

LSCS-UFSAR

CHAPTER 13.0 - CONDUCT OF OPERATIONS

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13.5-1

Diagram of "At the Controls" Area

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13.1 ORGANIZATIONAL STRUCTURE OF LICENSEE

13.1.1 Management and Technical Support Organization

The EGC corporate organization and its functions and responsibilities are described in Section 1.0 of Topical Report NO-AA-10 (latest NRC approved revision).

13.1.1.1 Operating Responsibilities

Technical services and backup support for the operating organization are provided throughout the life of the plant from departments within EGC as described in Topical Report NO-AA-10, and from support groups outside the company through contractual agreements.

13.1.1.2 Organizational Arrangement

The corporate management which relates to the operation of the station is described in Section 1.0 of Topical Report NO-AA-10.

13.1.1.3 Qualifications

General responsibility and activities of the support groups are described in Section 1.0 of Topical Report NO-AA-10.

Historical Perspective on Qualification

The original supportive department heads were generally employees of 12 to 25 years' experience who had operations-related experience derived from several years of CEC's nuclear power generation starting with Dresden-1. It was the policy of Commonwealth Edison Company to rotate personnel through development assignments with increasing responsibility. Experiences recorded on the resumes in Attachment 13A of the FSAR were typical for individuals utilized throughout the company for these responsibilities.

Current Perspective on Qualification

Off-site personnel will have a combination of education, experience, and skills commensurate with their level of responsibility. Their experience will be derived either from EGC nuclear power plant assignments, or other industry assignments. These qualifications provide reasonable assurance that decisions and actions during all normal and abnormal conditions will be such that the plant is operated in a safe and efficient manner.

Technical Support for Operations - Engineering

Section 1 of Topical Report NO-AA-10 outlines the specific engineering responsibilities for nuclear station design. Section 3 of Topical Report NO-AA-10 outlines the design control function; other chapters treat the related functions of procurement, inspection, quality assurance, etc.

Technical Support for Operations - Testing, Calibration, and Inservice Inspection Support, Laboratory Services, and Computer Maintenance

Descriptions of the Transmission and Distribution (T & D) Operational Analysis Department and the Systems Material Analysis Department are contained in Topical Report NO-AA-10.

Licensing and Plant Support Services

Licensing and Plant Support Services organizations are described in Topical Report NO-AA-10.

The corporate-level support for fueling and refueling activities at a nuclear station is provided by the Nuclear Fuels organization as described in Topical Report NO-AA-10.

The functional responsibilities and qualifications of the personnel of the onsite and offsite review groups are provided in Section 5.0 of the Technical Specifications.

13.1.2 Operating Organization

This subsection describes the structure, functions, and responsibilities of the onsite organization established to operate and maintain the plant.

All core alternations shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

13.1.2.1 Plant Organization

An organization chart is provided in Topical Report NO-AA-10.

Table 13.1-1 states the titles and the positions for which reactor operator and senior reactor operator licenses are required. The minimum shift crew composition is as follows:

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POSITION	MINIMUM CREW NUMBER		
	EACH UNIT IN MODE 1, 2, OR 3	ONE UNIT IN MODE 1, 2, OR 3, AND ONE UNIT IN MODE 4 OR 5 OR DEFUELED	EACH UNIT IN MODE 4 OR 5 OR DEFUELED
SM ¹	1	1	1
SRO ²	1	1	None
RO ³	3	3	2
STA ⁴	1	1	None

Notes:

1. SM – Shift Manager with a Senior Reactor Operator license for each unit whose reactor contains fuel.
2. SRO – Individual with a Senior Reactor Operator license for each unit whose reactor contains fuel.
3. RO – An individual with a Reactor Operator license or a Senior Reactor Operator license for the unit assigned. Individuals acting as relief operators shall hold a license for both units. Otherwise, for each unit, provide a relief operator who holds a license for the unit assigned.
4. STA – Shift Technical Advisor.

13.1.2.2 Plant Personnel Responsibilities and Authorities

The functions, responsibilities, and authorities of plant positions other than the Shift Manager are described in Topical Report NO-AA-10.

Shift Manager

The Shift Manager is responsible for directing and commanding the overall operation of the facility on his/her shift. During his/her shift, the Shift Manager is in charge of the entire plant operation. The primary management responsibility of the Shift Manager shall be for safe operation of the nuclear facility on his/her shift under all conditions. Standing operating orders are issued to delineate the authority and responsibility of the operations shift supervisors and shift crews which include:

- a. The reactor operator's authority and responsibility for shutting down the reactor when he determines that the safety of the reactor is in jeopardy or when operating parameters exceed any of the reactor protection circuit setpoints and automatic shutdown does not occur.
- b. The responsibility to determine the circumstance, cause, and limits under which operations can safely proceed before the reactor is returned to power following a trip or an unscheduled or unexplained power reduction.
- c. The senior reactor operator's responsibility to be present at the plant and to provide direction for returning the reactor to power following a trip or an unscheduled or unexplained power reduction.
- d. The responsibility to believe and respond conservatively to instrument indications unless they are proven to be incorrect.
- e. The responsibility to adhere to the plant Technical Specifications.
- f. The responsibility to review routine operating data to assure safe operation.
- g. The responsibility to meet the requirements of 10 CFR 50.54, (i), (j), (k), (l), (m).
- h. The responsibility to adhere to plant operating procedure and the requirements for their use. (However, in case of emergency, operations personnel are authorized to depart from approved procedures where necessary to prevent injury to personnel, including the public, or damage to the facility. Such changes shall be documented and incorporated in the revision to the affected procedure.)

13.1.2.3 Operating Shift Crews

Position titles are identified in the Quality Assurance Topical Report (QATR) NO-AA-10 and applicable operator licensing requirements are identified in the Technical Specifications. The minimum number of personnel planned for each shift is provided in the Technical Specifications.

13.1.3 Qualifications of Nuclear Plant Personnel

13.1.3.1 Qualification Requirements

Qualification of the station management and operating staff shall meet minimum acceptable levels as described in ANSI N18.1, "Selection and Training of Nuclear Power Plant Personnel," dated March 8, 1971. The Radiation Protection Manager shall meet the requirements of Radiation Protection Manager of Regulatory Guide 1.8, September, 1975. Table 13.1-1 lists plant staff positions and designates ANSI N18.1, 1971 equivalent titles.

13.1.3.2 Qualifications of Plant Personnel

The qualifications of the initial personnel on the LSCS staff holding key managerial and supervisory positions are provided in the resumes included in Attachment 13.A of the FSAR.

