

Westinghouse Electric Company Hematite Decommissioning Project 3300 State Road P Festus, MO 63028 USA

ATTN: Document Control Desk

Director, Office of Nuclear Material Safety and

Safeguards

U.S. Nuclear Regulatory Commission

Washington, DC 20555-0001

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Our ref: HEM-08-44

Date: April 29, 2008

Subject:

Hematite Effluent Monitoring Reports for the Ten Reporting Periods From

July 1, 2002, Through June 30, 2007 (License No. SNM-33, Docket No. 70-36)

Reference:

1. 10 CFR 70.59, "Effluent monitoring reporting requirements"

- 2. Westinghouse Electric Company LLC (Westinghouse) Letter No. HEM-08-15, "Hematite Effluent Monitoring Report for the Period from July 1, 2007, Through December 31, 2007 (License No. SNM-33, Docket No. 70-36)," dated February 29, 2008
- 3. Westinghouse Letter No. HEM-08-37, "Hematite Effluent Monitoring Reports for the Ten Reporting Periods From July 1, 2002, Through June 30, 2007 (License No. SNM-33, Docket No. 70-36)," dated April 27, 2008

Dear Sirs:

This letter transmits to the Nuclear Regulatory Commission (NRC) ten separate Hematite Decommissioning Project effluent monitoring reports containing the information specified in 10 CFR 70.59 (Reference 1). The reports, provided as attachments to this letter, cover each semi-annual reporting period from July 1, 2002, through June 30, 2007. Submittal of these reports fulfills Westinghouse's commitment to provide these reports to the NRC by April 30, 2008, as specified in Westinghouse Letter No. HEM-08-15 dated February 29, 2008 (Reference 2).

The attached effluent monitoring reports replace in their entirety the interim draft versions of these reports that inadvertently were transmitted to the NRC by Westinghouse Letter No. HEM-08-37 dated April 27, 2008 (Reference 3). Westinghouse regrets any inconvenience resulting from this administrative error, and hereby respectfully requests to withdraw the draft reports transmitted by the April 27, 2008 letter.

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If you have any questions concerning this letter or the attached reports, please contact Matt Featherston, Hematite Licensing Manager, at 314-810-3361.

Sincerely,

Leveled of Rood for EK Hackmann

E. Kurt Hackmann

Director, Hematite Decommissioning Project

Attachments

cc:

- J. J. Hayes, NRC/FSME/DWMEP/DURLD
- B. A. Watson, NRC/FSME/DWMEP/DURLD
- P. L. Louden, NRC Region III/DNMS/DB
- G. M. McCann, NRC Region III/DNMS/DB
- E. Gilstrap, MDNR
- J. A. McCully, Westinghouse

ATTACHMENT 1 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2002, Through December 31, 2002

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2002, Through December 31, 2002

I. Introduction

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from July 1, 2002, through December 31, 2002. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from July 1, 2002, through December 31, 2002, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table notes, it is estimated that approximately 395g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 13 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 41 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1
Liquid Effluent Monitoring Summary Data

	Gross	Alpha Rad	•	Gross Beta Radioactivity		
Reporting Period	Average	Average	Period Quantity	Average	Average	Period Quantity
July - December, 2002	Conc. (µCi/ml)	Fraction of Limit	Discharged (Ci)	Conc. (µCi/ml)	Fraction of Limit	Discharged (Ci)
Site Dam	5.8E-09	1.9E-02	7.2E-04	8.6E-09	1.7E-03	1.1E-03
Sewage Treat. Outfall	4.0E-08	1.3E-01	7.0E-05	3.0E-08	6.0E-03	5.2E-05
Total Quantity of Gross Alpha/Beta Discharged			7.9E+02 μCi			1.2E+03 μCi
Total Quantity of Uranium Discharged Note 1			395 g			
Maximum Concentration	Sewa	ge Treatme	nt Outfall			
for Sampling Period		(μCi/ml))	Site Dam (µCi/ml)		
Gross Alpha (Dissolved)	1.2E-07	Max. Frac	ction of Limit	1.9E-08	Max. Frac	ction of Limit
Gross Alpha (Suspended)	8.0E-10	4.1E-(01 (alpha)	1.1E-09	6.7E-0	02 (alpha)
Gross Beta (Dissolved)	7.4E-08	2.8E-	02 (beta)	2.0E-08	4.3E-	·03 (beta)
Gross Beta (Suspended)	6.5E-08			1.3E-09		
U-233/234	Note 2			Note 3		
U-235	Note 2			Note 3		
U-238	Note 2			Note 3		

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 μ Ci/g, which is representative of the enrichments present during this period.

