		3.6	3.6	3.6	3.6
'.08	Plant air system pressure: Plant-Specific		3.2	3.2	3.2	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 239003

System Name: MSIV Leakage Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between MSIV LEAKAGE CONTROL SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Main steam system: BWR-4, 5, 6(P-Spec)~~		3.3*	3.4	3.3	3.4
K1.02	~~Standby gas treatment system: BWR-4, 5, 6(P-Spec)~~		2.9	3.0	2.9	3.0
K1.03	~~Main steam line pressure instrumentation: BWR-4,5,6(P-Spec)~~		3.1	3.1	3.1	3.1
K1.04	~~A.C. electrical distribution: BWR-4, 5, 6(P-Spec)~~		2.7	2.8	2.7	2.8
K1.05	~~Steam tunnel: BWR-4, 5, 6(P-Spec)~~		2.9	2.9	2.9	2.9
K1.06	~~Radwaste system: BWR-4, 5, 6(P-Spec)~~		2.4	2.4	2.4	2.4
K1.07	~~Floor drainage system: BWR-4, 5, 6(P-Spec)~~		1.9*	1.9*	1.9	1.9
K1.08	~~Nuclear boiler instrumentation: BWR-4, 5, 6(P-Spec)~~		2.8	2.8	2.8	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	~~Motor operated valves: BWR-4, 5, 6(P-Spec)~~		2.3*	2.3*	2.3	2.3
K2.02	~~Leakage control system blowers: BWR-4, 5, 6(P-Spec)~~		1.9*	2.0*	1.9	2.0
K2.03	~~Leakage control system heaters: BWR-4, 5, 6(P-Spec)~~		1.9*	2.0*	1.9	2.0
K3	**Knowledge of the effect that a loss or malfunction of the MSIV LEAKAGE CONTROL SYSTEM will have on following:**	**(41.5 / 45.3)**				
3.01	~~Radiation release to the environment: BWR-4, 5, 6(P-Spec)~~		3.3	4.0	3.3	4.0
K4	**Knowledge of MSIV LEAKAGE CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Performance of its safety function following a loss of offsite power: BWR-4, 5, 6(P-Spec)~~		3.2	3.5	3.2	3.5
K4.02	~~Performance of intended safety function following any single active component failure: BWR-4, 5, 6(P-Spec)~~		3.0	3.4	3.0	3.4
K4.03	~~The prevention of inadvertent system operation: BWR-4,5,6(P-Spec)~~		2.9	3.2	2.9	3.2
K4.04	~~Surveillance for system operability: BWR-4, 5, 6~~		2.4	2.8	2.4	2.8
K4.05	~~Assurance that any MSIV leakage will pass through the system and into standby gas treatment prior to release to the atmosphere: BWR-4, 5, 6(P-Spec)~~		2.4*	2.8*	2.4	2.8
K4.06	~~The depressurization of main steam piping prior to routing leakage through system: BWR-4, 5, 6~~		3.1	3.3	3.1	3.3
K4.07	~~The reduction of MSIV leakage temperature: BWR-4,5,6(P-Spec)~~		2.4	2.4	2.4	2.4
K4.08	~~Prevention of collected condensate in system bleed lines: BWR-4, 5, 6(P-Spec)~~		2.3	2.4	2.3	2.4
K4.09	~~The dilution of MSIV leakage: BWR-4, 5, 6(P-Spec)~~		2.4	2.4	2.4	2.4

Facility: CPS Printed: 08/11/2006

System Number: 239003

System Name: MSIV Leakage Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the MSIV LEAKAGE CONTROL SYSTEM:**	**(41.7 / 45.7)**				
K6.01	~~A.C. electrical distribution: BWR-4, 5, 6(P-Spec)~~		2.8	3.0	2.8	3.0
K6.02	~~Standby gas treatment system: BWR-4, 5, 6(P-Spec)~~		2.8	3.0	2.8	3.0
K6.03	~~Nuclear boiler instrumentation: BWR-4, 5, 6(P-Spec)~~		2.6	2.9	2.6	2.9
A1	**Ability to predict and/or monitor changes in parameters associated with operating the MSIV LEAKAGE CONTROL SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	~~Main steam line pressure: BWR-4, 5, 6~~		3.1	3.1	3.1	3.1
A1.02	~~Heater operation: BWR-4, 5, 6(P-Spec)~~		2.6	2.6	2.6	2.6
A1.03	~~Dilution air flow: BWR-4, 5, 6(P-Spec)~~		2.4	2.6	2.4	2.6
A1.04	~~Status indicating lights and alarms: BWR-4, 5, 6~~		2.9	2.8	2.9	2.8
A1.05	~~System lineup: BWR-4, 5, 6~~		3.0	2.9	3.0	2.9
A1.06	~~MSIV leakage flow: BWR-4, 5, 6~~		2.6	2.6	2.6	2.6
A1.07	~~Reactor building temperature: BWR-4, 5, 6(P-Spec)~~		2.7	2.7	2.7	2.7
A2	**Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	~~Inboard MSIV valve leakage: BWR-4, 5, 6~~		2.8	3.2	2.8	3.2
A2.02	~~Outboard MSIV valves leakage: BWR-4, 5, 6~~		2.9	3.3	2.9	3.3
A2.03	~~Low dilution air flow (inboard and/or outboard): BWR-4, 5, 6(P-Spec)~~		2.6	2.7	2.6	2.7
A2.04	~~Outboard system logic failure: BWR-4, 5, 6~~		2.7	3.0	2.7	3.0
A2.05	~~Inboard system logic failure: BWR-4, 5, 6~~		2.7	3.0	2.7	3.0
A2.06	~~Blower failure: BWR-4, 5, 6(P-Spec)~~		2.6	2.6	2.6	2.6
A2.07	~~Heater failure: BWR-4, 5, 6(P-Spec)~~		2.4	2.4	2.4	2.4
A2.08	~~Motor operator valve failure(s): BWR-4, 5, 6~~		2.8	3.0	2.8	3.0
A2.09	~~Outboard main steamline high pressure: BWR-4, 5, 6(P-Spec)~~		2.8	3.0	2.8	3.0
A2.10	~~A.C. distribution power failures: BWR-4, 5, 6~~		2.8	3.0	2.8	3.0
A2.11	~~High reactor pressure: BWR-4, 5, 6~~		3.1	3.3	3.1	3.3
A2.12	~~MSIV valve failure to close: BWR-4, 5, 6~~		3.5	3.8	3.5	3.8

Facility: CPS Printed: 08/11/2006

System Number: 239003

System Name: MSIV Leakage Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A3	**Ability to monitor automatic operations of the MSIV LEAKAGE CONTROL SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	~~System logic initiation: BWR-4, 5, 6(P-Spec)~~		3.0	2.8	3.0	2.8
A3.02	~~Main steamline pressures: BWR-4, 5, 6~~		3.1	2.8	3.1	2.8
A3.03	~~Dilution air flows: BWR-4, 5, 6(P-Spec)~~		2.4	2.3	2.4	2.3
A3.04	~~MSIV leakage flows: BWR-4, 5, 6(P-Spec)~~		2.8	2.6	2.8	2.6
A3.05	~~Heater operation: BWR-4, 5, 6(P-Spec)~~		2.3	2.2	2.3	2.2
A3.06	~~System status lights and alarms: BWR-4, 5, 6~~		2.8	2.8	2.8	2.8
A3.07	~~System lineups: BWR-4, 5, 6~~		3.1	2.9	3.1	2.9
A3.08	~~Blower operation: BWR-4, 5, 6(P-Spec)~~		2.4	2.3	2.4	2.3
A3.09	~~Reactor building temperature: BWR-4, 5, 6(P-Spec)~~		2.6	2.5	2.6	2.5
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	~~Manually initiate system operation: BWR-4, 5, 6(P-Spec)~~		3.2	3.2	3.2	3.2
A4.02	~~†Surveillance testing: BWR-4, 5, 6(P-Spec)~~		2.5	2.8	2.5	2.8
A4.03	~~Main steamline pressures: BWR-4, 5, 6~~		3.3	3.2	3.3	3.2
A4.04	~~Dilution air flows: BWR-4, 5, 6(P-Spec)~~		2.5	2.4	2.5	2.4
A4.05	~~MSIV leakage flows: BWR-4, 5, 6(P-Spec)~~		2.8	2.7	2.8	2.7
A4.06	~~Heater operation: BWR-4, 5, 6(P-Spec)~~		2.4	2.3	2.4	2.3
A4.07	~~System status lights and alarms: BWR-4, 5, 6(P-Spec)~~		2.8	2.7	2.8	2.7
A4.08	~~System lineups: BWR-4, 5, 6(P-Spec)~~		3.1	2.9	3.1	2.9
A4.09	~~System reset: BWR-4, 5, 6(P-Spec)~~		3.0	2.8	3.0	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 241000

System Name: Reactor/Turbine Pressure Regulating System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR/TURBINE PRESSURE REGULATING SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Reactor power		3.8	3.9	3.8	3.9
K1.02	Reactor pressure		3.9	4.1	3.9	4.1
K1.03	Reactor water level		3.6	3.7	3.6	3.7
K1.04	Reactor steam flow		3.7	3.7	3.7	3.7
K1.05	Main turbine steam flow		3.5	3.6	3.5	3.6
K1.06	Bypass valves		3.8	3.9	3.8	3.9
K1.07	Main stop/throttle valves		3.4	3.6	3.4	3.6
K1.08	Control/governor valves		3.6	3.7	3.6	3.7
K1.09	Combined intermediate valves: Plant Specific		3.1	3.4	3.1	3.4
K1.10	Front standard trip system		3.2	3.3	3.2	3.3
K1.11	RPS		3.7	3.8	3.7	3.8
K1.12	FW extraction non-return valves		2.4	2.6	2.4	2.6
K1.13	Bearing oil		2.6	2.6	2.6	2.6
K1.14	A.C. electrical power		2.8	2.9	2.8	2.9
K1.15	~~D.C. electrical power~~		2.6	2.7	2.6	2.7
K1.16	Component cooling water systems		2.5	2.6	2.5	2.6
K1.17	Turbine chest warming: EHC-Only		2.4	2.5	2.4	2.5
K1.18	Turbine shell warming: EHC-Only		2.4	2.5	2.4	2.5
K1.19	Turbine acceleration		2.6	2.6	2.6	2.6
K1.20	Turbine speed		2.7	2.7	2.7	2.7
K1.21	Turbine inlet pressure		2.7	2.7	2.7	2.7
K1.22	Turbine trip		3.4	3.5	3.4	3.5
K1.23	~~Recirculation flow control system: Plant-Specific~~		3.0	3.1	3.0	3.1
K1.24	Main generator		2.7	2.8	2.7	2.8
K1.25	Stator water cooling: Plant-Specific		2.8	2.8	2.8	2.8
K1.26	Main turbine PMG: Plant-Specific		2.4	2.6	2.4	2.6
K1.27	Condenser vacuum		3.1	3.1	3.1	3.1
K1.28	Reactor startup		3.2	3.2	3.2	3.2
K1.29	Reactor heatup		3.4	3.4	3.4	3.4
K1.30	Reactor cooldown		3.2	3.3	3.2	3.3
K1.31	Turbine protection		3.1	3.2	3.1	3.2
K1.32	Turbine monitoring		2.7	2.8	2.7	2.8
K1.33	FW extraction steam valves		2.4	2.6	2.4	2.6
K1.34	EGC system: Plant-Specific		2.8	3.3	2.8	3.3
K1.35	Low pressure stop and control valves: Plant-Specific		2.4	2.6	2.4	2.6
K1.36	Primary water system: Plant-Specific		2.1*	2.1*	2.1	2.1
K1.37	~~Turbine stress evaluator: Plant-Specific~~		1.8*	1.8*	1.8	1.8
K1.38	PCIS/NSSSS: Plant-Specific		2.7	2.8	2.7	2.8
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Pumps		2.1*	2.2*	2.1	2.
K2.02	Controls		2.1*	2.1*	2.1	2.

Facility: CPS Printed: 08/11/2006

System Number: 241000

System Name: Reactor/Turbine Pressure Regulating System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SR
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR/TURBINE PRESSURE REGULATING SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor power		4.1	4.1	4.1	4.1
K3.02	Reactor pressure		4.2*	4.3*	4.2	4.3
K3.03	Reactor water level		3.7	3.8	3.7	3.8
K3.04	Reactor steam flow		3.8	3.9	3.8	3.9
K3.05	Main turbine steam flow		3.7	3.7	3.7	3.7
K3.06	Bypass valves		4.1*	4.1	4.1	4.1
K3.07	Main stop/throttle valves		3.3	3.3	3.3	3.3
K3.08	Control/governor valves		3.7	3.7	3.7	3.7
K3.09	Combined intermediate valves: Plant-Specific		3.2	3.3	3.2	3.3
K3.10	Front standard trip system		2.9	3.0	2.9	3.0
K3.11	RPS		3.8	3.8	3.8	3.8
K3.12	FW extraction steam valves: Plant-Specific		2.4	2.4	2.4	2.4
K3.13	FW extraction non-return valves		2.3	2.4	2.3	2.4
K3.14	Component cooling water systems: Plant-Specific		2.3	2.3	2.3	2.3
K3.15	Turbine chest warming: EHC-Only		2.8	2.8	2.8	2.8
K3.16	Turbine shell warming: EHC-Only		2.8	2.8	2.8	2.8
K3.17	Turbine acceleration		2.7	2.8	2.7	2.8
K3.18	Turbine speed		2.9	2.9	2.9	2.9
K3.19	Turbine inlet pressure		2.7	2.7	2.7	2.7
K3.20	Turbine trip		3.3	3.4	3.3	3.4
K3.21	~~Recirculation flow control system: Plant-Specific~~		2.8	2.8	2.8	2.8
K3.22	Main generator		2.6	2.6	2.6	2.6
K3.23	Turbine trip testing: Plant-Specific		2.8	2.9	2.8	2.9
K3.24	Reactor heatup		3.2	3.2	3.2	3.2
K3.25	Reactor cooldown		3.3	3.3	3.3	3.3
K3.26	Turbine protection		3.1	3.2	3.1	3.2
K3.27	Turbine monitoring		2.4	2.6	2.4	2.6
K3.28	Low pressure stop and control valves: Plant-Specific		2.5	2.5	2.5	2.5
K3.29	PCIS/NSSSS		2.9	3.1	2.9	3.1
K3.30	EGC: Plant-Specific		3.0	3.0	3.0	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 241000

System Name: Reactor/Turbine Pressure Regulating System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K4	**Knowledge of REACTOR/TURBINE PRESSURE REGULATING SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Reactor pressure control		3.8	3.8	3.8	3.8
K4.02	Turbine inlet pressure control		3.3	3.3	3.3	3.3
K4.03	Turbine speed control		3.0	3.1	3.0	3.1
K4.04	Turbine acceleration control		2.8	2.8	2.8	2.8
K4.05	Reactor scram		3.7*	3.8*	3.7	3.8
K4.06	Turbine trip		3.6	3.7*	3.6	3.7
K4.07	Generator runback: Plant-Specific		3.2	3.5	3.2	3.5
K4.08	Feedwater heater isolation: Plant-Specific		2.6	2.9	2.6	2.9
K4.09	Turbine chest warming: EHC-Only		2.4	2.4	2.4	2.4
K4.10	Turbine shell warming: EHC-Only		2.5	2.5	2.5	2.5
K4.11	~~Load following: Plant-Specific~~		2.6	2.6	2.6	2.6
K4.12	~~Recirculation flow control: Plant-Specific~~		2.9	2.9	2.9	2.9
K4.13	Turbine trip testing: Plant-Specific		2.9	3.0	2.9	3.0
K4.14	Reactor/turbine pressure regulating system oil cooling: Plant-Specific		2.2	2.4	2.2	2.4
K4.15	Automatic pump start		2.6	2.6	2.6	2.6
K4.16	Reactor cooldown		3.3	3.4	3.3	3.4
K4.17	Turbine monitoring		2.4	2.6	2.4	2.6
K4.18	Turbine protection		2.8	2.8	2.8	2.8
4.19	Steam bypass valve control		3.6	3.7	3.6	3.7
K5	**Knowledge of the operational Implications of the following concepts as they apply to REACTOR/TURBINE PRESSURE REGULATING SYSTEM:**	**(41.5 / 45.3)**				
K5.01	~~Accumulator operation: Plant-Specific~~		2.0*	2.1*	2.0	2.1
K5.02	Limit switch operation		2.1*	2.2*	2.1	2.2
K5.03	Reactor power vs. reactor pressure		3.5	3.6	3.5	3.6
K5.04	Turbine inlet pressure vs. reactor pressure		3.3	3.3	3.3	3.3
K5.05	Turbine inlet pressure vs. turbine load		2.8	2.9	2.8	2.9
K5.06	Turbine speed measurement		2.3	2.3	2.3	2.3
K5.07	~~Unitized actuator operation: Fermi-Only~~		2.5	3.0	2.5	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 241000

System Name: Reactor/Turbine Pressure Regulating System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR/TURBINE PRESSURE REGULATING SYSTEM:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical power		2.8	2.9	2.8	2.9
K6.02	D.C. electrical power		2.6	2.7	2.6	2.7
K6.03	Component cooling water systems		2.5	2.5	2.5	2.5
K6.04	~~Recirculation flow control system: Plant-Specific~~		3.0	3.0	3.0	3.0
K6.05	Condenser vacuum		3.4	3.4	3.4	3.4
K6.06	Reactor pressure		3.8*	3.9*	3.8	3.9
K6.07	Turbine inlet pressure		3.4	3.4	3.4	3.4
K6.08	Reactor power		3.6	3.7	3.6	3.7
K6.09	Main turbine steam flow		3.1	3.1	3.1	3.1
K6.10	Bypass valves		3.6	3.7	3.6	3.7
K6.11	Main stop/throttle valves		3.4	3.4	3.4	3.4
K6.12	Control/governor valves		3.3	3.4	3.3	3.4
K6.13	Combined intermediate valves: Plant-Specific		3.1	3.2	3.1	3.2
K6.14	Bearing oil		2.7	2.8	2.7	2.8
K6.15	Turbine speed signal		2.3	2.4	2.3	2.4
K6.16	Stator water cooling system: Plant-Specific		2.9	3.1	2.9	3.1
K6.17	Main turbine PMG: Plant-Specific		2.7	2.8	2.7	2.8
K6.18	Low pressure stop and control valves: Plant-Specific		2.6	2.7	2.6	2.7
K6.19	Primary water system: Plant-Specific		2.1*	2.3*	2.1	2.3
K6.20	Main generator		2.8	3.0	2.8	3

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 241000

System Name: Reactor/Turbine Pressure Regulating System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the REACTOR/TURBINE PRESSURE REGULATING SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Reactor pressure		3.9	3.8	3.9	3.8
A1.02	Reactor power		4.1*	3.9	4.1	3.9
A1.03	Reactor water level		3.7	3.7	3.7	3.7
A1.04	Main turbine inlet pressure		3.1	3.1	3.1	3.1
A1.05	Reactor steam flow		3.5	3.6	3.5	3.6
A1.06	Main turbine steam flow		3.2	3.2	3.2	3.2
A1.07	Bypass valve position		3.8	3.7	3.8	3.7
A1.08	Control/governor valve position		3.3	3.2	3.3	3.2
A1.09	Main stop throttle valve position		3.3	3.3	3.3	3.3
A1.10	Combined intermediate valve position: Plant-Specific		3.3	3.2	3.3	3.2
A1.11	Reactor/turbine pressure regulating system oil pressure: Plant-Specific		2.7	2.7	2.7	2.7
A1.12	Reactor/turbine pressure regulating system load set/reference: Plant-Specific		2.9	2.8	2.9	2.8
A1.13	Main turbine speed		2.7	2.7	2.7	2.7
A1.14	Pressure setpoint/pressure demand		3.4	3.4	3.4	3.4
A1.15	Maximum combined flow limit		3.1	3.1	3.1	3.1
A1.16	Load limit set: Plant-Specific		3.3	3.3	3.3	3.3
A1.17	Reactor/turbine pressure regulating system oil pump current: Plant-Specific		1.9*	1.9*	1.9	1.9
A1.18	Reactor/turbine pressure regulating system reservoir oil level: Plant-Specific		2.1*	2.0*	2.1	2.0
A1.19	Reactor/turbine pressure regulating system reservoir oil temperature: Plant-Specific		2.0*	1.9*	2.0	1.9
A1.20	Servo valve currents: Plant-Specific		2.2*	2.1*	2.2	2.1
A1.21	Main condenser vacuum		3.4	3.4	3.4	3.4
A1.22	Reactor cooldown		3.4	3.3	3.4	3.3
A1.23	Main turbine vibration		2.8	2.8	2.8	2.8
A1.24	Main turbine eccentricity		2.6	2.7	2.6	2.7
A1.25	Main turbine expansion		2.6	2.6	2.6	2.6
A1.26	Governor valve limit: Plant-Specific		2.4	2.4	2.4	2.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 241000

System Name: *Reactor/Turbine Pressure Regulating System*

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A2	**Ability to (a) predict the impacts of the following on the REACTOR/TURBINE PRESSURE REGULATING SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Loss of turbine inlet pressure signal		3.5	3.7	3.5	3.7
A2.02	High reactor pressure		3.7	3.7	3.7	3.7
A2.03	Failed open/closed bypass valve(s)		4.1*	4.2*	4.1	4.2
A2.04	Failed open/closed control/governor valve(s)		3.7	3.8	3.7	3.8
A2.05	Failed open/closed main stop valve(s)		3.8	3.9	3.8	3.9
A2.06	Low reactor/turbine pressure regulating system oil pressure: Plant-Specific		3.1	3.2	3.1	3.2
A2.07	Loss of condenser vacuum		3.7	3.6	3.7	3.6
A2.08	Main turbine overspeed		3.3	3.3	3.3	3.3
A2.09	Loss of generator load		3.4	3.5	3.4	3.5
A2.10	Loss of stator water cooling: Plant-Specific		3.1	3.2	3.1	3.2
A2.11	Loss of A.C. electrical power		3.1	3.1	3.1	3.1
A2.12	Loss of D.C. electrical power		2.7	2.8	2.7	2.8
A2.13	Loss of component cooling water systems		2.6	2.7	2.6	2.7
A2.14	Loss of main turbine PMG: Plant-Specific		2.7	2.7	2.7	2.7
A2.15	Loss of main turbine speed feedback: Plant-Specific		2.4	2.4	2.4	2.4
A2.16	Low turbine inlet pressure (loss of pressure signal)		3.4	3.4	3.4	3.4
A2.17	Turbine trip: Plant-Specific		3.8	3.8	3.8	
A2.18	Generator trip: Plant-Specific		3.5	3.5	3.5	3.5
A2.19	Reactor scram		3.8	3.8	3.8	3.8
A2.20	Low reactor/turbine pressure regulating system oil level: Plant-Specific		2.5	2.6	2.5	2.6
A2.21	Reactor/turbine pressure regulating system pump trip: Plant-Specific		2.7	2.8	2.7	2.8
A2.22	Turbine high vibration		2.8	2.9	2.8	2.9
A2.23	Turbine high eccentricity		2.6	2.6	2.6	2.6
A2.24	Turbine high differential expansion		2.4	2.5	2.4	2.5
A2.25	Loss of primary water system: Plant-Specific		2.1	2.3	2.1	2.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 241000

System Name: Reactor/Turbine Pressure Regulating System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A3	***Ability to monitor automatic operations of the* REACTOR/TURBINE PRESSURE REGULATING SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Turbine speed control: Plant-Specific		2.8	2.7	2.8	2.7
A3.02	Turbine acceleration control: Plant-Specific		2.6	2.6	2.6	2.6
A3.03	Turbine inlet pressure control		2.8	2.7	2.8	2.7
A3.04	Reactor/turbine pressure regulating system pump start: Plant-Specific		2.5	2.5	2.5	2.5
A3.05	Reactor/turbine pressure regulating system low pressure turbine trip: Plant-Specific		2.9	2.8	2.9	2.8
A3.06	Reactor/turbine pressure regulating system low pressure reactor scram: Plant-Specific		3.3	3.3	3.3	3.3
A3.07	Reactor/turbine pressure regulating system oil temperature control: Plant-Specific		2.3	2.3	2.3	2.3
A3.08	Steam bypass valve operation		3.8	3.8	3.8	3.8
A3.09	Control/governor valve operation		3.3	3.2	3.3	3.2
A3.10	Main stop/throttle valve operation		3.3	3.3	3.3	3.3
A3.11	Combined intermediate valve operation: Plant-Specific		3.3	3.1	3.3	3.1
A3.12	Turbine trip testing		2.9	2.9	2.9	2.9
A3.13	FW heater isolation: Plant-Specific		2.7	2.8	2.7	2.8
A3.14	Grid load following: Plant-Specific		2.6	2.7	2.6	2.7
A3.15	~~Recirculation pump flow control: Plant-Specific~~		2.8	2.9	2.8	2.9
A3.16	Lights and alarms		3.0	2.9	3.0	2.9
A3.17	Turbine runback		3.3	3.4	3.3	3.4
A3.18	Turbine startup: Plant-Specific		3.0	3.0	3.0	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 241000

System Name: Reactor/Turbine Pressure Regulating System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Reactor power		3.9	4.0	3.9	4.0
A4.02	Reactor pressure		4.1*	4.1*	4.1	4.1
A4.03	Reactor water level		3.8*	3.9	3.8	3.9
A4.04	Reactor steam flow		3.6	3.7	3.6	3.7
A4.05	Main turbine steam flow		3.3	3.3	3.3	3.3
A4.06	Bypass valves (operation)		3.9	3.9	3.9	3.9
A4.07	Main stop/throttle valves (operation)		3.5	3.4	3.5	3.4
A4.08	Control/governor valves (operation)		3.5	3.4	3.5	3.4
A4.09	Combined intermediate valves (operation): Plant-Specific		3.2	3.1	3.2	3.1
A4.10	Reactor/turbine pressure regulating system pumps: Plant-Specific		2.9	2.9	2.9	2.9
A4.11	Turbine speed		3.1	3.1	3.1	3.1
A4.12	Turbine acceleration		3.0	3.0	3.0	3.0
A4.13	Turbine inlet pressure		2.9	2.9	2.9	2.9
A4.14	Turbine trip		3.8	3.7	3.8	3.7
A4.15	Generator load		3.2	3.2	3.2	3.2
A4.16	Lights and alarms		3.3	3.2	3.3	3.2
A4.17	Turbine chest warming: Plant-Specific		2.9	2.8	2.9	2.8
A4.18	Turbine shell warming: Plant-Specific		2.9	2.8	2.9	2.8
A4.19	Turbine panel controls		3.5	3.4	3.5	3.4

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 245000

System Name: Main Turbine Generator and Auxiliary Systems

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	A.C. electrical distribution		3.2	3.3	3.2	3.3
K1.02	Condensate system		2.5	2.5	2.5	2.5
K1.03	Main steam system		3.2	3.3	3.2	3.3
K1.04	Reactor protection system		3.6	3.7	3.6	3.7
K1.05	Extraction steam system		2.7	2.7	2.7	2.7
K1.06	Component cooling water systems		2.6	2.6	2.6	2.6
K1.07	Plant air systems		2.5	2.5	2.5	2.5
K1.08	Reactor/turbine pressure control system: Plant-Specific		3.4	3.5	3.4	3.5
K1.09	D.C. electrical distribution		2.7	2.7	2.7	2.7
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Stator water cooling pumps: Plant-Specific		2.4*	2.4	2.4	2.4
K2.02	Main lubricating oil pumps		2.3*	2.4*	2.3	2.4
K2.03	~~Amplidyne: Plant-Specific~~		1.7*	1.8*	1.7	1.8
K2.04	Hydrogen seal oil pumps		2.3*	2.3*	2.3	2.3
K2.05	~~Air seal oil pumps: Plant-Specific~~		1.7*	1.7*	1.7	1.7
K2.06	Turbine supervisory instrumentation		2.3	2.5	2.3	2.5
K3	**Knowledge of the effect that a loss or malfunction of the MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS will have on following:**	**(41.7 / 45.4)**				
K3.01	A.C. electrical distribution		3.4	3.7	3.4	3.7
K3.02	Reactor pressure		3.9	4.0	3.9	4.0
K3.03	Reactor power		3.9	4.0	3.9	4.0
K3.04	Reactor feedwater system (feedwater heaters)		3.3	3.5	3.3	3.5
K3.05	Reactor feedwater pump: Plant-Specific		2.7	2.8	2.7	2.8
K3.06	Condensate system		2.5	2.5	2.5	2.5
K3.07	Reactor protection system		3.6	3.7	3.6	3.7
K3.08	Reactor/turbine pressure control system: Plant-Specific		3.7	3.8	3.7	3.8

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 245000

System Name: Main Turbine Generator and Auxiliary Systems

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I SRO
K4	**Knowledge of MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Bearing lubrication		2.4	2.5	2.4	2.5
K4.02	Generator cooling		2.4	2.5	2.4	2.5
K4.03	Sealing to prevent hydrogen leakage		2.7	2.8	2.7	2.8
K4.04	Hydrogen cooling		2.4	2.5	2.4	2.5
K4.05	Turbine protection		2.9	3.0	2.9	3.0
K4.06	Generator protection		2.7	2.8	2.7	2.8
K4.07	Generator voltage regulation		2.5	2.6	2.5	2.6
K4.08	Moisture removal from turbine steam		2.4	2.4	2.4	2.4
K4.09	Turbine control		3.1	3.2	3.1	3.2
K4.10	Extraction steam		2.6	2.7	2.6	2.7
K5	**Knowledge of the operational implications of the following concepts as they apply to MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS:**	**(41.5 / 45.3)**				
K5.01	Heat exchanger operation		2.3*	2.3*	2.3	2.3
K5.02	Turbine operation and limitations		2.8	3.1	2.8	3.1
K5.03	Hydraulically operated valve operation		2.6	2.6	2.6	2.6
K5.04	Turbine speed measurement		1.9*	2.0*	1.9	2.0
K5.05	Relief valve operation		2.4	2.4	2.4	
K5.06	Turbine shaft sealing		2.5	2.6	2.5	2.6
K5.07	Generator operations and limitations		2.6	2.9	2.6	2.9
K5.08	Generator cooling		2.4	2.6	2.4	2.6
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS:**	**(41.7 / 45.7)**				
K6.01	Gland seal		2.8	2.9	2.8	2.9
K6.02	Reactor/turbine pressure control system: Plant-Specific		3.5	3.7	3.5	3.7
K6.03	Hydrogen seal oil		2.8	3.1	2.8	3.1
K6.04	Hydrogen cooling		2.6*	2.7	2.6	2.7
K6.05	Stator water cooling		2.9	2.9	2.9	2.9
K6.06	Electrical distribution		3.0	3.2	3.0	3.2
K6.07	Extraction steam		2.4	2.5	2.4	2.5
K6.08	Main steam		3.0	3.1	3.0	3.1
K6.09	Voltage regulation		2.3*	2.5*	2.3	2.5
K6.10	Lube oil system		2.8*	2.9	2.8	2.9

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 245000

System Name: Main Turbine Generator and Auxiliary Systems

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS controls including:**	**(41.5 / 45.5)**				
A1.01	Generator megawatts		2.7	2.7	2.7	2.7
A1.02	Turbine speed		2.6	2.5	2.6	2.5
A1.03	Turbine valve position		2.7	2.7	2.7	2.7
A1.04	Steam flow		2.7	2.8	2.7	2.8
A1.05	Reactor pressure		3.5	3.4	3.5	3.4
A1.06	Condenser vacuum		3.3	3.2	3.3	3.2
A1.07	First stage turbine pressure		2.8	2.8	2.8	2.8
A1.08	Generator output voltage/reactive load		2.5*	2.7	2.5	2.7
A2	**Ability to (a) predict the impacts of the following on the MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Turbine trip		3.7	3.9	3.7	3.9
A2.02	Loss of lube oil		3.3	3.5	3.3	3.5
A2.03	Loss of condenser vacuum		3.5	3.6	3.5	3.6
A2.04	Reactor scram		3.7	3.8	3.7	3.8
A2.05	Generator trip		3.6	3.8	3.6	3.8
A2.06	Loss of extraction steam		2.9	3.1	2.9	3.1
A2.07	Loss of reactor/turbine pressure control system: Plant-Specific		3.8	3.9	3.8	3.9
A2.08	Turbine rotor bow		2.3	2.6	2.3	2.6
A2.09	Turbine vibration		2.5	2.8	2.5	2.8
A3	**Ability to monitor automatic operations of the MAIN TURBINE GENERATOR AND AUXILIARY SYSTEMS including:**	**(41.7 / 45.7)**				
A3.01	Turbine trip		3.6	3.6	3.6	3.6
A3.02	Turbine roll to rated speed		2.8	2.8	2.8	2.8
A3.03	Generator megawatt output		2.8	2.9	2.8	2.9
A3.04	Turbine speed		2.7*	2.8	2.7	2.8
A3.05	Control valve operation		3.0	3.1	3.0	3.1
A3.06	Turbine lube oil pressure		2.5	2.6	2.5	2.6
A3.07	Hydrogen seal oil pressure		2.5	2.6	2.5	2.6
A3.08	Hydrogen gas pressure		2.5	2.6	2.5	2.6
A3.09	Hydrogen gas temperature		2.4	2.5	2.4	2.5
A3.10	Generator output voltage/reactive load		2.5	2.6	2.5	2.6
A3.11	Generator power factor: Plant-Specific		1.8*	2.1*	1.8	2.1
A3.12	Automatic turbine control: Plant-Specific		3.3	3.5	3.3	3.5

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 245000

System Name: Main Turbine Generator and Auxiliary Systems

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I SRO
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Turbine lube oil pumps		2.7	2.7	2.7	2.7
A4.02	Generator controls		3.1	2.9	3.1	2.9
A4.03	Stator water cooling pumps: Plant-Specific		2.7	2.8	2.7	2.8
A4.04	Hydrogen seal oil pumps		2.7	2.7	2.7	2.7
A4.05	Generator megawatt output		2.7	2.7	2.7	2.7
A4.06	Turbine speed		2.7	2.6	2.7	2.6
A4.07	Turbine valve position		2.9	2.9	2.9	2.9
A4.08	Turbine oil pressure		2.7	2.0	2.7	2.0
A4.09	Hydrogen seal oil pressure		2.6	2.6	2.6	2.6
A4.10	Hydrogen gas pressure		2.6	2.6	2.6	2.6
A4.11	Hydrogen gas temperature		2.4	2.4*	2.4	2.4
A4.12	Generator output voltage		2.6	2.6	2.6	2.6
A4.13	Generator power factor: Plant-Specific		1.9*	2.1*	1.9	2.1
A4.14	Generator megavar output		2.5	2.5	2.5	2.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 256000

System Name: Reactor Condensate System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR CONDENSATE SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Main turbine		2.6	2.6	2.6	2.6
K1.02	Reactor feedwater system		3.3	3.3	3.3	3.3
K1.03	~~HPCI: Plant-Specific~~		3.5	3.5	3.5	3.5
K1.04	RCIC: Plant-Specific		3.2	3.3	3.2	3.3
K1.05	CRD hydraulics system		3.1	3.1	3.1	3.1
K1.06	Extraction steam system		2.7	2.7	2.7	2.7
K1.07	SJAE condenser		2.9	2.9	2.9	2.9
K1.08	Gland seal steam condenser: Plant-Specific		2.7	2.7	2.7	2.7
K1.09	Offgas condenser: Plant-Specific		2.9	3.0	2.9	3.0
K1.10	Exhaust hood spray system		2.4	2.5	2.4	2.5
K1.11	Plant air systems: Plant-Specific		2.4	2.5	2.4	2.5
K1.12	~~Isolation condenser: Plant-Specific~~		3.3*	3.3*	3.3	3.3
K1.13	Reactor water level		3.5	3.5	3.5	3.5
K1.14	~~RHR (LPCI): Plant-Specific~~		3.0	3.0	3.0	3.0
K1.15	HPCS: Plant-Specific		3.6	3.6	3.6	3.6
K1.16	RWCU		2.6	2.6	2.6	2.6
K1.17	~~ECCS keep fill system: Plant-Specific~~		2.9*	2.9	2.9	2.9
K1.18	Circulating water system		2.9	3.0	2.9	3.0
K1.19	Component cooling water (secondary equipment) systems: Plant-Specific		2.7	2.7	2.7	2.7
K1.20	Demineralized water storage and makeup system		2.5	2.6	2.5	2.6
K1.21	Steam seal evaporator: Plant-Specific		2.4	2.4	2.4	2.4
K1.22	Offgas system		2.8	2.8	2.8	2.8
K1.23	Auxiliary steam system: Plant-Specific		2.5	2.5	2.5	2.5
K1.24	Radwaste system: Plant-Specific		2.4	2.4	2.4	2.4
K1.25	Main steam system: Plant-Specific		3.0	3.1	3.0	3.1
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	System pumps		2.7*	2.8	2.7	2.8
K2.02	Motor operated valves		1.9*	2.0*	1.9	2.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 256000