The ANSI N18.1-1971 qualification requirements for Radiation Protection Technician may also be met by either one of the following alternatives:

- a. Individuals who have completed the Radiation Protection Technician training program and have accrued one year of working experience in the specialty, or
- b. Individuals who have completed the Radiation Protection Technician training program, but have not yet accrued one year of working experience in the specialty, who are supervised by on-shift Radiation Protection supervision who meet the requirements of ANSI N18.1-1971 Section 4.3.2, Supervisor Not Requiring AEC Licenses, or Section 4.4.4, Radiation Protection.

The ANSI N18.1-1971 qualification requirements for Chemistry Technician may also be met by either one of the following alternatives:

- a. Individuals who have completed the Chemistry Technician training program and have accrued one year of working experience in the specialty, or

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- b. Individuals who have completed the Chemistry Technician training program, but have not yet accrued one year of working experience in the specialty, who are supervised by on-shift Chemistry supervision who meet the requirements of ANSI N18.1-1971 Section 4.3.2, Supervisor Not Requiring AEC Licenses.

The LSCS technical specifications require a Radiation Protection technician shall be onsite when the fuel is in the reactor. An individual shall be considered qualified in radiation protection procedures upon certification by the licensee that he is capable of successfully accomplishing the following activities:

- a. Conduct special and routine radiation, contamination and airborne radioactivity surveys and evaluate the results.
- b. Establish protective barriers and post appropriate radiological signs.
- c. Establish means of limiting exposure rates and accumulated radiation doses, including the use of protective clothing and respiratory protection equipment.
- d. Perform operability checks of radiation monitors and survey meters.
- e. Recommend appropriate immediate actions in the event of a radiological problem and perform necessary activities until the arrival of Radiation Protection personnel.
- f. Conduct other routine radiological duties (e.g., Technical Specification surveillance items) as may be required on backshifts or weekends.

All Radiation Protection Technicians on the backshift shall be trained per the Radiation Protection Technician Training Program. All such Technicians shall also have satisfactorily completed the following emergency response training.

- a. Tasks to be performed during the first 60 minutes of a serious emergency on the backshift;
- b. In-plant radiation surveys during an accident;
- c. Use and interpretation of both portable and fixed area radiation monitoring equipment.
- d. First aid and bioassay techniques; and
- e. Use of respiratory equipment during emergency situations.

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All Chemistry Technicians on the backshift shall be trained per the Chemistry Technician Training Program. All such Technicians shall also have satisfactorily completed the following emergency response training.

- a. Tasks to be performed during the first 60 minutes of a serious emergency on the backshift; and
- b. Interpretation of critical effluent monitoring data for assisting the Shift Engineer during the first hour of an accident (i.e., station vent monitor and standby gas treatment monitor).

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TABLE 13.1-1
(Sheet 1 of 2)

LASALLE ORGANIZATIONAL TITLES AND ANSI N18.1 EQUIVALENT TITLES

ANSI N18.1 TITLE	TYPE OF LICENSE REQUIRED	LSCS TITLE
Plant Manager		Plant General Manager Plant Manager Operations Manager
Maintenance Manager		Maintenance Manager
Operations Manager	SRO	Operations Manager or Shift Operations Supervisor
Technical Manager		Supervisor in charge of the Systems Engineering Function Support Engineering Supervisor Design Engineering Supervisor Site Engineer Manager
Supervisors Requiring NRC Licenses	SRO**** SRO SRO SRO (Limited)	Shift Manager Unit Supervisor Work Execution Center Operations SRO# Fuel Handling Supervisor
Supervisors Not Requiring NRC Licenses		Field Supervisor Work Control Manager* Outage Manager Site Services Director* Mechanical Maintenance Supt. Electrical Maintenance Supt. Regulatory Assurance Supervisor* Quality Control Supervisor* Security Administrator* Office Support Supervisor* Material Management Supervisor*
Reactor Engineering & Physics		Station Reactor Engineer***
Instrument and Control Engineer		Instrument Maintenance Supt.
Radiochemistry		Chemistry Manager**
Radiation Protection		Health Physics Supervisor

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TABLE 13.1-1 (Cont'd)
(Sheet 2 of 2)

LASALLE ORGANIZATIONAL TITLES AND ANSI N18.1 EQUIVALENT TITLES

ANSI N18.1 TITLE	TYPE OF LICENSE REQUIRED	LSCS TITLE
Operator-Licensed	RO	Nuclear Station Operator
Technicians		Radiation Protection Technician Chemistry Technician
Repairmen		Senior Nuclear Mechanic Senior Nuclear Electrician Control Systems Technician “A” Fuel Handler
Technical Support Engineer in Charge		Assistant to the Supervisor in charge of Systems Engineering Function

* This position is not defined by ANSI N18.1, 1971.

** This qualification may alternately be met by technical personnel reporting to the Chemistry Manager.

*** Qualified as Reactor Engineer in accordance with ANSI 3.1-1981.

**** Specific administrative duties for these positions may be fulfilled by holders of an RO license if properly qualified per ANSI N18.1, 1971.

Either the Work Execution Center (WEC) Operations SRO or one of the Unit Supervisors will be qualified as a Station Technical Advisor (STA) or, whenever a SRO/STA is not included in the shift crew composition, a non-licensed STA will be used.

13.2 TRAINING

13.2.1 LSCS Training Program

The LaSalle County Station (LSCS) training program's goal is to implement training in areas that help ensure individuals performing functions important to plant safety meet associated performance requirements. This is accomplished through a systematic approach to training. This includes processes of job analysis, human factors analysis, developing knowledge and skills of the trainee, and an evaluation format for the trainees and related programs.

The training program acknowledges a combination of education, experience, and skills commensurate with an individual's level of responsibility. This provides reasonable assurances that decisions and actions taken during all plant conditions will be made consistent with plant safety procedures and operational limits established to protect the public health and safety.

This training program incorporates applicable federal guidelines and will be implemented in accordance with these guidelines. The ultimate responsibility for the training program is with the Plant General Manager. This responsibility has been delegated to the Training Manager for administering and monitoring its effectiveness.

13.2.1.1 Program Description

In order to achieve the objective of the training program, courses of instruction have been developed to ensure adequate qualification of personnel assigned functions important to the safe operation of the plant. The following subsections contain the primary categories of training and a brief description of classroom course content and "hands on" training given prior to qualification, if applicable. Retraining or requalification training programs are also addressed in each category as required by federal guidelines or as needed from Company requirements and commitments.