Note 2: Isotopic data not available.

Note 3: Consistent with the guidance provided in Regulatory Guide 4.16, isotopic analysis was not performed for samples collected from the Site Dam during these reporting periods in consideration of the operational knowledge regarding radionuclide composition; and the low individual sample concentrations which did not exceed ten percent of the annual effluent limit.

B. Gaseous (Airborne) Effluents

The quantity of radioactivity released to unrestricted areas in airborne effluents from July 1, 2002, through December 31, 2002, is summarized in Tables 2 and 3 below, for the third and fourth quarters of 2002, respectively.

As indicated in Tables 2 and 3, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the activity measurements, the average quantity released in airborne effluents was approximately 13 percent of the quarterly effluent limit. This data confirms that the maximum potential radiation dose to

the public resulting from airborne effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 2
Stack Exhaust Air Sampling Summary Data for Third Quarter, 2002

Reporting Period	Quarter 3				
Quarter 3 July - September, 2002	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit	
Stack 052	1.7E-14	3.5E-01	2.5E-01	1.7E-03	
Stack 053	1.2E-14	2.4E-01	2.6E-01	1.7E-03	
Stack 054	5.5E-15	1.1E-01	6.4E-02	4.3E-04	
Stack 055	1.9E-14	3.8E-01	5.0E-01	3.3E-03	
Stack 121	2.4E-14	4.7E-01	1.3E+00	8.6E-03	
Stack 228	2.6E-14	5.3E-01	7.3E-01	4.9E-03	
Stack 240	4.3E-14	8.7E-01	1.4E+00	9.1E-03	
Stack 241	1.7E-13	3.5E+00	5.4E+00	3.6E-02	
Stack 242	2.0E-13	4.0E+00	3.5E+00	2.4E-02	
Stack 243	1.7E-13	3.3E+00	1.6E+00	1.0E-02	
Stack 301	1.3E-14	2.5E-01	3.8E-02	2.5E-04	
Stack 302	3.9E-14	7.8E-01	1.3E-01	8.6E-04	
Stack 303	6.9E-15	1.4E-01	3.4E-01	2.3E-03	
Stack 304	5.4E-14	1.1E+00	2.2E+00	1.4E-02	
Stack 305	2.5E-15	5.0E-02	5.4E-02	3.6E-04	
Stack 401	5.2E-15	1.0E-01	1.3E-01	8.6E-04	
Stack 501	4.8E-14	9.5E-01	6.3E-01	4.2E-03	
Stack 601	1.4E-16	2.8E-03	2.3E-03	1.6E-05	
Stack 602	9.7E-16	1.9E-02	1.6E-02	1.1E-04	
Total Quantity Released	18 μCi				
Fraction of Quantity Limit	0.12				

Table 3

Stack Exhaust Air Sampling Summary Data for Fourth Quarter, 2002

Reporting Period	Quarter 4					
Quarter 4	Average	Fraction of Conc.	Quantity	Fraction of		
October - December, 2002	Conc. (µCi/ml)	Control Limit	Released (µCi)	Quantity Limit		
Stack 052	9.9E-15	2.0E-01	1.6E-01	1.0E-03		
Stack 053	5.7E-15	1.1E-01	1.2E-01	8.3E-04		
Stack 054	6.8E-15	1.4E-01	7.6E-02	5.0E-04		
Stack 055	1.2E-14	2.4E-01	3.4E-01	2.2E-03		
Stack 121	9.3E-15	1.9E-01	4.8E-01	3.2E-03		
Stack 228	4.7E-14	9.3E-01	1.2E+00	8.0E-03		
Stack 240	1.0E-13	2.0E+00	3.2E+00	2.1E-02		
Stack 241	2.6E-13	5.2E+00	8.4E+00	5.6E-02		
Stack 242	1.7E-14	3.5E-01	3.1E-01	2.1E-03		
Stack 243	4.1E-13	8.3E+00	4.0E+00	2.6E-02		
Stack 302	3.6E-14	7.1E-01	3.9E-01	2.6E-03		
Stack 303	1.0E-14	2.0E-01	5.5E-01	3.7E-03		
Stack 304	1.0E-14	2.0E-01	3.5E-01	2.3E-03		
Stack 305	5.6E-15	1.1E-01	1.0E-01	6.9E-04		
Stack 401	5.3E-15	1.1E-01	1.4E-01	9.2E-04		
Stack 501	3.4E-14	6.7E-01	4.7E-01	3.1E-03		
Stack 601	1.6E-15	3.1E-02	2.4E-02	1.6E-04		
Stack 602	2.0E-15	4.0E-02	3.1E-02	2.1E-04		
Total Quantity Released	20 μCi					
Fraction of Quantity Limit	-	0	.14			