System Name: Reactor Condensate System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR CONDENSATE SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Main turbine/main generator		3.2	3.2	3.2	3.2
K3.02	CRD hydraulics system		3.2	3.3	3.2	3.3
K3.03	Extraction steam system		2.6	2.6	2.6	2.6
K3.04	Reactor feedwater system		3.6	3.7	3.6	3.7
K3.05	~~HPCI: Plant-Specific~~		3.3	3.3	3.3	3.3
K3.06	RCIC: Plant-Specific		3.2	3.2	3.2	3.2
K3.07	~~Isolation condenser: Plant-Specific~~		3.0*	3.0*	3.0	3.0
K3.08	SJAE		2.8	2.8	2.8	2.8
K3.09	Offgas		2.8	2.8	2.8	2.8
K3.10	Gland seal steam system: Plant-Specific		2.5	2.5	2.5	2.5
K3.11	Reactor water level		3.9	3.9	3.9	3.9
K3.12	HPCS: Plant-Specific		3.3	3.3	3.3	3.3
K3.13	Main steam system		3.3	3.3	3.3	3.3
K4	**Knowledge of REACTOR CONDENSATE SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Condensate and/or booster pump auto start: Plant-Specific~~		3.4	3.4	3.4	3.4
K4.02	CRD pump suction		2.9	2.9	2.9	
K4.03	Condensate and/or booster pump protection		2.8	2.8	2.8	2.~
K4.04	Maintenance of water quality		2.7	2.7	2.7	2.7
K4.05	Maintenance of 100% system flow if a feedwater string isolates: Plant-Specific		3.0	3.0	3.0	3.0
K4.06	Control of extraction steam		2.8	2.8	2.8	2.8
K4.07	*Cascading heater drains*		2.4	2.4	2.4	2.4
K4.08	~~Dedicated ECCS water supply: Plant-Specific~~		3.6	3.6	3.6	3.6
K4.09	Initial main condenser vacuum		2.8	2.8	2.8	2.8
K4.10	Non-condensable gas removal		2.7	2.7	2.7	2.7
K4.11	Isolation of SJAE's on low flow: Plant-Specific		2.9	3.0	2.9	3.0
K5	**Knowledge of the operational implications of the following concepts as they apply to REACTOR CONDENSATE SYSTEM:**	**(41.5 / 45.3)**				
K5.01	System venting		2.5	2.5	2.5	2.5
K5.02	Water conductivity measurement		2.4*	2.4	2.4	2.4
K5.03	Heat exchanger level operation		2.6	2.7	2.6	2.7
K5.04	Ion exchange process		2.1*	2.1*	2.1	2.1
K5.05	Deaeration of condensate		2.2*	2.2	2.2	2.2
K5.06	Air operated valve operation		2.4	2.4	2.4	2.4
K5.07	Level controller operation		2.7	2.7	2.7	2.7
K5.08	Heat removal (transfer) mechanisms		2.6	2.7	2.6	2.7
K5.09	Pump cavitation		2.5	2.6	2.5	2.~
K5.10	Air ejection operation		2.8	2.8	2.8	

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 256000

System Name: Reactor Condensate System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR CONDENSATE SYSTEM:**	**(41.7 / 45.7)**				
K6.01	Plant air systems		2.8	2.8	2.8	2.8
K6.02	Circulating water system		3.1	3.1	3.1	3.1
K6.03	Extraction steam system		2.9	2.9	2.9	2.9
K6.04	A.C. power		2.8	2.8	2.8	2.8
K6.05	Component cooling water systems		2.9	2.9	2.9	2.9
K6.06	Reactor feedwater system		3.3	3.3	3.3	3.3
K6.07	Demineralized water storage and makeup system		2.4	2.4	2.4	2.4
K6.08	Main turbine		2.9	2.9	2.9	2.9
K6.09	Offgas system		2.6	2.6	2.6	2.6
K6.10	Main steam system		2.9	2.9	2.9	2.9
A1	**Ability to predict and/or monitor changes in parameters associated with operating the REACTOR CONDENSATE SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	System flow		2.9	2.9	2.9	2.9
A1.02	Pump amps		2.3*	2.3	2.3	2.3
A1.03	System pressure		2.8	2.8	2.8	2.8
A1.04	Hotwell level		2.9	2.9	2.9	2.9
A1.05	Condensate storage tank level		2.9	3.0	2.9	3.0
A1.06	Reactor water level		3.5	3.5	3.5	3.5
A1.07	System lineup		3.1	31	3.1	31
A1.08	System water quality		2.7	2.9	2.7	2.9
A1.09	Feedwater temperature		3.1	3.1	3.1	3.1
A1.10	Condenser vacuum		3.1	3.1	3.1	3.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 256000

System Name: Reactor Condensate System

		CFR	NRC Imp RO	NRC Imp SRO	Facility RO	Facility
A2	**Ability to (a) predict the impacts of the following on the REACTOR CONDENSATE SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Pump trips		3.3	3.3	3.3	3.3
A2.02	Valve closures		2.8	2.9	2.8	2.9
A2.03	Valve openings		2.8	2.9	2.8	2.9
A2.04	A.C. power failures		2.9	3.0	2.9	3.0
A2.05	Inadequate system flow		2.9	2.9	2.9	2.9
A2.06	Low hotwell level		3.2	3.2	3.2	3.2
A2.07	High hotwell level		2.9	2.9	2.9	2.9
A2.08	High feedwater heater level		3.1	3.1	3.1	3.1
A2.09	Low feedwater heater level		2.8	2.8	2.8	2.8
A2.10	Main turbine trip		3.1	3.1	3.1	3.1
A2.11	Loss of circulating water system		3.2	3.2	3.2	3.2
A2.12	Loss of equipment component cooling water systems		3.1	3.1	3.1	3.1
A2.13	Loss of applicable plant air systems		2.9	3.0	2.9	3.0
A2.14	Low Condensate storage tank level		3.3	3.4	3.3	3.4
A2.15	Abnormal water quality		2.8	3.1	2.8	3.1
A2.16	High demineralizer differential pressure		2.8	2.8	2.8	2.8
A2.17	Feedwater heater string trip: Plant-Specific		2.9	2.9	2.9	2.9
A2.18	Loss of SJAE		2.9	2.9	2.9	
A3	**Ability to monitor automatic operations of the REACTOR CONDENSATE SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Valve operation		2.7	2.7	2.7	2.7
A3.02	Pump starts		3.0	2.9	3.0	2.9
A3.03	System pressure		2.9	2.9	2.9	2.9
A3.04	System flow		3.0	3.0	3.0	3.0
A3.05	Lights and alarms		3.0	2.9	3.0	2.9
A3.06	Hotwell level		3.0	2.9	3.0	2.9
A3.07	Feedwater heater level		2.9	2.9	2.9	2.9
A3.08	Feedwater temperature		3.1	3.1	3.1	3.1
A3.09	Feedwater heater drain tank level: Plant-Specific		2.8	2.9	2.8	2.9

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 256000

System Name: Reactor Condensate System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Hotwell condensate/ condensate booster pumps		3.3	3.3	3.3	3.3
A4.02	System motor operated valves		2.8	2.8	2.8	2.8
A4.03	Hotwell level controls		3.2	3.1	3.2	3.1
A4.04	Minimum flow valves		2.8	2.7	2.8	2.7
A4.05	System flow		3.1	3.1	3.1	3.1
A4.06	System pressure		3.1	3.0	3.1	3.0
A4.07	Lights and alarms		2.9	2.9	2.9	2.9
A4.08	Reactor water level		3.7	3.7	3.7	3.7
A4.09	System water quality		2.9	3.1	2.9	3.1
A4.10	Feedwater temperature		3.2	3.2	3.2	3.2
A4.11	Condensate storage tank level		3.2	3.4	3.2	3.4
A4.12	Feedwater heater level: Plant-Specific		3.0	3.0	3.0	3.0
A4.13	Condenser vacuum		3.3	3.4	3.3	3.4
A4.14	Feedwater heater drain tank level: Plant-Specific		2.7	2.7	2.7	2.7
A4.15	Air ejectors: Plant-Specific		3.1	3.0	3.1	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 259001

System Name: Reactor Feedwater System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR FEEDWATER SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Reactor vessel		3.6	3.7	3.6	3.7
K1.02	~~HPCI: Plant-Specific~~		3.6	3.8	3.6	3.8
K1.03	RWCU		2.7	2.9	2.7	2.9
K1.04	Extraction steam system		2.8	2.9	2.8	2.9
K1.05	Condensate system		3.2	3.2	3.2	3.2
K1.06	Plant air systems		2.9	3.0	2.9	3.0
K1.07	A.C. electrical power		2.9	2.9	2.9	2.9
K1.08	Reactor water level control system		3.6	3.7	3.6	3.7
K1.09	Reactor water level		3.8	3.8	3.8	3.8
K1.10	Component cooling water systems		2.7	2.7	2.7	2.7
K1.11	RFP lube oil system		2.7	2.7	2.7	2.7
K1.12	RFP turbine seal steam system: TDRFP's-Only		2.6	2.6	2.6	2.6
K1.13	Main turbine generator: Plant-Specific		2.5	2.5	2.5	2.5
K1.14	RCIC: Plant-Specific		3.1	3.1	3.1	3.1
K1.15	RHR: Plant-Specific		3.0	3.1	3.0	3.1
K1.16	Recirculation		3.1	3.1	3.1	3.1
K1.17	Heater drains: Plant-Specific		2.4	2.4	2.4	2.4
K1.18	~~Fire protection system (emergency cooling): Plant-Specific~~		2.4	2.6	2.4	2.6
K1.19	~~†Redundant reactivity control system: Plant-Specific~~		3.0	3.3*	3.0	3.3
K1.20	Main steam system: TDRFPs-Only		3.1	3.2	3.1	3.2
K1.21	D.C. electric power		2.4	2.5	2.4	2.5
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Reactor feedwater pump(s): Motor-Driven-Only		3.3	3.3	3.3	3.3
K2.02	System motor operated valves		2.1*	2.2*	2.1	2.2
K2.03	RFP auxiliary oil pumps		2.3*	2.4	2.3	2.4
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR FEEDWATER SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor water level		3.9	3.9	3.9	3.9
K3.02	Reactor water level control system		3.8	3.8	3.8	3.8
K3.03	~~HPCI: Plant-Specific~~		3.3	3.3	3.3	3.3
K3.04	RWCU		2.5	2.5	2.5	2.5
K3.05	Recirculation pump NPSH		2.9	2.9	2.9	2.9
K3.06	Core inlet subcooling		3.1	3.1	3.1	3.1
K3.07	Condensate system		3.0	3.0	3.0	3.0
K3.08	RCIC		2.9	2.9	2.9	2.9
K3.09	Extraction steam system		2.5	2.6	2.5	2.6
K3.10	HPCS: Plant-Specific		3.2*	3.2*	3.2	3.2
K3.11	RHR: Plant-Specific		3.3*	3.3*	3.3	3.3
K3.12	Reactor power		3.8	3.9	3.8	3.9

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 259001

System Name: Reactor Feedwater System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
K4	**Knowledge of REACTOR FEEDWATER SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Auto start of the RFP's: Plant-Specific		3.8	4.0	3.8	4.0
K4.02	Feedwater heating		2.8	2.9	2.8	2.9
K4.03	RFP minimum flow		2.7	2.7	2.7	2.7
K4.04	Dispersal of feedwater in the reactor vessel		2.5	2.6	2.5	2.6
K4.05	RFP protection		2.7	2.8	2.7	2.8
K4.06	RFP lubrication		2.5	2.6	2.5	2.6
K4.07	RFP motor cooling: Motor-Driven-Only		2.5	2.6	2.5	2.6
K4.08	RFP turbine seals: TDRFP's-Only		2.3	2.4	2.3	2.4
K4.09	System isolation from the reactor vessel (check valves, double valve isolation inside/ outside containment)		3.2	3.3	3.2	3.3
K4.10	~~Feedpump runbacks: Plant-Specific~~		3.1	3.4	3.1	3.4
K4.11	Recirculation runbacks: Plant-Specific		3.5	3.5	3.5	3.5
K5	**Knowledge of the operational implications of the following concepts as they apply to REACTOR FEEDWATER SYSTEM:**	**(41.5 / 45.3)**				
K5.01	System venting		2.4*	2.4	2.4	2.4
K5.02	Water hammer		2.5	2.5	2.5	2.5
K5.03	Turbine operation: TDRFP's-Only		2.8	2.8	2.8	
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR FEEDWATER SYSTEM:**	**(41.7 / 45.7)**				
K6.01	Plant air systems		3.0	3.0	3.0	3.0
K6.02	Condensate system		3.3	3.4	3.3	3.4
K6.03	A.C. electrical power		2.9	3.1	2.9	3.1
K6.04	Extraction steam		2.8	2.9	2.8	2.9
K6.05	Component cooling water systems		2.7	2.7	2.7	2.7
K6.06	Plant service water		2.7	2.7	2.7	2.7
K6.07	Reactor water level control system		3.8	3.8	3.8	3.8
K6.08	Reactor feedwater pump motor ventilation: Motor-Driven-Only		2.1*	2.1*	2.1	2.1
K6.09	Reactor feedwater pump lube oil system		2.8	2.9	2.8	2.9
K6.10	RFP turbine seal system: TDRFP's-Only		2.5	2.5	2.5	2.5
K6.11	Main steam: TDRFP's-Only		2.8	2.8	2.8	2.8
K6.12	D.C. electrical power		2.3	2.4	2.3	2.4
K6.13	†Redundant reactivity control: Plant-Specific		3.0	3.3*	3.0	3.3
K6.14	Heater drains: Plant-Specific		2.5	2.6	2.5	2.6

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 259001

System Name: Reactor Feedwater System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the REACTOR FEEDWATER SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Feedwater flow/pressure		3.3	3.3	3.3	3.3
A1.02	Feedwater inlet temperature		3.2	3.3	3.2	3.3
A1.03	RFP motor amps: Motor-Driven-Only		2.8	2.8	2.8	2.8
A1.04	RFP turbine speed: Turbine-Driven-Only		2.8	2.7	2.8	2.7
A1.05	RFP turbine control valve position: Turbine-Driven-Only		2.8	2.7	2.8	2.7
A1.06	Feedwater heater level		2.7	2.7	2.7	2.7
A2	**Ability to (a) predict the impacts of the following on the REACTOR FEEDWATER SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Pump trip		3.7	3.7	3.7	3.7
A2.02	Feedwater heater isolation		3.1	3.3	3.1	3.3
A2.03	Loss of condensate pump(s)		3.6	3.6	3.6	3.6
A2.04	Loss of extraction steam		3.3	3.4	3.3	3.4
A2.05	Loss of applicable plant air systems		3.0	3.0	3.0	3.0
A2.06	Loss of A.C. electrical power		3.2	3.2	3.2	3.2
A2.07	Reactor water level control system malfunctions		3.7	3.8	3.7	3.8
A2.08	Loss of D.C. electrical power		2.5	2.6	2.5	2.6
A2.09	†TDRFP steam inlet pressure flow: Plant-Specific		2.6	2.7	2.6	2.7
A3	**Ability to monitor automatic operations of the REACTOR FEEDWATER SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	RFP auto start: Plant-Specific		3.3	3.5	3.3	3.5
A3.02	Motor amps: Motor-Driven-Only		2.4	2.5	2.4	2.5
A3.03	System flow		3.3	3.2	3.3	3.2
A3.04	Reactor water level		3.8	3.7	3.8	3.7
A3.05	Feedwater inlet temperature		3.1	3.2	3.1	3.2
A3.06	Pump discharge pressure		3.1	3.1	3.1	3.1
A3.07	FWRV position		3.2	3.2	3.2	3.2
A3.08	Turbine speed: TDRFP'S-Only		2.8	2.7	2.8	2.7
A3.09	Lights and alarms		3.0	3.0	3.0	3.0
A3.10	Pump trips		3.4	3.4	3.4	3.4
A3.11	~~Reactor feedpump runbacks: Plant-Specific~~		3.2	3.7	3.2	3.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 259001

System Name: Reactor Feedwater System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	System flow		3.6	3.5	3.6	3.5
A4.02	Manually start/control a RFP/TDRFP		3.9	3.7	3.9	3.7
A4.03	Feedwater heater/drain controls		2.9	3.0	2.9	3.0
A4.04	System valves		3.1	2.9	3.1	2.9
A4.05	Reactor water level		4.0	3.9	4.0	3.9
A4.06	Feedwater inlet temperature		3.4	3.5	3.4	3.5
A4.07	Pump discharge pressure		3.3	3.2	3.3	3.2
A4.08	FWRV position		3.3	3.3	3.3	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 259002

System Name: Reactor Water Level Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR WATER LEVEL CONTROL SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	RPS		3.8	3.9	3.8	3.9
K1.02	Main steam flow		3.2	3.3	3.2	3.3
K1.03	Reactor water level		3.8	3.9	3.8	3.9
K1.04	Reactor feedwater flow		3.5	3.6	3.5	3.6
K1.05	Reactor feedwater system		3.6	3.7	3.6	3.7
K1.06	Plant air systems		3.0	3.1	3.0	3.1
K1.07	~~Rod worth minimizer: Plant-Specific~~		2.6	2.7	2.6	2.7
K1.08	Recirculation system: Plant-Specific		3.2	3.2	3.2	3.2
K1.09	P sat/T sat (compensation)		2.9	3.0	2.9	3.0
K1.10	~~Emergency generator(s): FWCI/HPCI~~		2.9	3.0	2.9	3.0
K1.11	~~Drywell pressure: FWCI/HPCI~~		3.0	3.2	3.0	3.2
K1.12	~~Emergency condensate transfer pump: FWCI/HPCI~~		2.6*	2.6*	2.6	2.6
K1.13	Condensate system		3.2	3.2	3.2	3.2
K1.14	Main turbine		2.9	3.0	2.9	3.0
K1.15	Recirculation flow control system		3.2	3.2	3.2	3.2
K1.16	~~HPCI: Plant-Specific~~		3.4	3.5	3.4	3.5
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Reactor water level control system circuits		2.4*	2.7	2.4	2.7
K2.02	~~Feedwater coolant injection (FWCI) initiation logic: FWCI/HPCI~~		3.5*	3.5*	3.5	3.5
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR WATER LEVEL CONTROL SYSTEM will have on following:**	**(41.7 / 45.4 41.7 / 45.5 to 45.)**				
K3.01	Reactor water level		3.8	3.8	3.8	3.8
K3.02	Reactor feedwater system		3.7	3.7	3.7	3.7
K3.03	~~Rod worth minimizer: Plant-Specific~~		2.7	2.9	2.7	2.9
K3.04	Recirculation system: Plant-Specific		2.9	3.0	2.9	3.0
K3.05	Recirculation flow control system		2.8	2.9	2.8	2.9
K3.06	Main turbine		2.8	2.8	2.8	2.8
K3.07	Reactor water level indication		3.4*	3.4	3.4	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 259002

System Name: Reactor Water Level Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
K4	**Knowledge of REACTOR WATER LEVEL CONTROL SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Ensuring adequate NPSH for recirculation pumps: Plant-Specific		3.0	3.1	3.0	3.1
K4.02	~~Bypassing of the RWM: Plant-Specific~~		2.8	2.0	2.8	2.0
K4.03	Reactor feedpump runout protection: MDFP		2.8*	2.8	2.8	2.8
K4.04	Reactor water level setpoint setdown following a reactor scram		2.9	2.9	2.9	2.9
K4.05	*P sat/T sat (compensation)*		2.9	2.9	2.9	2.9
K4.06	Control signal failure		3.1	3.2	3.1	3.2
K4.07	~~TDRFP 20% power interlock: TDRFP~~		3.1	3.3	3.1	3.3
K4.08	TDRFP speed control: TDRFP		2.9	3.0	2.9	3.0
K4.09	Single element control (reactor water level provides the only input)		3.1	3.1	3.1	3.1
K4.10	Three element control (main steam flow, reactor feedwater flow and reactor water level provide input)		3.4	3.4	3.4	3.4
K4.11	DP control: Plant-Specific		3.3	3.3	3.3	3.3
K4.12	Manual and automatic control of the system		3.5	3.4	3.5	3.4
K4.13	FWRV lockup		3.5	3.6	3.5	3.6
K4.14	Selection of various instruments to provide reactor water level input		3.4	3.4	3.4	3.4
K4.15	~~Automatic initiation of the feedwater system upon receipt of an ECCS initiation signal: FWCI/HPCI~~		4.5*	4.5*	4.5	
K4.16	~~Dedication of feedwater string(s) to ECCS: FWCI/HPCI~~		4.5*	4.5*	4.5	4.5
K4.17	Simultaneous Manual and Auto operation of the system (i.e. 1 FP in Auto, 1 FP in Manual)		3.1	3.0	3.1	3.0
K5	**Knowledge of the operational implications of the following concepts as they apply to REACTOR WATER LEVEL CONTROL SYSTEM:**	**(41.5 / 45.3)**				
K5.01	GEMAC/Foxboro/Bailey controller operation: Plant-Specific		3.1	3.1	3.1	3.1
K5.02	Electro/ Pneumatic converter operation		2.2*	2.4	2.2	2.4
K5.03	Water level measurement		3.1	3.2	3.1	3.2
K5.04	Moisture carryover		2.3	2.3	2.3	2.3
K5.05	Moisture carryunder		2.3	2.3	2.3	2.3
K5.06	Pump runout		2.4	2.4	2.4	2.4
K5.07	*Turbine speed control mechanisms:* TDRFP		2.7	2.7	2.7	2.7
K5.08	~~Heat removal mechanisms: FWCI~~		3.6	3.8	3.6	3.8
K5.09	~~†Adequate core cooling: FWCI~~		3.8	4.2*	3.8	4.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 259002

System Name: Reactor Water Level Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR WATER LEVEL CONTROL SYSTEM:**	**(41.7 / 45.7)**				
K6.01	Plant air systems		3.2	3.2	3.2	3.2
K6.02	A.C. power		3.3	3.4	3.3	3.4
K6.03	Main steam flow input		3.1	3.1	3.1	3.1
K6.04	Reactor feedwater flow input		3.1	3.1	3.1	3.1
K6.05	Reactor water level input		3.5	3.5	3.5	3.5
K6.06	Reactor pressure/temperature input (for water level input compensation)		2.6	2.7	2.6	2.7
K6.07	~~Drywell pressure input: FWCI~~		3.0*	3.0*	3.0	3.0
K6.08	Loss of dP across startup level control bypass valve: Plant-Specific		2.7	2.7	2.7	2.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the REACTOR WATER LEVEL CONTROL SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Reactor water level		3.8	3.8	3.8	3.8
A1.02	Reactor feedwater flow		3.6	3.5	3.6	3.5
A1.03	Reactor power		3.8	3.8	3.8	3.8
A1.04	Reactor water level control controller indications		3.6	3.6	3.6	3.6
A1.05	FWRV/startup level control position: Plant-Specific		2.9	2.9	2.9	2.9
A1.06	~~Feedwater string(s) selected for FWCI: FWCI~~		3.3*	3.3*	3.3	3.3
A1.07	TDRFP speed: TDRFP		2.6	2.6	2.6	2.6
A2	**Ability to (a) predict the impacts of the following on the REACTOR WATER LEVEL CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Loss of any number of main steam flow inputs		3.3	3.4	3.3	3.4
A2.02	Loss of any number of reactor feedwater flow inputs		3.3	3.4	3.3	3.4
A2.03	Loss of reactor water level input		3.6	3.7	3.6	3.7
A2.04	RFP runout condition: Plant-Specific		3.0	3.1	3.0	3.1
A2.05	Loss of applicable plant air systems		3.2	3.4	3.2	3.4
A2.06	Loss of controller signal output		3.3	3.4	3.3	3.4
A2.07	Loss of comparator bias signal		2.4	2.5	2.4	2.5
A2.08	~~Receipt of an ECCS initiation signal: FWCI~~		4.5*	4.5*	4.5	4.5
A2.09	~~FWCI system failure alarm: FWCI~~		4.0	4.0	4.0	4.0

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 259002

System Name: Reactor Water Level Control System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
A3	**Ability to monitor automatic operations of the REACTOR WATER LEVEL CONTROL SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Runout flow control: Plant-Specific		3.0*	3.0*	3.0	3.0
A3.02	Changes in reactor water level		3.4	3.4	3.4	3.4
A3.03	Changes in main steam flow		3.2	3.2	3.2	3.2
A3.04	Changes in reactor feedwater flow		3.2	3.2	3.2	3.2
A3.05	Changes in reactor power		3.4	3.4	3.4	3.4
A3.06	Reactor water level setpoint setdown following a reactor scram: Plant-Specific		3.0	3.0	3.0	3.0
A3.07	FWRV lockup		3.5	3.6	3.5	3.6
A3.08	~~FWCI system initiation: FWCI~~		4.0	4.0	4.0	4.0
A3.09	Transfer of system from flow control to level control mode: FWCI		4.3*	4.0	4.3	4.0
A3.10	TDRFP lockup: TDRFP		3.1	3.0	3.1	3.0
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	All individual component controllers in the manual mode		3.8	3.6	3.8	3.6
A4.02	All individual component controllers in the automatic mode		3.7	3.6	3.7	3.6
A4.03	All individual component controllers when transferring from manual to automatic modes		3.8	3.6	3.8	3.6
A4.04	FWRV lockup reset controls		3.7	3.6	3.7	3.[illegible]
A4.05	Runout flow control reset controls: Plant-Specific		3.8	3.5	3.8	3.5
A4.06	DP/Single/three element control selector switch: Plant-Specific		3.1	3.2	3.1	3.2
A4.07	All individual component controllers when transferring from automatic to manual mode		3.8	3.6	3.8	3.6
A4.08	~~Manually initiate FWCI: FWCI~~		4.5*	4.5*	4.5	4.5
A4.09	TDRFP lockout reset: TDRFP		3.4	3.1	3.4	3.1
A4.10	Setpoint setdown reset controls: Plant-Specific		3.1	2.9	3.1	2.9
A4.11	High level lockout reset controls: Plant-Specific		3.5	3.3	3.5	3.3

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 261000

System Name: Standby Gas Treatment System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between STANDBY GAS TREATMENT SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Reactor building ventilation system		3.4	3.6	3.4	3.6
K1.02	Drywell		3.2	3.4	3.2	3.4
K1.03	Suppression pool		2.9	3.1	2.9	3.1
K1.04	High radiation sampling system		2.5	2.8	2.5	2.8
K1.05	Radwaste system: Plant-Specific		2.3	2.4	2.3	2.4
K1.06	~~High pressure coolant injection system: Plant-Specific~~		3.0	3.1	3.0	3.1
K1.07	Elevated release stack		3.1	3.2	3.1	3.2
K1.08	Process radiation monitoring system		2.8	3.1	2.8	3.1
K1.09	Primary containment isolation system		3.2	3.4	3.2	3.4
K1.10	Plant air systems		2.2*	2.3	2.2	2.3
K1.11	Primary containment pressure		3.2	3.3	3.2	3.3
K1.12	Primary containment purge system: Plant-Specific		3.1	3.2	3.1	3.2
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Fan power		2.1*	2.3*	2.1	2.3
K2.02	Valve power		1.9*	2.0*	1.9	2.0
K2.03	Initiation logic		2.3*	2.5*	2.3	2.5
K2.04	Heater power		1.7*	1.9*	1.7	1.9
K3	**Knowledge of the effect that a loss or malfunction of the STANDBY GAS TREATMENT SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	Secondary containment and environment differential pressure		3.3	3.6	3.3	3.6
K3.02	Off-site release rate		3.6	3.9	3.6	3.9
K3.03	~~Primary containment pressure: Mark-I&II~~		3.2	3.4	3.2	3.4
K3.04	~~High pressure coolant injection system: Plant-Specific~~		3.1	3.1	3.1	3.1
K3.05	Secondary containment radiation/ contamination levels		3.2	3.5	3.2	3.5
K3.06	~~Primary containment oxygen content: Mark-I&II~~		3.0	3.3	3.0	3.3
K4	**Knowledge of STANDBY GAS TREATMENT SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Automatic system initiation		3.7	3.8	3.7	3.8
K4.02	Charcoal bed decay heat removal		2.6	2.8	2.6	2.8
K4.03	Moisture removal		2.5	2.7	2.5	2.7
K4.04	Radioactive particulate filtration		2.7	2.9	2.7	2.9
K4.05	Fission product gas removal		2.6	2.8	2.6	2.8
K4.06	Charcoal bed retention		2.4*	2.6	2.4	2.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 261000

System Name: Standby Gas Treatment System

			CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
K5	**Knowledge of the operational implications of the following concepts as they apply to STANDBY GAS TREATMENT SYSTEM:**	**(41.5 / 45.3)**					
K5.01	Heat removal mechanisms			2.3*	2.6*	2.3	2.6
K5.02	Air operated valves: Plant-Specific			2.3*	2.5*	2.3	2.5
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the STANDBY GAS TREATMENT SYSTEM:**	**(41.7 / 45.7)**					
K6.01	A.C. electrical distribution			2.9	3.0	2.9	3.0
K6.02	D.C. electrical distribution			2.4*	2.6*	2.4	2.6
K6.03	Emergency diesel generator system			3.0	3.1	3.0	3.1
K6.04	Process radiation monitoring			2.9	3.1	2.9	3.1
K6.05	Reactor protection system: Plant-Specific			3.1	3.2	3.1	3.2
K6.06	Plant air systems			2.4	2.6	2.4	2.6
K6.07	Primary containment atmosphere sampling system: Plant-Specific			2.2*	2.4	2.2	2.4
K6.08	Reactor vessel level: Plant-Specific			3.1	3.1	3.1	3.1
K6.09	Primary containment high pressure: Plant-Specific			3.1	3.3	3.1	3.3
A1	**Ability to predict and/or monitor changes in parameters associated with operating the STANDBY GAS TREATMENT SYSTEM controls including:**	**(41.5 / 45.5)**					
A1.01	System flow			2.9	3.1	2.9	3.1
A1.02	Primary containment pressure			3.1	3.2	3.1	3.2
A1.03	†Off-site release levels			3.2	3.8	3.2	3.8
A1.04	Secondary containment differential pressure			3.0	3.3	3.0	3.3
A1.05	~~Primary containment oxygen level: Mark-I&II~~			2.7*	2.9*	2.7	2.9
A1.06	~~Drywell and suppression chamber differential pressure: Mark-I~~			2.7	3.0	2.7	3.0
A1.07	SBGTS train temperature			2.8	2.9	2.8	2.9

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 261000

System Name: Standby Gas Treatment System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the STANDBY GAS TREATMENT SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Low system flow		2.9	3.1	2.9	3.1
A2.02	High system flow		2.9	3.1	2.9	3.1
A2.03	High train temperature		2.9	3.2	2.9	3.2
A2.04	High train moisture content		2.5	2.7	2.5	2.7
A2.05	Fan trips		3.0	3.1	3.0	3.1
A2.06	Valve closures		2.9*	2.9*	2.9	2.9
A2.07	A.C. electrical failure		2.7*	2.8	2.7	2.8
A2.08	D.C. electrical failure		2.4*	2.7*	2.4	2.7
A2.09	Plant air system failure		2.4*	2.6*	2.4	2.6
A2.10	Low reactor water level: Plant-Specific		3.1	3.2	3.1	3.2
A2.11	High containment pressure		3.2	3.3	3.2	3.3
A2.12	High fuel pool ventilation radiation: Plant-Specific		3.2	3.4	3.2	3.4
A2.13	High secondary containment ventilation exhaust radiation		3.4	3.7	3.4	3.7
A2.14	High system pressure: Plant-Specific		3.0	3.2	3.0	3.2
A2.15	High area radiation by refuel bridge: Plant-Specific		3.0	3.4	3.0	3.4
A3	**Ability to monitor automatic operations of the STANDBY GAS TREATMENT SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	System flow		3.2	3.3	3.2	3.3
A3.02	Fan start		3.2	3.1	3.2	3.1
A3.03	Valve operation		3.0	2.9	3.0	2.9
A3.04	System temperature		3.0	3.1	3.0	3.1
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	†Off-site release levels: Plant-Specific		3.2*	4.0*	3.2	4.0
A4.02	Suction valves		3.1	3.1	3.1	3.1
A4.03	Fan		3.0	3.0	3.0	3.0
A4.04	Primary containment pressure		3.3	3.4	3.3	3.4
A4.05	~~Drywell to suppression chamber/torus differential pressure: Mark-I,II~~		2.9	3.2	2.9	3.2
A4.06	Reactor building differential pressure		3.3	3.6	3.3	3.6
A4.07	System flow		3.1	3.2	3.1	3.2
A4.08	System temperature		2.6	2.7	2.6	2.7
A4.09	Ventilation valves/dampers		2.7	2.7	2.7	2.7

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: **262001**

System Name: **A.C. Electrical Distribution**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between A.C. ELECTRICAL DISTRIBUTION and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Emergency generators (diesel/jet)		3.8	4.3*	3.8	4.3
K1.02	D.C. electrical distribution		3.3	3.6	3.3	3.6
K1.03	Off-site power sources		3.4	3.8	3.4	3.8
K1.04	Uninterruptible power supply		3.1	3.4	3.1	3.4
K1.05	Main turbine/generator		3.0	3.2	3.0	3.2
K1.06	Alternate shutdown system: Plant-Specific		3.6	3.9	3.6	3.9
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Off-site sources of power		3.3	3.6	3.3	3.6
K3	**Knowledge of the effect that a loss or malfunction of the A.C. ELECTRICAL DISTRIBUTION will have on following:**	**(41.7 / 45.4)**				
K3.01	Major system loads		3.5	3.7	3.5	3.7
K3.02	Emergency generators		3.8	4.2	3.8	4.2
K3.03	D.C. electrical distribution		2.9	3.2	2.9	3.2
K3.04	Uninterruptible power supply		3.1	3.3	3.1	3.3
K3.05	Off-site power system		3.2	3.5	3.2	3.5
K3.06	Reactor protection system		3.8	4.1*	3.8	4.1
K4	**Knowledge of A.C. ELECTRICAL DISTRIBUTION design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Bus lockouts		3.0	3.4	3.0	3.4
K4.02	Circuit breaker automatic trips		2.9	3.3	2.9	3.3
K4.03	Interlocks between automatic bus transfer and breakers		3.1	3.4	3.1	3.4
K4.04	Protective relaying		2.8	3.1	2.8	3.1
K4.05	Paralleling of A.C. sources (synchroscope)		3.4	3.6	3.4	3.6
K4.06	Redundant power sources to vital buses		3.6	3.9	3.6	3.9
K5	**Knowledge of the operational implications of the following concepts as they apply to A.C. ELECTRICAL DISTRIBUTION:**	**(41.5 / 45.3)**				
K5.01	Principle involved with paralleling two A.C. sources		3.1	3.4	3.1	3.4
K5.02	Breaker control		2.6	2.9	2.6	2.9

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 262001

System Name: A.C. Electrical Distribution

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the A.C. ELECTRICAL DISTRIBUTION:**	**(41.7 / 45.7)**				
K6.01	D.C. power		3.1	3.4	3.1	3.4
K6.02	Off-site power		3.6	3.9	3.6	3.9
K6.03	Generator trip		3.5	3.7	3.5	3.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the A.C. ELECTRICAL DISTRIBUTION controls including:**	**(41.5 / 45.5)**				
A1.01	Effect on instrumentation and controls of switching power supplies		3.1	3.4	3.1	3.4
A1.02	Effects of loads when energizing a bus		3.1	3.5	3.1	3.5
A1.03	Bus voltage		2.9	3.1	2.9	3.1
A1.04	Load currents		2.7	2.9	2.7	2.9
A1.05	Breaker lineups		3.2	3.5	3.2	3.5
A2	**Ability to (a) predict the impacts of the following on the A.C. ELECTRICAL DISTRIBUTION; and (b) based on those predictions, use procedures to correct, control, or *mitigate the consequences of those abnormal conditions* or operations:**	**(41.5 / 45.6)**				
A2.01	Turbine/generator trip		3.4	3.6	3.4	3.6
A2.02	Loss of coolant accident		3.6	3.9	3.6	3.9
A2.03	Loss of off-site power		3.9	4.3*	3.9	4.3
A2.04	Types of loads that, if deenergized, would degrade or hinder plant operation		3.8	4.2	3.8	4.2
A2.05	Bus grounds		2.9	3.3	2.9	3.3
A2.06	Deenergizing a plant bus		2.7	2.9	2.7	2.9
A2.07	Energizing a dead bus		3.0	3.2	3.0	3.2
A2.08	Opening a disconnect under load		3.3	3.6	3.3	3.6
A2.09	Exceeding voltage limitations		3.1	3.4	3.1	3.4
A2.10	Exceeding current limitations		2.9	3.4	2.9	3.4
A2.11	Degraded system voltages		3.2	3.6	3.2	3.6
A3	**Ability to monitor automatic operations of the A.C. ELECTRICAL DISTRIBUTION including:**	**(41.7 / 45.7)**				
A3.01	Breaker tripping		3.1	3.2	3.1	3.2
A3.02	Automatic bus transfer		3.2	3.3	3.2	3.3
A3.03	Load shedding		3.4	3.5	3.4	3.5
A3.04	Load sequencing		3.4	3.6	3.4	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: **262001**