13.2.1.1.1 Plant Staff Training

a. New Employee Orientation

The Orientation program at LaSalle County Station is designed for EGC personnel new to the Station.

The purpose of this class is to acquaint the new employee with various facets of nuclear power, the philosophy and organization of the Company, and selected Station and Company programs and policies

13.2.1.1.2 Non Licensed Operator Training

The Equipment Attendant training program is intended to introduce the prospective non-licensed operator to LaSalle plant systems and provide familiarization of the job responsibilities. The course presents the trainee with concepts and skills to enhance performance as an operator.

The training course combines classroom instruction with OJT shift experience. The Non Licensed Operator training program was accredited by the National Academy for Nuclear Training in October of 1985.

The non-licensed operator continuing training program is intended to review plant systems and the operator's job responsibilities, including plant modifications, procedure revisions, and experience items. This course is designed to enhance performance of the operator.

The continuing training program for non-licensed operators was accredited by the National Academy for Nuclear Training in October of 1985.

13.2.1.1.3 Nuclear General Employee Training (N-GET) Course

Nuclear General Employee Training (N-GET) is required for all personnel who require unescorted access to the protected area at LaSalle County Nuclear Station (LSCS). N-GET must be completed prior to an individual receiving dosimetry as required by the Station. The program is designed for new employees and personnel already familiar with nuclear power stations. Training qualification is valid for four calendar quarters after the completion date. The course includes, but is not limited to, the following subject areas:[m2]

1. Overall description of a nuclear power plant,
2. Radiological health and safety,
3. Appropriate emergency plans and procedures,
4. Industrial safety and fire protection,
5. Plant controlled access areas,
6. Protective clothing and equipment,
7. Plant Security,
8. Fitness For Duty, and
9. Quality Assurance/Quality Control.

Initial Radiation Worker Training (RWT) or equivalence contains the radiological training required for unescorted access to the Radiologically Protected Area. Initial RWT provides a worker with the basic knowledge and skills necessary to enter and work safely within a Radiologically Protected Area (RPA). Demonstrations on radiological health and safety are presented during the course. Written examination(s) are administered following the lectures. The required criterion for successful completion is a score of at least 80% of the written examinations. The training is valid for four full calendar quarters following the completion date.

Similar training is administered, as required, for contractor and temporary personnel who work in radiation/or security controlled areas.

After the initial training has been successfully completed, a requalification evaluation to be taken on an annual basis. The requalification consists of Plant Security GSEP, QA, QC, Fitness For Duty and any major changes to the initial training program that have occurred in the previous year. Written examination(s) are administered covering the initial course topics.

A plant access training (PAT) or equivalent course is mandatory training at LaSalle County Nuclear Station for all personnel who require unescorted access to LaSalle Station's protected area but who do not enter the Radiologically Protected Area (RPA) unless continuously escorted by a qualified basic radiation worker. Also, this PAT level of N-GET provides for the issuance of a primary, thermo luminescence dosimeter (TLD) to monitor personal accumulated dose. An awareness overview of radiation protection hazards is covered in the PAT level N-GET Program. The significant portion of this N-GET is the acknowledgement signature of females for the 500 mrem allowance limit during the gestation period of a pregnancy.

Examinations are taken on Fitness For Duty and all other required topics. Pass criteria is a score of at least 80% on the written examination. Subjects covered:

1. Fitness For Duty
2. N-GET Introduction
3. Security
4. Industrial Safety – including fire safety, hazardous material, and chemical control
5. Quality Programs – including environmental qualification and employee concerns
6. Generating Station Emergency Plan Response
7. Radiation Protection Introduction

A Radiation Worker Training (RWT) or equivalent course is necessary for granting personnel unescorted access to the RPA. This level of N-GET is more specific to the RPA requirements. This training covers all required items. In addition to the PAT N-GET course examination requiring an 80% pass criteria, the RWT Initial N-GET also requires 80% pass criteria for successful completion covering all the required topics.

Subjects covered:

1. Radiological Orientation
2. Sources of Radiation
3. Types and Measurements of Radiation
4. Biological Effects
5. Limits and Guidelines
6. ALARA
7. Radiation Dosimetry
8. Contamination
9. Internal Exposure
10. Radiation Work Permit (RWP)
11. Postings
12. Radiological Alarms
13. Radioactive Waste
14. Rights and Responsibilities
15. Practical Factors
 - a. Protective clothing, don and remove
 - b. Proper frisking techniques
 - c. Exhibit proper work habits

Both the PAT and RWT N-GET Programs now have the flexibility for individuals to “Challenge the Exam” on an annual basis. Challenge exams cover all required topics.

13.2.1.1.4 High Voltage Switching Training

The High Voltage Switching course is designed to prepare operator candidates to qualify for electrical switching responsibilities. The objective of this training course is to certify generating station personnel engaged in electrical switching, consistent with EGC guidelines. The program was accredited by the National Academy for Nuclear Training in October of 1985 as part of the operator training program.

Continuing training is done in conjunction with the non-licensed operators as described in 13.2.1.1.2.

13.2.1.1.5 Licensed Operator Training

The Reactor Operator/Senior Reactor Operator training program for prospective license candidates includes classroom, in plant, on shift and simulator training modules. The LaSalle training programs for Reactor Operator and Senior Reactor Operator were accredited by the National Academy for Nuclear Training in October of 1985. A systems approach to training was used in preparing the programs for accreditation.

13.2.1.1.6 Radwaste Operator Training

The Radwaste Operator Training course is designed to prepare operator candidates to qualify for radwaste operating responsibilities. The objective of this training course is to certify generating station personnel engaged in radwaste operation consistent with EGC guidelines. The program was accredited in December of 1989. |

Continuing training is done in conjunction with the non-licensed operators as described in 13.2.1.1.2.

13.2.1.2 Program Schedule for Preoperational Test, Fuel Loading, and Power Operations

Both units are licensed for 100% full power operations. Tables 13.2-1 and 13.2-2 represent training being conducted to support operational status of the units.

13.2.1.2.1 Evaluation of Training Program

The effectiveness of the training program is normally evaluated by one or more of the following methods:

- a. Satisfactory job performance of employees as determined by periodic supervisor evaluation and observation of responses during actual or simulated plant conditions. This type of evaluation provides informal feedback to the Training Department on common deficient areas of specific personnel weaknesses.
- b. Satisfactory performance of plant personnel on oral or written examinations administered by the Station Training Department, training consultant organization or other management personnel.
- c. Periodic observations of lectures and other training exercises are conducted by line managers and the training personnel or his designee.
- d. The Self Evaluation process used to maintain INPO accreditation.