III. Conclusion

ATTACHMENT 2 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2003, Through June 30, 2003

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2003, Through June 30, 2003

I. <u>Introduction</u>

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from January 1, 2003, through June 30, 2003. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from January 1, 2003, through June 30, 2003, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table note, it is estimated that approximately 676g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 12 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 47 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1
Liquid Effluent Monitoring Summary Data

	Gross	Alpha Rad	•	Gross Beta Radioactivity		
Reporting Period	Average	Average	Period Quantity	Average	Average	Period Quantity
January - June, 2003	Conc. (µCi/ml)	Fraction of Limit	Discharged (Ci)	Conc. (µCi/ml)	Fraction of Limit	Discharged (Ci)
Site Dam	2.2E-09	7.3E-03	1.3E-03	5.9E-09	1.2E-03	3.5E-03
Sewage Treat. Outfall	3.6E-08	1.2E-01	5.1E-05	3.3E-08	6.6E-03	4.6E-05
Total Quantity of Gross Alpha/Beta Discharged			1.4E+03 μCi			3.5E+03 μCi
Total Quantity of Uranium Discharged Note 1			676 g			
Maximum Concentration for Sampling Period	Sewa	ge Treatme (µCi/ml		Site Dam (μCi/ml)		
Gross Alpha (Dissolved)	3.7E-08	Max. Frac	ction of Limit	1.4E-08	Max. Frac	ction of Limit
Gross Alpha (Suspended)	1.0E-07	4.7E-0	01 (alpha)	1.4E-09	5.0E-0	02 (alpha)
Gross Beta (Dissolved)	6.4E-08	2.3E-	02 (beta)	2.0E-08	4.3E-	03 (beta)
Gross Beta (Suspended)	5.5E-08			1.3E-09		
U-233/234	2.3E-08			3.2E-10		
U-235	7.4E-10			6.8E-11		
U-238	3.3E-09			4.2E-10		

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 μ Ci/g, which is representative of the enrichments present during this period.

B. Gaseous (Airborne) Effluents

The quantity of radioactivity released to unrestricted areas in airborne effluents from January 1, 2003, through June 30, 2003, is summarized in Tables 2 and 3 below, for the first and second quarters of 2003, respectively.

As indicated in Tables 2 and 3, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the activity measurements, the average quantity released in airborne effluents was approximately 5 percent of the quarterly effluent limit. This data confirms that the maximum potential radiation dose to the public resulting from airborne effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 2
Stack Exhaust Air Sampling Summary Data for First Quarter, 2003