System Name: **A.C. Electrical Distribution**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	All breakers and disconnects (including available switch yard): Plant-Specific		3.4	3.7	3.4	3.7
A4.02	Synchroscope, including understanding of running and incoming voltages		3.4	3.4	3.4	3.4
A4.03	Local operation of breakers		3.2	3.4	3.2	3.4
A4.04	Synchronizing and paralleling of different A.C. supplies		3.6	3.7	3.6	3.7
A4.05	Voltage, current, power, and frequency on A.C. buses		3.3	3.3	3.3	3.3

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 262002

System Name: Uninterruptable Power Supply (A.C./D.C.)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Feedwater level control: Plant-Specific		2.8	3.0	2.8	3.0
K1.02	RFPT control: Plant-Specific		2.8	3.0	2.8	3.0
K1.03	Rod position information: Plant-Specific		2.7	2.9	2.7	2.9
K1.04	~~Reactor manual control: Plant-Specific~~		2.8	3.0	2.8	3.0
K1.05	Reactor/turbine pressure control system control unit: Plant-Specific		2.7	2.9	2.7	2.9
K1.06	Unit computer: Plant-Specific		2.6	2.7	2.6	2.7
K1.07	~~Rod worth minimizer: Plant-Specific~~		2.8	2.9	2.8	2.9
K1.08	Containment isolation system: Plant-Specific		2.9	3.1	2.9	3.1
K1.09	~~Drywell ventilation control: Plant-Specific~~		2.5	2.7	2.5	2.7
K1.10	~~Fire protection system: Plant-Specific~~		2.6	2.8	2.6	2.8
K1.11	Control room recorders: Plant-Specific		2.5	2.6	2.5	2.6
K1.12	Generator hydrogen and stator cooling water controls: Plant-Specific		2.1*	2.3*	2.1	2.3
K1.13	~~Recirculation pump speed control: Plant-Specific~~		2.5	2.6	2.5	2.6
K1.14	Main steam line radiation monitors: Plant-Specific		2.8	3.0	2.8	3.0
K1.15	~~Stack gas monitors: Plant-Specific~~		2.7	3.0	2.7	3.0
K1.16	MSIV's: Plant-Specific		3.1	3.2	3.1	3.2
K1.17	Scram solenoid valves: Plant-Specific		3.1	3.3	3.1	3.3
K1.18	Process radiation monitoring system: Plant-Specific		2.5	2.7	2.5	2.7
K1.19	Power range neutron monitoring system: Plant-Specific		2.9*	3.1*	2.9	3.1
K1.20	~~Plant communications equipment: Plant-Specific~~		2.4	2.7	2.4	2.7
K3	**Knowledge of the effect that a loss or malfunction of the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) will have on following:**	**(41.7 / 45.4)**				
K3.01	Water level control: Plant-Specific		3.1	3.3	3.1	3.3
K3.02	~~Recirculation pump speed: Plant-Specific~~		2.9	2.9	2.9	2.9
K3.03	RFPT speed: Plant-Specific		3.0	3.1	3.0	3.1
K3.04	~~Fire protection system: Plant-Specific~~		2.5	2.7	2.5	2.7
K3.05	~~Rod worth minimizer: Plant-Specific~~		2.9	3.0	2.9	3.0
K3.06	Rod position indication: Plant-Specific		2.8	2.9	2.8	2.9
K3.07	Movement of control rods: Plant-Specific		2.6	2.8	2.6	2.8
K3.08	Computer operation: Plant-Specific		2.7	2.8	2.7	2.8
K3.09	~~Drywell ventilation control: Plant-Specific~~		2.4	2.7	2.4	2.7
K3.10	Containment isolation: Plant-Specific		2.7	2.8	2.7	2.8
K3.11	MSIV's: Plant-Specific		2.8	2.9	2.8	2.9
K3.12	Control rod drive mechanism: Plant-Specific		2.3*	2.5	2.3	2.5
K3.13	Rx pressure: Plant-Specific		2.7	2.9	2.7	2.9
K3.14	Rx power: Plant-Specific		2.8	3.1	2.8	3.1
K3.15	Main turbine operation: Plant-Specific		2.6	2.7	2.6	2.7
K3.16	Control room recorders: Plant-Specific		2.4	2.6	2.4	2.6
K3.17	Process monitoring: Plant-Specific		2.9	3.1	2.9	3.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 262002

System Name: **Uninterruptable Power Supply (A.C./D.C.)**

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
K4	**Knowledge of UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Transfer from preferred power to alternate power supplies		3.1	3.4	3.1	3.4
K5	**Knowledge of the operational implications of the following concepts as they apply to UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.):**	**(41.5 / 45.3)**				
K5.01	General principles of static inverter operation		2.3*	2.3	2.3	2.3
K5.02	~~General principles of motor generator operation: Plant-Specific~~		2.4	2.5	2.4	2.5
K5.03	~~General principles of inertia fly wheel operation: Plant-Specific~~		2.3*	2.4*	2.3	2.4
K5.04	General principles of static switch operation		2.1*	2.2*	2.1	2.2
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.):**	**(41.7 / 45.7)**				
K6.01	A.C. electrical power		2.7	2.9	2.7	2.9
K6.02	D.C. electrical power		2.8	3.1	2.8	3.1
K6.03	Static inverter		2.7	2.9	2.7	2.9
A1	**Ability to predict and/or monitor changes in parameters associated with operating the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) controls including:**	**(41.5 / 45.5)**				
A1.01	Inverter electrical outputs		2.4	2.6	2.4	2.6
A1.02	~~Motor generator outputs~~		2.5	2.9	2.5	2.9
A2	**Ability to (a) predict the impacts of the following on the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Under voltage		2.6	2.8	2.6	2.8
A2.02	Over voltage		2.5	2.7	2.5	2.7
A2.03	Frequency changes in the system		2.4*	2.6	2.4	2.6
A2.04	~~Abnormal battery operation: BWR-1~~		3.2	3.4	3.2	3.4
A3	**Ability to monitor automatic operations of the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) including:**	**(41.7 / 45.7)**				
A3.01	Transfer from preferred to alternate source		2.8	3.1	2.8	3.1

KA Catalog

Facility: CPS

Printed: 08/11/2006

System Number: 262002

System Name: Uninterruptable Power Supply (A.C./D.C.)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Transfer from alternative source to preferred source		2.8	3.1	2.8	3.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 263000

System Name: D.C. Electrical Distribution

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between D.C. ELECTRICAL DISTRIBUTION and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	A.C. electrical distribution		3.3	3.5	3.3	3.5
K1.02	Battery charger and battery		3.2	3.3	3.2	3.3
K1.03	Battery ventilation		2.6	2.8	2.6	2.8
K1.04	Ground detection		2.6	2.9	2.6	2.9
K2	***Knowledge of electrical power supplies to the following:***	**(41.7)**				
K2.01	Major D.C. loads		3.1	3.4	3.1	3.4
K2.02	*Battery room ventilation*		2.1*	2.2*	2.1	2.2
K3	**Knowledge of the effect that a loss or malfunction of the D.C. ELECTRICAL DISTRIBUTION will have on following:**	**(41.7 / 45.4)**				
K3.01	Emergency generators: Plant-Specific		3.4	3.8	3.4	3.8
K3.02	Components using D.C. control power (i.e. breakers)		3.5	3.8	3.5	3.8
K3.03	Systems with D.C. components (i.e. valves, motors, solenoids, etc.)		3.4	3.8	3.4	3.8
4	**Knowledge of D.C. ELECTRICAL DISTRIBUTION design feature(s) and/or *interlocks which provide for the* following:**	**(41.7)**				
K4.01	Manual/ *automatic transfers of control:* Plant-Specific		3.1	3.4	3.1	3.4
K4.02	Breaker interlocks, permissives, bypasses and cross ties: *Plant-Specific*		3.1	3.5	3.1	3.5
K5	**Knowledge of the operational implications of the following concepts as they apply to D.C. ELECTRICAL DISTRIBUTION:**	**(41.5 / 45.3)**				
K5.01	Hydrogen generation during battery charging		2.6	2.9	2.6	2.9
K6	**Knowledge of the effect that a *loss or malfunction of the* following will have on the D.C. ELECTRICAL DISTRIBUTION:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical distribution		3.2	3.5	3.2	3.5
K6.02	Battery ventilation		2.5	2.6	2.5	2.6
A1	**Ability to predict and/or monitor changes in parameters associated with operating the D.C. ELECTRICAL DISTRIBUTION controls including:**	**(41.5 / 45.5)**				
A1.01	Battery charging/discharging rate		2.5	2.8	2.5	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 263000

System Name: D.C. Electrical Distribution

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir
A2	**Ability to (a) predict the impacts of the following on the D.C. ELECTRICAL DISTRIBUTION; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Grounds		2.8	3.2	2.8	3.2
A2.02	Loss of ventilation during charging		2.6	2.9	2.6	2.9
A3	**Ability to monitor automatic operations of the D.C. ELECTRICAL DISTRIBUTION including:**	**(41.7 / 45.7)**				
A3.01	Meters, dials, recorders, alarms, and indicating lights		3.2	3.3	3.2	3.3
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Major breakers and control power fuses: Plant-Specific		3.3	3.5	3.3	3.5
A4.02	Battery voltage indicator: Plant-Specific		3.2	3.1	3.2	3.1
A4.03	Battery discharge rate: Plant-Specific		2.7	2.8	2.7	2.8
A4.04	Ground detection circuit: Plant-Specific		3.0	3.2	3.0	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 264000

System Name: Emergency Generators (Diesel/Jet)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between EMERGENCY GENERATORS (DIESEL/JET) and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	A.C. electrical distribution		3.8	4.1	3.8	4.1
K1.02	D.C. electrical distribution		3.3	3.4	3.3	3.4
K1.03	Fire protection system		2.9	3.2	2.9	3.2
K1.04	Emergency generator cooling water system		3.2	3.3	3.2	3.3
K1.05	Emergency generator fuel oil supply system		3.2	3.3	3.2	3.3
K1.06	Starting system		3.2	3.2	3.2	3.2
K1.07	Emergency core cooling systems		3.9	4.1	3.9	4.1
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Air compressor		2.2*	2.4*	2.2	2.4
K2.02	Fuel oil pumps		2.2*	2.4*	2.2	2.4
K2.03	~~Turning gear (jet engine): Plant-Specific~~		2.0*	2.3*	2.0	2.3
K2.04	~~Ignition system (jet engine): Plant-Specific~~		2.3*	2.7*	2.3	2.7
K2.05	Lube oil pumps		2.3*	2.4*	2.3	2.4
K2.06	Battery charger		2.2*	2.2*	2.2	2.2
K2.07	Emergency generator field flash		2.1*	2.3*	2.1	2.3
K3	**Knowledge of the effect that a loss or malfunction of the EMERGENCY GENERATORS (DIESEL/JET) will have on following:**	**(41.7 / 45.4)**				
K3.01	Emergency core cooling systems		4.2*	4.4*	4.2	4.4
K3.02	A.C. electrical distribution		3.9	4.0	3.9	4.0
K3.03	Major loads powered from electrical buses fed by the emergency generator(s)		4.1*	4.2*	4.1	4.2
K4	**Knowledge of EMERGENCY GENERATORS (DIESEL/JET) design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Emergency generator trips (normal)		3.5	3.7	3.5	3.7
K4.02	Emergency generator trips (emergency/LOCA)		4.0	4.2	4.0	4.2
K4.03	Speed droop control		2.5	2.7	2.5	2.7
K4.04	Field flashing		2.6	2.7	2.6	2.7
K4.05	Load shedding and sequencing		3.2	3.5	3.2	3.5
K4.06	Governor control		2.6	2.7	2.6	2.7
K4.07	Local operation and control		3.3	3.4	3.3	3.4
K4.08	Automatic startup		3.8	3.7	3.8	3.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 264000

System Name: Emergency Generators (Diesel/Jet)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir
K5	**Knowledge of the operational implications of the following concepts as they apply to EMERGENCY GENERATORS (DIESEL/JET):**	**(41.5 / 45.3)**				
K5.01	Definition of frequency and synchronous frequency		2.0*	2.1*	2.0	2.1
K5.02	Reactive power control		2.0*	2.1*	2.0	2.1
K5.03	Real power control		2.4*	2.4*	2.4	2.4
K5.04	Governor control		2.4	2.5	2.4	2.5
K5.05	Paralleling A.C. power sources		3.4	3.4	3.4	3.4
K5.06	Load sequencing		3.4	3.5	3.4	3.5
K5.07	Speed droop		2.3	2.4	2.3	2.4
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the EMERGENCY GENERATORS (DIESEL/JET):**	**(41.7 / 45.7)**				
K6.01	Starting air		3.8	3.9	3.8	3.9
K6.02	Fuel oil pumps		3.6	3.6	3.6	3.6
K6.03	Lube oil pumps		3.5	3.7	3.5	3.7
K6.04	~~Turning gear (jet engine): Plant-Specific~~		2.5	2.5	2.5	2.5
K6.05	~~Ignition system (jet engine): Plant-Specific~~		2.5	3.0	2.5	3.0
K6.06	Battery charger		2.9	3.1	2.9	3.1
K6.07	Cooling water system		3.8	3.9	3.8	3.9
K6.08	A.C. power		3.6	3.7	3.6	3
K6.09	D.C. power		3.3	3.5	3.3	3.
A1	**Ability to predict and/or monitor changes in parameters associated with operating the EMERGENCY GENERATORS (DIESEL/JET) controls including:**	**(41.5 / 45.5)**				
A1.01	Lube oil temperature		3.0*	3.0*	3.0	3.0
A1.02	Fuel consumption rate		2.2*	2.3	2.2	2.3
A1.03	Operating voltages, currents, and temperatures		2.8	2.9	2.8	2.9
A1.04	Crank case temperature and pressure		2.6	2.7	2.6	2.7
A1.05	Cylinder temperature differential		2.4*	2.4*	2.4	2.4
A1.06	Emergency generator room temperature		2.3	2.4	2.3	2.4
A1.07	~~Gas generator temperature: Plant-Specific~~		2.0*	2.5	2.0	2.5
A1.08	~~Gas generator speed: Plant-Specific~~		2.0*	2.5	2.0	2.5
A1.09	Maintaining minimum load on emergency generator (to prevent reverse power)		3.0	3.1	3.0	3.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 264000

System Name: Emergency Generators (Diesel/Jet)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the EMERGENCY GENERATORS (DIESEL/JET); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Parallel operation of emergency generator		3.5	3.6	3.5	3.6
A2.02	Unloading prior to securing emergency generator		3.1	3.1	3.1	3.1
A2.03	Operating unloaded, lightly loaded, and highly loaded		3.4	3.4	3.4	3.4
A2.04	Consequences of operating under/over excited		2.9	3.0	2.9	3.0
A2.05	Synchronization of the emergency generator with other electrical supplies		3.6	3.6	3.6	3.6
A2.06	Opening normal and/or alternate power to emergency bus		3.4	3.4	3.4	3.4
A2.07	Loss of off-site power during full-load testing		3.5	3.7	3.5	3.7
A2.08	Initiation of emergency generator room fire protection system		3.3	3.7	3.3	3.7
A2.09	Loss of A.C. power		3.7	4.1	3.7	4.1
A2.10	LOCA		3.9	4.2*	3.9	4.2
A3	**Ability to monitor automatic operations of the EMERGENCY GENERATORS (DIESEL/JET) including:**	**(41.7 / 45.7)**				
A3.01	Automatic starting of compressor and emergency generator		3.0	3.1	3.0	3.1
A3.02	Minimum time for load pick up		3.1	3.1	3.1	3.1
A3.03	Indicating lights, meters, and recorders		3.4	3.4	3.4	3.4
A3.04	Operation of the governor control system on frequency and voltage control		3.1	3.1	3.1	3.1
A3.05	Load shedding and sequencing		3.4	3.5	3.4	3.5
A3.06	Cooling water system operation		3.1	3.2	3.1	3.2
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Adjustment of exciter voltage		3.3	3.4	3.3	3.4
A4.02	Synchroscope		3.4	3.4	3.4	3.4
A4.03	Transfer of emergency control between manual and automatic		3.2	3.4	3.2	3.4
A4.04	Manual start, loading, and stopping of emergency generator: Plant-Specific		3.7	3.7	3.7	3.7
A4.05	Transfer of emergency generator (with load) to grid		3.6	3.7	3.6	3.7
A4.06	Droop setting		2.4*	2.8*	2.4	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 268000

System Name: Radwaste

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RADWASTE and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Condensate system		2.2	2.6	2.2	2.6
K1.02	Plant air systems		2.1*	2.4	2.1	2.4
K1.03	Reactor building equipment drains: Plant-Specific		2.6	2.9	2.6	2.9
K1.04	Reactor building floor drains: Plant-Specific		2.7	2.9	2.7	2.9
K1.05	Drywell equipment drains		2.9	3.2	2.9	3.2
K1.06	Drywell floor drains		2.9	3.2	2.9	3.2
K1.07	Reactor water cleanup		2.8	2.9	2.8	2.9
K1.08	Fuel pool		2.7	2.8	2.7	2.8
K1.09	ECCS systems		2.6	2.8	2.6	2.8
K1.10	Auxiliary steam: Plant-Specific		2.2	2.4	2.2	2.4
K1.11	Applicable component cooling water system		2.2	2.4	2.2	2.4
K1.12	Suppression pool		2.3	2.5	2.3	2.5
K1.13	Auxiliary building floor drains: Plant-Specific		2.2	2.4	2.2	2.4
K1.14	Auxiliary building equipment drains: Plant-Specific		2.0*	2.2	2.0	2.2
K1.15	Offgas system		2.3	2.5	2.3	2.5
K1.16	Circulating water system: Plant-Specific		2.3	2.5	2.3	2.5
K3	**Knowledge of the effect that a loss or malfunction of the RADWASTE will have on following:**	**(41.5 / 45.3)**				
K3.01	RWCU system		2.4	2.5	2.4	2.5
K3.02	Condensate system		2.2	2.4	2.2	2.4
K3.03	ECCS systems		2.2*	2.4	2.2	2.4
K3.04	Drain sumps		2.7	2.8	2.7	2.8
K3.05	Fuel pools: Plant-Specific		2.1	2.4	2.1	2.4
K5	**Knowledge of the operational implications of the following concepts as they apply to RADWASTE:**	**(41.5 / 45.3)**				
K5.01	Units of radiation, dose and dose rate		2.7	3.0	2.7	3.0
K5.02	Radiation hazards and ALARA concept		3.1	3.6*	3.1	3.6
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RADWASTE:**	**(41.7)**				
K6.01	Applicable component cooling water system		2.2	2.5	2.2	2.5
K6.02	Plant air systems		2.3	2.6	2.3	2.6
K6.03	Building ventilation		2.4	2.7	2.4	2.7
K6.04	Circulating water		2.1*	2.3	2.1	2.3
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RADWASTE controls including:**	**(41.5 / 45.5)**				
A1.01	Radiation level		2.7*	3.1*	2.7	3.1
A1.02	Off-site release		2.6*	3.6*	2.6	3.6

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 268000

System Name: Radwaste

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A2	**Ability to (a) predict the impacts of the following on the RADWASTE; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	System rupture		2.9	3.5	2.9	3.5
A2.02	High turbidity water		2.3	2.7	2.3	2.7
A2.03	Loss of steam supply		2.0*	2.3	2.0	2.3
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Sump integrators		3.4	3.6	3.4	3.6

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: **271000**

System Name: **Offgas System**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between OFFGAS SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Condenser air removal system		3.1	3.1	3.1	3.1
K1.02	Process radiation monitoring system		3.1	3.3	3.1	3.3
K1.03	Elevated release point		2.7	3.0	2.7	3.0
K1.04	Condensate system		2.7	2.7	2.7	2.7
K1.05	Radwaste system		2.3	2.5	2.3	2.5
K1.06	Main steam system		2.8	2.9	2.8	2.9
K1.07	Plant air systems		2.7	2.7	2.7	2.7
K1.08	~~Oxygen injection system: Plant-Specific~~		2.3	2.3	2.3	2.3
K1.09	Component cooling water systems		2.6	2.6	2.6	2.6
K1.10	Condenser vacuum		3.2	3.3	3.2	3.3
K1.11	†Station radioactive release rate		3.1	3.6	3.1	3.6
K1.12	A.C. electrical distribution		2.3	2.4	2.3	2.4
K1.13	Process sampling system		2.6	2.8	2.6	2.8
K1.14	Radwaste sparging air: Design-Specific		2.0*	2.5	2.0	2.5
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Glycol pumps		1.5*	1.5*	1.5	1.5
K3	**Knowledge of the effect that a loss or malfunction of the OFFGAS SYSTEM will have on following:**	**(41.5 / 45.3)**				
K3.01	Condenser vacuum		3.5	3.5	3.5	3.5
K3.02	†Off-site radioactive release rate		3.3	3.9	3.3	3.9
K4	**Knowledge of OFFGAS SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Dilution of hydrogen gas concentration		2.9	3.3	2.9	3.3
K4.02	Prevention of the poisoning of the recombiner catalyst by the presence of water		2.6	2.7	2.6	2.7
K4.03	Maintenance of sufficient oxygen gas inventory to allow for complete hydrogen recombination: Plant-Specific		2.4	2.8	2.4	2.8
K4.04	The prevention of hydrogen explosions and/or fires		3.3	3.6	3.3	3.6
K4.05	Redundancy		2.6	2.6	2.6	2.6
K4.06	Decay of fission product gases to particulate daughters		2.7	2.9	2.7	2.9
K4.07	Maximizing charcoal bed efficiency		2.6	2.7	2.6	2.7
K4.08	Automatic system isolation		3.1	3.3	3.1	3.3
K4.09	Filtration of radioactive particulate		2.8	3.1	2.8	3.1

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 271000

System Name: Offgas System

			CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
K5	**Knowledge of the operational implications of the following concepts as they apply to OFFGAS SYSTEM:**	**(41.7 / 45.4)**					
K5.01	Air operated valve operation			2.3	2.3	2.3	2.3
K5.02	Heat removal mechanisms			2.2	2.2	2.2	2.2
K5.03	Heat addition mechanisms			2.2*	2.2	2.2	2.2
K5.04	Hydrogen concentration measurement			2.9	3.1	2.9	3.1
K5.05	~~Oxygen concentration measurement~~			2.8	2.9	2.8	2.9
K5.06	Catalytic recombination			2.7	2.7	2.7	2.7
K5.07	Radioactive decay			2.7	2.9	2.7	2.9
K5.08	Charcoal absorption of fission product gases			2.5	2.6	2.5	2.6
K5.09	Hydrogen and oxygen recombination			2.6	2.8	2.6	2.8
K5.10	Decontamination factors			2.0*	2.3*	2.0	2.3
K5.11	Explain the necessity of reducing relative humidity for carbon bed filters			2.6	2.8	2.6	2.8
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the OFFGAS SYSTEM:**	**(41.7 / 45.7)**					
K6.01	Plant air systems			2.7	2.8	2.7	2.8
K6.02	Process radiation monitoring system			3.0	3.2	3.0	3.2
K6.03	Component cooling water systems			2.4	2.4	2.4	2.4
K6.04	Dilution steam			2.8	2.8	2.8	2.8
K6.05	Heating steam: Design-Specific			2.4	2.4	2.4	?
K6.06	~~Oxygen injection system: Plant-Specific~~			2.5	2.5	2.5	2.__
K6.07	A.C. electrical distribution			2.4	2.5	2.4	2.5
K6.08	Condenser air removal system			2.9	3.0	2.9	3.0
K6.09	Fuel cladding integrity			3.4	3.6	3.4	3.6
K6.10	Condensate system flow			2.7	2.8	2.7	2.8
K6.11	Condenser vacuum			3.2	3.3	3.2	3.3
K6.12	Glycol system			2.1*	2.2*	2.1	2.2
K6.13	~~Plant exhaust: BWR-1~~			3.0	3.4	3.0	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 271000

System Name: Offgas System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the OFFGAS SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Condenser vacuum		3.3	3.2	3.3	3.2
A1.02	Station radioactive release rate		3.0	3.6	3.0	3.6
A1.03	Preheater discharge temperature		2.2*	2.3	2.2	2.3
A1.04	Recombiner catalyst temperature		2.2*	2.4	2.2	2.4
A1.05	Cooler condenser discharge temperature		2.2*	2.4	2.2	2.4
A1.06	Filter differential pressure		2.4*	2.5	2.4	2.5
A1.07	Charcoal bed humidity		2.4*	2.5	2.4	2.5
A1.08	System flow		3.1	3.1	3.1	3.1
A1.09	Charcoal bed temperature		2.1*	2.4	2.1	2.4
A1.10	Charcoal vault temperature		2.3	2.5	2.3	2.5
A1.11	Offgas condenser temperatures		2.3	2.4	2.3	2.4
A1.12	Process radiation monitoring indications		3.1	3.5	3.1	3.5
A1.13	Hydrogen gas concentration		3.2	3.7	3.2	3.7
A1.14	Oxygen gas concentration		2.7	3.0	2.7	3.0
A1.15	Steam supply pressures		2.7	2.8	2.7	2.8
A2	**Ability to (a) predict the impacts of the following on the OFFGAS SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Low condenser vacuum		3.1	3.3	3.1	3.3
A2.02	Low dilution steam flow		2.9	3.1	2.9	3.1
A2.03	Main steamline high radiation		3.5	3.8	3.5	3.8
A2.04	Offgas system high radiation		3.7	4.1	3.7	4.1
A2.05	High charcoal bed humidity		2.5	2.9	2.5	2.9
A2.06	Offgas system holdup volume explosion/ fire		3.5	3.9	3.5	3.9
A2.07	~~Low oxygen injection flow: Plant Specific~~		2.7	3.3	2.7	3.3
A2.08	A.C. distribution failures		2.5	2.7	2.5	2.7
A2.09	Valve closures		2.6	2.8	2.6	2.8
A2.10	Offgas system high flow		3.1	3.3	3.1	3.3
A2.11	Offgas system low flow		2.8	2.9	2.8	2.9
A2.12	Recombiner high temperature		2.7	2.9	2.7	2.9
A2.13	Recombiner low temperature		2.4	2.8	2.4	2.8
A2.14	Offgas filter high differential pressure		2.6	2.8	2.6	2.8
A2.15	Air intrusion		2.7	2.9	2.7	2.9
A2.16	Loss of offgas system loop seals		2.9	3.2	2.9	3.2
A2.17	Reactor power changes		2.9	3.1	2.9	3.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 271000

System Name: Offgas System

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A3	**Ability to monitor automatic operations of the OFFGAS SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Automatic system isolations		3.3	3.3	3.3	3.3
A3.02	System flows		2.9	2.8	2.9	2.8
A3.03	System temperatures		2.8	2.8	2.8	2.8
A3.04	†Station radioactive release rate: Plant-Specific		3.0	3.8	3.0	3.8
A3.05	System indicating lights and alarms		2.9	2.9	2.9	2.9
A3.06	System differential pressures		2.5	2.5	2.5	2.5
A3.07	†Process radiation monitoring system indications		3.4	3.6	3.4	3.6
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Reset system isolations		2.8	2.8	2.8	2.8
A4.02	System flows		2.9	2.9	2.9	2.9
A4.03	System temperatures		2.8	2.8	2.8	2.8
A4.04	Condenser vacuum		3.4	3.5	3.4	3.5
A4.05	Station radioactive release rate		3.2	3.9	3.2	3.9
A4.06	System indicating lights and alarms		3.3	3.2	3.3	3.2
A4.07	System differential pressures		2.4	2.5	2.4	2.5
A4.08	Process radiation monitoring system		3.2	3.6	3.2	3.6
A4.09	Offgas system controls/components		3.3	3.2	3.3	3.2

Facility: CPS Printed: 08/11/2006

System Number: 272000

System Name: Radiation Monitoring System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between RADIATION MONITORING SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Main steam system		3.6	3.8	3.6	3.8
K1.02	Offgas system (augmented offgas): Plant-Specific		3.2	3.5	3.2	3.5
K1.03	Stack gas: Plant-Specific		3.3	3.6	3.3	3.6
K1.04	Applicable component cooling water system		2.9	2.9	2.9	2.9
K1.05	Radwaste system		2.8	3.1	2.8	3.1
K1.06	~~Reactor building ventilation system: Plant-Specific~~		3.2	3.3	3.2	3.3
K1.07	~~Isolation condenser: Plant-Specific~~		3.0	3.2	3.0	3.2
K1.08	Reactor protection system		3.6	3.9	3.6	3.9
K1.09	Primary containment isolation system		3.6	3.8	3.6	3.8
K1.10	Reactor building refuel floor: Plant-Specific		3.4	3.6	3.4	3.6
K1.11	Reactor building overhead crane; Plant-Specific		2.1*	2.4*	2.1	2.4
K1.12	~~Reactor building~~		3.1	3.2	3.1	3.2
K1.13	Turbine building		2.9	3.0	2.9	3.0
K1.14	Radwaste building: Plant-Specific		2.7	2.8	2.7	2.8
K1.15	~~Filter building: Plant-Specific~~		2.8	3.0	2.8	3.0
K1.16	Process computer		2.2*	2.3*	2.2	2.3
K1.17	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.4	2.7	2.4	2.7
K1.18	Primary containment/containment building: Plant-Specific		3.1	3.1	3.1	3.1
K1.19	Drywell		3.1	3.2	3.1	3.2
.20	†Auxiliary building: Plant-Specific		2.8	3.0	2.8	3.0
K1.21	Circulating water: Plant-Specific		2.3	2.4	2.3	2.4
K1.22	Fuel building: mark-III		3.0	3.4	3.0	3.4
K1.23	Continuous air monitoring: Plant-Specific		3.0	3.3	3.0	3.3
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Main steamline radiation monitors		2.5	2.8	2.5	2.8
K2.02	Offgas radiation monitoring system		2.5	2.8	2.5	2.8
K2.03	Stack gas radiation monitoring system		2.5	2.8	2.5	2.8
K2.04	Process liquid radiation monitoring system		2.3	2.5	2.3	2.5
K2.05	Reactor building ventilation monitors: Plant-Specific		2.6	2.9	2.6	2.9
K2.06	Area radiation monitors		2.1*	2.2*	2.1	2.2
K2.07	Control room ventilation monitors: Plant-Specific		2.2*	2.4*	2.2	2.4

Facility: CPS Printed: 08/11/2006

System Number: 272000

System Name: Radiation Monitoring System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Iv RO	Facility Iv S.
K3	**Knowledge of the effect that a loss or malfunction of the RADIATION MONITORING System will have on following:**	**(41.5 / 45.3)**				
K3.01	†Station liquid effluent release monitoring		3.2	3.8	3.2	3.8
K3.02	†Station gaseous effluent release monitoring		3.1	3.8	3.1	3.8
K3.03	Station area radiation monitoring		3.2	3.4	3.2	3.4
K3.04	Main steam system		3.7	3.8	3.7	3.8
K3.05	Offgas system		3.5	3.7	3.5	3.7
K3.06	Reactor building ventilation: Plant-Specific		3.4	3.6	3.4	3.6
K3.07	Reactor building overhead crane operation: Plant-Specific		2.4*	3.0	2.4	3.0
K3.08	†Auxiliary building ventilation: Plant-Specific		2.9	3.3	2.9	3.3
K3.09	Radwaste building ventilation: Plant-Specific		2.8	3.1	2.8	3.1
K3.10	Control room ventilation: Plant-Specific		2.9	3.3	2.9	3.3
K4	**Knowledge of RADIATION MONITORING System design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Redundancy		2.7	2.8	2.7	2.8
K4.02	Automatic actions to contain the radioactive release in the event that the predetermined release rates are exceeded		3.7	4.1	3.7	4.1
K4.03	Fail safe tripping of process radiation monitoring logic during conditions of instrument failure		3.6	3.9	3.6	3.9
K5	**Knowledge of the operational implications of the following concepts as they apply to RADIATION MONITORING SYSTEM:**	**(41.7 / 45.4)**				
K5.01	Hydrogen injection operation's effect on process radiation indications: Plant-Specific		3.2	3.5	3.2	3.5
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the RADIATION MONITORING SYSTEM:**	**(41.7 / 45.7)**				
K6.01	Reactor protection system		3.0	3.2	3.0	3.2
K6.02	D.C. power		2.5	2.7	2.5	2.7
K6.03	A.C. power		2.8	3.0	2.8	3.0
K6.04	Process computer		2.0*	2.2*	2.0	2.2
K6.05	SPDS/ERIS/CRID/GDS: Plant-Specific		2.1*	2.4*	2.1	2.4
K6.06	Continuous air monitoring: Plant-Specific		2.4	2.6	2.4	2.6

Facility: CPS Printed: 08/11/2006

System Number: 272000

System Name: Radiation Monitoring System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A1	**Ability to predict and/or monitor changes in parameters associated with operating the RADIATION MONITORING SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	Lights, alarms, and indications associated with normal operations		3.2	3.2	3.2	3.2
A1.02	Lights, alarms, and indications associated with surveillance testing		2.9	2.9	2.9	2.9
A2	**Ability to (d) predict the impacts of the following on the RADIATION MONITORING SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Fuel element failure		3.7	4.1	3.7	4.1
A2.02	Reactor protection system power failure		3.3	3.6	3.3	3.6
A2.03	A.C. electrical failure		2.9	3.1	2.9	3.1
A2.04	D.C. electrical failure		2.7	2.8	2.7	2.8
A2.05	Loss of dilution steam		2.5	2.6	2.5	2.6
A2.06	Downscale trips		2.8	2.9	2.8	2.9
A2.07	~~Hydrogen injection operation: Plant-Specific~~		2.6	2.8	2.6	2.8
A2.08	Offgas system failure		2.9	3.1	2.9	3.1
A2.09	Low fuel pool level		3.1	3.3	3.1	3.3
A2.10	Loss of coolant accident		3.9	4.1	3.9	4.1
A2.11	Leakage and/or breaks from contaminated systems to atmosphere or to other process systems		3.4	3.7	3.4	3.7
A2.12	†Refuel floor handling accidents/operations		3.3	4.0	3.3	4.0
A2.13	Low reactor water level during refueling operations		3.3	3.8	3.3	3.8
A2.14	Loss of, or inadequate, shielding		3.2	3.5	3.2	3.5
A2.15	Maintenance operations		2.5	2.7	2.5	2.7
A2.16	Instrument malfunctions		2.7	2.9	2.7	2.9
A3	**Ability to monitor automatic operations of the RADIATION MONITORING SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Main steam isolation indications		3.8	3.9	3.8	3.9
A3.02	Offgas system isolation indications		3.6	3.7	3.6	3.7
A3.03	Liquid radwaste isolation indications		3.1	3.5	3.1	3.5
A3.04	Radwaste handing interlocks		2.3	2.8	2.3	2.8
A3.05	Refuel floor overhead crane operation interrupt: Plant-Specific		3.0	3.1	3.0	3.1
A3.06	Ventilation system isolation indications		3.4	3.4	3.4	3.4
A3.07	Recorder indications		2.8	2.9	2.8	2.9
A3.08	Meter indications		2.9	2.9	2.9	2.9
A3.09	Containment isolation indications		3.6	3.5	3.6	3.5
A3.10	Lights and alarms		3.3	3.2	3.3	3.2
A3.11	~~Circulating water system blowdown isolations: Plant-Specific~~		2.6	2.7	2.6	2.7