13.2.2 Replacement and Retraining

Retraining and replacement of Station personnel shall be in accordance with ANSI N18.1, "Selection and Training of Nuclear Power Plant Personnel", dated March 8, 1971 and Appendix "A" of 10 CFR Part 55, and shall include familiarization with relevant industry operational experience.

Retraining shall be conducted at intervals not exceeding 2 years.

Replacement training for licensed operators is described in subsection 13.2.1.1.5. Requalification training for licensed operators is described in subsection 13.2.2.1. Refresher training for all station employees is described in subsection 13.2.2.2. Replacement training for certain specialty groups is described in subsection 13.2.2.3.

13.2.2.1 Licensed Operator Requalification

The LaSalle training programs for Reactor Operator and Senior Reactor Operator were accredited by the National Academy for Nuclear Training in October of 1985 and renewed in December of 1989. A systems approach to training was used in preparing the programs for accreditation. In accordance with the provisions of Generic Letter 87-07 as it applies to 10CFR55.59(c) (Commonwealth Edison)

notified the NRC on May 31, 1988 of its intention to substitute the system based license requalification program for the previously approved program.

13.2.2.2 Refresher Training for all Station Employees

Technical refresher training is conducted for the respective departments on a periodic basis to ensure minimum qualifications of all station personnel. Record keeping for refresher training is the responsibility of the Training Manager.

13.2.2.3 Replacement Training of Certain Specialties

Specialists in the field of radiation protection, radiochemistry, nuclear physics, mechanical-electrical-instrument maintenance, fuel handling, fire brigade, and first aid receive training by manufacturers, equipment vendors, and/or special in-house or Company designed programs. Such courses vary in depth and length from several hours to several weeks duration. A few of the major programs are as follows:

a. Mechanical Maintenance

LaSalle Station Mechanics attend generic courses of instruction at the Production Training Center. The purpose of these courses is to develop generic job related skills using theory, knowledge, and practical shop exercises designed for the generating stations Mechanical Maintenance Group. There are approximately 26 generic course modules which equal approximately 17 weeks of generic mechanical training. Course topics include theory, basic mechanical skills, use of machine shop tools, lubrication, and welding basics. In addition, LaSalle Station Mechanics attend LaSalle Site-specific training to prepare and qualify for upper Mechanical related job responsibilities. Training involves required Station training and Mechanical related maintenance tasks. Course content and structure are designed to meet the National Academy for Nuclear Training accreditation criteria and may change when the criteria warrant change.

b. Electrical Maintenance

LaSalle Station Electricians attend generic courses of instruction at the Production Training Center. The purpose of these courses is to develop generic job related skills using theory, knowledge, and shop exercises designed for the generating stations Electrical Maintenance Group. There are approximately 16 generic course modules

which equal approximately 24 weeks of generic electrical training. Course topics include solid state components, troubleshooting techniques, switchgear maintenance, and motor repair. In addition, LaSalle Station Electricians attend LaSalle Site-specific training to prepare and qualify for upper Electrical related job responsibilities. Training involves required Station training and Electrical related maintenance tasks. Course content and structure are designed to meet the National Academy for Nuclear Training accreditation criteria and may change when the criteria warrant change.

c. Instrument Maintenance

LaSalle Station Instrument Mechanics attend generic courses of instruction at the Production Training Center. The purpose of these courses is to develop generic job related skills using theory, knowledge, and practical shop exercises designed for the generating Stations Instrument Maintenance Group. There are approximately 18 generic course modules which equal approximately 27 weeks of generic instrument skills. Course topics include electronics, math, transmitters, and control loops. In addition, LaSalle Station Instrument Mechanics attend LaSalle Site-specific training to prepare and qualify for additional Journeyman level and Control System Technician related job responsibilities. Training involves required Station training and Instrument related maintenance tasks. Course content and structure are designed to meet the National Academy for Nuclear Training acceptance criteria and may change when the criteria warrant change.

d. Health Physics Technicians

Health Physics Technicians complete an initial course at the Production Training Center which lasts approximately 14 weeks. The trainee then returns to the plant for classroom training that lasts a total of approximately 6 weeks and a practical OJT course which lasts a total of approximately 16 weeks. The two courses provide prospective Radiation Protection Technicians with sufficient expertise to perform related job responsibilities. Course content and structure are designed to meet the National Academy for Nuclear Training Acceptance Criteria and may change when the criteria change.

e. Engineering

1. Nuclear

Several LSCS Nuclear Engineers have in the past attended training conducted by General Electric in San Jose, California, as well as fulfilled training assignments at one or more of EGC's operating reactor plants. Such training is included as part of the LaSalle Qualified Nuclear Engineer training program, using similar materials. This training program consists of many other requirements, which may change when conditions warrant it.

2. Other

In addition, LaSalle Station Engineers must complete site-specific training to prepare and qualify for job related activities. Course content and structure are designed to meet the National Academy for Nuclear Training acceptance criteria and may change when the criteria warrants change.

f. Instructor Training

Workshops in Instructor teaching techniques, program development, and lesson plan preparation have been attended by nearly all LSCS Instructors, with many attending multiple workshops.

g. Shift Manager Training

The goal of Shift Manager Training is to prepare the candidate to be the senior management person on shift. The training is designed to prepare him/her to make decisions in that capacity. The program stresses awareness of available resources to accomplish normal duties and responsibilities as well as to make decisions in abnormal conditions. This training may include classroom, OJT, simulator and workshops. The program was accredited in February 1993.

h. Maintenance Supervisor Training Program

LaSalle Station Maintenance First Line Supervisors attend training to support reliable and safe plant operation in

accordance with operating licenses, plant procedures, quality assurance programs, applicable regulatory requirements, company/station policies, directives, and procedures at various company locations. The purpose of these courses is to develop the First Line Supervisor effectiveness in his daily work activities. There are approximately 4 weeks of training. Course topics consist of NGET, Strategies of Supervisors, First Line Supervisor Administrative, Fitness for Duty, Time Management, Affirmative Action, Problem Solving & Decision Making, VETIP, Conduct of Maintenance, On-the-Job Familiarization, and Building Team Relationships. Course content and structure are designed to meet the National Academy for Nuclear Training accreditation and may change when the criteria warrant change.

i. Chemistry Technicians

Chemistry Technicians complete an initial course at the Production Training Center which lasts approximately 12 weeks. The trainee then returns to the plant for site-specific training and a practical OJT course lasting approximately 22 weeks. The course provides prospective Chemistry Technicians with sufficient expertise to perform related job responsibilities. Course content and structure are designed to meet the National Academy for Nuclear Training Acceptance Criteria and may change when the criteria change.