Reporting Period		Qua	rter 1	_		
Quarter 1	Average	Fraction of Conc.	Quantity	Fraction of		
January - March, 2003	Conc. (µCi/ml)	Control Limit	Released (µCi)	Quantity Limit		
Stack 052	2.7E-15	5.5E-02	4.3E-02	2.9E-04		
Stack 053	5.0E-15	9.9E-02	1.1E-01	7.2E-04		
Stack 054	4.5E-15	9.0E-02	5.0E-02	3.3E-04		
Stack 055	4.2E-15	8.4E-02	1.1E-01	7.6E-04		
Stack 121	4.8E-15	9.6E-02	2.5E-01	1.6E-03		
Stack 228	2.4E-14	4.8E-01	3.3E-01	2.2E-03		
Stack 240	2.3E-15	4.6E-02	7.3E-02	4.8E-04		
Stack 241	3.6E-13	7.2E+00	1.1E+01	7.2E-02		
Stack 242	5.4E-15	1.1E-01	9.5E-02	6.3E-04		
Stack 243	1.3E-13	2.7E+00	4.2E-01	2.8E-03		
Stack 302	5.5E-15	1.1E-01	1.2E-01	7.9E-04		
Stack 303	2.9E-15	5.7E-02	1.3E-01	8.6E-04		
Stack 304	2.3E-15	4.6E-02	6.6E-02	4.4E-04		
Stack 305	2.5E-15	5.0E-02	4.6E-02	3.1E-04		
Stack 401	3.3E-15	6.6E-02	8.6E-02	5.7E-04		
Stack 501	5.1E-15	1.0E-01	6.9E-02	4.6E-04		
Stack 601	7.2E-16	1.4E-02	1.1E-02	7.2E-05		
Stack 602	2.4E-15	4.8E-02	3.8E-02	2.5E-04		
Total Quantity Released	13 μCi					
Fraction of Quantity Limit	0.09					

Table 3

Stack Exhaust Air Sampling Summary Data for Second Quarter, 2003

Reporting Period		Qua	rter 2		
Quarter 2 April - June, 2003	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit	
Stack 052	1.4E-14	2.8E-01	4.6E-02	3.1E-04	
Stack 053	3.5E-15	7.0E-02	1.6E-02	1.1E-04	
Stack 054	2.6E-15	5.2E-02	2.7E-02	1.8E-04	
Stack 055	1.2E-14	2.5E-01	7.0E-02	4.7E-04	
Stack 121	2.2E-14	4.4E-01	2.3E-01	1.6E-03	
Stack 240	3.5E-15	7.1E-02	3.6E-02	2.4E-04	
Stack 242	1.8E-14	3.7E-01	3.0E-01	2.0E-03	
Stack 302	1.7E-13	3.3E+00	7.5E-01	5.0E-03	
Stack 305	2.2E-15	4.4E-02	3.7E-02	2.5E-04	
Stack 401	3.9E-16	7.8E-03	2.1E-03	1.4E-05	
Stack 501	7.6E-15	1.5E-01	2.2E-02	1.4E-04	
Stack 601	1.1E-15	2.1E-02	1.5E-02	9.8E-05	
Stack 602 ⁽¹⁾	2.4E-15	4.8E-02	3.5E-02	2.3E-04	
Total Quantity Released	1.6 μCi				
Fraction of Quantity Limit	0.01				

⁽¹⁾ The sampling pump located at Stack 602 was not operational during the Second Quarter of 2003. Therefore, the average concentration at Stack 602 during the First Quarter of 2003 was used to estimate the quantity of radioactive material released (µCi) from Stack 602 during the Second Quarter of 2003.

III. Conclusion

ATTACHMENT 3 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2003, Through December 31, 2003

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2003, Through December 31, 2003

I. Introduction

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from July 1, 2003, through December 31, 2003. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from July 1, 2003, through December 31, 2003, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table notes, it is estimated that approximately 560g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 10 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 25 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1
Liquid Effluent Monitoring Summary Data

	Gross	Alpha Rad	ioactivity	Gross Beta Radioactivity		
Reporting Period			Period			Period
Reporting Fortou	Average	Average	Quantity	Average	Average	Quantity
July - December, 2003	Conc. (µCi/ml)	Fraction of Limit	Discharged	Conc.	Fraction of Limit	Discharged (Ci)
G'A Daw	` '		(Ci)	(μCi/ml)		(Ci)
Site Dam	4.8E-09	1.6E-02	1.1E-03	1.2E-08	2.4E-03	2.7E-03
Sewage Treat. Outfall	2.9E-08	1.0E-01	1.9E-05	4.4E-08	8.7E-03	2.8E-05
Total Quantity of Gross						
Alpha/Beta Discharged			1.1E+03 μCi			2.7E+03 μCi
Total Quantity of						
Uranium Discharged Note 1			560 g			
Maximum Concentration	Sewa	ge Treatme	nt Outfall			
for Sampling Period		(μCi/ml))	Site Dam (µCi/ml)		
Gross Alpha (Dissolved)	7.2E-08	Max. Frac	ction of Limit	5.5E-08	Max. Frac	ction of Limit
Gross Alpha (Suspended)	3.1E-09	2.5E-0	01 (alpha)	8.0E-10	1.8E-0	01 (alpha)
Gross Beta (Dissolved)	1.1E-07	2.3E-	02 (beta)	1.7E-07	3.4E-	02 (beta)
Gross Beta (Suspended)	4.9E-09			1.5E-09		
U-233/234	Note 2			Note 3		
U-235	Note 2			Note 3		
U-238	Note 2			Note 3_		