System Number: 272000

System Name: Radiation Monitoring System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Im RO	Facility Im S.
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Recorder indications		2.9	2.9	2.9	2.9
A4.02	Meter indications		3.0	3.0	3.0	3.0
A4.03	Power supply status indicators		2.6	2.6	2.6	2.6
A4.04	SPDS/ERIS/CRID/GDS		2.5*	2.7*	2.5	2.7
A4.05	†Convert process radiation monitor readings to offsite release rates: Plant-Specific		2.3*	3.7*	2.3	3.7
A4.06	†Manually trip process radiation monitor logic		2.9	3.2	2.9	3.2

Facility: CPS Printed: 08/11/2006

System Number: 286000

System Name: *Fire Protection System*

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between FIRE PROTECTION SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Component cooling water systems~~		2.7	2.8	2.7	2.8
K1.02	~~Isolation condenser: Plant-Specific~~		3.8	3.8	3.8	3.8
K1.03	Reactor water level: Plant-Specific		2.9	3.0	2.9	3.0
K1.04	D.C. electrical distribution: Plant-Specific		2.6	2.6	2.6	2.6
K1.05	Main generator hydrogen system: Plant-Specific		3.1	3.1	3.1	3.1
K1.06	~~Auxiliary (boiler) steam system: Plant-Specific~~		2.2*	2.3*	2.2	2.3
K1.07	A.C. power supplies		2.8	2.9	2.8	2.9
K1.08	~~Intake canals: Plant-Specific~~		3.0	3.0	3.0	3.0
K1.09	Emergency generator rooms: Plant-Specific		3.2	3.3	3.2	3.3
K1.10	Main generator exciter: Plant-Specific		2.8	2.8	2.8	2.8
K1.11	Screen wash system: Plant-Specific		2.5	2.5	2.5	2.5
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	†Fire protection supervisory circuits: Plant-Specific		2.3*	2.5*	2.3	2.5
K2.02	Pumps		2.9	3.1	2.9	3.1
K2.03	Fire detection system: Plant-Specific		2.5*	2.7*	2.5	2.7
K3	**Knowledge of the effect that a loss or malfunction of the FIRE PROTECTION SYSTEM will have on following:**	**(41.7 / 45.4)**				
K3.01	The ability to detect fires		3.2	3.4	3.2	3.4
K3.02	Personnel protection		3.2	3.4	3.2	3.4
K3.03	Plant protection		3.6	3.8	3.6	3.8
K4	**Knowledge of FIRE PROTECTION SYSTEM design feature(s) and/or interlocks which provide for the following:**	**(41.7 41.5 / 45.3 41.7 / 45.7 4)**				
K4.01	Adequate supply of water for the fire protection system		3.4	3.6	3.4	3.6
K4.02	Automatic system initiation		3.3	3.5	3.3	3.5
K4.03	Maintenance of fire header pressure		3.3	3.4	3.3	3.4
K4.04	Personnel safety during halon and/or carbon dioxide system actuation		3.6	3.7	3.6	3.7
K4.05	Fire protection capability during loss of off-site power		3.7	3.8	3.7	3.8
K4.06	Fire suppression capability that does not rely on the displacement of oxygen (Halon): Plant-Specific		3.4	3.4	3.4	3.4
K4.07	Diesel engine protection		3.3	3.3	3.3	3.3

Facility: CPS Printed: 08/11/2006

System Number: 286000

System Name: Fire Protection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp S.
K5	**Knowledge of the operational implications of the following concepts as they apply to FIRE PROTECTION SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Effect of carbon dioxide on fires		2.6	2.7	2.6	2.7
K5.02	Effect of Halon on fires: Plant-Specific		2.6	2.6	2.6	2.6
K5.03	Effect of water spray on electrical components		3.3	3.4	3.3	3.4
K5.04	Valve operation		2.9	2.9	2.9	2.9
K5.05	Diesel operations		3.0*	3.1*	3.0	3.1
K5.06	Heat detection		2.6	2.7	2.6	2.7
K5.07	Smoke detection		2.6	2.7	2.6	2.7
K5.08	~~Gas refrigeration: Plant-Specific~~		2.4*	2.5*	2.4	2.5
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the FIRE PROTECTION SYSTEM**	**(41.7 / 45.7)**				
K6.01	A.C. electrical distribution: Plant-Specific		3.1	3.1	3.1	3.1
K6.02	D.C. electrical distribution		2.8*	2.9*	2.8	2.9
K6.03	~~Applicable component cooling water system: Plant-Specific~~		2.4*	2.5*	2.4	2.5
K6.04	Diesel fuel transfer system: Plant-Specific		2.8	3.0	2.8	3.0
K6.05	Screen wash system: Plant-Specific		2.6	2.6	2.6	2.6
A1	**Ability to predict and/or monitor changes in parameters associated with operating the FIRE PROTECTION SYSTEM controls including:**	**(41.5 / 45.5)**				
A1.01	System pressure		2.9	2.9	2.9	2.9
A1.02	System flow		2.4*	2.5*	2.4	2.5
A1.03	Fire doors		2.8	3.1	2.8	3.1
A1.04	Fire dampers		2.8	3.1	2.8	3.1
A1.05	System lineups		3.2	3.2	3.2	3.2
A1.06	~~Tank pressure: Plant-Specific~~		2.9	3.0	2.9	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 286000

System Name: Fire Protection System

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
A2	**Ability to (a) predict the impacts of the following on the FIRE PROTECTION SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	System logic failure: Plant-Specific		2.7	2.9	2.7	2.9
A2.02	D.C. distribution failure: Plant-Specific		2.6*	2.7*	2.6	2.7
A2.03	A.C. distribution failure: Plant-Specific		2.9	3.0	2.9	3.0
A2.04	~~Applicable component cooling water system failure: Plant-Specific~~		2.3*	2.4*	2.3	2.4
A2.05	Fire protection diesel trips		3.1	3.3	3.1	3.3
A2.06	Low fire main pressure: Plant-Specific		3.1	3.2	3.1	3.2
A2.07	Inadvertent system initiation		2.9	2.9	2.9	2.9
A2.08	Failure to actuate when required		3.2	3.3	3.2	3.3
A2.09	Valve closures: Plant-Specific		2.7	2.8	2.7	2.8
A2.10	Valve openings: Plant-Specific		2.6*	2.7	2.6	2.7
A2.11	Pump trips: Plant-Specific		3.1	3.2	3.1	3.2
A2.12	Low diesel fuel supply: Plant-Specific		3.1	3.3	3.1	3.3
A3	**Ability to monitor automatic operations of the FIRE PROTECTION SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Fire water pump start		3.4	3.4	3.4	3.4
A3.02	Fire main pressure		3.1	3.2	3.1	3.2
A3.03	Actuation of fire detectors		3.3	3.3	3.3	3.3
A3.04	System initiation		3.2	3.3	3.2	3.3
A3.05	Fire doors		3.0	3.1	3.0	3.1
A3.06	Fire dampers		3.0	3.1	3.0	3.1
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	System alarms and indicating lights		3.3	3.2	3.3	3.2
A4.02	~~Applicable component cooling water system: Plant-Specific~~		2.5*	2.5*	2.5	2.5
A4.03	~~Applicable component cooling water pressure~~		2.5*	2.5*	2.5	2.5
A4.04	Fire main pressure: Plant-Specific		2.8	2.8	2.8	2.8
A4.05	Fire pump		3.3	3.3	3.3	3.3
A4.06	Fire diesel		3.4	3.4	3.4	3.4

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 288000

System Name: Plant Ventilation Systems

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between PLANT VENTILATION SYSTEMS and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	A.C. electrical		2.6	2.6	2.6	2.6
K1.02	Secondary containment		3.4	3.4	3.4	3.4
K1.03	Standby gas treatment		3.7	3.7	3.7	3.7
K1.04	Applicable component cooling water system: Plant-Specific		2.6	2.6	2.6	2.6
K1.05	Process radiation monitoring system		3.3	3.6	3.3	3.6
K1.06	Plant air systems		2.7	2.7	2.7	2.7
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Reactor building supply and exhaust fans: Plant-Specific		2.4	2.4	2.4	2.4
K2.02	Auxiliary building supply and exhaust fans (turbine building/ radwaste building): Plant-Specific		1.8*	1.9*	1.8	1.9
K3	**Knowledge of the effect that a loss or malfunction of the PLANT VENTILATION SYSTEMS will have on following:**	**(41.5 / 45.3)**				
K3.01	Secondary containment temperature: Plant-Specific		2.8	3.0	2.8	3.0
K3.02	~~Reactor building temperature: Plant-Specific~~		2.9	3.1	2.9	3.1
K3.03	Auxiliary building temperature: Plant-Specific		2.5	2.5	2.5	2.5
K3.04	Secondary containment pressure: Plant-Specific		3.2	3.3	3.2	3.3
K3.05	~~Reactor building pressure: Plant-Specific~~		3.1	3.3	3.1	3.3
K3.06	Auxiliary building pressure: Plant-Specific		2.4*	2.4	2.4	2.4
K4	**Knowledge of PLANT VENTILATION SYSTEMS design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Automatic initiation of standby gas treatment system		3.7	3.9	3.7	3.9
K4.02	Secondary containment isolation		3.7	3.8	3.7	3.8
K4.03	Automatic starting and stopping of fans		2.8	2.9	2.8	2.9
K5	**Knowledge of the operational implications of the following concepts as they apply to PLANT VENTILATION SYSTEMS:**	**(41.7 / 45.4)**				
K5.01	Airborne contamination control		3.1	3.2	3.1	3.2
K5.02	Differential pressure control		3.2	3.4	3.2	3.4
K5.03	Temperature control		2.5	2.6	2.5	2.6

Facility: CPS Printed: 08/11/2006

System Number: 288000

System Name: Plant Ventilation Systems

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SR
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the PLANT VENTILATION SYSTEMS:**	**(41.7 / 45.7)**				
K6.01	A.C. electrical		2.7	2.7	2.7	2.7
K6.02	Applicable component cooling water system: Plant-Specific		2.5	2.5	2.5	2.5
K6.03	Plant air systems		2.7	2.7	2.7	2.7
A1	**Ability to predict and/or monitor changes in parameters associated with operating the PLANT VENTILATION SYSTEMS controls including:**	**(41.5 / 45.5)**				
A1.01	Filter differential pressure		2.2*	2.2*	2.2	2.2
A1.02	Fan differential pressure		2.2*	2.2*	2.2	2.2
A1.03	Area temperatures		2.4	2.4	2.4	2.4
A2	**Ability to (a) predict the impacts of the following on the PLANT VENTILATION SYSTEMS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	High drywell pressure: Plant-Specific		3.3	3.4	3.3	3.4
A2.02	Low reactor water level: Plant-Specific		3.4	3.6	3.4	3.6
A2.03	Loss of coolant accident: Plant-Specific		3.5	3.7	3.5	3.7
A2.04	High radiation: Plant-Specific		3.7	3.8	3.7	3.8
A2.05	Extreme outside weather conditions: Plant-Specific		2.6	2.7	2.6	2.7
A3	**Ability to monitor automatic operations of the PLANT VENTILATION SYSTEMS including:**	**(41.7 / 45.7)**				
A3.01	Isolation/initiation signals		3.8	3.8	3.8	3.8
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Start and stop fans		3.1	2.9	3.1	2.9
A4.02	Area temperature		2.8	2.8	2.8	2.8

KA Catalog

Facility: CPS *Printed:* 08/11/2006

System Number: **290001**

System Name: **Secondary Containment**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between SECONDARY CONTAINMENT and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Reactor building ventilation: Plant-Specific~~		3.3	3.5	3.3	3.5
K1.02	Primary containment system: Plant-Specific		3.4	3.6	3.4	3.6
K1.03	Radwaste building ventilation: Plant-Specific		2.4	2.7	2.4	2.7
K1.04	SBGT		3.7	3.9	3.7	3.9
K1.05	Auxiliary building ventilation: Plant-Specific		3.1	3.3	3.1	3.3
K1.06	Auxiliary building isolation: BWR-6		3.4	3.6	3.4	3.6
K1.07	Turbine building ventilation (steam tunnel): Plant-Specific		3.0	3.1	3.0	3.1
K1.08	~~Exhaust stack: BWR-2, 3, 4~~		3.2	3.3	3.2	3.3
K1.09	Plant air systems		2.9	2.9	2.9	2.9
K1.10	~~Auxiliary boiler system: BWR-2, 3, 4~~		1.6*	1.6*	1.6	1.6
K3	**Knowledge of the effect that a loss or malfunction of the SECONDARY CONTAINMENT will have on following:**	**(41.7 / 45.4)**				
K3.01	†Off-site radioactive release rates		4.0	4.4*	4.0	4.4
K4	**Knowledge of SECONDARY CONTAINMENT design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	Personnel access without breaching secondary containment: Plant-System		3.5	3.8	3.5	3.8
K4.02	Protection against over pressurization: Plant-System		3.4	3.5	3.4	3.5
K4.03	Fluid leakage collection		2.8	2.9	2.8	2.9
K4.04	Auxiliary building isolation: BWR-6		3.4	3.4	3.4	3.4
K5	**Knowledge of the operational implications of the following concepts as they apply to SECONDARY CONTAINMENT:**	**(41.5 / 45.3)**				
K5.01	~~Vacuum breaker operation: BWR-4~~		3.3*	3.4*	3.3	3.4
K5.02	~~Flow measurement: BWR-3~~		2.2	2.2	2.2	2.2
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the SECONDARY CONTAINMENT:**	**(41.7 / 45.7)**				
K6.01	~~Reactor building ventilation: Plant-Specific~~		3.5	3.6	3.5	3.6
K6.02	Radwaste building ventilation: Plant-Specific		2.4	2.6	2.4	2.6
K6.03	SBGT		3.8	4.0	3.8	4.0
K6.04	Primary containment system		3.9	4.1	3.9	4.1
K6.05	Auxiliary building ventilation: Plant-Specific		2.9	3.0	2.9	3.0
K6.06	Turbine building ventilation: Plant-Specific		2.2*	2.3	2.2	2.3
K6.07	~~Auxiliary boiler system: BWR-3, 4~~		1.9*	2.0*	1.9	2.0
K6.08	Plant air systems		2.7	2.8	2.7	2.8
K6.09	A.C. power: BWR-6		3.4	3.6	3.4	3.6

Facility: CPS Printed: 08/11/2006

System Number: 290001

System Name: Secondary Containment

		CFR	NRC Imp RO	NRC Imp SRO	Facility Im RO	Facility Im S
A1	**Ability to predict and/or monitor changes in parameters associated with operating the SECONDARY CONTAINMENT controls including:**	**(41.5 / 45.5)**				
A1.01	System lineups		3.1	3.1	3.1	3.1
A1.02	High area temperature: BWR-6		3.6	3.6	3.6	3.6
A2	**Ability to (a) predict the impacts of the following on the SECONDARY CONTAINMENT; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	†Personnel airlock failure		3.3	3.7	3.3	3.7
A2.02	†Excessive outleakage		3.5	3.7	3.5	3.7
A2.03	High area radiation		3.4	3.6	3.4	3.6
A2.04	High airborne radiation		3.4	3.7	3.4	3.7
A2.05	High area temperature		3.1	3.3	3.1	3.3
A2.06	Auxiliary building isolation: BWR-6		3.7	4.0	3.7	4.0
A3	**Ability to monitor automatic operations of the SECONDARY CONTAINMENT including:**	**(41.7 / 45.7)**				
A3.01	Secondary containment isolation		3.9	4.0	3.9	4.0
A3.02	Normal building differential pressure: Plant-Specific		3.5	3.5	3.5	3..
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Reactor building differential pressure: Plant-Specific		3.3	3.4	3.3	3.4
A4.02	Reactor building area temperatures: Plant-Specific		3.3	3.4	3.3	3.4
A4.03	Auxiliary building differential pressure: Plant-Specific		2.6*	2.7	2.6	2.7
A4.04	Auxiliary building area temperature: Plant-Specific		2.6*	2.7	2.6	2.7
A4.05	Fuel building differential pressure: Plant-Specific		3.3	3.5	3.3	3.5
A4.06	Fuel building area temperature: Plant-Specific		3.3	3.5	3.3	3.5
A4.07	Radwaste building differential pressure: Plant-Specific		2.3	2.5	2.3	2.5
A4.08	Radwaste building area temperature: Plant-Specific		2.2	2.4	2.2	2.4
A4.09	System status lights and alarms: Plant-Specific		3.2	3.2	3.2	3.2
A4.10	System lineups: Plant-Specific		3.4	3.3	3.4	3.3
A4.11	System reset: Plant-Specific		3.4	3.4	3.4	3.4
A4.12	Surveillance testing: Plant-Specific		2.8	3.2	2.8	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 290002

System Name: Reactor Vessel Internals

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between REACTOR VESSEL INTERNALS and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Main steam system		3.2	3.2	3.2	3.2
K1.02	Recirculation system		3.2	3.2	3.2	3.2
K1.03	Reactor feedwater system		3.2	3.2	3.2	3.2
K1.04	~~HPCI: Plant-Specific~~		3.4	3.5	3.4	3.5
K1.05	RHR: Plant-Specific		3.1	3.2	3.1	3.2
K1.06	HPCS: Plant-Specific		3.1	3.1	3.1	3.1
K1.07	~~Isolation condenser: Plant-Specific~~		3.4	3.4	3.4	3.4
K1.08	RCIC: Plant-Specific		3.1	3.1	3.1	3.1
K1.09	LPCI: Plant-Specific		3.2	3.3	3.2	3.3
K1.10	CRD hydraulic system		3.1	3.1	3.1	3.1
K1.11	CRD mechanism		2.9	2.9	2.9	2.9
K1.12	SBLC		3.4	3.5	3.4	3.5
K1.13	Relief/safety valves		3.4	3.5	3.4	3.5
K1.14	RWCU		2.9	3.1	2.9	3.1
K1.15	Nuclear boiler instrumentation		3.4	3.5	3.4	3.5
K1.16	LPCS		3.2	3.4	3.2	3.4
K1.17	ADS		3.3	3.4	3.3	3.4
K1.18	Lost parts monitoring: Plant-Specific		2.1*	2.2*	2.1	2.2
K1.19	TIP		2.5	2.6	2.5	2.6
K1.20	Nuclear instrumentation		3.2	3.3	3.2	3.3
K3	**Knowledge of the effect that a loss or malfunction of the REACTOR VESSEL INTERNALS will have on following:**	**(41.7 / 45.4)**				
K3.01	Reactor water level		3.2	3.3	3.2	3.3
K3.02	Reactor pressure		2.9	3.0	2.9	3.0
K3.03	Reactor power		3.3	3.4	3.3	3.4
K3.04	Plant radiation levels		2.9	3.2	2.9	3.2
K3.05	Off-site radiation levels		2.9	3.2	2.9	3.2
K3.06	PCIS/NSSSS		3.1	3.1	3.1	3.1
K3.07	Nuclear boiler instrumentation		3.1	3.1	3.1	3.1
K4	**Knowledge of REACTOR VESSEL INTERNALS design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	2/3 core coverage following a DBA LOCA		3.7	3.9	3.7	3.9
K4.02	Separation of fluid flow paths within the vessel		3.1	3.2	3.1	3.2
K4.03	Core orificing		3.2	3.3	3.2	3.3
K4.04	Moisture removal from generated steam		2.8	2.8	2.8	2.8
K4.05	Natural circulation		3.3	3.5	3.3	3.5

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 290002

System Name: Reactor Vessel Internals

			CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
K5	**Knowledge of the operational implications of the following concepts as they apply to REACTOR VESSEL INTERNALS:**	**(41.5 / 45.3)**					
K5.01	†Thermal limits			3.5	3.9	3.5	3.9
K5.02	Fission product poisons			2.9	3.1	2.9	3.1
K5.03	Burnable poisons			2.7	3.0	2.7	3.0
K5.04	†PCIOMR: Plant-Specific			3.1	3.7	3.1	3.7
K5.05	Brittle fracture			3.1	3.3	3.1	3.3
K5.06	*Heat transfer mechanisms*			2.8	3.2	2.8	3.2
K5.07	†Safety limits			3.9	4.4*	3.9	4.4
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR VESSEL INTERNALS:**	**(41.7 / 45.7)**					
K6.01	CRD *hydraulic system*			2.8	2.9	2.8	2.9
K6.02	CRD mechanism			2.9	2.9	2.9	2.9
K6.03	Recirculation system			3.1	3.2	3.1	3.2
K6.04	Reactor feedwater system			3.0	3.1	3.0	3.1
K6.05	SBLC			3.3	3.4	3.3	3.4
K6.06	Relief/safety valves			3.0	3.2	3.0	3.2
K6.07	RWCU			2.6	2.7	2.6	2.7
K6.08	Nuclear boiler instrumentation			2.9	3.2	2.9	
K6.09	LPCS			3.2	3.3	3.2	
K6.10	~~HPCI: Plant-Specific~~			3.0	3.3	3.0	3.3
K6.11	RHR: Plant-Specific			3.1	3.2	3.1	3.2
K6.12	~~Isolation condenser: Plant-Specific~~			3.0	3.2	3.0	3.2
K6.13	RCIC: Plant-Specific			2.7	2.8	2.7	2.8
K6.14	LPCI: Plant-Specific			3.1	3.3	3.1	3.3
K6.15	ADS			3.1	3.4	3.1	3.4
K6.16	Loss parts monitoring			2.0*	2.0*	2.0	2.0
K6.17	TIP			2.5	2.5	2.5	2.5
K6.18	Nuclear instrumentation			3.0	3.1	3.0	3.1
K6.19	HPCS: Plant-Specific(BWR-5&6)			3.0	3.1	3.0	3.1
K6.20	Main steam system			2.9	3.1	2.9	3.1
A2	**Ability to (a) predict the impacts of the following on the REACTOR VESSEL INTERNALS; and (b) based on those predictions, use procedures to correct, control, or *mitigate the consequences* of those abnormal conditions or operations:**	**(41.5 / 45.6)**					
A2.01	LOCA			3.7	4.0	3.7	4.0
A2.02	†Overpressurization transient			3.6	3.9	3.6	3.9
A2.03	†Control rod drop accident			3.6	3.9	3.6	3.9
A2.04	Excessive heatup/cooldown rate			3.7	4.1	3.7	4.1
A2.05	†Exceeding thermal limits			3.7	4.2	3.7	4.2
A2.06	†Exceeding safety limits			4.0	4.5*	4.0	

KA Catalog

Facility: CPS

Printed: 08/11/2006

System Number: 290002

System Name: Reactor Vessel Internals

		NRC Imp		Facility Imp	
	CFR	RO	SRO	RO	SRO

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 290003

System Name: Control Room HVAC

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between CONTROL ROOM HVAC and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Radiation monitors		3.4	3.5	3.4	3.5
K1.02	~~Chlorine amonia detectors: Plant-Specific~~		2.9	3.1	2.9	3.1
K1.03	Remote air intakes: Plant-Specific		2.8	2.9	2.8	2.9
K1.04	Nuclear steam supply shut off system (NSSSS/PCIS): Plant-Specific		3.2	3.3	3.2	3.3
K1.05	Component cooling water systems		2.8	3.0	2.8	3.0
K1.06	Plant air systems		2.6	2.7	2.6	2.7
K1.07	Fire protection		2.9	3.0	2.9	3.0
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Fans		2.2*	2.4*	2.2	2.4
K2.02	Chiller units		2.4*	2.4*	2.4	2.4
K2.03	Motor operated valves: Plant-Specific		1.9*	2.0*	1.9	2.0
K3	**Knowledge of the effect that a loss or malfunction of the CONTROL ROOM HVAC will have on following:**	**(41.7 / 45.4)**				
K3.01	Control room habitability		3.5	3.8	3.5	3.8
K3.02	Computer/instrumentation: Plant-Specific		3.3	3.6	3.3	3.6
K3.03	Control room temperature		2.9	3.1	2.9	3.1
K3.04	Control room pressure		2.8*	2.9	2.8	2.9
K4	**Knowledge of CONTROL ROOM HVAC design feature(s) and/or interlocks which provide for the following:**	**(41.7)**				
K4.01	System initiations/reconfiguration: Plant-Specific		3.1	3.2	3.1	3.2
K4.02	Control room temperatures		2.4*	2.6	2.4	2.6
K5	**Knowledge of the operational implications of the following concepts as they apply to CONTROL ROOM HVAC**	**(41.5 / 45.3)**				
K5.01	Airborne contamination (e.g., radiological, toxic gas, smoke) control		3.2	3.5	3.2	3.5
K5.02	Differential pressure control		2.8	2.8	2.8	2.8
K5.03	Temperature control		2.6	2.7	2.6	2.7
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the CONTROL ROOM HVAC:**	**(41.7 / 45.7)**				
K6.01	Electrical power		2.7	2.9	2.7	2.9
K6.02	Component cooling water systems		2.7	2.9	2.7	2.9
K6.03	Plant air systems		2.4	2.6	2.4	2.6
K6.04	Fire protection: Plant-Specific		2.6	2.8	2.6	2.8

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 290003

System Name: *Control Room HVAC*

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
A1	**Ability to predict and/or monitor changes in parameters associated with operating the CONTROL ROOM HVAC controls including:**	**(41.5 / 45.5)**				
A1.01	Filter differential pressure		2.1*	2.1*	2.1	2.1
A1.02	Fan differential pressure		2.1*	2.1*	2.1	2.1
A1.03	Area temperatures		2.6	2.6	2.6	2.6
A1.04	Control room pressure		2.5	2.8	2.5	2.8
A1.05	Radiation monitoring (control room)		3.2	3.3	3.2	3.3
A2	**Ability to (a) predict the impacts of the following on the CONTROL ROOM HVAC; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations:**	**(41.5 / 45.6)**				
A2.01	Initiation/reconfiguration		3.1	3.2	3.1	3.2
A2.02	Extreme environmental conditions		3.1	3.4	3.1	3.4
A2.03	Initiation/reconfiguration failure		3.4	3.6	3.4	3.6
A2.04	Initiation/failure of fire protection system		3.1	3.3	3.1	3.3
A3	**Ability to monitor automatic operations of the CONTROL ROOM HVAC including:**	**(41.7 / 45.7)**				
A3.01	Initiation/reconfiguration		3.3	3.5	3.3	
A3.02	Initiation/failure of fire protection system		3.0	3.4	3.0	3.4
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Initiate/reset system		3.2	3.2	3.2	3.2
A4.02	Fans		2.8	2.8	2.8	2.8
A4.03	Reposition dampers		2.8	2.8	2.8	2.8
A4.04	Environmental conditions		2.8	3.0	2.8	3.0

Facility: CPS Printed: 08/11/2006

System Number:

System Name: **GENERIC**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
2.1	**Conduct of Operations**					
2.1.1	Knowledge of conduct of operations requirements.	(41.10 / 45.13)	3.7	3.8	3.7	3.8
2.1.2	Knowledge of operator responsibilities during all modes of plant operation.	(41.10 / 45.13)	3.0	4.0	3.0	4.0
2.1.3	Knowledge of shift turnover practices.	(41.10 / 45.13)	3.0	3.4	3.0	3.4
2.1.4	Knowledge of shift staffing requirements.	(41.10 / 43.2)	2.3	3.4	2.3	3.4
2.1.5	Ability to locate and use procedures and directives related to shift staffing and activities.	(41.10 / 43.5 / 45.12)	2.3	3.4	2.3	3.4
2.1.6	Ability to supervise and assume a management role during plant transients and upset conditions.	(43.5 / 45.12 / 45.13)	2.1	4.3	2.1	4.3
2.1.7	Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation.	(43.5 / 45.12 / 45.13)	3.7	4.4	3.7	4.4
2.1.8	Ability to coordinate personnel activities outside the control room.	(45.5 / 45.12 / 45.13)	3.8	3.6	3.8	3.6
2.1.9	Ability to direct personnel activities inside the control room.	(45.5 / 45.12 / 45.13)	2.5	4.0	2.5	4.0
2.1.10	Knowledge of conditions and limitations in the facility license.	(43.1 / 45.13)	2.7	3.9	2.7	3.9
2.1.11	Knowledge of less than one hour technical specification action statements for systems.	(43.2 / 45.13)	3.0	3.8	3.0	3.8
2.1.12	Ability to apply technical specifications for a system.	(43.2 / 43.5 / 45.3)	2.9	4.0	2.9	4.0
2.1.13	Knowledge of facility requirements for controlling vital / controlled access.	(41.10 / 43.5 / 45.9 / 45.10)	2.0	2.9	2.0	2.9
2.1.14	Knowledge of system status criteria which require the notification of plant personnel.	(43.5 / 45.12)	2.5	3.3	2.5	3.3
2.1.15	Ability to manage short-term information such as night and standing orders.	(45.12)	2.3	3.0	2.3	3.0
2.1.16	Ability to operate plant phone, paging system, and two-way radio.	(41.10 / 45.12)	2.9	2.8	2.9	2.8
2.1.17	Ability to make accurate, clear and concise verbal reports.	(45.12 / 45.13)	3.5	3.6	3.5	3.6
2.1.18	Ability to make accurate, clear and concise logs, records, status boards, and reports.	(45.12 / 45.13)	2.9	3.0	2.9	3.0
2.1.19	Ability to use plant computer to obtain and evaluate parametric information on system or component status.	(45.12)	3.0	3.0	3.0	3.(
2.1.20	Ability to execute procedure steps.	(41.10 / 43.5 / 45.12)	4.3	4.2	4.3	4.2
2.1.21	Ability to obtain and verify controlled procedure copy.	(45.10 / 45.13)	3.1	3.2	3.1	3.2
2.1.22	Ability to determine Mode of Operation.	(43.5 / 45.13)	2.8	3.3	2.8	3.
2.1.23	Ability to perform specific system and integrated plant procedures during different modes of plant operation.	(45.2 / 45.6)	3.9	4.0	3.9	4.
2.1.24	Ability to obtain and interpret station electrical and mechanical drawings.	(45.12 / 45.13)	2.8	3.1	2.8	3.
2.1.25	Ability to obtain and interpret station reference materials such as graphs, monographs, and tables which contain performance data.	(41.10 / 43.5 / 45.12)	2.8	3.1	2.8	3
2.1.26	Knowledge of non-nuclear safety procedures (e.g. rotating equipment, electrical, high temperature, high pressure, caustic, chlorine, oxygen and hydrogen).	(41.10 / 45.12)	2.2	2.6	2.2	2
2.1.27	Knowledge of system purpose and/or function.	(41.7)	2.8	2.9	2.8	

System Number:

System Name: GENERIC

		CFR	NRC Imp RO	NRC Imp SRO	Facility Im RO	Facility Im S.
2.1	**Conduct of Operations**					
2.1.28	Knowledge of the purpose and function of major system components and controls.	(41.7)	3.2	3.3	3.2	3.3
2.1.29	Knowledge of how to conduct and verify valve lineups.	(41.10 / 45.1 / 45.12)	3.4	3.3	3.4	3.3
2.1.30	Ability to locate and operate components, including local controls.	(41.7 / 45.7)	3.9	3.4	3.9	3.4
2.1.31	Ability to locate control room switches, controls and indications and to determine that they are correctly reflecting the desired plant lineup.	(45.12)	4.2	3.9	4.2	3.9
2.1.32	Ability to explain and apply system limits and precautions.	(41.10 / 43.2 / 45.12)	3.4	3.8	3.4	3.8
2.1.33	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.	(43.2 / 43.3 / 45.3)	3.4	4.0	3.4	4.0
2.1.34	Ability to maintain primary and secondary plant chemistry within allowable limits.	(41.10 / 43.5 / 45.12)	2.3	2.9	2.3	2.9

Facility: CPS Printed: 08/11/2006

System Number:

System Name: GENERIC

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
2.2	**Equipment Control**					
2.2.1	Ability to perform pre-startup procedures for the facility, including operating those controls associated with plant equipment that could affect reactivity.	(45.1)	3.7	3.6	3.7	3.6
2.2.2	Ability to manipulate the console controls as required to operate the facility between shutdown and designated power levels.	(45.2)	4.0	3.5	4.0	3.5
2.2.3	~~(multi-unit) Knowledge of the design, procedural, and operati(41)/~~ ~~differences between units.~~	43 / 45)	3.1	3.3	3.1	3.3
2.2.4	~~(multi-unit) Ability to explain the variations in control board layouts, systems, instrumentation and procedural actions between units at a facility.~~	(45.1-45.13)	2.8	3.0*	2.8	3.0
2.2.5	Knowledge of the process for making changes in the facility as described in the safety analysis report.	(43.3 / 45.13)	1.6	2.7	1.6	2.7
2.2.6	Knowledge of the process for making changes in procedures as described in the safety analysis report.	(43.3 / 45.13)	2.3	3.3	2.3	3.3
2.2.7	Knowledge of the process for conducting tests or experiments not described in the safety analysis report.	(43.3 / 45.13)	2.0	3.2	2.0	3.2
2.2.8	Knowledge of the process for determining if the proposed change, test, or experiment involves an unreviewed safety question.	(43.3 / 45.13)	1.8	3.3	1.8	3.3
2.2.9	Knowledge of the process for determining if the proposed change, test or experiment increases the probability of occurrence or consequences of an accident during the change, test or experiment.	(43.3 / 45.13)	2.0	3.3	2.0	3.3
2.2.10	Knowledge of the process for determining if the margin of safety, as defined in the basis of any technical specification is reduced by a proposed change, test or experiment.	(43.3 / 45.13)	1.9	3.3	1.9	3.3
2.2.11	Knowledge of the process for controlling temporary changes.	(41.10 / 43.3 / 45.13)	2.5	3.4*	2.5	3.4
2.2.12	Knowledge of surveillance procedures.	(41.10 / 45.13)	3.0	3.4	3.0	3.4
2.2.13	Knowledge of tagging and clearance procedures.	(41.10 / 45.13)	3.6	3.8	3.6	3.8
2.2.14	Knowledge of the process for making configuration changes.	(43.3 / 45.13)	2.1	3.0	2.1	3.0
2.2.15	Ability to identify and utilize as-built design and configuration change documentation to ascertain expected current plant configuration and operate the plant.	(43.3 / 45.13)	2.2	2.9	2.2	2.9
2.2.16	Knowledge of the process for making of field changes.	(41.10 / 45.13)	1.9	2.6*	1.9	2.6
2.2.17	Knowledge of the process for managing maintenance activities during power operations.	(43.5 / 45.13)	2.3	3.5	2.3	3.5
2.2.18	Knowledge of the process for managing maintenance activities during shutdown operations.	(43.5 / 45.13)	2.3	3.6	2.3	3.6
2.2.19	Knowledge of maintenance work order requirements.	(43.5 / 45.13)	2.1	3.1	2.1	3.
2.2.20	Knowledge of the process for managing troubleshooting activities.	(43.5 / 45.13)	2.2	3.3	2.2	3.
2.2.21	Knowledge of pre and post maintenance operability requirements.	(43.2)	2.3	3.5	2.3	3
2.2.22	Knowledge of limiting conditions for operations and safety limits.	(43.2 / 45.2)	3.4	4.1	3.4	4
2.2.23	Ability to track limiting conditions for operations.	(43.2 / 45.13)	2.6	3.8	2.6	3

Facility: CPS Printed: 08/11/2006

System Number:

System Name: GENERIC

		CFR	NRC Imp RO	NRC Imp SRO	Facility Iv RO	Facility Iv S
2.2	**Equipment Control**					
2.2.24	Ability to analyze the affect of maintenance activities on LCO status.	(43.2 / 45.13)	2.6	3.8	2.6	3.8
2.2.25	Knowledge of bases in technical specifications for limiting conditions for operations and safety limits.	(43.2)	2.5	3.7	2.5	3.7
2.2.26	Knowledge of refueling administrative requirements.	(43.5 / 45.13)	2.5	3.7	2.5	3.7
2.2.27	Knowledge of the refueling process.	(43.6 / 45.13)	2.6	3.5	2.6	3.5
2.2.28	Knowledge of new and spent fuel movement procedures.	(43.7 / 45.13)	2.6	3.5	2.6	3.5
2.2.29	Knowledge of SRO fuel handling responsibilities.	(43.6 / 45.12)	1.6	3.8	1.6	3.8
2.2.30	Knowledge of RO duties in the control room during fuel handling such as alarms from fuel handling area / communication with fuel storage facility / systems operated from the control room in support of fueling operations / and supporting instrumentation.	(45.12)	3.5	3.3	3.5	3.3
2.2.31	Knowledge of procedures and limitations involved in initial core loading.	(43.6)	2.2	2.9*	2.2	2.9
2.2.32	Knowledge of the effects of alterations on core configuration.	(43.6)	2.3	3.3	2.3	3.3
2.2.33	Knowledge of control rod programming.	(43.6)	2.5	2.9	2.5	2.9
2.2.34	Knowledge of the process for determining the internal and external effects on core reactivity.	(43.6)	2.8	3.2*	2.8	3.2
2.3	**Radiological Controls**					
2.3.1	Knowledge of 10 CFR 20 and related facility radiation control requirements.	(41.12 / 43.4 / 45.9 / 45.10)	2.6	3.0	2.6	3.0
2.3.2	Knowledge of facility ALARA program.	(41.12 / 43.4 / 45.9 / 45.10)	2.5	2.9	2.5	2.9
2.3.3	Knowledge of SRO responsibilities for auxiliary systems that are outside the control room (e.g., waste disposal and handling systems).	(43.4 / 45.10)	1.8	2.9	1.8	2.9
2.3.4	Knowledge of radiation exposure limits and contamination control, including permissible levels in excess of those authorized.	(43.4 / 45.10)	2.5	3.1	2.5	3.1
2.3.5	Knowledge of use and function of personnel monitoring equipment.	(41.11 / 45.9)	2.3	2.5	2.3	2.5
2.3.6	Knowledge of the requirements for reviewing and approving release permits.	(43.4 / 45.10)	2.1	3.1	2.1	3.1
2.3.7	Knowledge of the process for preparing a radiation work permit.	(41.10 / 45.12)	2.0	3.3	2.0	3.3
2.3.8	Knowledge of the process for performing a planned gaseous radioactive release.	(43.4 / 45.10)	2.3	3.2	2.3	3.2
2.3.9	Knowledge of the process for performing a containment purge.	(43.4 / 45.10)	2.5	3.4	2.5	3.4
2.3.10	Ability to perform procedures to reduce excessive levels of radiation and guard against personnel exposure.	(43.4 / 45.10)	2.9	3.3	2.9	3.3
2.3.11	Ability to control radiation releases.	(45.9 / 45.10)	2.7	3.2	2.7	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number:

System Name: **GENERIC**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
2.4	**Emergency Procedures and Plan**					
2.4.1	Knowledge of EOP entry conditions and immediate action steps.	(41.10 / 43.5 / 45.13)	4.3	4.6	4.3	4.6
2.4.2	Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions.	(41.7 / 45.7 / 45.8)	3.9	4.1	3.9	4.1
2.4.3	Ability to identify post-accident instrumentation.	(41.6 / 45.4)	3.5	3.8	3.5	3.8
2.4.4	Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures.	(41.10 / 43.2 / 45.6)	4.0	4.3	4.0	4.3
2.4.5	Knowledge of the organization of the operating procedures network for normal, abnormal, and emergency evolutions.	(41.10 / 43.5 / 45.13)	2.9	3.6	2.9	3.6
2.4.6	Knowledge symptom based EOP mitigation strategies.	(41.10 / 43.5 / 45.13)	3.1	4.0	3.1	4.0
2.4.7	Knowledge of event based EOP mitigation strategies.	(41.10 / 43.5 / 45.13)	3.1	3.8	3.1	3.8
2.4.8	Knowledge of how the event-based emergency/abnormal operating procedures are used in conjunction with the symptom-based EOPs.	(41.10 / 43.5 / 45.13)	3.0	3.7	3.0	3.7
2.4.9	Knowledge of low power / shutdown implications in accident (e.g. LOCA or loss of RHR) mitigation strategies.	(41.10 / 43.5 / 45.13)	3.3	3.9	3.3	3.9
2.4.10	Knowledge of annunciator response procedures.	(41.10 / 43.5 / 45.13)	3.0	3.1	3.0	3.1
2.4.11	Knowledge of abnormal condition procedures.	(41.10 / 43.5 / 45.13)	3.4	3.6	3.4	3.6
2.4.12	Knowledge of general operating crew responsibilities during emergency operations.	(41.10 / 45.12)	3.4	3.9	3.4	3.9
2.4.13	Knowledge of crew roles and responsibilities during EOP flowchart use.	(41.10 / 45.12)	3.3	3.9	3.3	3.9
2.4.14	Knowledge of general guidelines for EOP flowchart use.	(41.10 / 45.13)	3.0	3.9	3.0	3.9
2.4.15	Knowledge of communications procedures associated with EOP implementation.	(41.10 / 45.13)	3.0	3.5	3.0	3.5
2.4.16	Knowledge of EOP implementation hierarchy and coordination with other support procedures.	(41.10 / 43.5 / 45.13)	3.0	4.0	3.0	4.0
2.4.17	Knowledge of EOP terms and definitions.	(41.10 / 45.13)	3.1	3.8	3.1	3.8
2.4.18	Knowledge of the specific bases for EOPs.	(41.10 / 45.13)	2.7	3.6	2.7	3.6
2.4.19	Knowledge of EOP layout, symbols, and icons.	(41.10 / 45.13)	2.7	3.7	2.7	3.7
2.4.20	Knowledge of operational implications of EOP warnings, cautions, and notes.	(41.10 / 45.13)	3.3	4.0	3.3	4.0
2.4.21	Knowledge of the parameters and logic used to assess the status of safety functions including: 1.Reactivity control 2.Core cooling and heat removal 3.Reactor coolant system integrity 4.Containment conditions 5.Radioactivity release control.	(43.5 / 45.12)	3.7	4.3	3.7	4.3
2.4.22	Knowledge of the bases for prioritizing safety functions during abnormal/emergency operations.	(43.5 / 45.12)	3.0	4.0	3.0	4.0
2.4.23	Knowledge of the bases for prioritizing emergency procedure implementation during emergency operations.	(41.10 / 45.13)	2.8	3.8	2.8	3.8
2.4.24	Knowledge of loss of cooling water procedures.	(41.10 / 45.13)	3.3	3.7	3.3	3.7
2.4.25	Knowledge of fire protection procedures.	(41.10 / 45.13)	2.9	3.4	2.9	3.4
2.4.26	Knowledge of facility protection requirements including fire brigade and portable fire fighting equipment usage.	(43.5 / 45.12)	2.9	3.3	2.9	3.3
2.4.27	Knowledge of fire in the plant procedure.	(41.10 / 43.5 / 45.13)	3.0	3.5	3.0	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number:

System Name: GENERIC

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
2.4	**Emergency Procedures and Plan**					
2.4.28	Knowledge of procedures relating to emergency response to sabotage.	(41.10 / 43.5 / 45.13)	2.3	3.3	2.3	3.3
2.4.29	Knowledge of the emergency plan.	(43.5 / 45.11)	2.6	4.0	2.6	4.0
2.4.30	Knowledge of which events related to system operations/status should be reported to outside agencies.	(43.5 / 45.11)	2.2	3.6	2.2	3.6
2.4.31	Knowledge of annunciators alarms and indications, and use of the response instructions.	(41.10 / 45.3)	3.3	3.4	3.3	3.4
2.4.32	Knowledge of operator response to loss of all annunciators.	(41.10 / 43.5 / 45.13)	3.3	3.5	3.3	3.5
2.4.33	Knowledge of the process used track inoperable alarms.	(41.10 / 43.5 / 45.13)	2.4	2.8	2.4	2.8
2.4.34	Knowledge of RO tasks performed outside the main control room during emergency operations including system geography and system implications.	(43.5 / 45.13)	3.8	3.6	3.8	3.6
2.4.35	Knowledge of local auxiliary operator tasks during emergency operations including system geography and system implications.	(43.5 / 45.13)	3.3	3.5	3.3	3.5
2.4.36	Knowledge of chemistry / health physics tasks during emergency operations.	(43.5)	2.0	2.8	2.0	2.8
2.4.37	Knowledge of the lines of authority during an emergency.	(45.13)	2.0	3.5	2.0	3.5
2.4.38	Ability to take actions called for in the facility emergency plan, including (if required)supporting or acting as emergency coordinator.	(43.5 / 45.11)	2.2	4.0	2.2	4.0
2.4.39	Knowledge of the RO's responsibilities in emergency plan implementation.	(45.11)	3.3	3.1	3.3	3
2.4.40	Knowledge of the SRO's responsibilities in emergency plan implementation.	(45.11)	2.3	4.0	2.3	4.0
2.4.41	Knowledge of the emergency action level thresholds and classifications.	(43.5 / 45.11)	2.3	4.1	2.3	4.1
2.4.42	Knowledge of emergency response facilities.	(45.11)	2.3	3.7	2.3	3.7
2.4.43	Knowledge of emergency communications systems and techniques.	(45.13)	2.8	3.5	2.8	3.5
2.4.44	Knowledge of emergency plan protective action recommendations.	(43.5 / 45.11)	2.1	4.0	2.1	4.0
2.4.45	Ability to prioritize and interpret the significance of each annunciator or alarm.	(43.5 / 45.3 / 45.12)	3.3	3.6	3.3	3.6
2.4.46	Ability to verify that the alarms are consistent with the plant conditions.	(43.5 / 45.3 / 45.12)	3.5	3.6	3.5	3.6
2.4.47	Ability to diagnose and recognize trends in an accurate and timely manner utilizing the appropriate control room reference material.	(41.10 / 43.5 / 45.12)	3.4	3.7	3.4	3.7
2.4.48	Ability to interpret control room indications to verify the status and operation of system, and understand how operator action s and directives affect plant and system conditions.	(43.5 / 45.12)	3.5	3.8	3.5	3.8
2.4.49	Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.	(41.10 / 43.2 / 45.6)	4.0	4.0	4.0	4.0
2.4.50	Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.	(45.3)	3.3	3.3	3.3	3

KA Catalog

Facility: CPS

Printed: 08/11/2006

System Number:

System Name: **GENERIC**

		NRC Imp		Facility Imp	
	CFR	RO	SRO	RO	SRO

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295001

System Name: Partial or Complete Loss of Forced Core Flow Circulation

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION:**	**(41.8 to 41.10)**				
AK1.01	Natural circulation		3.5	3.6	3.5	3.6
AK1.02	Power/flow distribution		3.3	3.5	3.3	3.5
AK1.03	†Thermal limits		3.6	4.1	3.6	4.1
AK1.04	†Limiting cycle oscillation: Plant-Specific		2.5	3.3	2.5	3.3
K2	**Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION and the following:**	**(41.7, 45.8)**				
AK2.01	Recirculation system		3.6	3.7	3.6	3.7
AK2.02	Nuclear boiler instrumentation		3.2	3.3	3.2	3.3
AK2.03	Reactor water level		3.6	3.7	3.6	3.7
AK2.04	Reactor/turbine pressure regulating system: Plant-Specific		3.3	3.3	3.3	3.3
AK2.05	~~LPCI loop select logic: Plant-Specific~~		3.2	3.6	3.2	3.6
AK2.06	Reactor power		3.8	3.8	3.8	3.8
AK2.07	Core flow indication		3.4	3.4	3.4	3.4
AK2.08	~~Standby liquid control: BWR-1~~		2.5*	2.8	2.5	2.8
.3	**Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION:**	**(41.5 / 45.6)**				
AK3.01	Reactor water level response		3.4	3.6	3.4	3.6
AK3.02	Reactor power response		3.7	3.8	3.7	3.8
AK3.03	Idle loop flow		2.8	2.9	2.8	2.9
AK3.04	Reactor SCRAM		3.4	3.6	3.4	3.6
AK3.05	Reduced loop operating requirements: Plant-Specific		3.2	3.6	3.2	3.6
AK3.06	Core flow indication		2.9	3.0	2.9	3.0
A1	**Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION:**	**(41.7 / 45.6)**				
AA1.01	Recirculation system		3.5	3.6	3.5	3.6
AA1.02	RPS		3.3	3.3	3.3	3.3
AA1.03	~~RMCS: Plant-Specific~~		2.6	2.7	2.6	2.7
AA1.04	Rod control and information system: BWR-5&6		2.6	2.8	2.6	2.8
AA1.05	Recirculation flow control system		3.3	3.3	3.3	3.3
AA1.06	Neutron monitoring system		3.3	3.4	3.3	3.4
AA1.07	Nuclear boiler instrumentation system		3.1	3.2	3.1	3.2
AA1.08	~~Standby liquid control: BWR-1~~		2.5*	2.8	2.5	2.8

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295001

System Name: Partial or Complete Loss of Forced Core Flow Circulation

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir .
A2	**Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Power/flow map		3.5	3.8	3.5	3.8
AA2.02	Neutron monitoring		3.1	3.2	3.1	3.2
AA2.03	Actual core flow		3.3	3.3	3.3	3.3
AA2.04	Individual jet pump flows: Not-BWR-1&2		3.0	3.1	3.0	3.1
AA2.05	Jet pump operability: Not-BWR-1&2		3.1	3.4	3.1	3.4
AA2.06	Nuclear boiler instrumentation		3.2	3.3	3.2	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295002

System Name: Loss of Main Condenser Vacuum

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to LOSS OF MAIN CONDENSER VACUUM:**	**(41.8 to 41.10)**				
AK1.01	Plant efficiency		2.1*	2.5	2.1	2.5
AK1.02	Turbine efficiency		2.2*	2.5	2.2	2.5
AK1.03	Loss of heat sink		3.6	3.8	3.6	3.8
AK1.04	Increased offgas flow		3.0	3.3	3.0	3.3
K2	**Knowledge of the interrelations between LOSS OF MAIN CONDENSER VACUUM and the following:**	**(41.7, 45.8)**				
AK2.01	RPS		3.5	3.5	3.5	3.5
AK2.02	Main turbine		3.1	3.2	3.1	3.2
AK2.03	PCIS/NSSSS		3.5	3.6	3.5	3.6
AK2.04	Reactor/turbine pressure regulating system		3.2	3.3	3.2	3.3
AK2.05	Feedwater system		2.7	2.7	2.7	2.7
AK2.06	Condensate system		2.6	2.7	2.6	2.7
AK2.07	Offgas system		3.1	3.1	3.1	3.1
AK2.08	Condenser circulating water system		3.1	3.2	3.1	3.2
AK2.09	Vacuum drag (low conductivity drain): Plant-Specific		2.4	2.4	2.4	2.4
AK2.10	Reactor recirculation system: Plant-Specific		2.4*	2.5	2.4	2.5
AK2.11	Seal steam: Plant-Specific		2.6	2.7	2.6	2.7
K3	**Knowledge of the reasons for the following responses as they apply to LOSS OF MAIN CONDENSER VACUUM:**	**(41.5 / 45.6)**				
AK3.01	Reactor SCRAM: Plant-Specific		3.7	3.8	3.7	3.8
AK3.02	Turbine trip		3.4	3.4	3.4	3.4
AK3.03	Reactor feedpump turbine trip: Plant-Specific		3.3	3.3	3.3	3.3
AK3.04	Bypass valve closure		3.4	3.6	3.4	3.6
AK3.05	Main steam isolation valve: Plant-Specific		3.4	3.4	3.4	3.4
AK3.06	Air ejector flow		2.9	2.9	2.9	2.9
AK3.07	Decreased main generator output		2.4	2.5	2.4	2.5
AK3.08	~~Recirculation system run-backs: Plant-Specific~~		2.8	2.9	2.8	2.9
AK3.09	Reactor power reduction		3.2	3.2	3.2	3.2

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295002

System Name: Loss of Main Condenser Vacuum

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
A1	**Ability to operate and/or monitor the following as they apply to LOSS OF MAIN CONDENSER VACUUM:**	**(41.7 / 45.6)**				
AA1.01	Condensate system		2.6	2.6	2.6	2.6
AA1.02	Offgas system		2.9	2.9	2.9	2.9
AA1.03	RPS		3.4	3.5	3.4	3.5
AA1.04	PCIS/NSSSS		3.3	3.4	3.3	3.4
AA1.05	Main turbine		3.2	3.2	3.2	3.2
AA1.06	Reactor/turbine pressure regulating system		3.0	3.1	3.0	3.1
AA1.07	Condenser circulating water system		3.1	2.9	3.1	2.9
AA1.08	Recirculating flow control system		2.6	2.7	2.6	2.7
AA1.09	Reactor manual control/rod control and information system		2.4	2.4	2.4	2.4
AA1.10	Feedwater system: Plant-Specific		2.7	2.7	2.7	2.7
A2	**Ability to determine and/or interpret the following as they apply to LOSS OF MAIN CONDENSER VACUUM:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Condenser vacuum/absolute pressure		2.9	3.1	2.9	3.1
AA2.02	Reactor power: Plant-Specific		3.2	3.3	3.2	3.3
AA2.03	Generator output		2.3	2.4	2.3	2.4
AA2.04	Offgas system flow		2.8	2.9	2.8	2.9

Facility: CPS Printed: 08/11/2006

System Number: 295003

System Name: Partial or Complete Loss of A.C. Power

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER:**	**(41.8 to 41.10)**				
AK1.01	Effect of battery discharge rate on capacity		2.7	2.9	2.7	2.9
AK1.02	Load shedding		3.1	3.4	3.1	3.4
AK1.03	Under voltage/degraded voltage effects on electrical loads		2.9	3.2	2.9	3.2
AK1.04	Electrical bus divisional separation		3.1	3.2	3.1	3.2
AK1.05	Failsafe component design		2.6	2.7	2.6	2.7
AK1.06	Station blackout: Plant-Specific		3.8	4.0*	3.8	4.0
K2	**Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF A.C. POWER and the following:**	**(41.7, 45.8)**				
AK2.01	Station batteries		3.2	3.2	3.2	3.2
AK2.02	Emergency generators		4.1*	4.2*	4.1	4.2
AK2.03	A.C. electrical distribution system		3.7	3.9	3.7	3.9
AK2.04	A.C. electrical loads		3.4	3.5	3.4	3.5
AK2.05	~~Isolation condenser: Plant-Specific~~		3.8*	4.0	3.8	4.0
AK2.06	D.C. electrical loads		3.4	3.5	3.4	3.5
K3	**Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER:**	**(41.5 / 45.6)**				
AK3.01	Manual and auto bus transfer		3.3	3.5	3.3	3.5
AK3.02	Selective tripping		2.9	3.1	2.9	3.1
AK3.03	Load shedding		3.5	3.6	3.5	3.6
AK3.04	Ground isolation		3.0	3.2	3.0	3.2
AK3.05	Reactor SCRAM		3.7	3.7	3.7	3.7
AK3.06	Containment isolation		3.7	3.7	3.7	3.7
AK3.07	~~Initiation of isolation condenser: Plant-Specific~~		3.8*	4.0*	3.8	4.0
A1	**Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER:**	**(41.7 / 45.6)**				
AA1.01	A.C. electrical distribution system		3.7	3.8	3.7	3.8
AA1.02	Emergency generators		4.2*	4.3*	4.2	4.3
AA1.03	Systems necessary to assure safe plant shutdown		4.4*	4.4*	4.4	4.4
AA1.04	D.C. electrical distribution system		3.6	3.7	3.6	3.7

System Number: 295003

System Name: Partial or Complete Loss of A.C. Power

		CFR	NRC Imp RO	NRC Imp SRO	Facility Im RO	Facility Im SR
A2	**Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER:**	(41.10 / 43.5 / 45.13)				
AA2.01	Cause of partial or complete loss of A.C. power		3.4	3.7	3.4	3.7
AA2.02	Reactor power, pressure, and level		4.2*	4.3*	4.2	4.3
AA2.03	Battery status: Plant-Specific		3.2	3.5	3.2	3.5
AA2.04	System lineups		3.5	3.7	3.5	3.7
AA2.05	Whether a partial or complete loss of A.C. power has occurred		3.9*	4.2*	3.9	4.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295004

System Name: **Partial or Complete Loss of D.C. Power**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER:**	**(41.8 to 41.10)**				
AK1.01	†~~Automatic load shedding: Plant-Specific~~		2.9	3.2	2.9	3.2
AK1.02	*Redundant D.C. power supplies: Plant-Specific*		3.2	3.4	3.2	3.4
AK1.03	Electrical bus divisional separation		2.9	2.9	2.9	2.9
AK1.04	Effect of battery discharge rate on capacity		2.8	2.9	2.8	2.9
AK1.05	Loss of breaker protection		3.3	3.4	3.3	3.4
AK1.06	Prevention of inadvertent system(s) actuation upon restoration of D.C. power		3.3	3.6	3.3	3.6
K2	**Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF D.C. POWER and the following:**	**(41.7, 45.8)**				
AK2.01	Battery charger		3.1	3.1	3.1	3.1
AK2.02	Batteries		3.0	3.1	3.0	3.1
AK2.03	D.C. bus loads		3.3	3.3	3.3	3.3
K3	**Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER:**	**(41.5 / 45.6)**				
AK3.01	†Load shedding: Plant-Specific		2.6	3.1	2.6	3.1
AK3.02	Ground isolation/fault determination		2.9	3.3	2.9	3.3
AK3.03	Reactor SCRAM: Plant-Specific		3.1	3.5	3.1	3.5
A1	**Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER:**	**(41.7 / 45.6)**				
AA1.01	D.C. electrical distribution systems		3.3	3.4	3.3	3.4
AA1.02	Systems necessary to assure safe plant shutdown		3.8	4.1	3.8	4.1
AA1.03	A.C. electrical distribution		3.4	3.6	3.4	3.6
A2	**Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Cause of partial or complete loss of D.C. power		3.2	3.6	3.2	3.6
AA2.02	Extent of partial or complete loss of D.C. power		3.5	3.9	3.5	3.9
AA2.03	Battery voltage		2.8	2.9	2.8	2.9
AA2.04	System lineups		3.2	3.3	3.2	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295005

System Name: *Main Turbine Generator Trip*

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to MAIN TURBINE GENERATOR TRIP:**	**(41.8 to 41.10)**				
AK1.01	Pressure effects on reactor power		4.0	4.1	4.0	4.1
AK1.02	†*Core thermal limit considerations*		3.2	3.6	3.2	3.6
AK1.03	Pressure effects on reactor level		3.5	3.7	3.5	3.7
K2	**Knowledge of the interrelations between MAIN TURBINE GENERATOR TRIP and the following:**	**(41.7, 45.8)**				
AK2.01	RPS		3.8	3.9	3.8	3.9
AK2.02	Feedwater temperature		2.9	3.0	2.9	3.0
AK2.03	Recirculation system		3.2	3.3	3.2	3.3
AK2.04	Main generator protection		3.3	3.3	3.3	3.3
AK2.05	Extraction steam system		2.6	2.7	2.6	2.7
AK2.06	Seal steam evaporator: Plant-Specific		1.9*	1.9*	1.9	1.9
AK2.07	Reactor pressure control		3.6	3.7	3.6	3.7
AK2.08	A.C. electrical distribution.:		3.2	3.3	3.2	3.3
AK2.09	~~Feedwater HPCI: BWR-2~~		4.0	4.3	4.0	4.3
K3	**Knowledge of the reasons for the following responses as they apply to MAIN TURBINE GENERATOR TRIP:**	**(41.5 / 45.6)**				
AK3.01	Reactor scram		3.8	3.8	3.8	3.8
AK3.02	Recirculation pump downshift/trip: Plant-Specific		3.4	3.5	3.4	3.5
AK3.03	Feedwater temperature decrease		2.8	3.0	2.8	3.0
AK3.04	Main generator trip		3.2	3.2	3.2	3.2
AK3.05	Extraction steam/moisture separator isolations		2.5	2.6	2.5	2.6
AK3.06	Realignment of electrical distribution		3.3	3.3	3.3	3.3
AK3.07	Bypass valve operation		3.8	3.8	3.8	3.8
AK3.08	~~Feedwater HPCI actuation: BWR-2~~		4.0	4.3	4.0	4.3
A1	**Ability to operate and/or monitor the following as they apply to MAIN TURBINE GENERATOR TRIP:**	**(41.7 / 45.6)**				
AA1.01	Recirculation system: Plant-Specific		3.1	3.3	3.1	3.3
AA1.02	RPS		3.6	3.6	3.6	3.6
AA1.03	Reactor manual control/rod control and information system		2.7	2.8	2.7	2.8
AA1.04	Main generator controls		2.7	2.8	2.7	2.8
AA1.05	Reactor/turbine pressure regulating system		3.6	3.6	3.6	3.6
AA1.06	Condenser vacuum breaker		2.4	2.6	2.4	2.6
AA1.07	A.C. electrical distribution		3.3	3.3	3.3	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295005

System Name: Main Turbine Generator Trip

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S
A2	**Ability to determine and/or interpret the following as they apply to MAIN TURBINE GENERATOR TRIP:**	(41.10 / 43.5 / 45.13)				
AA2.01	Turbine speed		2.6	2.7	2.6	2.7
AA2.02	Turbine vibration		2.4	2.7	2.4	2.7
AA2.03	Turbine valve position		3.1	3.1	3.1	3.1
AA2.04	Reactor pressure		3.7	3.8	3.7	3.8
AA2.05	Reactor power		3.8	3.9	3.8	3.9
AA2.06	Feedwater temperature		2.6	2.7	2.6	2.7
AA2.07	Reactor water level		3.5	3.6	3.5	3.6
AA2.08	Electrical distribution status		3.2	3.3	3.2	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295006

System Name: SCRAM

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to SCRAM:**	**(41.8 to 41.10)**				
AK1.01	Decay heat generation and removal.		3.7	3.9	3.7	3.9
AK1.02	Shutdown margin		3.4	3.7	3.4	3.7
AK1.03	Reactivity control		3.7	4.0	3.7	4.0
K2	**Knowledge of the interrelations between SCRAM and the following:**	**(41.7, 45.8)**				
AK2.01	RPS		4.3*	4.4*	4.3	4.4
AK2.02	Reactor water level control system		3.8	3.8	3.8	3.8
AK2.03	CRD hydraulic system		3.7	3.8	3.7	3.8
AK2.04	Turbine trip logic: Plant-Specific		3.6	3.7	3.6	3.7
AK2.05	CRD mechanism		3.1	3.3	3.1	3.3
AK2.06	Reactor power		4.2*	4.3*	4.2	4.3
AK2.07	Reactor pressure control		4.0	4.1	4.0	4.1
K3	**Knowledge of the reasons for the following responses as they apply to SCRAM:**	**(41.5 / 45.6)**				
AK3.01	Reactor water level response		3.8	3.9	3.8	3.9
AK3.02	Reactor power response		4.1*	4.2*	4.1	4.2
AK3.03	Reactor pressure response		3.8	3.9*	3.8	3.9
AK3.04	Reactor water level setpoint setdown: Plant-Specific		3.1	3.3	3.1	3.3
AK3.05	Direct turbine generator trip: Plant-Specific		3.8	4.0	3.8	4.0
AK3.06	Recirculation pump speed reduction: Plant-Specific		3.2	3.3	3.2	3.3
A1	**Ability to operate and/or monitor the following as they apply to SCRAM:**	**(41.7 / 45.6)**				
AA1.01	RPS		4.2*	4.2*	4.2	4.2
AA1.02	Reactor water level control system		3.9	3.8	3.9	3.8
AA1.03	Reactor/turbine pressure regulating system		3.7	3.7	3.7	3.7
AA1.04	Recirculation system		3.1	3.2	3.1	3.2
AA1.05	Neutron monitoring system		4.2*	4.2*	4.2	4.2
AA1.06	CRD hydraulic system		3.5	3.6	3.5	3.6
AA1.07	Control rod position		4.1	4.1	4.1	4.1
A2	**Ability to determine and/or interpret the following as they apply to SCRAM:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Reactor power		4.5*	4.6*	4.5	4.6
AA2.02	Control rod position		4.3*	4.4*	4.3	4.4
AA2.03	Reactor water level		4.0	4.2*	4.0	4.2
AA2.04	Reactor pressure		4.1	4.1*	4.1	4.1
AA2.05	Whether a reactor SCRAM has occurred		4.6*	4.6*	4.6	4.6
AA2.06	Cause of reactor SCRAM		3.5	3.8	3.5	3.8

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295007

System Name: High Reactor Pressure

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH REACTOR PRESSURE:**	**(41.8 to 41.10)**				
AK1.01	Pump shutoff head		2.9	3.2	2.9	3.2
AK1.02	Decay heat generation		3.1	3.4	3.1	3.4
AK1.03	Pressure effects on reactor power		3.8	3.9	3.8	3.9
AK1.04	Turbine load		2.7	2.8	2.7	2.8
K2	**Knowledge of the interrelations between HIGH REACTOR PRESSURE and the following:**	**(41.7, 45.8)**				
AK2.01	Reactor/turbine pressure regulating system		3.5	3.7	3.5	3.7
AK2.02	Reactor power		3.8	3.8	3.8	3.8
AK2.03	RHR/LPCI: Plant-Specific		3.1	3.2	3.1	3.2
AK2.04	LPCS		3.2	3.3	3.2	3.3
AK2.05	Shutdown cooling: Plant-Specific		2.9	3.1	2.9	3.1
AK2.06	PCIS/NSSSS: Plant-Specific		3.5	3.7	3.5	3.7
K3	**Knowledge of the reasons for the following responses as they apply to HIGH REACTOR PRESSURE:**	**(41.5 / 45.6)**				
AK3.01	~~Isolation condenser operation: Plant-Specific~~		4.0	4.2	4.0	4.2
AK3.02	~~HPCI operation: Plant-Specific~~		3.7	3.8*	3.7	3.8
AK3.03	RCIC operation: Plant-Specific		3.4	3.5	3.4	3.5
AK3.04	Safety/relief valve operation: Plant-Specific		4.0	4.1*	4.0	4.1
AK3.05	Low pressure system isolation		3.0	3.2	3.0	3.2
AK3.06	Reactor/turbine pressure regulating system operation		3.7	3.8	3.7	3.8
A1	**Ability to operate and/or monitor the following as they apply to HIGH REACTOR PRESSURE:**	**(41.7 / 45.6)**				
AA1.01	~~Isolation condenser: Plant-Specific~~		4.0	4.2	4.0	4.2
AA1.02	~~HPCI: Plant-Specific~~		3.5	3.7*	3.5	3.7
AA1.03	RCIC: Plant-Specific		3.4	3.5	3.4	3.5
AA1.04	Safety/relief valve operation: Plant-Specific		3.9	4.1*	3.9	4.1
AA1.05	Reactor/turbine pressure regulating system		3.7	3.8	3.7	3.8
A2	**Ability to determine and/or interpret the following as they apply to HIGH REACTOR PRESSURE:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Reactor pressure		4.1*	4.1*	4.1	4.1
AA2.02	Reactor power		4.1*	4.1*	4.1	4.1
AA2.03	Reactor water level		3.7	3.7	3.7	3.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295008

System Name: High Reactor Water Level

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH REACTOR WATER LEVEL:**	**(41.8 to 41.10)**				
AK1.01	Moisture carryover		3.0	3.2	3.0	3.2
AK1.02	Component erosion/damage		2.8	2.8	2.8	2.8
AK1.03	Feed flow/steam flow mismatch		3.2	3.2	3.2	3.2
AK1.04	~~Containment integrity: Allis-Chalmers~~		4.0	4.3*	4.0	4.3
K2	**Knowledge of the interrelations between HIGH REACTOR WATER LEVEL and the following:**	**(41.7, 45.8)**				
AK2.01	RPS: Plant-Specific		3.7	3.8	3.7	3.8
AK2.02	Reactor feedwater system		3.6	3.8	3.6	3.8
AK2.03	Reactor water level control		3.6	3.7	3.6	3.7
AK2.04	PCIS/NSSSS: Plant-Specific		3.1	3.3	3.1	3.3
AK2.05	~~HPCI: Plant-Specific~~		3.8	3.9	3.8	3.9
AK2.06	RCIC: Plant-Specific		3.4	3.6	3.4	3.6
AK2.07	HPCS: Plant-Specific		2.9	3.0	2.9	3.0
AK2.08	Main turbine: Plant-Specific		3.4	3.5	3.4	3.5
AK2.09	Reactor water cleanup system (ability to drain): Plant-Specific		3.1	3.1	3.1	3.1
AK2.10	RHR (ability to drain): Plant-Specific		2.7	2.8	2.7	2.8
K2.11	Main steam		3.1	3.3	3.1	3.3
K3	**Knowledge of the reasons for the following responses as they apply to HIGH REACTOR WATER LEVEL:**	**(41.5 / 45.6)**				
AK3.01	Main turbine trip		3.4	3.5	3.4	3.5
AK3.02	Reactor SCRAM: Plant-Specific		3.6*	3.9*	3.6	3.9
AK3.03	PCIS/NSSSS initiation: Plant-Specific		2.9	3.1	2.9	3.1
AK3.04	Reactor feed pump trip: Plant-Specific		3.3	3.5	3.3	3.5
AK3.05	~~HPCI turbine trip: Plant-Specific~~		3.5	3.6	3.5	3.6
AK3.06	RCIC turbine trip: Plant-Specific		3.4	3.5	3.4	3.5
AK3.07	HPCS isolation: Plant-Specific		3.2	3.3	3.2	3.3
AK3.08	RCIC steam supply valve closure: Plant-Specific		3.4	3.5	3.4	3.5
AK3.09	HPCS injection valve closure: Plant-Specific		3.3	3.4	3.3	3.4

Facility: CPS Printed: 08/11/2006

System Number: 295008

System Name: High Reactor Water Level

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir
A1	**Ability to operate and/or monitor the following as they apply to HIGH REACTOR WATER LEVEL:**	**(41.7 / 45.6)**				
AA1.01	Reactor water level control: Plant-Specific		3.7	3.7	3.7	3.7
AA1.02	Reactor water cleanup (ability to drain): Plant-Specific		3.3	3.3	3.3	3.3
AA1.03	Main steam system: Plant-Specific		3.1	3.1	3.1	3.1
AA1.04	~~HPCI: Plant-Specific~~		3.5	3.5	3.5	3.5
AA1.05	RCIC: Plant-Specific		3.3	3.3	3.3	3.3
AA1.06	HPCS: Plant-Specific		2.8	2.8	2.8	2.8
AA1.07	Main turbine: Plant-Specific		3.4	3.4	3.4	3.4
AA1.08	Feedwater system		3.5	3.5	3.5	3.5
AA1.09	Ability to drain: Plant-Specific		3.3	3.3	3.3	3.3
A2	**Ability to determine and/or interpret the following as they apply to HIGH REACTOR WATER LEVEL:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Reactor water level		3.9	3.9	3.9	3.9
AA2.02	Steam flow/feedflow mismatch		3.4	3.4	3.4	3.4
AA2.03	Reactor water cleanup blowdown flow		2.9	3.0	2.9	3.0
AA2.04	Heatup rate: Plant-Specific		3.1	3.3	3.1	3.3
AA2.05	Swell		2.9	3.1	2.9	3.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: **295009**

System Name: **Low Reactor Water Level**

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to LOW REACTOR WATER LEVEL:**	**(41.8 to 41.10)**				
AK1.01	Steam carryunder		2.7	2.9	2.7	2.9
AK1.02	Recirculation pump net positive suction head: Plant-Specific		3.0	3.1	3.0	3.1
AK1.03	Jet pump net positive suction head: Not-BWR-1&2		2.7	2.7	2.7	2.7
AK1.04	Jet pump efficiency: Not-BWR-1&2		2.4	2.4	2.4	2.4
AK1.05	Natural circulation		3.3	3.4	3.3	3.4
K2	**Knowledge of the interrelations between LOW REACTOR WATER LEVEL and the following:**	**(41.7, 45.8)**				
AK2.01	Reactor water level indication		3.9	4.0	3.9	4.0
AK2.02	Reactor water level control		3.9	3.9	3.9	3.9
AK2.03	Recirculation system		3.1	3.2	3.1	3.2
AK2.04	Reactor water cleanup		2.6	2.6	2.6	2.6
K3	**Knowledge of the reasons for the following responses as they apply to LOW REACTOR WATER LEVEL:**	**(41.5 / 45.6)**				
AK3.01	Recirculation pump run back: Plant-Specific		3.2	3.3	3.2	3.3
AK3.02	Reactor feedpump runout flow control: Plant-Specific		2.7	2.8	2.7	2.8
A1	**Ability to operate and/or monitor the following as they apply to LOW REACTOR WATER LEVEL:**	**(41.7 / 45.6)**				
AA1.01	Reactor feedwater		3.9	3.9	3.9	3.9
AA1.02	Reactor water level control		4.0	4.0	4.0	4.0
AA1.03	Recirculation system: Plant-Specific		3.0	3.1	3.0	3.1
AA1.04	Reactor water cleanup		2.7	2.7	2.7	2.7
A2	**Ability to determine and/or interpret the following as they apply to LOW REACTOR WATER LEVEL:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Reactor water level		4.2	4.2	4.2	4.2
AA2.02	Steam flow/feedflow mismatch		3.6	3.7	3.6	3.7
AA2.03	Reactor water cleanup blowdown rate		2.9	2.9	2.9	2.9