13.2.2.3.1 Specialist Refresher Training

Specialist refresher training is periodically provided by manufacturers, equipment vendors, and/or special in-house or company programs. Such courses vary in length and depth and designed to meet the individual department needs.

13.2.2.4 Training Program Levels Due to Previous Experience

Personnel with previous nuclear power training or experience are evaluated individually and provided appropriate license training in accordance with ANSI 18.1-1971. The LSCS training program has been established to meet the requirements for neophytes. Individuals who have either held an NRC license or had reactor experience without a license, receive training as specified in the accredited program.

Equivalency of selected individual training requirements for all training programs may be granted following review and verification of equivalent knowledge and/or skills as appropriate. This review may be accomplished through use of proficiency exams or documented experience/education.

13.2.3 Applicable NRC Documents

The following is a list of documents referenced in preparation of LSCS training program. The guidance provided by the applicable portions of these documents as of January, 1989, have been incorporated into the LaSalle County Station training programs.

- a. 10CFR50
- b. 10CFR55
- c. 10CFR19
- d. 10CFR20
- e. ANSI N18.1-1971
- f. Regulatory Guide 0737
- g. Regulatory Guides 8.2, 8.13, 8.8, 8.10

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TABLE 13.2-1

OPERATIONAL TRAINING OF PERSONNEL

<u>TITLE</u>	<u>N-GET</u>	<u>NEO</u>	<u>LOT</u>	<u>LRO</u>	<u>SPEC</u>
Shift Operations Supervisor	X	X	X	X	
Shift Manager	X	X	X	X	X
Field Supervisor	X	X			X
Unit Supervisor	X	X	X	X	
Work Control Center Operations SRO	X	X	X	X	
Nuclear Station Operator	X	X	X	X	X
Equipment Operator	X	X			X
Equipment Attendant	X	X			X
Auxiliary Operator	X	X			X
Engineering Staff	X	X			X
Instructor Staff	X	X			X
Radiation Protection	X	X			X
Maintenance Personnel	X	X			X
Station Laborer	X	X			X
Administrative	X	X			
Storeroom	X	X			
Security Contract Personnel	X				X

N-GET - General Employee Training Course

NEO - New Employee Orientation

LOT - Licensed Operator Training

LRQ - Licensed Requalification Training

SPEC - Specialist Training as applicable to job classification

13.3 EMERGENCY PLANNING

Plans for coping with emergency situations are contained in the Exelon Nuclear Radiological Emergency Plan. The Emergency Plan dated December 1977 was filed on September 11, 1978, for LaSalle County Station Dockets 50-373 and 50-374. That filing also applied for Dresden Units 1, 2, and 3; Quad Cities Units 1 and 2; and Zion Units 1 and 2 (Dockets 50-010, 50-237, 50-249, 50-254, 50-265, 50-295, and 50-304). Revisions to the Emergency Plan are filed with the Nuclear Regulatory Commission (NRC). NRC's evaluation of the Emergency Plan are provided in the Safety Evaluation Report (NUREG 0519) dated March 1981, and in its Supplements.

The Exelon Nuclear Standardized Radiological Emergency Plan is a written emergency plan that establishes the concepts, evaluation and assessment criteria, and recommended protective actions necessary to limit and mitigate the consequences of potential or actual nuclear power plant emergencies. The Emergency Plan provides the necessary prearrangements, directions, and organization to ensure nuclear emergencies can be effectively and efficiently resolved in order to safeguard station personnel, property, and the general public.

The Emergency Plan has been developed based on the emergency planning and preparedness requirements specified in Appendix E to 10 CFR Part 50 and Regulatory Guide 1.101, Revision 3 of August 1992. The Emergency Plan has been submitted to and approved by the NRC. It is reviewed annually, any changes or revisions that pertain to regulatory requirements are submitted to the NRC for approval.

The Emergency Plan includes site-specific Annexes which contain additional information and guidance that are unique to each nuclear generating facility. The site-specific Annexes are not independent of the Emergency Plan. Each site has Standardized Emergency Plan Implementing Procedures which implement the Emergency Plan appropriately.

The Emergency Plan identifies onsite and offsite facilities and equipment available for emergency assessment, communications, first aid and medical care, and damage control. The Emergency Response Facilities (ERFs) consist of the: Control Room (CR), Technical Support Center (TSC), Operational Support Center (OSC), Emergency Operations Facility (EOF), and the Joint Public Information Center (JPIC).

The Emergency Plan provides for classification of emergencies into five categories (listed in order of increasing severity): *Unusual Event*, *Alert*, *Site Emergency*, *General Emergency*, and *Recovery*. *Unusual Event* classifications are events that are in progress or have occurred which indicate a potential degradation of the level of safety of the plant. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs. *Alert* classifications are events that are in progress or have occurred which involve an

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actual or potential substantial degradation of the level of safety of the plant. Any radioactive material releases are expected to be limited to small fractions of the Environmental Protection Agency (EPA) Protective Action Guideline exposure levels. *Site Emergency* classifications are events that are in progress or have occurred which involve actual or likely major failures of the plant functions needed for protection of the public. Any radioactive material releases are not expected to exceed EPA Protective Action Guideline exposure levels except near the site boundary. *General Emergency* classifications are events that are in progress or have occurred which involve actual or imminent substantial core degradation or melting with potential for loss of containment integrity. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels for more than the immediate site area. *Recovery* is that period when the emergency phase is over and actions are being taken to return the situation to a non-emergency state. The plant is under control and no potential for further degradation of the plant or the environment is believed to exist.

The Emergency Plan includes notification requirements for classified events, including prompt and accurate notifications to Federal, State, and local governments.

Training is conducted for all emergency response personnel to ensure their proficiency. The training programs for emergency response personnel are based on the requirements of Appendix E to 10 CFR Part 50.

Evaluated Exercises are conducted biennially at each nuclear station to demonstrate the adequacy of timing and content of the Standardized Emergency Plan Implementing Procedures and methods; to test emergency equipment and communication networks; and to ensure that emergency response personnel are familiar with their duties. Each Exercise involves participation by Federal, State, and/or local personnel as prescribed by regulatory requirements.