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 μ Ci/g, which is representative of the enrichments present during this period.

Note 2: Isotopic data not available.

Note 3: Consistent with the guidance provided in Regulatory Guide 4.16, isotopic analysis was not performed for samples collected from the Site Dam during these reporting periods in consideration of the operational knowledge regarding radionuclide composition; and the low individual sample concentrations which did not exceed ten percent of the annual effluent limit.

B. Gaseous (Airborne) Effluents

The quantity of radioactivity released to unrestricted areas in airborne effluents from July 1, 2003, through December 31, 2003, is summarized in Tables 2 and 3 below, for the third and fourth quarters of 2003, respectively.

As indicated in Tables 2 and 3, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the activity measurements, the average quantity released in airborne effluents was less than 1 percent of the quarterly effluent limit. This data confirms that the maximum potential radiation dose to the

public resulting from airborne effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 2

Stack Exhaust Air Sampling Summary Data for Third Quarter, 2003

Reporting Period		Quarter 3					
Quarter 3 July - September, 2003	Average Conc. (µCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit			
Stack 054	7.0E-15	1.4E-01	7.9E-02	5.2E-04			
Stack 240	6.0E-13	1.2E+01	9.3E-03	6.2E-05			
Stack 242	1.0E-14	2.0E-01	1.9E-01	1.3E-03			
Stack 305	2.8E-15	5.5E-02	5.4E-02	3.6E-04			
Stack 601	2.6E-15	5.2E-02	2.6E-02	1.7E-04			
Stack 602	1.1E-14	2.1E-01	1.7E-01	1.1E-03			
Total Quantity Released	0.5 μCi						
Fraction of Quantity Limit	3.5E-03						

Table 3

Stack Exhaust Air Sampling Summary Data for Fourth Quarter, 2003

Reporting Period					
Quarter 4 October - December, 2003	Average Conc. (µCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit	
Stack 054	1.5E-15	3.1E-02	1.6E-02	1.1E-04	
Stack 242	3.2E-15	6.4E-02	5.5E-02	3.6E-04	
Stack 305	1.2E-15	2.5E-02	2.2E-02	1.5E-04	
Total Quantity Released	0.09 μCi				
Fraction of Quantity Limit	6.2E-04				

III. Conclusion

ATTACHMENT 4 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2004, Through June 30, 2004

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2004, Through June 30, 2004

I. Introduction

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from January 1, 2004, through June 30, 2004. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from January 1, 2004, through June 30, 2004, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table note, it is estimated that approximately 532g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 15 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 78 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1
Liquid Effluent Monitoring Summary Data

	Gross	Alpha Rad	ioactivity	Gross Beta Radioactivity			
Reporting Period			Period			Period	
Reporting Ferrod	Average	Average	Quantity	Average	Average	Quantity	
January - June, 2004	Conc.	Fraction	Discharged	Conc.	Fraction	Discharged	
	(µCi/ml)	of Limit	(Ci)	(μCi/ml)	of Limit	(Ci)	
Site Dam	1.6E-09	5.4E-03	1.0E-03	5.4E-09	1.1E-03	3.3E-03	
Sewage Treat. Outfall	4.6E-08	1.5E-01	6.4E-05	5.3E-08	1.1E-02	7.4E-05	
Total Quantity of Gross							
Alpha/Beta Discharged			1.1E+03 μCi			3.4E+03 μCi	
Total Quantity of							
Uranium Discharged Note 1			532 g				
Maximum Concentration	Sewa	ge Treatme	nt Outfall				
for Sampling Period		(μCi/ml))	Si	Site Dam (µCi/ml)		
Gross Alpha (Dissolved)	6.2E-08	Max. Frac	ction of Limit	1.2E-09	Max. Fra	ction of Limit	
Gross Alpha (Suspended)	1.7E-07	7.8E-0	01 (alpha)	3.3E-09	1.5E-0	02 (alpha)	
Gross Beta (Dissolved)	1.5E-07	3.8E-	02 (beta)	8.4E-09	2.4E-	03 (beta)	
Gross Beta (Suspended)	4.2E-08			3.6E-09			
U-233/234	1.4E-08			5.6E-09			
U-235	3.7E-10			1.7E-10			
U-238	2.0E-09			7.9E-10			