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295010

System Name: High Drywell Pressure

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL PRESSURE:**	**(41.8 to 41.10)**				
AK1.01	~~Downcomer submergence: Mark-I&II~~		3.0	3.4	3.0	3.4
AK1.02	Submergence vent control: Mark-III		2.8	3.1*	2.8	3.1
AK1.03	Temperature increases		3.2	3.4	3.2	3.4
K2	**Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following:**	**(41.7, 45.8)**				
AK2.01	Suppression pool level		3.2	3.3	3.2	3.3
AK2.02	~~Drywell/suppression chamber differential pressure: Mark-I&II~~		3.3	3.5	3.3	3.5
AK2.03	Drywell/containment differential pressure: Mark-III		3.0	3.1	3.0	3.1
AK2.04	~~Nitrogen makeup system: Plant-Specific~~		2.6	2.8	2.6	2.8
AK2.05	Drywell cooling and ventilation		3.7	3.8	3.7	3.8
K3	**Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL PRESSURE:**	**(41.5 / 45.6)**				
AK3.01	Drywell venting		3.8	4.0*	3.8	4.0
AK3.02	Increased drywell cooling		3.4	3.4	3.4	3.4
AK3.03	Radiation level monitoring		3.2	3.5	3.2	3.5
AK3.04	Leak investigation		3.5	3.8	3.5	3.8
AK3.05	Temperature monitoring		3.5	3.4	3.5	3.4
AK3.06	~~Termination of drywell inerting: Plant-Specific~~		2.4*	2.8	2.4	2.8
A1	**Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE:**	**(41.7 / 45.6)**				
AA1.01	Drywell ventilation/cooling		3.4	3.5	3.4	3.5
AA1.02	Drywell floor and equipment drain sumps		3.6	3.6	3.6	3.6
AA1.03	~~Nitrogen makeup: Plant-Specific~~		2.6	2.6	2.6	2.6
AA1.04	Drywell sampling system		3.1	3.0	3.1	3.0
AA1.05	Drywell/suppression vent and purge		3.1	3.4	3.1	3.4
AA1.06	Leakage detection systems		3.3	3.5	3.3	3.5
AA1.07	Containment (drywell) atmosphere control		3.2	3.4	3.2	3.4
A2	**Ability to determine and/or interpret the following as they apply to HIGH DRYWELL PRESSURE:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	†Leak rates		3.4	3.8	3.4	3.8
AA2.02	Drywell pressure		3.8	3.9	3.8	3.9
AA2.03	Drywell radiation levels		3.3	3.6	3.3	3.6
AA2.04	~~Drywell humidity: Plant-Specific~~		2.8	3.0	2.8	3.0
AA2.05	Drywell air cooler drain flow: BWR-6		3.3	3.3	3.3	3.3
AA2.06	Drywell temperature		3.6	3.6	3.6	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295011

System Name: High Containment Temperature (Mark III Containment Only)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):**	**(41.8 to 41.10)**				
AK1.01	Containment pressure: Mark-III		4.0*	4.1*	4.0	4.1
K2	**Knowledge of the interrelations between HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY) and the following:**	**(41.7, 45.8)**				
AK2.01	Containment ventilation/cooling: Mark-III		3.7	4.0	3.7	4.0
K3	**Knowledge of the reasons for the following responses as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):**	**(41.5 / 45.6)**				
AK3.01	Increased containment cooling: Mark-III		3.6	3.9	3.6	3.9
A1	**Ability to operate and/or monitor the following as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):**	**(41.7 / 45.6)**				
AA1.01	Containment ventilation/cooling system: Mark-III		3.6	3.9	3.6	3.9
A2	**Ability to determine and/or interpret the following as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Containment temperature: Mark-III		3.6	3.9	3.6	3.9
AA2.02	Containment pressure: Mark-III		4.0*	4.1*	4.0	4.1
AA2.03	~~Containment humidity: Mark-III~~		2.8	3.2	2.8	3.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295012

System Name: High Drywell Temperature

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL TEMPERATURE:**	**(41.8 to 41.10)**				
AK1.01	Pressure/temperature relationship		3.3	3.5	3.3	3.5
AK1.02	Reactor power level control		3.1	3.2	3.1	3.2
K2	**Knowledge of the interrelations between HIGH DRYWELL TEMPERATURE and the following:**	**(41.7, 45.8)**				
AK2.01	Drywell ventilation		3.4	3.5	3.4	3.5
AK2.02	Drywell cooling		3.6	3.7	3.6	3.7
K3	**Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL TEMPERATURE:**	**(41.5 / 45.6)**				
AK3.01	Increased drywell cooling		3.5	3.6	3.5	3.6
A1	**Ability to operate and/or monitor the following as they apply to HIGH DRYWELL TEMPERATURE:**	**(41.7 / 45.6)**				
AA1.01	Drywell ventilation system		3.5	3.6	3.5	3.6
AA1.02	Drywell cooling system		3.8	3.8	3.8	3.8
A2	**Ability to determine and/or interpret the following as they apply to HIGH DRYWELL TEMPERATURE:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Drywell temperature		3.8	3.9	3.8	3.9
AA2.02	Drywell pressure		3.9	4.1	3.9	4.1
AA2.03	~~Drywell humidity: Plant-Specific~~		2.8	3.1	2.8	3.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295013

System Name: High Suppression Pool Temperature

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH SUPPRESSION POOL TEMPERATURE:**	**(41.8 to 41.10)**				
AK1.01	Pool stratification		2.5	2.6	2.5	2.6
AK1.02	Ambient temperature effects.		2.4	2.5	2.4	2.5
AK1.03	Localized heating		3.0	3.3	3.0	3.3
AK1.04	Complete condensation		2.9	3.2	2.9	3.2
K2	**Knowledge of the interrelations between HIGH SUPPRESSION POOL TEMPERATURE and the following:**	**(41.7, 45.8)**				
AK2.01	Suppression pool cooling		3.6	3.7	3.6	3.7
K3	**Knowledge of the reasons for the following responses as they apply to HIGH SUPPRESSION POOL TEMPERATURE:**	**(41.5 / 45.6)**				
AK3.01	Suppression pool cooling operation		3.6	3.8	3.6	3.8
AK3.02	Limiting heat additions		3.6	3.8	3.6	3.8
A1	**Ability to operate and/or monitor the following as they apply to HIGH SUPPRESSION POOL TEMPERATURE:**	**(41.7 / 45.6)**				
AA1.01	Suppression pool cooling		3.9	3.9	3.9	3.9
AA1.02	Systems that add heat to the suppression pool		3.9	3.9	3.9	3.9
A2	**Ability to determine and/or interpret the following as they apply to HIGH SUPPRESSION POOL TEMPERATURE:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Suppression pool temperature		3.8	4.0	3.8	4.0
AA2.02	Localized heating/stratification		3.2	3.5	3.2	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295014

System Name: Inadvertent Reactivity Addition

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to INADVERTENT REACTIVITY ADDITION:**	**(41.8 to 41.10)**				
AK1.01	~~Prompt critical~~		3.7	3.8	3.7	3.8
AK1.02	Reactivity anomaly		3.3	3.7	3.3	3.7
AK1.03	Shutdown margin		3.7	4.0	3.7	4.0
AK1.04	PCIOMR: Plant-Specific		3.0	3.4	3.0	3.4
AK1.05	†Fuel thermal limits		3.7	4.2*	3.7	4.2
AK1.06	Abnormal reactivity additions.		3.8	3.9	3.8	3.9
K2	**Knowledge of the interrelations between INADVERTENT REACTIVITY ADDITION and the following:**	**(41.7, 45.8)**				
AK2.01	RPS		3.9	4.1	3.9	4.1
AK2.02	†Fuel thermal limits		3.7	4.2*	3.7	4.2
AK2.03	Fuel temperature		3.3	3.4	3.3	3.4
AK2.04	Void concentration		3.2	3.3	3.2	3.3
AK2.05	Neutron monitoring system		4.0	4.1*	4.0	4.1
AK2.06	Moderator temperature		3.4	3.5	3.4	3.5
AK2.07	Reactor power		3.9	3.9	3.9	3.9
AK2.08	~~RMCS: Plant-Specific~~		3.4	3.5	3.4	3.5
AK2.09	Rod control and information system: Plant-Specific		3.4	3.6	3.4	3.6
AK2.10	Safety limits		4.1	4.5*	4.1	4.5
AK2.11	Recirculation flow control		3.6	3.7	3.6	3.7
K3	**Knowledge of the reasons for the following responses as they apply to INADVERTENT REACTIVITY ADDITION:**	**(41.5 / 45.6)**				
AK3.01	Reactor SCRAM		4.1*	4.1	4.1	4.1
AK3.02	Control rod blocks		3.7	3.7	3.7	3.7
A1	**Ability to operate and/or monitor the following as they apply to INADVERTENT REACTIVITY ADDITION:**	**(41.7 / 45.6)**				
AA1.01	RPS		4.0	4.1*	4.0	4.1
AA1.02	Recirculation flow control system		3.6	3.8	3.6	3.8
AA1.03	~~RMCS: Plant-Specific~~		3.5	3.5	3.5	3.5
AA1.04	Rod control and information system: Plant-Specific		3.2	3.3	3.2	3.3
AA1.05	Neutron monitoring system		3.9	3.9	3.9	3.9
AA1.06	Reactor/turbine pressure regulating system		3.3	3.4	3.3	3.4
AA1.07	Cold water injection		4.0	4.1	4.0	4.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: **295014**

System Name: **Inadvertent Reactivity Addition**

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A2	**Ability to determine and/or interpret the following as they apply to INADVERTENT REACTIVITY ADDITION:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Reactor power		4.1*	4.2*	4.1	4.2
AA2.02	Reactor period		3.9	3.9	3.9	3.9
AA2.03	Cause of reactivity addition		4.0	4.3	4.0	4.3
AA2.04	†Violation of fuel thermal limits		4.1	4.4	4.1	4.4
AA2.05	†Violation of safety limits		4.2*	4.6*	4.2	4.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295015

System Name: Incomplete SCRAM

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to INCOMPLETE SCRAM:**	**(41.8 to 41.10)**				
AK1.01	Shutdown margin		3.6*	3.9*	3.6	3.9
AK1.02	Cooldown effects on reactor power		3.9	4.1	3.9	4.1
AK1.03	Reactivity effects		3.8	3.9	3.8	3.9
AK1.04	Reactor pressure: Plant-Specific		3.8	3.8	3.8	3.8
K2	**Knowledge of the interrelations between INCOMPLETE SCRAM and the following:**	**(41.7, 45.8)**				
AK2.01	CRD hydraulics		3.8	3.9	3.8	3.9
AK2.02	~~RMCS: Plant-Specific~~		3.6	3.7	3.6	3.7
AK2.03	Rod control and information system: Plant-Specific		3.2	3.6	3.2	3.6
AK2.04	RPS		4.0	4.1	4.0	4.1
AK2.05	~~Rod worth minimizer: Plant-Specific~~		2.6	2.8	2.6	2.8
AK2.06	~~RSCS: Plant-Specific~~		2.6	2.8	2.6	2.8
AK2.07	CRD mechanism		3.3	3.4	3.3	3.4
AK2.08	Neutron monitoring system		3.6	3.7	3.6	3.7
AK2.09	~~RPIS~~		3.5	3.6	3.5	3.6
AK2.10	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.8	3.0	2.8	3.0
AK2.11	Instrument air		3.5	3.7	3.5	3.7
K3	**Knowledge of the reasons for the following responses as they apply to INCOMPLETE SCRAM:**	**(41.5 / 45.6)**				
AK3.01	Bypassing rod insertion blocks		3.4	3.7	3.4	3.7
A1	**Ability to operate and/or monitor the following as they apply to INCOMPLETE SCRAM:**	**(41.7 / 45.6)**				
AA1.01	CRD hydraulics		3.8	3.9	3.8	3.9
AA1.02	RPS		4.0	4.2*	4.0	4.2
AA1.03	~~RMCS: Plant-Specific~~		3.6	3.8	3.6	3.8
AA1.04	Rod control and information system: Plant-Specific		3.4	3.7	3.4	3.7
AA1.05	~~Rod worth minimizer: Plant-Specific~~		2.5*	2.8*	2.5	2.8
AA1.06	~~RSCS: Plant-Specific~~		2.7	2.9	2.7	2.9
AA1.07	Neutron monitoring system		3.6	3.7	3.6	3.7
AA1.08	Process computer/SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.7	2.9	2.7	2.9
A2	**Ability to determine and/or interpret the following as they apply to INCOMPLETE SCRAM:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Reactor power		4.1*	4.3*	4.1	4.3
AA2.02	Control rod position		4.1	4.2*	4.1	4.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295016

System Name: Control Room Abandonment

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K2	**Knowledge of the interrelations between CONTROL ROOM ABANDONMENT and the following:**	**(41.7, 45.8)**				
AK2.01	Remote shutdown panel: Plant-Specific		4.4*	4.5*	4.4	4.5
AK2.02	Local control stations: Plant-Specific		4.0*	4.1*	4.0	4.1
AK2.03	Control room HVAC		2.9*	3.1*	2.9	3.1
K3	**Knowledge of the reasons for the following responses as they apply to CONTROL ROOM ABANDONMENT:**	**(41.5 / 45.6)**				
AK3.01	Reactor SCRAM		4.1*	4.2*	4.1	4.2
AK3.02	Turbine trip		3.7*	3.8*	3.7	3.8
AK3.03	Disabling control room controls		3.5	3.7*	3.5	3.7
A1	**Ability to operate and/or monitor the following as they apply to CONTROL ROOM ABANDONMENT:**	**(41.7 / 45.6)**				
AA1.01	RPS		3.8	3.9	3.8	3.9
AA1.02	~~Reactor/turbine pressure regulating system~~		2.9*	3.1*	2.9	3.1
AA1.03	RPIS		3.0*	3.1	3.0	3.1
AA1.04	A.C. electrical distribution		3.1	3.2	3.1	3.2
AA1.05	D.C. electrical distribution		2.8	2.9	2.8	2.9
AA1.06	Reactor water level		4.0	4.1	4.0	4.1
AA1.07	Control room/local control transfer mechanisms		4.2*	4.3*	4.2	4.3
AA1.08	Reactor pressure		4.0	4.0	4.0	4.0
AA1.09	~~Isolation/emergency condenser(s): Plant-Specific~~		4.0	4.0	4.0	4.0
A2	**Ability to determine and/or interpret the following as they apply to CONTROL ROOM ABANDONMENT:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Reactor power		4.1*	4.1*	4.1	4.1
AA2.02	Reactor water level		4.2*	4.3*	4.2	4.3
AA2.03	Reactor pressure		4.3*	4.4*	4.3	4.4
AA2.04	Suppression pool temperature		3.9	4.1	3.9	4.1
AA2.05	~~Drywell pressure~~		3.8	3.9	3.8	3.9
AA2.06	Cooldown rate		3.3	3.5	3.3	3.5
AA2.07	~~Suppression chamber pressure~~		3.2	3.4	3.2	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295017

System Name: High Off-Site Release Rate

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH OFF-SITE RELEASE RATE:**	**(41.8 to 41.10)**				
AK1.01	Biological effects of radioisotope ingestion		2.2*	2.5	2.2	2.5
AK1.02	†Protection of the general public		3.8*	4.3*	3.8	4.3
AK1.03	†Meteorological effects on off-site release		2.7	3.4	2.7	3.4
K2	**Knowledge of the interrelations between HIGH OFF-SITE RELEASE RATE and the following:**	**(41.7, 45.8)**				
AK2.01	Fission product production versus reactor power		2.8	3.3	2.8	3.3
AK2.02	Radwaste		2.8	3.1	2.8	3.1
AK2.03	Off-gas system		3.3	3.5	3.3	3.5
AK2.04	Plant ventilation systems		3.1	3.3	3.1	3.3
AK2.05	Stack-gas monitoring system: Plant-Specific		3.4	3.6	3.4	3.6
AK2.06	†Site emergency plan		3.4	4.6*	3.4	4.6
AK2.07	Control room ventilation		3.2	3.4	3.2	3.4
AK2.08	SPDS/ERIS/CRIDS/GDS		2.8	3.3	2.8	3.3
AK2.09	Condenser air removal system: Plant-Specific		2.8	2.9	2.8	2.9
AK2.10	Process radiation monitoring system		3.3	3.6	3.3	3.6
AK2.11	MSIV leakage control: Plant-Specific		3.1	3.2	3.1	3.2
AK2.12	Standby gas treatment/FRVS		3.4	3.7	3.4	3.7
AK2.13	RPS		3.4	3.7	3.4	3.7
AK2.14	PCIS/NSSSS		4.0	4.1	4.0	4.1
K3	**Knowledge of the reasons for the following responses as they apply to HIGH OFF-SITE RELEASE RATE:**	**(41.5 / 45.6)**				
AK3.01	System isolations		3.6	3.9	3.6	3.9
AK3.02	Plant ventilation		3.3	3.5	3.3	3.5
AK3.03	†Implementation of site emergency plan		3.3	4.5*	3.3	4.5
AK3.04	Power reduction		3.6	3.8	3.6	3.8
AK3.05	Control room ventilation: Plant-Specific		3.3	3.6	3.3	3.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295017

System Name: High Off-Site Release Rate

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S.
A1	**Ability to operate and/or monitor the following as they apply to HIGH OFF-SITE RELEASE RATE:**	**(41.7 / 45.6)**				
AA1.01	Radwaste		2.7*	3.1	2.7	3.1
AA1.02	Off-gas system		3.5	3.7	3.5	3.7
AA1.03	Plant ventilation systems		3.4	3.4	3.4	3.4
AA1.04	Stack-gas monitoring system: Plant-Specific		3.6	3.8	3.6	3.8
AA1.05	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.7	3.2	2.7	3.2
AA1.06	Condenser air removal system: Plant-Specific		3.2	3.2	3.2	3.2
AA1.07	Process radiation monitoring system		3.4	3.6	3.4	3.6
AA1.08	MSIV leakage control: Plant-Specific		3.1	3.4	3.1	3.4
AA1.09	Standby gas treatment/FRVS		3.6	3.8	3.6	3.8
AA1.10	RPS		3.6	3.7	3.6	3.7
AA1.11	PCIS/NSSSS		3.9	4.1	3.9	4.1
AA1.12	†Meteorological data		2.5*	3.9	2.5	3.9
A2	**Ability to determine and/or interpret the following as they apply to HIGH OFF-SITE RELEASE RATE:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	†Off-site release rate: Plant-Specific		2.9*	4.2*	2.9	4.2
AA2.02	†Total number of curies released: Plant-Specific		2.4*	3.5*	2.4	3.5
AA2.03	†Radiation levels: Plant-Specific		3.1	3.9	3.1	3.9
AA2.04	†Source of off-site release		3.6	4.3*	3.6	4.3
AA2.05	†Meteorological data		2.5	3.8	2.5	

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295018

System Name: Partial or Complete Loss of Component Cooling Water

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER:**	**(41.8 to 41.10)**				
AK1.01	Effects on component/system operations		3.5	3.6	3.5	3.6
K2	**Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER and the following:**	**(41.7, 45.8)**				
AK2.01	System loads		3.3	3.4	3.3	3.4
AK2.02	Plant operations		3.4	3.6	3.4	3.6
K3	**Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER:**	**(41.5 / 45.6)**				
AK3.01	Isolation of non-essential heat loads: Plant-Specific		2.9	3.2	2.9	3.2
AK3.02	Reactor power reduction		3.3	3.4	3.3	3.4
AK3.03	Securing individual components (prevent equipment damage)		3.1	3.3	3.1	3.3
AK3.04	Starting standby pump		3.3	3.3	3.3	3.3
AK3.05	Placing standby heat exchanger in service		3.2	3.3	3.2	3.3
K3.06	Increasing cooling water flow to heat exchangers		3.3	3.3	3.3	3.3
AK3.07	Cross-connecting with backup systems		3.1	3.2	3.1	3.2
A1	**Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER:**	**(41.7 / 45.6)**				
AA1.01	Backup systems		3.3	3.4	3.3	3.4
AA1.02	System loads		3.3	3.4	3.3	3.4
AA1.03	Affected systems so as to isolate damaged portions		3.3	3.4	3.3	3.4
A2	**Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Component temperatures		3.3	3.4	3.3	3.4
AA2.02	Cooling water temperature		3.1	3.2	3.1	3.2
AA2.03	Cause for partial or complete loss		3.2	3.5	3.2	3.5
AA2.04	System flow		2.9	2.9	2.9	2.9
AA2.05	System pressure		2.9	2.9	2.9	2.9

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295019

System Name: Partial or Complete Loss of Instrument Air

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K2	**Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR and the following:**	**(41.7, 45.8)**				
AK2.01	CRD hydraulics		3.8	3.9	3.8	3.9
AK2.02	Component cooling water		2.9	3.0	2.9	3.0
AK2.03	Reactor feedwater		3.2	3.3	3.2	3.3
AK2.04	Reactor water cleanup		2.8	2.8	2.8	2.8
AK2.05	Main steam system		3.4	3.4	3.4	3.4
AK2.06	Offgas system		2.8	2.9	2.8	2.9
AK2.07	Condensate system		3.2	3.2	3.2	3.2
AK2.08	Plant ventilation		2.8	2.9	2.8	2.9
AK2.09	Containment		3.3	3.3	3.3	3.3
AK2.10	Fuel pool cooling		2.8	2.8	2.8	2.8
AK2.11	Radwaste		2.5	2.6	2.5	2.6
AK2.12	Standby gas treatment/FRVS		3.3	3.4	3.3	3.4
AK2.13	~~Isolation condenser: Plant-Specific~~		3.2*	3.2*	3.2	3.2
AK2.14	Plant air systems		3.2	3.2	3.2	3.2
AK2.15	Standby liquid control system		2.3*	2.6	2.3	2.6
AK2.16	Reactor core isolation cooling		2.8	2.8	2.8	2.8
AK2.17	~~High pressure coolant injection: Plant-Specific~~		2.7	2.7	2.7	2.7
AK2.18	ADS: Plant-Specific		3.5	3.5	3.5	3.5
AK2.19	RHR/LPCI: Plant-Specific		2.7	2.8	2.7	2.8
K3	**Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR:**	**(41.5 / 45.6)**				
AK3.01	Backup air system supply: Plant-Specific		3.3	3.4	3.3	3.4
AK3.02	Standby air compressor operation		3.5	3.4	3.5	3.4
AK3.03	Service air isolations: Plant-Specific		3.2	3.2	3.2	3.2
A1	**Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR:**	**(41.7 / 45.6)**				
AA1.01	Backup air supply		3.5	3.3	3.5	3.3
AA1.02	Instrument air system valves: Plant-Specific		3.3	3.1	3.3	3.1
AA1.03	Instrument air compressor power supplies		3.0	3.0	3.0	3.0
AA1.04	Service air isolations valves: Plant-Specific		3.3	3.2	3.3	3.2
A2	**Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Instrument air system pressure		3.5	3.6	3.5	3.6
AA2.02	Status of safety-related instrument air system loads (see AK2.1-AK2.19)		3.6	3.7	3.6	3.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295020

System Name: Inadvertent Containment Isolation

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to INADVERTENT CONTAINMENT ISOLATION:**	**(41.8 to 41.10)**				
AK1.01	Loss of normal heat sink		3.7	3.9	3.7	3.9
AK1.02	Power/reactivity control		3.5	3.8	3.5	3.8
AK1.03	Water chemistry		2.3	2.5	2.3	2.5
AK1.04	Bottom head thermal stratification		2.5	2.8	2.5	2.8
AK1.05	Loss of drywell/containment cooling		3.3	3.6	3.3	3.6
K2	**Knowledge of the interrelations between INADVERTENT CONTAINMENT ISOLATION and the following:**	**(41.7, 45.8)**				
AK2.01	Main steam system		3.6	37	3.6	37
AK2.02	Sampling system		2.6	2.8	2.6	2.8
AK2.03	Drywell/containment ventilation/cooling: Plant-Specific		3.1	3.3	3.1	3.3
AK2.04	RWCU system		3.1	3.1	3.1	3.1
AK2.05	~~Isolation condenser: Plant-Specific~~		4.2*	4.2	4.2	4.2
AK2.06	~~HPCI: Plant-Specific~~		3.8	3.8	3.8	3.8
AK2.07	RCIC: Plant-Specific		3.4	3.4	3.4	3.4
AK2.08	~~Traversing in-core probes: Plant-Specific~~		2.5*	2.6	2.5	2.6
AK2.09	RHR/shutdown cooling: Plant-Specific		3.1	3.3	3.1	3.3
AK2.10	Drywell equipment/floor drain sumps		2.9	3.1	2.9	3.1
AK2.11	Standby gas treatment system/FRVS: Plant-Specific		3.2	3.4	3.2	3.4
AK2.12	Instrument air/nitrogen: Plant-Specific		3.1	3.2	3.1	3.2
K3	**Knowledge of the reasons for the following responses as they apply to INADVERTENT CONTAINMENT ISOLATION:**	**(41.5 / 45.6)**				
AK3.01	Reactor SCRAM		3.8	3.8	3.8	3.8
AK3.02	Drywell/containment pressure response		3.3	3.5	3.3	3.5
AK3.03	Drywell/containment temperature response		3.2	3.2	3.2	3.2
AK3.04	Reactor pressure response		4.1	4.1	4.1	4.1
AK3.05	Reactor water level response		3.8	3.9	3.8	3.9
AK3.06	Suppression pool water level response		3.3	3.4	3.3	3.4
AK3.07	Suppression pool temperature response		3.4	3.6	3.4	3.6
AK3.08	~~Suppression chamber pressure response~~		3.3	3.5	3.3	3.5
A1	**Ability to operate and/or monitor the following as they apply to INADVERTENT CONTAINMENT ISOLATION:**	**(41.7 / 45.6)**				
AA1.01	PCIS/NSSSS		3.6	3.6	3.6	3.6
AA1.02	Drywell ventilation/cooling system		3.2	3.2	3.2	3.2
AA1.03	Containment ventilation system: Plant-Specific		2.9	3.1	2.9	3.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295020

System Name: Inadvertent Containment Isolation

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp
A2	**Ability to determine and/or interpret the following as they apply to INADVERTENT CONTAINMENT ISOLATION:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Drywell/containment pressure		3.6	3.7	3.6	3.7
AA2.02	Drywell/containment temperature		3.3	3.4	3.3	3.4
AA2.03	Reactor power		3.7	3.7	3.7	3.7
AA2.04	Reactor pressure		3.9	3.9	3.9	3.9
AA2.05	Reactor water level		3.6	3.6	3.6	3.6
AA2.06	Cause of isolation		3.4	3.8	3.4	3.8

AA2

Facility: CPS Printed: 08/11/2006

System Number: 295021

System Name: Loss of Shutdown Cooling

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to LOSS OF SHUTDOWN COOLING:**	**(41.8 to 41.10)**				
AK1.01	Decay heat		3.6	3.8	3.6	3.8
AK1.02	Thermal stratification		3.3	3.4	3.3	3.4
AK1.03	Adequate core cooling.		3.9	3.9	3.9	3.9
AK1.04	Natural circulation		3.6	3.7	3.6	3.7
K2	**Knowledge of the interrelations between LOSS OF SHUTDOWN COOLING and the following:**	**(41.7, 45.8)**				
AK2.01	Reactor water temperature		3.6	3.7	3.6	3.7
AK2.02	Reactor water cleanup		3.2	3.3	3.2	3.3
AK2.03	RHR/shutdown cooling		3.6	3.6	3.6	3.6
AK2.04	Component cooling water systems: Plant-Specific		3.0	3.1	3.0	3.1
AK2.05	Fuel pool cooling and cleanup system		2.7	2.8	2.7	2.8
AK2.06	Reactor vessel headspray: Plant-Specific		2.5	2.6	2.5	2.6
AK2.07	Reactor recirculation		3.1	3.2	3.1	3.2
K3	**Knowledge of the reasons for the following responses as they apply to LOSS OF SHUTDOWN COOLING:**	**(41.5 / 45.6)**				
AK3.01	Raising reactor water level		3.3	3.4	3.3	3.4
AK3.02	Feeding and bleeding reactor vessel		3.3	3.4	3.3	3.4
AK3.03	Increasing drywell cooling		2.9	2.9	2.9	2.9
AK3.04	Maximizing reactor water cleanup flow		3.3	3.4	3.3	3.4
AK3.05	Establishing alternate heat removal flow paths		3.6	3.8	3.6	3.8
A1	**Ability to operate and/or monitor the following as they apply to LOSS OF SHUTDOWN COOLING:**	**(41.7 / 45.6)**				
AA1.01	Reactor water cleanup system		3.4	3.4	3.4	3.4
AA1.02	RHR/shutdown cooling		3.5	3.5	3.5	3.5
AA1.03	Component cooling water systems: Plant-Specific		3.1	3.1	3.1	3.1
AA1.04	Alternate heat removal methods		3.7	3.7	3.7	3.7
AA1.05	Reactor recirculation		3.0	3.0	3.0	3.0
AA1.06	Containment/ drywell temperature		2.8	3.0	2.8	3.0
A2	**Ability to determine and/or interpret the following as they apply to LOSS OF SHUTDOWN COOLING:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Reactor water heatup/cooldown rate		3.5	3.6	3.5	3.
AA2.02	RHR/shutdown cooling system flow		3.4	3.4	3.4	3
AA2.03	Reactor water level		3.5	3.6	3.5	3
AA2.04	Reactor water temperature		3.6	3.5	3.6	3
AA2.05	Reactor vessel metal temperature		3.4	3.3	3.4	3
AA2.06	Reactor pressure		3.2	3.3	3.2	3
AA2.07	Reactor recirculation flow		2.9	3.1	2.9	3

Facility: CPS

Printed: 08/11/2006

System Number: 295021

System Name: Loss of Shutdown Cooling

	NRC Imp		Facility Iw	
CFR	RO	SRO	RO	S.

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295022

System Name: Loss of CRD Pumps

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to LOSS OF CRD PUMPS:**	**(41.8 to 41.10)**				
AK1.01	Reactor pressure vs. rod insertion capability		3.3	3.4	3.3	3.4
AK1.02	Reactivity control		3.6	3.7	3.6	3.7
K2	**Knowledge of the interrelations between LOSS OF CRD PUMPS and the following:**	**(41.7, 45.8)**				
AK2.01	Recirculation system: Plant-Specific		2.8	3.0	2.8	3.0
AK2.02	CRD mechanism		3.1	3.1	3.1	3.1
AK2.03	Accumulator pressures.		3.4	3.4	3.4	3.4
AK2.04	Reactor water level		2.5	2.7	2.5	2.7
AK2.05	Reactor water cleanup: Plant-Specific		2.4	2.5	2.4	2.5
AK2.06	~~Shared components with other units: Plant-Specific~~		2.6	2.8	2.6	2.8
AK2.07	Reactor pressure (SCRAM assist): Plant-Specific		3.4	3.6	3.4	3.6
K3	**Knowledge of the reasons for the following responses as they apply to LOSS OF CRD PUMPS:**	**(41.5 / 45.6)**				
AK3.01	Reactor SCRAM		3.7*	3.9*	3.7	3.9
AK3.02	CRDM high temperature		2.9	3.1	2.9	3.1
A1	**Ability to operate and/or monitor the following as they apply to LOSS OF CRD PUMPS:**	**(41.7 / 45.6)**				
AA1.01	CRD hydraulic system		3.1	3.2	3.1	3.2
AA1.02	RPS		3.6	3.6	3.6	3.6
AA1.03	Recirculation system: Plant-Specific		2.7	2.8	2.7	2.8
AA1.04	Reactor water cleanup system: Plant-Specific		2.5	2.6	2.5	2.6
A2	**Ability to determine and/or interpret the following as they apply to LOSS OF CRD PUMPS:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Accumulator pressure		3.5	3.6	3.5	3.6
AA2.02	CRD system status		3.3	3.4	3.3	3.4
AA2.03	CRD mechanism temperatures		3.1	3.2	3.1	3.2

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295023

System Name: Refueling Accidents

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to REFUELING ACCIDENTS:**	**(41.8 to 41.10)**				
AK1.01	Radiation exposure hazards		3.6	4.1	3.6	4.1
AK1.02	Shutdown margin		3.2	3.6	3.2	3.6
AK1.03	Inadvertent criticality		3.7	4.0	3.7	4.0
K2	**Knowledge of the interrelations between REFUELING ACCIDENTS and the following:**	**(41.7, 45.8)**				
AK2.01	Fuel handling equipment		3.3	3.7	3.3	3.7
AK2.02	Fuel pool cooling and cleanup system		2.9	3.2	2.9	3.2
AK2.03	Radiation monitoring equipment		3.4	3.6	3.4	3.6
AK2.04	RMCS/Rod control and information system		3.2	3.4	3.2	3.4
AK2.05	Secondary containment ventilation		3.5	3.7	3.5	3.7
AK2.06	Containment ventilation: Mark-III		3.4	3.8	3.4	3.8
AK2.07	Standby gas treatment/FRVS		3.6	3.9	3.6	3.9
K3	**Knowledge of the reasons for the following responses as they apply to REFUELING ACCIDENTS:**	**(41.5 / 45.6)**				
AK3.01	Refueling floor evacuation		3.6	4.3	3.6	4.3
AK3.02	Interlocks associated with fuel handling equipment		3.4	3.8*	3.4	3.8
AK3.03	Ventilation isolation		3.3	3.6	3.3	3.6
AK3.04	Non-coincident SCRAM function		3.0	3.5	3.0	3.5
AK3.05	~~Initiation of SLC/shut-down cooling: Plant-Specific(BWR-1)~~		3.5	4.0	3.5	4.0
A1	**Ability to operate and/or monitor the following as they apply to REFUELING ACCIDENTS:**	**(41.7 / 45.6)**				
AA1.01	Secondary containment ventilation		3.3	3.5	3.3	3.5
AA1.02	Fuel pool cooling and cleanup system		2.9	3.1	2.9	3.1
AA1.03	Fuel handling equipment		3.3	3.6	3.3	3.6
AA1.04	Radiation monitoring equipment		3.4	3.7	3.4	3.7
AA1.05	Fuel transfer system: Plant-Specific		2.8	3.5	2.8	3.5
AA1.06	Neutron monitoring		3.3	3.4	3.3	3.4
AA1.07	Standby gas treatment/FRVS		3.6	3.6	3.6	3.6
AA1.08	†Containment building ventilation: Mark-III		3.3	3.4	3.3	3.4
A2	**Ability to determine and/or interpret the following as they apply to REFUELING ACCIDENTS:**	**(41.10 / 43.5 / 45.13)**				
AA2.01	Area radiation levels		3.6	4.0	3.6	4.0
AA2.02	Fuel pool level		3.4	3.7	3.4	3.7
AA2.03	Airborne contamination levels		3.3*	3.8*	3.3	3.8
AA2.04	†Occurrence of fuel handling accident		3.4	4.1	3.4	4.1
AA2.05	†Entry conditions of emergency plan		3.2	4.6*	3.2	4.6