The Emergency Plan manuals and Station Annexes are distributed on a controlled basis to all stations and emergency facilities requiring them, including appropriate Federal, State, and local agencies.

13.4 REVIEW AND AUDIT

The review and audit program for LaSalle County Station is conducted in accordance with Quality Assurance Topical Report NO-AA-10.

13.5 PLANT PROCEDURES

The LaSalle County Station (LSCS) day-to-day operations are governed by procedure manuals within assigned areas of responsibility which govern employees' actions and establish standards for plant operation.

A formalized system of written procedures which contains administrative and operating instructions in conformance with ANSI N18.7-1976, and which acknowledges the safety provisions of the facility license and Technical Specifications, is employed to ensure that all normal and reasonably foreseeable abnormal or emergency activities are conducted in a safe manner.

Detailed station/site procedures (i.e., plant procedures), administrative procedures, and safety-related operating procedures, are approved by members of the site management staff. Technical Specifications provide a list of items for which written procedures are required to be established, implemented, and maintained.

Planned safety-related operations are conducted in accordance with these detailed site procedures.

The site reviews all site procedures as required by the Quality Assurance Topical Report NO-AA-10 for LSCS.

13.5.1 Administrative Procedures

13.5.1.1 Conformance with Federal Guidelines

The LSCS site procedures are written to conform to applicable federal guidelines.

13.5.1.2 Preparation of Procedures

The LSCS administrative procedures are prepared by members of the site management organization and/or the nuclear generation organization. The responsibility for writing and preparing procedures is assigned to the appropriate department heads of the site administrative organization as indicated in Table 13.5-1. When procedures are common for all nuclear sites, the nuclear generation organization writes and prepares generic procedures for approval at all sites.

Operating and maintenance procedures and necessary changes are reviewed and approved by a Shift Operations Supervisor in areas of operation or fuel handling and by the Maintenance Manager in areas of plant maintenance, instrument maintenance, and plant inspection.

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Radiation control procedures are reviewed and approved by the Radiation Protection Manager.

All procedures enumerated in this section must be authorized by the responsible approval authority as required by Quality Assurance Topical Report NO-AA-10 before being implemented.

Revisions to these procedures which change the intent or are more than clarifications or corrections to typographical errors, must be authorized by the responsible approval authority as required by Quality Assurance Topical Report NO-AA-10 before being implemented.

Temporary changes to procedures listed in the Technical Specifications may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the site management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed and approved in accordance with the Quality Assurance Topical Report NO-AA-10 within 14 days of implementation.

13.5.1.3 Procedures

The LSCS Administrative Procedures identify site organization and responsibilities, control room procedures, operating criteria, requirements for records, a summary of required tests, analyses, and calibrations to be performed, their frequency of performance, and the group responsible for this work.

Format of the Administrative procedures is controlled through an administrative writers guide.

13.5.1.3.1 Standing Operating Orders

Standing operating orders are issued to delineate the authority and responsibility of the operations shift supervisors and shift crews and include the following:

- a. The reactor operator's authority and responsibility to shutdown the reactor when he determines that the safety of the reactor is in jeopardy or when operating parameters exceed any of the reactor protection circuit setpoints and automatic shutdown does not occur.

- b. The responsibility to determine the circumstances, analyze the cause, and determine that operation can be safely resumed before the reactor is returned to power following a trip or an unscheduled or unexplained power reduction.
- c. The senior reactor operator's responsibility to be present at the plant and to provide direction for returning the reactor to power following a trip or an unscheduled or unexplained power reduction.
- d. The responsibility to review and respond conservatively to instrument indications unless they are proved to be incorrect.
- e. The responsibility to adhere to the LaSalle County Station Technical Specifications.
- f. The responsibility to review routine operating data to assure safe operation.
- g. The responsibility to meet the requirements of 10 CFR 50.54(i), (j), (k), (l), and (m).
- h. The responsibility to adhere to plant operating procedures, and the requirements for their use. (However, in case of emergency, operations personnel shall be authorized to depart from approved procedures where necessary to prevent injury to personnel, including the public, or damage to the facility. Such changes are to be documented and incorporated into the revision to the affected procedure.)
- i. The area designated for the "at the controls" area of the control room (Figure 13.5-1).

13.5.1.3.2 Special Operating Orders

Special operating orders contain nonroutine station operating limits with specific explanations and clarifications of definitions or terms. Special orders are also used to explain temporary operations that may apply to a particular unit.

Special orders are typically used for items which have a short-term applicability and as such include an effective date and a cancellation date. A special operating order is not used as a substitute for a revised procedure or a temporary change to a procedure.

13.5.1.3.3 Equipment Control Procedures

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Equipment control procedures are established to provide for the necessary control of equipment to maintain plant equipment and personnel safety and to avoid unauthorized operation of equipment. These instructions utilize the control measure of tagging to secure and identify equipment in a controlled status. Independent verification is employed to ensure that tagging has been implemented correctly.

The use of equipment locks is employed in certain circumstances in conjunction with the tagging procedure or in the absence of the tagging procedure to ensure that the safety of plant equipment and personnel is not jeopardized. Verification and control procedures are utilized when equipment locking is necessary.

13.5.1.3.4 Control of Maintenance and Modification

Control of maintenance and modification is provided for in the Quality Assurance Topical Report. Prescribed procedures are found in Sections 3, 5, 6, 9 and 10 of this quality assurance manual.

13.5.1.3.5 Master Surveillance Testing Schedule

A master surveillance testing schedule is prepared which prescribes the surveillance to be performed and the frequency as outlined in the Technical Specifications.

13.5.1.3.6 Procedures for Logbook

Procedures for logbook usage and control are provided to ensure adequate documentation of various unit operations and conditions. Logbooks are maintained in the control area and shift supervisor's office. Entry items in these books include unit or plant equipment status, malfunction, scrams (including the reasons), changes in operating conditions, test and measurements performed, and other significant information noted by the operating crew or supervisors. Review of the appropriate logs shall be performed as a function of shift change activities as directed by the shift change administrative procedure.

13.5.1.3.7 Temporary Procedures

Temporary procedures may be issued to direct specific operator actions during testing and maintenance, to provide guidance in unusual situations not within the scope of the station operating procedures, and to ensure orderly and uniform operations for short periods when the plant, a system, or a component of a system is performing in a manner such that portions of existing procedures do not apply.

