

May 15, 2008

MEMORANDUM TO: Christiana H. Lui, Director  
Division of Risk Assessment  
Office of Nuclear Regulatory Research

FROM: Jack W. Foster, Chief */RA/*  
Operating Experience and Generic Issues Branch  
Division of Risk Assessment  
Office of Nuclear Regulatory Research

SUBJECT: GENERIC ISSUE MANAGEMENT CONTROL SYSTEM  
REPORT (FY 2008, Q2)

Enclosed please find the Generic Issue Management Control System (GIMCS) report for the second quarter of FY 2008. For your convenience, the following table summarizes the status of the open Generic Issues (GIs) and the subsequent paragraphs provide a narrative summary of the current status of these GIs. The enclosure provides the related GIMCS report details.

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Status Summary of Active Generic Issues during Q2 of FY 2008						
GI No.	Title	Current Stage	Status	Planned Closure	Months Open	Regulatory Impacts
163	Multiple Steam Generator Tube Leakage	Regulatory Office Implementation		04/2009	189	NUREG-1430, NUREG-1431, and NUREG-1432; GL 2006-01; PWR Technical Specifications
186	Potential Risk and Consequences of Heavy Load Drops in Nuclear Power Plants	Implementation and Verification	Active	12/2008	107	NUREG-1774; Standard Review Plan 9.1.5 (NUREG-0800)
189	Susceptibility of Ice Condenser and Mark III Containments to Early Failure from Hydrogen Combustion During a Severe Accident	Regulatory Office Implementation		09/2010	79	Title 10, Sections 50.34 and 50.44, of the <i>Code of Federal Regulations</i> (10 CFR 50.34 and 50.44)
191	Assessment of Debris Accumulation on PWR Sump Performance	Regulatory Office Implementation		06/2009	138	Regulatory Guide 1.82, Rev. 3; NUREG-0800; GL 1985-22; Bulletin 2003-01; GL 2004-02
193	BWR ECCS Suction Concerns	Technical Assessment	Active	09/2009	70	To Be Determined
199	Implications of Updated Probabilistic Seismic Hazard Estimates in Central and Eastern United States for Existing Plants	Safety/Risk Assessment	Active	06/2009	34	To Be Determined

### **Reactor Generic Issues**

**GI-163, Regulatory Office Implementation, Multiple Steam Generator Tube Leakage** (pages 1–4 of the GIMCS report): As of September 30, 2007, all pressurized-water reactor (PWR) licensees have modified their technical specifications in response to NRC Generic Letter 2006-01, "Steam Generator Tube Integrity and Associated Technical Specifications," and in accordance with Technical Specification Task Force (TSTF)-449. The staff has completed the relevant task items defined in the Steam Generator Action Plan, with the exception of task 3.1.k. Task 3.1.k involves evaluation of the

conditional probabilities of multiple tube failures for risk assessment pertaining to alternative steam generator repair criteria. The staff is targeting April 30, 2009, for issuing a memorandum to the Executive Director of Operations (EDO) documenting the resolution of GSI-163 and the supporting technical bases.

**GI-186, Implementation and Verification, Potential Risk and Consequences of Heavy Load Drops in Nuclear Power Plants** (pages 5–7 of the GIMCS report): On September 14, 2007, the Nuclear Energy Institute (NEI) notified the NRC that the nuclear industry had approved a formal initiative that specifies actions each plant will take to ensure that heavy load lifts continue to be conducted safely and that plant licensing bases accurately reflect plant practices. On December 13, 2007, the NRC staff participated in a public meeting with NEI to discuss implementation of the initiative and acceptance criteria for analyses of reactor vessel head drops. The staff plans to brief the Advisory Committee on Reactor Safeguards on the implementation of recommendations in October 2008, and issue a closure memorandum to the EDO in December 2008.