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295024

System Name: High Drywell Pressure

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL PRESSURE:**	**(41.8 to 41.10)**				
EK1.01	Drywell integrity: Plant-Specific		4.1	4.2*	4.1	4.2
EK1.02	Containment building integrity: Mark-III		3.9	4.1	3.9	4.1
K2	**Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following:**	**(41.7, 45.8)**				
EK2.01	~~HPCI (FWCI): Plant-Specific~~		3.9	4.0	3.9	4.0
EK2.02	HPCS: Plant-Specific		3.7	3.7	3.7	3.7
EK2.03	LPCS: Plant-Specific		3.8	3.8	3.8	3.8
EK2.04	RHR/LPCI		3.9	3.9	3.9	3.9
EK2.05	RPS		3.9	4.0	3.9	4.0
EK2.06	Emergency generators		3.9	4.0	3.9	4.0
EK2.07	PCIS/NSSSS		3.9	3.9	3.9	3.9
EK2.08	ADS: Plant-Specific		4.0	4.1	4.0	4.1
EK2.09	Suppression pool makeup: Plant-Specific		2.9	3.1	2.9	3.1
EK2.10	A.C. distribution		3.5	3.5	3.5	3.5
EK2.11	~~Drywell spray (RHR) logic: Mark-I&II~~		4.2	4.2*	4.2	4.2
EK2.12	Suppression pool cooling		3.5	3.5	3.5	3.5
EK2.13	~~Suppression pool spray: Plant-Specific~~		3.8	3.8	3.8	3.8
EK2.14	Containment pressure: Mark-III		3.9	3.9	3.9	3.9
EK2.15	Containment spray logic: Plant-Specific		3.8	3.9	3.8	3.9
EK2.16	SPDS/ERIS/CRIDS: Plant-Specific		3.2	3.2	3.2	3.2
EK2.17	~~Auxiliary building isolation logic: Plant-Specific~~		3.0	3.3	3.0	3.3
EK2.18	Ventilation		3.3	3.4	3.3	3.4
EK2.19	Feedwater and condensate: Plant-Specific		2.9	2.9	2.9	2.9
EK2.20	D.C. distribution: Plant-Specific		2.8	2.9	2.8	2.9
K3	**Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL PRESSURE:**	**(41.5 / 45.6)**				
EK3.01	~~Drywell spray operation: Mark-I&II~~		3.6	4.0	3.6	4.0
EK3.02	~~Suppression pool spray operation: Plant-Specific~~		3.5	3.8	3.5	3.8
EK3.03	Containment venting: Mark-III		3.6	4.1	3.6	4.1
EK3.04	†Emergency depressurization		3.7	4.1	3.7	4.1
EK3.05	†RPV flooding		3.5	3.8	3.5	3.8
EK3.06	Reactor SCRAM		4.0*	4.1	4.0	4.1
EK3.07	Drywell venting		3.5	4.0	3.5	4.0
EK3.08	Containment spray: Plant Specific		3.7	4.1	3.7	4.1
EK3.09	~~Auxiliary building isolation: Plant-Specific.~~		3.1	3.6	3.1	3.6

Facility: CPS Printed: 08/11/2006

System Number: 295024

System Name: High Drywell Pressure

		CFR	NRC Imp RO	NRC Imp SRO	Facility Im RO	Facility Im S.
A1	**Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE:**	**(41.7 / 45.6)**				
EA1.01	~~HPCI (FWCI): Plant-Specific~~		4.1*	4.0	4.1	4.0
EA1.02	HPCS: Plant-Specific		3.8	3.7	3.8	3.7
EA1.03	LPCS: Plant-Specific		4.0	3.9	4.0	3.9
EA1.04	RHR/LPCI		4.1	3.9	4.1	3.9
EA1.05	RPS		3.9	4.0	3.9	4.0
EA1.06	Emergency generators		3.7	3.7	3.7	3.7
EA1.07	PCIS/NSSSS		3.8	3.9	3.8	3.9
EA1.08	ADS: Plant-Specific		3.9	3.9	3.9	3.9
EA1.09	Suppression pool makeup: Plant-Specific		2.9	3.0	2.9	3.0
EA1.10	A.C. distribution		3.4	3.6	3.4	3.6
EA1.11	~~Drywell spray: Mark-I&II~~		4.2*	4.2*	4.2	4.2
EA1.12	~~Suppression pool spray: Mark-I&II~~		3.8	3.8	3.8	3.8
EA1.13	Suppression pool cooling		3.6	3.6	3.6	3.6
EA1.14	Drywell ventilation system		3.4	3.5	3.4	3.5
EA1.15	Containment/drywell atmospheric monitoring		3.6	3.7	3.6	3.7
EA1.16	Containment/drywell vacuum breakers		3.4	3.4	3.4	3.4
EA1.17	Containment spray: Plant-Specific		3.9	3.9	3.9	3.9
EA1.18	Containment ventilation system: Mark-III		3.6	3.6	3.6	3.6
EA1.19	Containment atmosphere control: Plant-Specific		3.3	3.4	3.3	3.4
EA1.20	Standby gas treatment/FRVS: Plant-Specific		3.5	3.6	3.5	3.6
EA1.21	~~Recirculation system (LPCI loop select logic): Plant-Specific~~		3.4	3.8	3.4	3.8
EA1.22	D.C. distribution: Plant-Specific		2.7	2.9	2.7	2.9
A2	**Ability to determine and/or interpret the following as they apply to HIGH DRYWELL PRESSURE:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Drywell pressure		4.2*	4.4*	4.2	4.4
EA2.02	Drywell temperature		3.9	4.0	3.9	4.0
EA2.03	Suppression pool level		3.8	3.8	3.8	3.8
EA2.04	~~Suppression chamber pressure: Plant-Specific~~		3.9	3.9	3.9	3.9
EA2.05	~~Suppression chamber air-space temperature: Plant-Specific~~		3.6	3.1	3.6	3.1
EA2.06	Suppression pool temperature		4.1	4.1	4.1	4.1
EA2.07	Containment radiation levels: Mark-III		3.4	3.9	3.4	3.9
EA2.08	Drywell radiation levels		3.6	4.0	3.6	4.0
EA2.09	Containment pressure: Mark-III		4.0*	4.1*	4.0	4.1
EA2.10	Containment temperature: Mark-III		3.7	3.9	3.7	3.9

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295025

System Name: High Reactor Pressure

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH REACTOR PRESSURE:**	**(41.8 to 41.10)**				
EK1.01	Pressure effects on reactor power		3.9	4.0	3.9	4.0
EK1.02	Reactor vessel integrity		4.1	4.2	4.1	4.2
EK1.03	Safety/relief valve tailpipe temperature/pressure relationships		3.6	3.8	3.6	3.8
EK1.04	Decay heat generation		3.6	3.9	3.6	3.9
EK1.05	†Exceeding safety limits		4.4*	4.7*	4.4	4.7
EK1.06	Pressure effects on reactor water level		3.5	3.6	3.5	3.6
K2	**Knowledge of the interrelations between HIGH REACTOR PRESSURE and the following:**	**(41.7, 45.8)**				
EK2.01	RPS		4.1*	4.1	4.1	4.1
EK2.02	~~Isolation condenser: Plant-Specific~~		4.2*	4.5*	4.2	4.5
EK2.03	~~RRCS: Plant-Specific~~		4.0	4.3	4.0	4.3
EK2.04	ARI/RPT/ATWS: Plant-Specific		3.9	4.1*	3.9	4.1
EK2.05	Safety/relief valves: Plant-Specific		4.1*	4.2	4.1	4.2
EK2.06	~~HPCI: Plant-Specific~~		3.8	3.8	3.8	3.8
EK2.07	RCIC: Plant-Specific		3.7	3.7	3.7	3.7
EK2.08	Reactor/turbine pressure regulating system: Plant-Specific		3.7	3.7	3.7	3.7
EK2.09	Reactor power		3.9	3.9	3.9	3.9
EK2.10	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.9	3.2	2.9	3.2
EK2.11	Reactor water level		3.5	3.6	3.5	3.6
K3	**Knowledge of the reasons for the following responses as they apply to *HIGH REACTOR PRESSURE:***	**(41.5 / 45.6)**				
EK3.01	Safety/relief valve opening		4.2	4.3*	4.2	4.3
EK3.02	Recirculation pump trip: Plant-Specific		3.9	4.1	3.9	4.1
EK3.03	~~HPCI operation: Plant-Specific~~		3.8	3.8	3.8	3.8
EK3.04	~~Isolation condenser initiation: Plant-Specific~~		4.5*	4.7*	4.5	4.7
EK3.05	RCIC operation: Plant-Specific		3.6	3.7	3.6	3.7
EK3.06	Alternate rod insertion: Plant-Specific		4.2*	4.4*	4.2	4.4
EK3.07	~~†RRCs initiation: Plant-Specific~~		3.3	3.7*	3.3	3.7
EK3.08	Reactor/turbine pressure regulating system operation		3.5	3.5	3.5	3.5
EK3.09	Low-low set initiation: Plant-Specific		3.7	3.7	3.7	3.7

Facility: CPS **Printed:** 08/11/2006

System Number: 295025

System Name: High Reactor Pressure

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir
A1	**Ability to operate and/or monitor the following as they apply to HIGH REACTOR PRESSURE:**	**(41.7 / 45.6)**				
EA1.01	Main steam line drains		2.9	3.0	2.9	3.0
EA1.02	Reactor/turbine pressure regulating system		3.8	3.8	3.8	3.8
EA1.03	Safety/relief valves: Plant-Specific		4.4*	4.4*	4.4	4.4
EA1.04	~~HPCI: Plant-Specific~~		3.8	3.9	3.8	3.9
EA1.05	RCIC: Plant-Specific		3.7	3.7	3.7	3.7
EA1.06	~~Isolation condenser: Plant-Specific~~		4.5*	4.5*	4.5	4.5
EA1.07	ARI/RPT/ATWS: Plant-Specific		4.1	4.1	4.1	4.1
EA1.08	~~†RRCS: Plant-Specific~~		3.3	3.7*	3.3	3.7
A2	**Ability to determine and/or interpret the following as they apply to HIGH REACTOR PRESSURE:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Reactor pressure		4.3*	4.3*	4.3	4.3
EA2.02	Reactor power		4.2*	4.2	4.2	4.2
EA2.03	Suppression pool temperature		3.9	4.1	3.9	4.1
EA2.04	Suppression pool level		3.9	3.9	3.9	3.9
EA2.05	Decay heat generation		3.4	3.6	3.4	3.6
EA2.06	Reactor water level		3.7	3.8	3.7	3.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295026

System Name: Suppression Pool High Water Temperature

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE:**	**(41.8 to 41.10)**				
EK1.01	Pump NPSH		3.0	3.4	3.0	3.4
EK1.02	Steam condensation		3.5	3.8	3.5	3.8
K2	**Knowledge of the interrelations between SUPPRESSION POOL HIGH WATER TEMPERATURE and the following:**	**(41.7, 45.8)**				
EK2.01	Suppression pool cooling		3.9	4.0	3.9	4.0
EK2.02	~~Suppression pool spray: Plant-Specific~~		3.6	3.8	3.6	3.8
EK2.03	~~Suppression chamber pressure: Mark-I&II~~		3.2	3.6	3.2	3.6
EK2.04	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.5	2.8	2.5	2.8
EK2.05	Containment pressure: Mark-III		3.0	3.3	3.0	3.3
EK2.06	Suppression pool level		3.5	3.7	3.5	3.7
K3	**Knowledge of the reasons for the following responses as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE:**	**(41.5 / 45.6)**				
EK3.01	Emergency/normal depressurization		3.8	4.1	3.8	4.1
EK3.02	Suppression pool cooling		3.9	4.0	3.9	4.0
EK3.03	~~Suppression pool spray: Plant-Specific~~		3.5	3.8	3.5	3.8
EK3.04	†SBLC injection		3.7	4.1*	3.7	4.1
EK3.05	Reactor SCRAM		3.9	4.1	3.9	4.1
A1	**Ability to operate and/or monitor the following as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE:**	**(41.7 / 45.6)**				
EA1.01	Suppression pool cooling		4.1	4.1	4.1	4.1
EA1.02	~~Suppression pool spray: Plant-Specific~~		3.6	3.8	3.6	3.8
EA1.03	Temperature monitoring		3.9*	3.9	3.9	3.9
A2	**Ability to determine and/or interpret the following as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Suppression pool water temperature		4.1*	4.2*	4.1	4.2
EA2.02	Suppression pool level		3.8	3.9	3.8	3.9
EA2.03	Reactor pressure		3.9	4.0	3.9	4.0

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295027

System Name: High Containment Temperature (Mark III Containment Only)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):**	**(41.8 to 41.10)**				
EK1.01	Equipment environmental qualifications: Mark-III		2.5	2.7	2.5	2.7
EK1.02	Reactor water level measurement: Mark-III		3.0	3.2	3.0	3.2
EK1.03	Containment integrity: Mark-III		3.8	3.8	3.8	3.8
K2	**Knowledge of the interrelations between HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY) and the following:**	**(41.7, 45.8)**				
EK2.01	Containment spray: Plant-Specific		3.2	3.4	3.2	3.4
EK2.02	Components internal to the containment: Mark-III		3.2	3.3	3.2	3.3
EK2.03	Containment ventilation/cooling: Mark-III		3.5	3.7	3.5	3.7
EK2.04	SPDS/ERIS/CRIDS/GDS: Mark-III		2.6	3.2	2.6	3.2
K3	**Knowledge of the reasons for the following responses as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):**	**(41.5 / 45.6)**				
EK3.01	Emergency depressurization: Mark-III		3.7	3.8	3.7	3.8
EK3.02	Containment spray: Plant-Specific		3.2	3.2	3.2	3.2
EK3.03	Reactor SCRAM: Mark-III		3.7	3.7	3.7	3.7
A1	**Ability to operate and/or monitor the following as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):**	**(41.7 / 45.6)**				
EA1.01	Containment spray: Plant-Specific		3.2	3.4	3.2	3.4
EA1.02	Containment ventilation/cooling: Mark-III		3.5	3.5	3.5	3.5
EA1.03	Emergency depressurization: Mark-III		3.5	3.8	3.5	3.8
A2	**Ability to determine and/or interpret the following as they apply to HIGH CONTAINMENT TEMPERATURE (MARK III CONTAINMENT ONLY):**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Containment temperature: Mark-III		3.7	3.7	3.7	3.7
EA2.02	Containment pressure: Mark-III		3.7	3.7	3.7	3.7
EA2.03	Reactor pressure: Mark-III		3.3	3.3	3.3	3.3
EA2.04	Containment radiation levels: Mark-III		3.3	3.7	3.3	3.7

Facility: CPS Printed: 08/11/2006

System Number: 295028

System Name: High Drywell Temperature

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH DRYWELL TEMPERATURE:**	**(41.8 to 41.10)**				
EK1.01	Reactor water level measurement		3.5	3.7	3.5	3.7
EK1.02	Equipment environmental qualification		2.9	3.1	2.9	3.1
K2	**Knowledge of the interrelations between HIGH DRYWELL TEMPERATURE and the following:**	**(41.7, 45.8)**				
EK2.01	~~†Drywell spray: Mark-I&II~~		3.7	4.1	3.7	4.1
EK2.02	Components internal to the drywell		3.2	3.3	3.2	3.3
EK2.03	Reactor water level indication		3.6	3.8	3.6	3.8
EK2.04	Drywell ventilation		3.6	3.6	3.6	3.6
EK2.05	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.3*	2.5	2.3	2.5
K3	**Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL TEMPERATURE:**	**(41.5 / 45.6)**				
EK3.01	Emergency depressurization		3.6	3.9	3.6	3.9
EK3.02	RPV flooding		3.5	3.8	3.5	3.8
EK3.03	~~†Drywell spray operation: Mark-I&II~~		3.6	3.9	3.6	3.9
EK3.04	Increased drywell cooling		3.6	3.8	3.6	3.8
EK3.05	Reactor SCRAM		3.6	3.7	3.6	3.7
EK3.06	ADS		3.4	3.7	3.4	3.7
A1	**Ability to operate and/or monitor the following as they apply to HIGH DRYWELL TEMPERATURE:**	**(41.7 / 45.6)**				
EA1.01	~~Drywell spray: Mark-I&II~~		3.8	3.9	3.8	3.9
EA1.02	Drywell ventilation system		3.9	3.8	3.9	3.8
EA1.03	Drywell cooling system		3.9	3.9	3.9	3.9
EA1.04	Drywell pressure		3.9	4.0	3.9	4.0
EA1.05	ADS		3.7	3.7	3.7	3.7
A2	**Ability to determine and/or interpret the following as they apply to HIGH DRYWELL TEMPERATURE:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Drywell temperature		4.0*	4.1*	4.0	4.1
EA2.02	Reactor pressure		3.8	3.9	3.8	3.9
EA2.03	Reactor water level		3.7	3.9	3.7	3.9
EA2.04	Drywell pressure		4.1	4.2	4.1	4.2
EA2.05	~~Torus/suppression chamber pressure: Plant-Specific~~		3.6	3.8	3.6	3.8
EA2.06	~~Torus/suppression chamber air space temperature: Plant-Specific~~		3.4	3.7	3.4	3.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295029

System Name: High Suppression Pool Water Level

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH SUPPRESSION POOL WATER LEVEL:**	**(41.8 to 41.10)**				
EK1.01	Containment integrity		3.4	3.7	3.4	3.7
K2	**Knowledge of the interrelations between HIGH SUPPRESSION POOL WATER LEVEL and the following:**	**(41.7, 45.8)**				
EK2.01	RHR/LPCI		3.0	3.3	3.0	3.3
EK2.02	~~HPCI: Plant-Specific~~		3.4	3.6	3.4	3.6
EK2.03	HPCS: Plant-Specific		3.3	3.5	3.3	3.5
EK2.04	Suppression pool cleanup system: Plant-Specific		2.4*	2.4*	2.4	2.4
EK2.05	Containment/drywell vacuum breakers		3.1	3.3	3.1	3.3
EK2.06	SRV's and discharge piping		3.4	3.5	3.4	3.5
EK2.07	Drywell/containment water level		3.1	3.2	3.1	3.2
EK2.08	Drywell/suppression chamber ventilation		2.6	2.9	2.6	2.9
EK2.09	RCIC: Plant-Specific		3.1	3.2	3.1	3.2
K3	**Knowledge of the reasons for the following responses as they apply to HIGH SUPPRESSION POOL WATER LEVEL:**	**(41.5 / 45.6)**				
EK3.01	Emergency depressurization		3.5	3.9*	3.5	3.9
EK3.02	Lowering suppression pool water level		3.6	4.0	3.6	4.0
EK3.03	Reactor SCRAM		3.4	3.5	3.4	3.5
A1	**Ability to operate and/or monitor the following as they apply to HIGH SUPPRESSION POOL WATER LEVEL:**	**(41.7 / 45.6)**				
EA1.01	~~HPCI: Plant-Specific~~		3.4*	3.5	3.4	3.5
EA1.02	HPCS: Plant-Specific		3.1	3.1	3.1	3.1
EA1.03	RHR/LPCI		2.9	3.0	2.9	3.0
EA1.04	RCIC: Plant-Specific		3.4	3.5	3.4	3.5
A2	**Ability to determine and/or interpret the following as they apply to HIGH SUPPRESSION POOL WATER LEVEL:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Suppression pool water level		3.5*	3.9*	3.5	3.9
EA2.02	Reactor pressure		3.5	3.6	3.5	3.6
EA2.03	Drywell/containment water level		3.4	3.5	3.4	3.5

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295030

System Name: Low Suppression Pool Water Level

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to LOW SUPPRESSION POOL WATER LEVEL:**	**(41.8 to 41.10)**				
EK1.01	Steam condensation		3.8*	4.1*	3.8	4.1
EK1.02	Pump NPSH		3.5	3.8	3.5	3.8
EK1.03	Heat capacity		3.8	4.1*	3.8	4.1
K2	**Knowledge of the interrelations between LOW SUPPRESSION POOL WATER LEVEL and the following:**	**(41.7, 45.8)**				
EK2.01	~~HPCI: Plant-Specific~~		3.8	3.9	3.8	3.9
EK2.02	RCIC: Plant-Specific		3.7	3.8	3.7	3.8
EK2.03	LPCS		3.8	3.9	3.8	3.9
EK2.04	RHR/LPCI		3.7	3.8	3.7	3.8
EK2.05	HPCS: Plant-Specific		3.8	3.9	3.8	3.9
EK2.06	Suppression pool make-up: Mark-III		3.9*	3.9*	3.9	3.9
EK2.07	Downcomer/ horizontal vent submergence		3.5	3.8	3.5	3.8
EK2.08	SRV discharge submergence		3.5	3.8	3.5	3.8
EK2.09	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.5	2.8	2.5	2.8
K3	**Knowledge of the reasons for the following responses as they apply to LOW SUPPRESSION POOL WATER LEVEL:**	**(41.5 / 45.6)**				
EK3.01	Emergency depressurization		3.8	4.1	3.8	4.1
EK3.02	~~HPCI operation: Plant-Specific~~		3.5	3.7	3.5	3.7
EK3.03	RCIC operation: Plant-Specific		3.6	3.7	3.6	3.7
EK3.04	HPCS operation: Plant-Specific		3.5	3.5	3.5	3.5
EK3.05	Suppression pool make-up operation: Mark-III		3.6	3.6	3.6	3.6
EK3.06	Reactor SCRAM		3.6	3.8	3.6	3.8
EK3.07	NPSH considerations for ECCS pumps		3.5	3.8	3.5	3.8
A1	**Ability to operate and/or monitor the following as they apply to LOW SUPPRESSION POOL WATER LEVEL:**	**(41.7 / 45.6)**				
EA1.01	ECCS systems (NPSH considerations): Plant-Specific		3.6	3.8	3.6	3.8
EA1.02	RCIC: Plant-Specific		3.4	3.5	3.4	3.5
EA1.03	HPCS: Plant-Specific		3.4	3.4	3.4	3.4
EA1.04	Suppression pool make-up system: Mark-III		4.0	4.0	4.0	4.0
EA1.05	~~HPCI~~		3.5	3.5	3.5	3.5
EA1.06	Condensate storage and transfer (make-up to the suppression pool): Plant-Specific		3.4	3.4	3.4	3.4

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295030

System Name: Low Suppression Pool Water Level

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A2	**Ability to determine and/or interpret the following as they apply to LOW SUPPRESSION POOL WATER LEVEL:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Suppression pool level		4.1*	4.2*	4.1	4.2
EA2.02	Suppression pool temperature		3.9	3.9	3.9	3.9
EA2.03	Reactor pressure		3.7	3.9	3.7	3.9
EA2.04	~~Drywell/ suppression chamber differential pressure: Mark I&II~~		3.5	3.7	3.5	3.7

Facility: CPS Printed: 08/11/2006

System Number: 295031

System Name: Reactor Low Water Level

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to REACTOR LOW WATER LEVEL:**	**(41.8 to 41.10)**				
EK1.01	Adequate core cooling.		4.6*	4.7*	4.6	4.7
EK1.02	Natural circulation: Plant-Specific		3.8	4.1	3.8	4.1
EK1.03	Water level effects on reactor power		3.7	4.1	3.7	4.1
K2	**Knowledge of the interrelations between REACTOR LOW WATER LEVEL and the following:**	**(41.7, 45.8)**				
EK2.01	Reactor water level indication		4.4*	4.4*	4.4	4.4
EK2.02	Reactor pressure		3.8	3.9	3.8	3.9
EK2.03	Low pressure core spray		4.2	4.3*	4.2	4.3
EK2.04	Reactor core isolation cooling: Plant-Specific		4.0	4.1	4.0	4.1
EK2.05	Low pressure coolant injection (RHR)		4.2	4.3	4.2	4.3
EK2.06	~~High pressure (feedwater) coolant injection (FWCI/HPCI): Plant-Specific~~		4.1	4.2	4.1	4.2
EK2.07	High pressure core spray: Plant-Specific		4.0	4.1	4.0	4.1
EK2.08	Automatic depressurization system		4.2*	4.3*	4.2	4.3
EK2.09	Recirculation system: Plant-Specific		3.3	3.4	3.3	3.4
EK2.10	Redundant reactivity control: Plant-Specific		4.0	4.0	4.0	4.0
EK2.11	Reactor Protection System		4.4*	4.4*	4.4	4.4
EK2.12	Primary containment isolation system/ Nuclear steam supply shutoff		4.5*	4.5*	4.5	4.5
EK2.13	ARI/RPT/ATWS: Plant-Specific		4.1	4.2	4.1	4.2
EK2.14	Emergency generators		3.9	4.0	3.9	4.0
EK2.15	A.C. distribution: Plant-Specific		3.2	3.2	3.2	3.2
EK2.16	Reactor water level control		4.1*	4.1	4.1	4.1
K3	**Knowledge of the reasons for the following responses as they apply to REACTOR LOW WATER LEVEL:**	**(41.5 / 45.6)**				
EK3.01	Automatic depressurization system actuation		3.9	4.2*	3.9	4.2
EK3.02	Core coverage		4.4*	4.7*	4.4	4.7
EK3.03	Spray cooling		4.1	4.4*	4.1	4.4
EK3.04	Steam cooling		4.0	4.3*	4.0	4.3
EK3.05	Emergency depressurization		4.2*	4.3*	4.2	4.3

Facility: CPS Printed: 08/11/2006

System Number: 295031

System Name: Reactor Low Water Level

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp S
A1	**Ability to operate and/or monitor the following as they apply to REACTOR LOW WATER LEVEL:**	**(41.7 / 45.6)**				
EA1.01	Low pressure coolant injection (RHR): Plant-Specific		4.4*	4.4*	4.4	4.4
EA1.02	~~High pressure (feedwater) coolant injection: Plant-Specific~~		4.5*	4.5*	4.5	4.5
EA1.03	Low pressure core spray		4.4*	4.4*	4.4	4.4
EA1.04	High pressure core spray: Plant-Specific		4.3*	4.2	4.3	4.2
EA1.05	Reactor core isolation system: Plant-Specific		4.3*	4.3*	4.3	4.3
EA1.06	Automatic depressurization system		4.4*	4.4*	4.4	4.4
EA1.07	Safety/relief valves		3.7*	3.7*	3.7	3.7
EA1.08	Alternate injection systems: Plant-specific		3.8	3.9	3.8	3.9
EA1.09	~~Isolation condenser: Plant-Specific~~		3.3*	3.5*	3.3	3.5
EA1.10	Control rod drive		3.6	3.7	3.6	3.7
EA1.11	Condensate		4.1	4.1	4.1	4.1
EA1.12	Feedwater		3.9	4.1*	3.9	4.1
EA1.13	Reactor water level control		4.3*	4.3*	4.3	4.3
A2	**Ability to determine and/or interpret the following as they apply to REACTOR LOW WATER LEVEL:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Reactor water level		4.6*	4.6*	4.6	4.6
EA2.02	Reactor power		4.0	4.2*	4.0	4.2
EA2.03	Reactor pressure		4.2*	4.2*	4.2	4.2
EA2.04	Adequate core cooling		4.6*	4.8*	4.6	4.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295032

System Name: High Secondary Containment Area Temperature

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH SECONDARY CONTAINMENT AREA TEMPERATURE:**	**(41.8 to 41.10)**				
EK1.01	Personnel protection		3.6	3.8	3.6	3.8
EK1.02	†Radiation releases		3.6	4.0	3.6	4.0
EK1.03	Secondary containment leakage detection: Plant-Specific		3.5	3.9	3.5	3.9
EK1.04	Impact of operating environment on components		3.1	3.6	3.1	3.6
K2	**Knowledge of the interrelations between HIGH SECONDARY CONTAINMENT AREA TEMPERATURE and the following:**	**(41.7, 45.8)**				
EK2.01	Area/room coolers		3.5	3.6	3.5	3.6
EK2.02	Secondary containment ventilation		3.6	3.7	3.6	3.7
EK2.03	Fire protection system		3.3	3.4	3.3	3.4
EK2.04	PCIS/NSSSS		3.6	3.8	3.6	3.8
EK2.05	Temperature sensitive instrumentation		3.2	3.4	3.2	3.4
EK2.06	Area temperature monitoring system		3.3	3.4	3.3	3.4
EK2.07	Leak detection system concept: Plant-Specific		3.6	3.8	3.6	3.8
EK2.08	Systems required for safe shut-down		3.8	3.9	3.8	3.9
K3	**Knowledge of the reasons for the following responses as they apply to HIGH SECONDARY CONTAINMENT AREA TEMPERATURE:**	**(41.5 / 45.6)**				
EK3.01	Emergency/normal depressurization		3.5	3.8	3.5	3.8
EK3.02	Reactor SCRAM		3.6	3.8	3.6	3.8
EK3.03	Isolating affected systems		3.8	3.9*	3.8	3.9
A1	**Ability to operate and/or monitor the following as they apply to HIGH SECONDARY CONTAINMENT AREA TEMPERATURE:**	**(41.7 / 45.6)**				
EA1.01	Area temperature monitoring system		3.6	3.7	3.6	3.7
EA1.02	Leak detection system concept: Plant-Specific		3.4	3.5	3.4	3.5
EA1.03	Secondary containment ventilation		3.7	3.7	3.7	3.7
EA1.04	Fire protection system		3.3	3.4	3.3	3.4
EA1.05	Affected systems so as to isolate damaged portions		3.7	3.9	3.7	3.9
A2	**Ability to determine and/or interpret the following as they apply to HIGH SECONDARY CONTAINMENT AREA TEMPERATURE:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Area temperature		3.8*	3.8	3.8	3.8
EA2.02	Equipment operability		3.3	3.5	3.3	3.5
EA2.03	Cause of high area temperature		3.8	4.0	3.8	4.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295033

System Name: High Secondary Containment Area Radiation Levels

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH SECONDARY CONTAINMENT AREA RADIATION LEVELS:**	**(41.8 to 41.10)**				
EK1.01	Component environmental qualifications		2.4	2.8*	2.4	2.8
EK1.02	Personnel protection		3.9	4.2*	3.9	4.2
EK1.03	†Radiation releases		3.9	4.2*	3.9	4.2
K2	***Knowledge of the interrelations* between HIGH SECONDARY CONTAINMENT AREA RADIATION LEVELS and the following:**	**(41.7, 45.8)**				
EK2.01	Area radiation monitoring system		3.8	4.0	3.8	4.0
EK2.02	Process radiation monitoring system		3.8	4.1	3.8	4.1
EK2.03	Secondary containment ventilation: Plant-Specific		3.7	3.9	3.7	3.9
EK2.04	Standby gas treatment system/FRVS		3.9	4.2	3.9	4.2
K3	***Knowledge of the reasons for* the following responses as they apply to HIGH SECONDARY CONTAINMENT AREA RADIATION LEVELS:**	**(41.5 / 45.6)**				
EK3.01	Emergency depressurization		3.3	3.5	3.3	3.5
EK3.02	Reactor SCRAM		3.5	3.6	3.5	3.6
EK3.03	Isolating affected systems		3.8	3.9	3.8	3.9
EK3.04	Personnel evacuation		4.0	4.4*	4.0	4.4
EK3.05	†Emergency plan		3.6	4.5*	3.6	4.5
A1	**Ability to operate and/or monitor the following as they apply to HIGH SECONDARY CONTAINMENT AREA RADIATION LEVELS:**	**(41.7 / 45.6)**				
EA1.01	Area radiation monitoring system		3.9	4.0	3.9	4.0
EA1.02	Process radiation monitoring system		3.7	3.8	3.7	3.8
EA1.03	Secondary containment ventilation		3.8	3.8	3.8	3.8
EA1.04	SBGT/FRVS		4.2*	4.2	4.2	4.2
EA1.05	Affected systems so as to isolate damaged portions		3.9	4.0	3.9	4.0
EA1.06	Portable radiation monitoring instruments		2.9*	3.1*	2.9	3.1
EA1.07	Personnel dosimetry		3.5	3.6	3.5	3.6
EA1.08	Control room ventilation: Plant-Specific		3.6	3.8	3.6	3.8
A2	**Ability to determine and/or interpret the following as they apply to HIGH SECONDARY CONTAINMENT AREA RADIATION LEVELS:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Area radiation levels		3.8	3.9	3.8	3.9
EA2.02	Equipment operability		3.1	3.2	3.1	3.2
EA2.03	†Cause of high area radiation		3.7	4.2	3.7	4.2

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295034

System Name: Secondary Containment Ventilation High Radiation

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to SECONDARY CONTAINMENT VENTILATION HIGH RADIATION:**	**(41.8 to 41.10)**				
EK1.01	Personnel protection		3.8	4.1	3.8	4.1
EK1.02	†Radiation releases		4.1	4.4*	4.1	4.4
K2	**Knowledge of the interrelations between SECONDARY CONTAINMENT VENTILATION HIGH RADIATION and the following:**	**(41.7, 45.8)**				
EK2.01	Process radiation monitoring system		3.9	4.2	3.9	4.2
EK2.02	Area radiation monitoring system		3.8	3.9	3.8	3.9
EK2.03	SBGT/FRVS: Plant-Specific		4.3*	4.5*	4.3	4.5
EK2.04	Secondary containment ventilation		3.9	3.9	3.9	3.9
EK2.05	Fuel building ventilation: Mark-III		3.5	3.7	3.5	3.7
EK2.06	PCIS/NSSSS: Plant-Specific		3.9	4.2	3.9	4.2
K3	**Knowledge of the reasons for the following responses as they apply to SECONDARY CONTAINMENT VENTILATION HIGH RADIATION:**	**(41.5 / 45.6)**				
EK3.01	Isolating secondary containment ventilation		3.8	4.1	3.8	4.1
EK3.02	Starting SBGT/FRVS: Plant-Specific		4.1	4.1	4.1	4.1
EK3.03	Personnel evacuation		4.0*	4.4*	4.0	4.4
EK3.04	Fuel building ventilation: Plant-Specific		3.7	3.8	3.7	3.8
EK3.05	Manual SCRAM and depressurization: Plant-Specific		3.6	3.9	3.6	3.9
A1	**Ability to operate and/or monitor the following as they apply to SECONDARY CONTAINMENT VENTILATION HIGH RADIATION:**	**(41.7 / 45.6)**				
EA1.01	Area radiation monitoring system		3.8	3.8	3.8	3.8
EA1.02	Process radiation monitoring system		3.9	4.0	3.9	4.0
EA1.03	Secondary containment ventilation		4.0	3.9	4.0	3.9
EA1.04	SBGT/FRVS: Plant-Specific		4.1*	4.2*	4.1	4.2
EA1.05	Fuel building ventilation: Plant-Specific		3.8	3.8	3.8	3.8
A2	**Ability to determine and/or interpret the following as they apply to SECONDARY CONTAINMENT VENTILATION HIGH RADIATION:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Ventilation radiation levels		3.8	4.2	3.8	4.2
EA2.02	Cause of high radiation levels		3.7	4.2*	3.7	4.2

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295035

System Name: Secondary Containment High Differential Pressure

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to SECONDARY CONTAINMENT HIGH DIFFERENTIAL PRESSURE:**	**(41.8 to 41.10)**				
EK1.01	Secondary containment integrity		3.9	4.2*	3.9	4.2
EK1.02	†Radiation release		3.7	4.2	3.7	4.2
K2	**Knowledge of the interrelations between SECONDARY CONTAINMENT HIGH DIFFERENTIAL PRESSURE and the following:**	**(41.7, 45.8)**				
EK2.01	Secondary containment ventilation		3.6	3.6	3.6	3.6
EK2.02	SBGT/FRVS		3.6	3.8	3.6	3.8
EK2.03	†Off-site release rate		3.3	4.1	3.3	4.1
EK2.04	~~Blow-out panels: Plant-Specific~~		3.3	3.7	3.3	3.7
K3	**Knowledge of the reasons for the following responses as they apply to SECONDARY CONTAINMENT HIGH DIFFERENTIAL PRESSURE:**	**(41.5 / 45.6)**				
EK3.01	~~Blow-out panel operation: Plant-Specific~~		2.8	3.1	2.8	3.1
EK3.02	Secondary containment ventilation response		3.3	3.5	3.3	3.5
A1	**Ability to operate and/or monitor the following as they apply to SECONDARY CONTAINMENT HIGH DIFFERENTIAL PRESSURE:**	**(41.7 / 45.6)**				
EA1.01	Secondary containment ventilation system		3.6	3.6	3.6	3.6
EA1.02	SBGT/FRVS		3.8	3.8	3.8	3.8
A2	**Ability to determine and/or interpret the following as they apply to SECONDARY CONTAINMENT HIGH DIFFERENTIAL PRESSURE:**	**(41.8 to 41.10)**				
EA2.01	Secondary containment pressure: Plant-Specific		3.8	3.9	3.8	3.9
EA2.02	†Off-site release rate: Plant-Specific		2.8*	4.1	2.8	4.1

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295036

System Name: Secondary Containment High Sump/Area Water Level

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to SECONDARY CONTAINMENT HIGH SUMP/AREA WATER LEVEL:**	**(41.8 to 41.10)**				
EK1.01	Radiation releases		2.9	3.1	2.9	3.1
EK1.02	Electrical ground/ circuit malfunction		2.6*	2.8*	2.6	2.8
K2	**Knowledge of the interrelations between SECONDARY CONTAINMENT HIGH SUMP/AREA WATER LEVEL and the following:**	**(41.7, 45.8)**				
EK2.01	Secondary containment equipment and floor drain system		3.1	3.2	3.1	3.2
EK2.02	~~Post-accident sampling system: Plant-Specific~~		2.6	2.9	2.6	2.9
EK2.03	Radwaste		2.8	3.1	2.8	3.1
K3	**Knowledge of the reasons for the following responses as they apply to SECONDARY CONTAINMENT HIGH SUMP/AREA WATER LEVEL:**	**(41.5 / 45.6)**				
EK3.01	Emergency depressurization		2.6	2.8	2.6	2.8
EK3.02	Reactor SCRAM		2.8	2.8	2.8	2.8
EK3.03	Isolating affected systems		3.5	3.6	3.5	3.6
EK3.04	Pumping secondary containment sumps		3.1	3.4	3.1	3.4
A1	**Ability to operate and/or monitor the following as they apply to SECONDARY CONTAINMENT HIGH SUMP/AREA WATER LEVEL:**	**(41.7 / 45.6)**				
EA1.01	Secondary containment equipment and floor drain systems		3.2	3.3	3.2	3.3
EA1.02	Affected systems so as to isolate damaged portions		3.5	3.6	3.5	3.6
EA1.03	Radwaste		2.8	3.0	2.8	3.0
EA1.04	Radiation monitoring: Plant-Specific		3.1	3.4	3.1	3.4
A2	**Ability to determine and/or interpret the following as they apply to SECONDARY CONTAINMENT HIGH SUMP/AREA WATER LEVEL:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Operability of components within the affected area		3.0	3.2	3.0	3.2
EA2.02	Water level in the affected area		3.1	3.1	3.1	3.1
EA2.03	Cause of the high water level		3.4	3.8	3.4	3.8