Temporary procedures require review and approval according to the station review process and must be authorized by the appropriate management representatives prior to implementation. (See Subsection 13.5.1.2)

13.5.2 Operating and Maintenance Procedures

13.5.2.1 Control Room Operating Procedures

The procedures described in this section are performed primarily by licensed operators or reflect licensed operator actions in the performance of the procedure.

Operating procedures are arranged categorically and include a descriptive title for each procedure identified within the classification.

13.5.2.1.1 Systems Operating Procedure

The procedures identified in this classification provide step-by-step instructions for energizing, filling, venting, draining, startup, shutdowns, and changing modes of operation. Each procedure in this classification is written following the format outlined in the station procedure writers guide.

13.5.2.1.2 General Operating Procedures

The general operating procedures provide instruction for the integrated operations of the plant. They are written in the degree of detail necessary to perform the evolution. Where appropriate, reference is made to detailed system procedures.

Format and content of the general operating procedures is controlled through an administrative writers guide.

13.5.2.1.3 Abnormal Operating Procedures

This procedure section is composed of systems abnormal operating procedures, which are directed to system and subsystem conditions. Format and content of the Abnormal Operating Procedures is controlled through an administrative writers guide.

Conditions for entry into the General Abnormal Operations procedures are required to be memorized.

13.5.2.1.4 Emergency Procedures

General Abnormal Operating Procedures are directed to plant conditions. The General Abnormal Operating Procedures are based on the BWR Owners Group Emergency Procedure Guidelines and are produced in flow chart form, with supporting documents produced in a format approved by the station.

Format of the General Abnormal Operating Procedure Flowcharts and support procedures are controlled through an administrative writers guide. Conditions for

entry into the General Abnormal Operations procedures are required to be memorized.

13.5.2.1.5 Annunciator Response Procedures

Annunciators in control areas are identified by an alphanumeric designation on the control panel which coincides with the alphanumeric designation of the annunciator response procedure in the appropriate procedure manual. The format and content of the annunciator procedures are controlled through an administrative writers guide.

13.5.2.1.6 Temporary Procedures

Temporary procedures are described in Subsection 13.5.1.3.7.

13.5.2.2 Other Procedures

This subsection describes how other operating and maintenance procedures are classified and what group within the operating organization has the responsibility for following each class of procedures and outlining the general objectives and character of each class.

13.5.2.2.1 Plant Radiation Protection Procedures

These procedures are described in Chapter 12.0.

13.5.2.2.2 Emergency Preparedness Procedures

These procedures are described in Section 13.3.

13.5.2.2.3 Instrument Calibration and Test Procedures

The instrumentation procedures section governs operation at checkout and calibration of instrumentation. These procedures apply to nuclear, process, area radiation monitoring, reactor protection and rod worth minimizer instrumentation. Checkoff lists are provided and completed for the more complex operations.

The instrument maintenance department prepares and follows these procedures.

13.5.2.2.4 Chemical/Radiochemical Control Procedures

The chemical control procedures section states the operating limits or ranges for chemical and radiochemical control and prescribes the personnel to be notified if these limits are approached.

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The Chemistry Technicians are provided with a schedule which describes the types of analysis to be performed and the frequency at which samples are taken. Step-by-step procedures are provided. In addition, a copy of these procedures is kept in the radiochemical laboratory for ready reference. Other copies are available per the controlled distribution of station procedures.

The Chemistry department prepares and follows the procedures described in this section.

13.5.2.2.5 Radioactive Waste Management Procedures

The radioactive waste management procedures prescribe the methods and modes of operation employed to collect, treat, store, and dispose of liquid and solid radioactive waste materials resulting from plant operations. The operations department prepares and follows the procedures described in this section.

13.5.2.2.6 Maintenance Procedures

The maintenance procedures prescribe the techniques, tools, and equipment used to perform inspection, repair, and overhaul of unit equipment. The maintenance department prepares and follows the procedures in this category.

13.5.2.2.7 Materials Control Procedures

Materials control requirements and procedures are identified in Sections 4, 7, 8, 13 and 15 of the Quality Assurance Topical Report.

13.5.2.2.8 Plant Security Procedures

The plant security procedures are acknowledged in the industrial security section of this UFSAR (Section 13.7).

13.5.2.2.9 Surveillance Procedures

The surveillance procedures prescribe the frequency at which major components and systems are inspected and tested. This includes unit equipment, the surveillance of which is not included in the Technical Specifications or instrumentation procedures.

13.5.2.2.10 Process Control Program

The Process Control Program procedures prescribe and or contain the current formulas, sampling, analysis, test, and determinations to be made to ensure that processing and packaging of solid radioactive wastes based on demonstrated processing of actual or simulated wet solid wastes will be accomplished in such a way as to assure compliance with 10 CFR parts 20, 61, and 71, State regulation, burial ground requirements, and other requirements governing the disposal of solid radioactive waste.

13.5.3 Procedure Review

It is the responsibility of the appropriate department requiring surveillance items to prepare and follow surveillance procedures.

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LaSalle Station has adopted the procedure review requirements of ANS 3.2, ANSI N18.7-1988, sections pertaining to biennial review of procedure, as defined in the ComED 1993 Topical Report.

Abnormal procedures (such as LGA, LOA, and LZPs) shall be reviewed at least every two years and to determine if revisions are required or desired, and revised as appropriate. |

All other procedures related activities will continue to be performed in accordance with the requirements of ANS 3.2, ANSI N18.7-1976 as required by Reg Guide 1.33.

RESPONSIBILITY FOR PREPARATION OFPLANT PROCEDURES

<u>PROCEDURE GROUP</u>	<u>RESPONSIBLE DEPARTMENT HEADS</u>
Administrative procedures	Cognizant Department Head
Operating/Surveillance procedures	Shift Operations Supervisor
Technical/Surveillance procedures	System Engineering Supervisor Support Engineering Supervisor Design Engineering Supervisor
Mechanical Maintenance/Surveillance procedures	Mechanical Maintenance Supt.
Electrical Maintenance/Surveillance procedures	Electrical Maintenance Supt.
Instrumentation/Surveillance procedures	Instrument Maintenance Supt.
Radiation Control procedures	Radiation Protection Manager
Chemical Control procedures	Chemistry Manager
Emergency Procedures	EOP Coordinator

13.6 PLANT RECORDS

Records of plant history are kept in accordance with 10 CFR 50, Appendix B, Section 17, April 30, 1975.