**GI-189, Regulatory Office Implementation, Susceptibility of Ice Condenser and Mark III Containments to Early Failure from Hydrogen Combustion During a Severe Accident** (pages 8–12 of the GIMCS report): In late February and early March 2007, the staff received industry proposals for design modifications that incorporate security insights. On the basis of industry proposals, the staff expects nearly all affected units to complete implementation of proposed modifications by January 2010. The staff expects to close this GI with a memo to the EDO by September 30, 2010.

**GI-191, Regulatory Office Implementation, Assessment of Debris Accumulation on PWR Sump Performance** (pages 13–18 of the GIMCS report): Planned strainer modifications are now complete at essentially all PWRs. These modifications typically increased strainer size by one to two orders of magnitude. The NRC believes these modifications have improved strainer performance. Head loss testing accounting for chemical effects is ongoing but expected to generally be complete in late 2008. Licensees have submitted supplemental responses to Generic Letter (GL) 2004-02 and staff's review of the responses is expected to be complete by the end of 2008. The staff is also verifying through Temporary Inspection (TI) Procedure TI-2515/166 that licensees have accomplished their committed activities related to GL 2004-02. Completion reports for the TI will be due in summer 2008. Extensions have been granted for some licensees to complete certain hardware modifications as late as spring 2009.

**GI-193, Technical Assessment, BWR ECCS Suction Concerns** (pages 19–21 of the GIMCS report): The Boiling Water Reactor Owners Group provided references to two research reports from the Lappeenranta University of Technology laboratory in Finland which have information relative to this GI. The staff independently pursued contact with Finland through the Office of International Programs and obtained the information (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML071640273 and ML071640280). The staff evaluated the information obtained from Finland. The staff's preliminary conclusion is that cavitation of the emergency core cooling system (ECCS) pumps is not as great a concern as when the issue was formulated. If more detailed examination of the applicability of the data to the plants subject to the concerns of GI-193 validates the preliminary assessment results, the risk model will be revised. The results will be used to complete the safety/risk assessment stage of the generic issue process.

**GI-199, Screening, Implications of Updated Probabilistic Seismic Hazard Estimates in Central and Eastern United States for Existing Plants** (pages 22–24 of the GIMCS report): The staff completed the screening analysis and determined that GI-199 meets the criteria to begin the Safety/Risk Assessment Stage of the Generic Issue Process. On February 6, 2008, the staff met with the stakeholders in a public meeting to discuss the results of the Screening Stage of GI-199. This meeting was provided to ensure that Electric Power Research Institute (EPRI) and other stakeholders are given the opportunity to participate in the safety/risk assessment stage for GI-199. RES staff is currently collecting and analyzing seismic hazard information from available sources, and seismic risk information from individual plant examination of external events analyses. EPRI has committed to provide up-to-date information on seismic source characterization and attenuation models in May 2008. The staff plans to review this information and use this information in the safety/risk assessment stage as appropriate.

Thus, six reactor GIs remain to be resolved.

### ***Nonreactor Generic Issues***

At the end of the reporting period, no nonreactor GIs remain to be resolved in FY 2008.

I will continue to keep you informed of the staff's progress in resolving the remaining reactor GIs and any future GIs, as well as any major problems that may surface during their resolution.

Enclosure:  
As stated

**GI-199, Screening, Implications of Updated Probabilistic Seismic Hazard Estimates in Central and Eastern United States for Existing Plants** (pages 22–24 of the GIMCS report): The staff completed the screening analysis and determined that GI-199 meets the criteria to begin the Safety/Risk Assessment Stage of the Generic Issue Process. On February 6, 2008, the staff met with the stakeholders in a public meeting to discuss the results of the Screening Stage of GI-199. This meeting was provided to ensure that Electric Power Research Institute (EPRI) and other stakeholders are given the opportunity to participate in the safety/risk assessment stage for GI-199. RES staff is currently collecting and analyzing seismic hazard information from available sources, and seismic risk information from individual plant examination of external events analyses. EPRI has committed to provide up-to-date information on seismic source characterization and attenuation models in May 2008. The staff plans to review this information and use this information in the safety/risk assessment stage as appropriate.

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