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295037

System Name: SCRAM Condition Present and Reactor Power Above APRM Downscale or Unknown

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN:**	**(41.8 to 41.10)**				
EK1.01	Reactor pressure effects on reactor power		4.1*	4.3*	4.1	4.3
EK1.02	Reactor water level effects on reactor power		4.1*	4.3*	4.1	4.3
EK1.03	Boron effects on reactor power (SBLC)		4.2	4.4*	4.2	4.4
EK1.04	Hot shutdown boron weight: Plant-Specific		3.4	3.6	3.4	3.6
EK1.05	Cold shutdown boron weight: Plant-Specific		3.4	3.6	3.4	3.6
EK1.06	Cooldown effects on reactor power		4.0*	4.2*	4.0	4.2
EK1.07	Shutdown margin		3.4	3.8	3.4	3.8
K2	**Knowledge of the interrelations between SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN and the following:**	**(41.7, 45.8)**				
EK2.01	RPS		4.2*	4.3*	4.2	4.3
EK2.02	~~RRCS: Plant-Specific~~		4.0	4.2	4.0	4.2
EK2.03	ARI/RPT/ATWS: Plant-Specific		4.1	4.2*	4.1	4.2
EK2.04	SBLC system		4.4*	4.5*	4.4	4.5
EK2.05	CRD hydraulic system		4.0	4.1	4.0	4.1
EK2.06	CRD mechanisms		3.5	3.6	3.5	3.6
EK2.07	Neutron monitoring system		4.0*	4.0	4.0	4.0
EK2.08	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.7	3.1	2.7	3.1
EK2.09	Reactor water level		4.0	4.2	4.0	4.2
EK2.10	Reactor pressure		3.8	4.1	3.8	4.1
EK2.11	~~RMCS: Plant-Specific~~		3.8	3.9	3.8	3.9
EK2.12	Rod control and information system: Plant-Specific		3.6	3.8	3.6	3.8
EK2.13	Alternate boron injection methods: Plant-Specific		3.4	4.1	3.4	4.1
EK2.14	~~RPIS: Plant-Specific~~		3.6	3.9	3.6	3.9
K3	**Knowledge of the reasons for the following responses as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN:**	**(41.5 / 45.6)**				
EK3.01	Recirculation pump trip/runback: Plant-Specific		4.1	4.2	4.1	4.2
EK3.02	SBLC injection		4.3*	4.5*	4.3	4.5
EK3.03	Lowering reactor water level		4.1*	4.5*	4.1	4.5
EK3.04	Hot shutdown boron weight: Plant-Specific		3.2	3.7	3.2	3.7
EK3.05	Cold shutdown boron weight: Plant-Specific		3.2	3.7	3.2	3.7
EK3.06	Maintaining heat sinks external to the containment		3.8	4.1	3.8	4.1
EK3.07	Various alternate methods of control rod insertion: Plant-Specific		4.2	4.3*	4.2	4.3
EK3.08	ATWS circuitry: Plant-Specific.		3.6*	3.9*	3.6	3.9

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 295037

System Name: SCRAM Condition Present and Reactor Power Above APRM Downscale or Unknown

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp S
A1	**Ability to operate and/or monitor the following as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN:**	**(41.7 / 45.6)**				
EA1.01	Reactor Protection System		4.6*	4.6*	4.6	4.6
EA1.02	~~RRCS: Plant-Specific~~		3.8	4.0	3.8	4.0
EA1.03	ARI/RPT/ATWS: Plant-Specific		4.1*	4.1*	4.1	4.1
EA1.04	SBLC		4.5*	4.5*	4.5	4.5
EA1.05	CRD hydraulics systems		3.9	4.0	3.9	4.0
EA1.06	Neutron monitoring system		4.1*	4.1	4.1	4.1
EA1.07	~~RMCS: Plant-Specific~~		3.9	4.0	3.9	4.0
EA1.08	Rod control and information system: Plant-Specific		3.6	3.6	3.6	3.6
EA1.09	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.8*	3.0	2.8	3.0
EA1.10	Alternate boron injection methods: Plant-Specific		3.7	3.9	3.7	3.9
EA1.11	PCIS/NSSSS		3.5	3.6	3.5	3.6
A2	**Ability to determine and/or interpret the following as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Reactor power		4.2*	4.3*	4.2	4.3
EA2.02	Reactor water level		4.1*	4.2*	4.1	4.2
EA2.03	SBLC tank level		4.3*	4.4*	4.3	4.4
EA2.04	Suppression pool temperature		4.0*	4.1*	4.0	4.1
EA2.05	Control rod position		4.2*	4.3*	4.2	4.3
EA2.06	Reactor pressure		4.0	4.1	4.0	4.1
EA2.07	Containment conditions/isolations		4.0	4.2*	4.0	4.2

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 295038

System Name: High Off-Site Release Rate

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH OFF-SITE RELEASE RATE:**	**(41.8 to 41.10)**				
EK1.01	Biological effects of radioisotope ingestion		2.5	3.1	2.5	3.1
EK1.02	†Protection of the general public		4.2*	4.4*	4.2	4.4
EK1.03	†Meteorological effects on off-site release		2.8	3.8	2.8	3.8
K2	**Knowledge of the interrelations between HIGH OFF-SITE RELEASE RATE and the following:**	**(41.7, 45.8)**				
EK2.01	Radwaste		3.1	3.4	3.1	3.4
EK2.02	Offgas system		3.6	3.8	3.6	3.8
EK2.03	Plant ventilation systems		3.6	3.8	3.6	3.8
EK2.04	Stack-gas monitoring system: Plant-Specific		3.9	4.2	3.9	4.2
EK2.05	†Site emergency plan		3.7	4.7*	3.7	4.7
EK2.06	Process liquid radiation monitoring system		3.4	3.7	3.4	3.7
EK2.07	Control room ventilation		3.5	3.7	3.5	3.7
EK2.08	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.6	3.1	2.6	3.1
EK2.09	Post accident sample system (PASS): Plant-Specific		2.9*	3.5*	2.9	3.5
EK2.10	Condenser air removal system		3.2	3.4	3.2	3.4
EK2.11	MSIV leakage control: Plant-Specific		3.1	3.4	3.1	3.4
EK2.12	Feedwater leakage control: BWR-6		3.5	3.8	3.5	3.8
K3	**Knowledge of the reasons for the following responses as they apply to HIGH OFF-SITE RELEASE RATE:**	**(41.5 / 45.6)**				
EK3.01	†Implementation of site emergency plan		3.6	4.5*	3.6	4.5
EK3.02	System isolations		3.9	4.2	3.9	4.2
EK3.03	Control room ventilation isolation: Plant-Specific		3.7	3.9	3.7	3.9
EK3.04	†Emergency depressurization		3.6	3.9	3.6	3.9
A1	**Ability to operate and/or monitor the following as they apply to HIGH OFF-SITE RELEASE RATE:**	**(41.7 / 45.6)**				
EA1.01	Stack-gas monitoring system: Plant-Specific		3.9	4.2	3.9	4.2
EA1.02	†Meteorological instrumentation		3.0*	3.8	3.0	3.8
EA1.03	Process liquid radiation monitoring system		3.7	3.9	3.7	3.9
EA1.04	SPDS/ERIS/CRIDS/GDS: Plant-Specific		2.8	3.2	2.8	3.2
EA1.05	~~Post accident sample system (PASS): Plant-Specific~~		3.0*	3.5*	3.0	3.5
EA1.06	Plant ventilation		3.5	3.6	3.5	3.6
EA1.07	Control room ventilation: Plant-Specific		3.6	3.8	3.6	3.8
A2	**Ability to determine and/or interpret the following as they apply to HIGH OFF-SITE RELEASE RATE:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	†Off-site		3.3*	4.3*	3.3	4.3
EA2.02	†Total number of curies released		2.5*	3.3*	2.5	3.3
EA2.03	†Radiation levels		3.5*	4.3*	3.5	4.3
EA2.04	Source of off-site release		4.1*	4.5*	4.1	4.5

KA Catalog

Facility: CPS

Printed: 08/11/2006

System Number: 295038

System Name: High Off-Site Release Rate

		NRC Imp		Facility Ir	
	CFR	RO	SRO	RO	S

KA Catalog

Facility: CPS **Printed:** 08/11/2006

System Number: 300000

System Name: Instrument Air System (IAS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the connections and/or cause effect relationships between INSTRUMENT AIR SYSTEM and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	~~Sensor air~~		2.4	2.7	2.4	2.7
K1.02	Service air		2.7	2.8	2.7	2.8
K1.03	Containment air		2.8	2.9	2.8	2.9
K1.04	Cooling water to compressor		2.8	2.9	2.8	2.9
K1.05	Main Steam Isolation Valve air		3.1	3.2	3.1	3.2
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	Instrument air compressor		2.8	2.8	2.8	2.8
K2.02	~~Emergency air compressor~~		3.0	3.0	3.0	3.0
K3	**Knowledge of the effect that a loss or malfunction of the (INSTRUMENT AIR SYSTEM) will have on the following:**	**(41.7 / 45.6)**				
K3.01	Containment air system		2.7	2.9	2.7	2.9
K3.02	Systems having pneumatic valves and controls		3.3	3.4	3.3	3.4
K3.03	~~Cross-tied units~~		2.9	3.0	2.9	3.0
K4	**Knowledge of (INSTRUMENT AIR SYSTEM) design feature(s) and or interlocks which provide for the following:**	**(41.7)**				
K4.01	~~Manual/automatic transfers of control~~		2.8	2.9	2.8	2.9
K4.02	Cross-over to other air systems		3.0	3.0	3.0	3.0
K4.03	Securing of IAS upon loss of cooling water		2.8	2.8	2.8	2.8
K5	**Knowledge of the operational implications of the following concepts as they apply to the INSTRUMENT AIR SYSTEM:**	**(41.5 / 45.3)**				
K5.01	Air compressors		2.5	2.5	2.5	2.5
K5.02	Pressure gauges		2.3	2.2	2.3	2.2
K5.03	Temperature indicators		2.1	2.1	2.1	2.1
K5.04	~~Service air refusal valve~~		2.3	2.3	2.3	2.3
K5.06	Air dryers		2.4	2.5	2.4	2.5
K5.07	Valves		2.4	2.3	2.4	2.3
K5.08	Sensors and detectors		2.3	2.5	2.3	2.5
K5.09	Controllers and positioners		2.0	2.1	2.0	2.1
K5.10	Motors		2.1	2.2	2.1	2.2
K5.11	Heat exchangers and condensers		2.2	2.2	2.2	2.2
K5.12	Breakers, relays and disconnects		2.2	2.2	2.2	2.2
K5.13	Filters		2.9	2.9	2.9	2.9

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 300000

System Name: Instrument Air System (IAS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Ir RO	Facility Ir S.
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the INSTRUMENT AIR SYSTEM:**	**(41.7 / 45.7)**				
K6.01	Air compressors		2.2	2.2	2.2	2.2
K6.02	Pressure gauges		2.4	2.4	2.4	2.4
K6.03	Temperature indicators		2.7	2.7	2.7	2.7
K6.04	~~Service air refusal valve~~		2.6	2.5	2.6	2.5
K6.06	Air dryers		2.3	2.3	2.3	2.3
K6.07	Valves		2.5	2.6	2.5	2.6
K6.08	Sensors and detectors		2.3	2.3	2.3	2.3
K6.09	Controllers and positioners		2.3	2.3	2.3	2.3
K6.10	Motors		2.3	2.4	2.3	2.4
K6.11	Heat exchangers and condensers		2.3	2.3	2.3	2.3
K6.12	Breakers, relays and disconnects		2.9	2.9	2.9	2.9
K6.13	Filters		2.8	2.3	2.8	2.3
A2	**Ability to (a) predict the impacts of the following on the INSTRUMENT AIR SYSTEM and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operation:**	**(41.5 / 45.6)**				
A2.01	Air dryer and filter malfunctions		2.9	2.8	2.9	2.8
A3	**Ability to monitor automatic operations of the INSTRUMENT AIR SYSTEM including:**	**(41.7 / 45.7)**				
A3.01	Air pressure		2.3	2.1	2.3	2.1
A3.02	Air temperature		2.9	2.7	2.9	2.7
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	Pressure gauges		2.6	2.7	2.6	2.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 400000

System Name: Component Cooling Water System (CCWS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the physical connections and/or cause-effect relationships between CCWS and the following:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
K1.01	Service water system		3.2	3.3	3.2	3.3
K1.02	Loads cooled by CCWS		3.2	3.4	3.2	3.4
K1.03	Radiation monitoring systems		2.7	3.0	2.7	3.0
K1.04	Reactor coolant system, in order to determine source (s) of RCS leakage into CCWS		2.9	3.1	2.9	3.1
K2	**Knowledge of electrical power supplies to the following:**	**(41.7)**				
K2.01	CCW pumps		2.9	3.0	2.9	3.0
K2.02	CCW valves		2.9	2.9	2.9	2.9
K3	**Knowledge of the effect that a loss or malfunction of the CCWS will have on the following:**	**(41.7 / 45.6)**				
K3.01	Loads cooled by CCWS		2.9	3.3	2.9	3.3
K4	**Knowledge of CCWS design feature(s) and or interlocks which provide for the following:**	**(41.7)**				
K4.01	Automatic start of standby pump		3.4	3.9	3.4	3.9
K5	**Knowledge of the operational implications of the following concepts as they apply to the CCWS:**	**(41.5 / 45.3)**				
K5.01	Chemistry control		1.9	2.0	1.9	2.0
K6	**Knowledge of the effect that a loss or malfunction of the following will have on the CCWS:**	**(41.7 / 45.7)**				
K6.01	Valves		2.7	2.8	2.7	2.8
K6.02	Sensors and detectors		2.3	2.4	2.3	2.4
K6.03	Controllers and positioners		2.4	2.6	2.4	2.6
K6.04	Pumps		3.0	3.1	3.0	3.1
K6.05	Motors		2.8	2.9	2.8	2.9
K6.06	Heat exchangers and condensers		2.9	2.9	2.9	2.9
K6.07	Breakers, relays, and disconnects		2.7	2.8	2.7	2.8
A1	**Ability to predict and/or monitor changes in parameters associated with operating the CCWS controls including:**	**(41.5 / 45.5)**				
A1.01	CCW flow rate		2.8	2.8	2.8	2.8
A1.02	CCW temperature		2.8	2.8	2.8	2.8
A1.03	CCW Pressure		2.7	2.7	2.7	2.7
A1.04	Surge tank level		2.8	2.8	2.8	2.8

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 400000

System Name: Component Cooling Water System (CCWS)

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A2	**Ability to (a) predict the impacts of the following on the CCWS and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operation:**	**(41.5 / 45.6)**				
A2.01	Loss of CCW pump		3.3	3.4	3.3	3.4
A2.02	High/low surge tank level		2.8	3.0	2.8	3.0
A2.03	High/low CCW temperature		2.9	3.0	2.9	3.0
A2.04	Radiation monitoring system alarm		2.9	3.0	2.9	3.0
A3	**Ability to monitor automatic operations of the CCWS including:**	**(41.7 / 45.7)**				
A3.01	Setpoints on instrument signal levels for normal operations, warnings, and trips that are applicable to the CCWS		3.0	3.0	3.0	3.0
A4	**Ability to manually operate and/or monitor in the control room:**	**(41.7 / 45.5 to 45.8)**				
A4.01	CCW indications and control		3.1	3.0	3.1	3.0

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 500000

System Name: High Containment Hydrogen Concentration

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operational implications of the following concepts as they apply to HIGH CONTAINMENT HYDROGEN CONCENTRATIONS:**	**(41.8 to 41.10)**				
EK1.01	Containment integrity		3.3	3.9	3.3	3.9
K2	**Knowledge of the interrelations between HIGH CONTAINMENT HYDROGEN CONCENTRATIONS the following:**	**(41.7, 45.8)**				
EK2.01	Containment hydrogen monitoring systems		3.1	3.5	3.1	3.5
EK2.02	Containment oxygen monitoring systems		3.1	3.5	3.1	3.5
EK2.03	Containment Atmosphere Control System		3.3	3.4	3.3	3.4
EK2.04	Drywell recirculating fan		2.7	2.9	2.7	2.9
EK2.05	Hydrogen and oxygen recombiners		3.2	3.3	3.2	3.3
EK2.06	~~Wetwell Spray system~~		3.0	3.4	3.0	3.4
EK2.07	Drywell vent system		3.2	3.7	3.2	3.7
EK2.08	~~Wet Well vent system~~		3.2	3.6	3.2	3.6
EK2.09	~~Drywell nitrogen purge system~~		3.0	3.3	3.0	3.3
K3	**Knowledge of the reasons for the following responses as they apply to HIGH PRIMARY CONTAINMENT HYDROGEN CONCENTRATIONS:**	**(41.5 / 45.6)**				
EK3.01	Initiation of containment atmosphere control system		2.9	3.3	2.9	3.3
EK3.02	Operation of drywell recirculating fans		2.8	3.0	2.8	3.0
EK3.03	Operation of hydrogen and oxygen recombiners		3.0	3.5	3.0	3.5
EK3.04	~~Emergency depressurization~~		3.1	3.9	3.1	3.9
EK3.05	~~Operation of wet well (suppression pool) sprays~~		2.9	3.4	2.9	3.4
EK3.06	~~Operation of wet well vent~~		3.1	3.7	3.1	3.7
EK3.07	Operation of drywell vent		3.1	3.7	3.1	3.7
EK3.08	~~Operation of drywell nitrogen purge system~~		3.1	3.6	3.1	3.6
A1	**Ability to operate and monitor the following as they apply to HIGH CONTAINMENT HYDROGEN CONTROL:**	**(41.7 / 45.6)**				
EA1.01	Primary containment hydrogen instrumentation		3.4	3.3	3.4	3.3
EA1.02	Primary containment oxygen instrumentation		3.3	3.2	3.3	3.2
EA1.03	Containment Atmosphere Control System		3.4	3.2	3.4	3.2
EA1.04	Drywell recirculating fans		2.9	2.9	2.9	2.9
EA1.05	~~Wetwell sprays~~		3.3	3.3	3.3	3.3
EA1.06	~~Drywell sprays~~		3.3	3.4	3.3	3.4
EA1.07	~~Nitrogen purge system~~		3.4	3.3	3.4	3.3

KA Catalog

Facility: CPS

Printed: 08/11/2006

System Number: 500000

System Name: High Containment Hydrogen Concentration

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I
A2	**Ability to determine and/or interpret the following as they apply to HIGH PRIMARY CONTAINMENT HYDROGEN CONCENTRATIONS:**	**(41.10 / 43.5 / 45.13)**				
EA2.01	Hydrogen monitoring system availability		3.1	3.5	3.1	3.5
EA2.02	Oxygen monitoring system availability		3.0	3.5	3.0	3.5
EA2.03	Combustible limits for drywell		3.3	3.8	3.3	3.8
EA2.04	~~Combustible limits for wetwell~~		3.3	3.3	3.3	3.3

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 600000

System Name: Plant Fire On Site

		CFR	NRC Imp RO	NRC Imp SRO	Facility Imp RO	Facility Imp SRO
K1	**Knowledge of the operation applications of the following concepts as they apply to Plant Fire On Site:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
AK1.01	Fire Classifications by type		2.5	2.8	2.5	2.8
AK1.02	Fire Fighting		2.9	3.1	2.9	3.1
K2	**Knowledge of the interrelations between PLANT FIRE ON SITE and the following:**	**(41.7)**				
AK2.01	Sensors, detectors and valves		2.6	2.7	2.6	2.7
AK2.02	Controllers and positioners		2.4	2.5	2.4	2.5
AK2.03	Motors		2.5	2.6	2.5	2.6
AK2.04	Breakers, relays, and disconnects		2.5	2.6	2.5	2.6
K3	**Knowledge of the reasons for the following responses as they apply to PLANT FIRE ON SITE:**	**(41.7 / 45.4)**				
AK3.01	Installation of fire detectors		2.0	2.1	2.0	2.1
AK3.02	Steps called our in the site fire protection plant, fire protection system manual, and fire zone manual		2.2	2.8	2.2	2.8
AK3.03	Fire detector surveillance test		2.0	2.2	2.0	2.2
AK3.04	Actions contained in the abnormal procedure for plant fire on site		2.8	3.4	2.8	3.4
A1	**Ability to operate and/or monitor the following as they apply to PLANT FIRE ON SITE:**	**(41.7 / 45.6)**				
AA1.01	Respirator air pack		3.0	2.9	3.0	2.9
AA1.02	Re-installation of fire detector		1.8	1.9	1.8	1.9
AA1.03	Bypass of fire zone detector		1.8	2.2	1.8	2.2
AA1.04	Bypass of heat detector		1.9	2.2	1.9	2.2
AA1.05	Plant and control room ventilation systems		3.0	3.1	3.0	3.1
AA1.06	Fire alarm		3.0	3.0	3.0	3.0
AA1.07	Fire alarm reset panel		2.3	2.4	2.3	2.4
AA1.08	Fire fighting equipment used on each class of fire		2.6	2.9	2.6	2.9
AA1.09	Plant fire zone panel (including detector location)		2.5	2.7	2.5	2.7

KA Catalog

Facility: CPS Printed: 08/11/2006

System Number: 600000

System Name: Plant Fire On Site

		CFR	NRC Imp RO	NRC Imp SRO	Facility I RO	Facility I S
A2	**Ability to determine and interpret the following as they apply to PLANT FIRE ON SITE:**	**(41.2 to 41.9 / 45.7 to 45.8)**				
AA2.01	Gas treatment system		2.2	2.3	2.2	2.3
AA2.02	Damper position		2.8	2.9	2.8	2.9
AA2.03	Fire alarm		2.8	3.2	2.8	3.2
AA2.04	The fire's extent of potential operational damage to plant equipment		2.8	3.1	2.8	3.1
AA2.05	Ventilation alignment necessary to secure affected area		2.9	3.0	2.9	3.0
AA2.06	Need for pressurizing control room (recirculating mode)		2.5	2.8	2.5	2.8
AA2.07	Whether malfunction is due to common-mode electrical failures		2.6	3.0	2.6	3.0
AA2.08	Limits of affected area		0	0	0	0
AA2.09	That a failed fire alarm detector exists		2.4	2.8	2.4	2.8
AA2.10	Time limit of long-term-breathing air system for control room		2.9	3.1	2.9	3.1
AA2.11	Time limit for use of respirators		2.9	3.0	2.9	3.0
AA2.12	Location of vital equipment within fire zone		3.1	3.5	3.1	3.5
AA2.13	Need for emergency plant shutdown		3.2	3.8	3.2	3.8
AA2.14	Equipment that will be affected by fire suppression activities in each zone		3.0	3.6	3.0	3.6
AA2.15	Requirements for establishing a fire watch		2.3	3.5	2.3	3.5
AA2.16	Vital equipment and control systems to be maintained and operated during a fire		3.0	3.5	3.0	3.5
AA2.17	Systems that may be affected by the fire		3.1	3.6	3.1	3.[illegible]

June 6, 2006

Mr. Christopher M. Crane
President and Chief Nuclear Officer
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: CONFIRMATION OF REQUESTED INITIAL OPERATOR LICENSE EXAMINATIONS: NRC REGION III EXELON STATIONS

Dear Mr. Crane:

In a letter dated September 30, 2005 (RS-05-122), your staff responded to Regulatory Issue Summary 2005-19 and requested we schedule initial license operator examinations for the Exelon Nuclear Plants. This letter confirms the requested examinations at the Region III power plants. Some dates are different than those requested in your letter. The individual station training departments have been notified by telephone of the schedule differences. We have tentatively scheduled the requested examinations to occur as follows:

Station	2006	2007	2008	2009
Braidwood	5/15/2006	12/3 & 12/10/2007	3/3 & 3/10/2008	N/R
Byron	6/19 & 6/26/2006	N/R	5/19 & 5/26/2008	8/24 & 8/31/2009
Clinton	N/R	8/13 & 8/20/2007	N/	8/31 & 9/07/2009
Dresden	02/06/2006	4/23 & 4/30/2007	7/14 & 7/21/2008	10/12 & 10/19/2009
LaSalle	11/13 & 11/20/2006	N/R	8/18 & 8/25/2008	N/R
Quad Cities	N/R	06/04 & 06/11/2007	N/R	05/25 & 06/01/2009

N/R - None Requested

On-site validation of the examinations is tentatively scheduled to occur approximately three weeks before each scheduled examination. In the unlikely event that we are unable to support the examination during the scheduled weeks, we will inform you and the affected station's training department immediately upon discovery of such conditions and make arrangements to administer the examination at a mutually acceptable date.

We understand that your staff will develop the proposed examinations with the exception of the November 2006 LaSalle Station examination, which will be developed by Region III examiners. Examinations scheduled for two weeks indicate a class size of ten or more candidates. Please inform us if the number of candidates declines below ten as this will impact the examination schedule. Please inform us at your earliest opportunity if you discover a need to change any of the dates of the scheduled examinations.

A supplementary letter will be sent to the station's training department approximately 120 days prior to the initial examination. The letter will provide the station with the chief examiner's name, an outline of examination security expectations, a list of the materials required by the NRC to conduct the examination, and it will reconfirm the examination dates and the number of candidates you have in the training program. If you have any questions concerning this information, please contact Mr. Dell McNeil of my staff at 630-829-9737.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's Agencywide Documents Access and Management System (ADAMS). ADAMS is accessible from the Electronic Reading Room page of the NRC's public Web site at http://www.nrc.gov/reading-rm/adams.html.

Sincerely,

/RA/

Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-456; 50-457
License Nos. NPF-72; NPF-77

See Attached Distributions

We understand that your staff will develop the proposed examinations with the exception of the November 2006 LaSalle Station examination, which will be developed by Region III examiners. Examinations scheduled for two weeks indicate a class size of ten or more candidates. Please inform us if the number of candidates declines below ten as this will impact the examination schedule. Please inform us at your earliest opportunity if you discover a need to change any of the dates of the scheduled examinations.

A supplementary letter will be sent to the station's training department approximately 120 days prior to the initial examination. The letter will provide the station with the chief examiner's name, an outline of examination security expectations, a list of the materials required by the NRC to conduct the examination, and it will reconfirm the examination dates and the number of candidates you have in the training program. If you have any questions concerning this information, please contact Mr. Dell McNeil of my staff at 630-829-9737.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's Agencywide Documents Access and Management System (ADAMS). ADAMS is accessible from the Electronic Reading Room page of the NRC's public Web site at http://www.nrc.gov/reading-rm/adams.html.

Sincerely,

/RA/

Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-456; 50-457
License Nos. NPF-72; NPF-77

See Attached Distributions

ADAMS DOCUMENT TITLE: G:\Work in Progress\Exelon 2007 Exams Ltr.wpd
☐ Publicly Available ☐ Non-Publicly Available ☐ Sensitive ☐ Non-Sensitive
To receive a copy of this document, indicate in the concurrence box "C" = Copy without attach/encl "E" = Copy with attach/encl "N" = No copy

OFFICE	DRS	DRS	DRS	
NAME	MABies:co	DMcNeil	HPeterson	
DATE	05/09/06	05/09/06	06/06/06	

OFFICIAL RECORD COPY

ML061650224

Docket Nos. 50-237; 50-249
License Nos. DPR-19; DPR-25

cc: Site Vice President - Dresden Nuclear Power Station
Dresden Nuclear Power Station Plant Manager
Regulatory Assurance Manager - Dresden
C. Symonds, Training Director, Dresden Nuclear Power Station

Docket Nos. 50-373; 50-374
License Nos. NPF-11; NPF-18

cc: Site Vice President - LaSalle County Station
LaSalle County Station Plant Manager
Regulatory Assurance Manager - LaSalle County Station
Senior Vice President - Mid-West Regional
Operating Group
Manager Licensing - Clinton and LaSalle
R. Ebright, Sr., Training Director, LaSalle County Station

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

cc: Site Vice President - Quad Cities Nuclear Power Station
Plant Manager - Quad Cities Nuclear Power Station
Regulatory Assurance Manager - Quad Cities Nuclear Power Station
Manager Licensing - Dresden and Quad Cities
Vice President - Law and Regulatory Affairs
Mid American Energy Company
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
D. Tubbs, Manager of Nuclear MidAmerican Energy Company
R. Armitage, QCNPP Training Director

cc: Site Vice President - Braidwood Station
Plant Manager - Braidwood Station
Regulatory Assurance Manager - Braidwood Station
Chief Operating Officer
Senior Vice President - Nuclear Services
Vice President - Operations Support
Vice President - Licensing and Regulatory Affairs
Director Licensing
Manager Licensing - Braidwood and Byron
Senior Counsel, Nuclear, Mid-West Regional
Operating Group
Document Control Desk - Licensing
Assistant Attorney General
Illinois Emergency Management Agency
State Liaison Officer - State of Illinois
Chairman, Illinois Commerce Commission
C. Dunn, Training Director, Braidwood Station

Docket Nos. 50-454; 50-455
License Nos. NPF-37; NPF-66

cc: Site Vice President - Byron Station
Plant Manager - Byron Station
Regulatory Assurance Manager - Byron Station
State Liaison Officer, State of Wisconsin
Chairman, Illinois Commerce Commission
B. Quigley, Byron Station
S. Sitmac, Training Director, Byron Station

Docket No. 50-461
License No. NPF-62

cc: Site Vice President - Clinton Power Station
Plant Manager - Clinton Power Station
Regulatory Assurance Manager - Clinton Power Station
Manager Licensing - Clinton Power Station
Senior Counsel, Nuclear, Mid-West Regional Operating Group
J. Lindsey, Training Director, Clinton Power Station

ADAMS Distribution:
GYS
DXC1
SPR
KNJ
BCD
BLB1
MXB
DRC1
DMS6
DEK
JBH1
RidsNrrDipmlipb
GEG
KGO
NXS
CAA1
DRPIII
DRSIII
PLB1
JRK1
NFO
MAB1

Exelon Nuclear
200 Exelon Way
Kennett Square, PA 19348

Exelon Nuclear

RIS 2005-19

RS-05-122
2130-05-20179
5928-05-20277

September 30, 2005

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. STN 50-456 and STN 50-457

Byron Station, Units 1 and 2
Facility Operating License Nos. NPF-37 and NPF-66
NRC Docket Nos. STN 50-454 and STN 50-455

Clinton Power Station, Unit 1
Facility Operating License No. NPF-62
NRC Docket No. 50-461

Dresden Nuclear Power Station, Units 2 and 3
Renewed Facility Operating License Nos. DPR-19 and DPR-25
NRC Docket Nos. 50-237 and 50-249

LaSalle County Station, Units 1 and 2
Facility Operating License Nos. NPF-11 and NPF-18
NRC Docket Nos. 50-373 and 50-374

Limerick Generating Station, Units 1 and 2
Facility Operating License Nos. NPF-39 and NPF-85
NRC Docket Nos. 50-352 and 50-353

Oyster Creek Generating Station
Facility Operating License No. DPR-16
NRC Docket No. 50-219

Peach Bottom Atomic Power Station, Units 2 and 3
Renewed Facility Operating License Nos. DPR-44 and DPR-56
NRC Docket Nos. 50-277 and 50-278

Three Mile Island Nuclear Station, Unit 1
Facility Operating License No. DPR-50
NRC Docket No. 50-289

Quad Cities Nuclear Power Station, Units 1 and 2
Renewed Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

Subject: Response to NRC Regulatory Issue Summary 2005-19
"Preparation and Scheduling of Operator Licensing Examinations"

Reference: NRC Regulatory Issue Summary (RIS) 2005-19, "Preparation and Scheduling of Operator Licensing Examinations," dated August 24, 2005.

In the Reference letter, the NRC requested licensees to voluntarily submit a response concerning initial operator licensing examination needs, including an estimate of the number of applicants to be examined and intentions regarding the preparation of the examinations.

Attachments 1 through 10 provide the responses to RIS 2005-19 for the Exelon Generation Company, LLC (Exelon) and AmerGen Energy Company, LLC (AmerGen) plants identified above. The operator licensing examination information for each facility is provided on a separate NRC Form 536.

During Calendar Year (CY) 2007, a total of 6 candidates are expected to take the dual site Limited Senior Reactor Operator (LSRO) initial license examination as indicated in Attachment 6. The schedule for the indicated exam is as follows: the written exam and part of the operating exam will be given at Limerick Generating Station (LGS); the remainder of the operating exam will be completed at Peach Bottom Atomic Power Station (PBAPS). Since the LSRO exam is a dual site exam that involves both LGS and PBAPS, rather than repeating the same information on both attachments, the exam scheduling information is provided on Attachment 6 for LGS only and Attachment 8 for PBAPS references Attachment 6.

If you have any questions or require additional information, please do not hesitate to contact us.

Sincerely,

D. P. Helker

David P. Helker
Manager, Licensing & Regulatory Affairs
Exelon Generation Company, LLC
AmerGen Energy Company, LLC

Attachments
Attachment 1 – Completed NRC Form 536 for Braidwood Station
Attachment 2 – Completed NRC Form 536 for Byron Station
Attachment 3 – Completed NRC Form 536 for Clinton Power Station
Attachment 4 – Completed NRC Form 536 for Dresden Nuclear Power Station
Attachment 5 – Completed NRC Form 536 for LaSalle County Station
Attachment 6 – Completed NRC Form 536 for Limerick Generating Station
Attachment 7 – Completed NRC Form 536 for Oyster Creek Generating Station
Attachment 8 – Completed NRC Form 536 for Peach Bottom Atomic Power Station
Attachment 9 – Completed NRC Form 536 for Three Mile Island Nuclear Station – Unit 1
Attachment 10 – Completed NRC Form 536 for Quad Cities Nuclear Power Station

NRC FORM 536 (8-2005) U.S NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO 3150-0131 EXPIRES 08/31/2008

Estimated burden per response to comply with this voluntary information collect on request: 1 hour. This information collection is used to plan budgets and resources for operator examinations. Send comments regarding burden estimate to the Records and FOIA/Privacy Services Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0131), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

OPERATOR LICENSING EXAMINATION DATA

FACILITY

Clinton Power Station, Unit 1
Facility Operating License No. NPF-62
NRC Docket No. 50-461

NRC REGION

III

A. PROPOSED EXAMINATION PREPARATION SCHEDULE

PROPOSED NUMBER	CY 2006	CY 2007	CY 2008	CY 2009
ESTIMATED NUMBER OF LICENSEE-PREPARED EXAMINATIONS	0	1	0	1
ESTIMATED NUMBER OF NRC-PREPARED EXAMINATIONS	0	0	0	0

B. INITIAL OPERATOR LICENSE EXAMINATIONS

PROPOSED NUMBER	CY 2006	CY 2007	CY 2008	CY 2009
NUMBER OF REACTOR OPERATORS	0	5	0	5
NUMBER OF SENIOR REACTOR OPERATORS-INSTANT	0	4	0	3
NUMBER OF SENIOR REACTOR OPERATORS-UPGRADE	0	3	0	4
NUMBER OF SENIOR REACTOR OPERATORS-LIMITED	0	0	0	0
PROPOSED DATES				
PRIMARY DATE	N/A	Week of 8/20	N/A	Week of 8/24
ALTERNATE DATE	N/A	Week of 8/27	N/A	Week of 8/31

C. PROPOSED GENERIC FUNDAMENTALS EXAMINATION (GFE) SCHEDULE

PROPOSED NUMBER	CY 2006				CY 2007			
	MARCH	JUNE	SEPTEMBER	DECEMBER	MARCH	JUNE	SEPTEMBER	DECEMBER
ESTIMATED NUMBER OF CANDIDATES	0	9	0	0	0	0	0	0