Operating logbooks, reviews, inspections, tests, audits, monitoring of work performance, analyses of materials, procedures, qualification of personnel records, and equipment records are microfilmed and retained in the plant archive room and the central file.

Records contain appropriate identification of inspectors, types of observation, acceptance, deficiencies, results, and action taken to correct deficiencies.

All records are indexed for identification and retrievability within a reasonable time.

All archive records are stored and retained in a manner consistent with regulatory requirements and Company Policy.

The Station Manager has overall responsibility for plant records. The office supervisor is assigned the responsibility of maintaining the records consistent with regulatory requirements and company policy.

13.6.1 Operating Records

Operating records as well as records of principal maintenance activities and abnormal occurrences during the life of the plant are microfilmed and retained in the plant archive room and the central file.

All operating records are indexed for identification and retrievability within a reasonable time.

Operating records are stored and retained in a manner consistent with regulatory requirements and Company Policy.

13.6.2 Event Records

Records pertaining to events such as radioactive releases and environmental surveys during the life of the plant are microfilmed and retained in the plant archive room and the central file.

All event records are indexed for identification and retrievability within a reasonable time.

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Event records are stored and retained in a manner consistent with regulatory requirements and Company Policy.

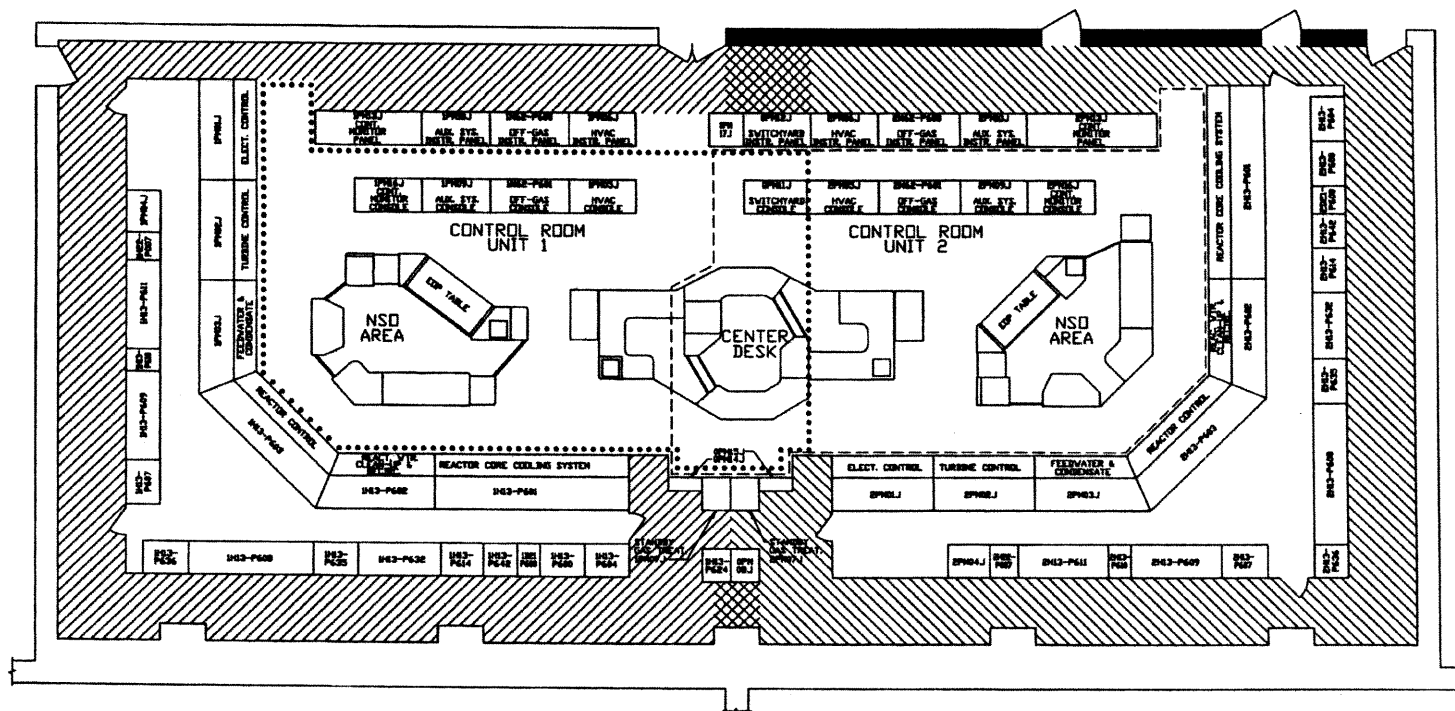
13.7 INDUSTRIAL SECURITY

The following security program plans and associated revisions were filed with the Nuclear Regulatory Commission for the LaSalle Nuclear Power Station:

"LaSalle Station Physical Security Plan", "LaSalle Station Contingency Plan" and "LaSalle Station Guard Training and Qualification Plan."

Based on the review of the submitted documents and visit to the site, the NRC concluded that the protection provided against radiological sabotage at the LaSalle Nuclear Station, Units 1 and 2, meets the requirements of 10 CFR Part 73, and the protection will ensure that the health and safety of the public will not be endangered. Details of the NRC evaluation is provided in the Safety Evaluation Report (SER) NUREG 0519, March 1981 and SER Supplement Nos. 1, 3 and 5, dated June 1981, April 1982 and August 1983, respectively.

LaSalle's security plan is being withheld from public disclosure in accordance with Section 2.790(d) (I) of 10 CFR Part 2.



LEGEND:

THE AREA WITHIN THE DOTTED LINE IS DESIGNATED AS 'AT THE CONTROLS' AREA, UNDER NON-EMERGENCY CONDITIONS.

THE HATCHED AREA IS DESIGNATED AS THE AREA IN WHICH THE OPERATOR MAY MOMENTARILY BE PRESENT IN ORDER TO VERIFY THE RECEIPT OF AN ANNUNCIATOR ALARM OR INITIATE CORRECTIVE ACTION IN THE EVENT OF AN EMERGENCY.

NOTE:

OPERATORS OF BOTH UNITS ARE ALLOWED ACCESS TO PANELS WHICH ARE COMMON TO BOTH UNITS (FOR EXAMPLE, PANELS OPH11J AND OPH12J).

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UPDATED FINAL SAFETY ANALYSIS REPORT

FIGURE 13.5-1

DIAGRAM OF 'AT THE CONTROLS' AREA