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 μ Ci/g, which is representative of the enrichments present during this period.

B. Gaseous (Airborne) Effluents

The quantity of radioactivity released to unrestricted areas in airborne effluents from January 1, 2004, through June 30, 2004, is summarized in Tables 2 and 3 below, for the first and second quarters of 2004, respectively.

As indicated in Tables 2 and 3, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the activity measurements, the average quantity released in airborne effluents was less than 1 percent of the quarterly effluent limit. This data confirms that the maximum potential radiation dose to the public resulting from airborne effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 2
Stack Exhaust Air Sampling Summary Data for First Quarter, 2004

Reporting Period	Quarter 1					
Quarter 1 January - March, 2004	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (μCi)	Fraction of Quantity Limit		
Stack 054	2.4E-15	4.7E-02	2.7E-02	1.8E-04		
Stack 242	4.2E-15	8.4E-02	7.9E-02	5.3E-04		
Stack 305	2.1E-14	4.2E-01	4.0E-01	2.7E-03		
Total Quantity Released	5.1E-01 μCi					
Fraction of Quantity Limit	3.4E-03					

Table 3

Stack Exhaust Air Sampling Summary Data for Second Quarter, 2004

Reporting Period	Quarter 2					
Quarter 2 April - June, 2004	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit		
Stack 054	2.0E-15	4.0E-02	2.2E-02	1.5E-04		
Stack 242	4.2E-15	8.3E-02	8.1E-02	5.4E-04		
Stack 305	2.3E-15	4.6E-02	4.5E-02	3.0E-04		
Total Quantity Released	1.5E-01 μCi					
Fraction of Quantity Limit	9.9E-04					

III. Conclusion

ATTACHMENT 5 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2004, Through December 31, 2004

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2004, Through December 31, 2004

I. Introduction

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from July 1, 2004, through December 31, 2004. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. <u>Effluent Monitoring Report</u>

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from July 1, 2004, through December 31, 2004, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table notes, it is estimated that approximately 345g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 25 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 62 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1

Liquid Effluent Monitoring Summary Data

	Gross	Alpha Rad	ioactivity	Gross Beta Radioactivity		
Reporting Period			Period			Period
Reporting 1 cried	Average	Average	Quantity	Average	Average	Quantity
July - December, 2004	Conc.	Fraction	Discharged	Conc.	Fraction	Discharged
l	(μCi/ml)	of Limit	(Ci)	(μCi/ml)	of Limit	(Ci)
Site Dam	2.8E-09	9.3E-03	5.7E-04	6.0E-09	1.2E-03	1.2E-03
Sewage Treat. Outfall	7.4E-08	2.5E-01	1.2E-04	9.7E-08	1.9E-02	1.6E-04
Total Quantity of Gross						
Alpha/Beta Discharged			6.9E+02 μCi			1.4E+03 μCi
Total Quantity of						
Uranium Discharged Note 1			345 g			
Maximum Concentration	Sewa	ge Treatme	nt Outfall		•	
for Sampling Period		(µCi/ml))	Si	te Dam (µ0	Ci/ml)
Gross Alpha (Dissolved)	6.5E-08	Max. Frac	ction of Limit	1.9E-08	Max. Frac	ction of Limit
Gross Alpha (Suspended)	1.2E-07	6.2E-0	01 (alpha)	3.4E-09	7.3E-0	02 (alpha)
Gross Beta (Dissolved)	1.6E-07	3.5E-	02 (beta)	6.1E-08	1.2E-	02 (beta)
Gross Beta (Suspended)	1.5E-08			1.4E-09		
U-233/234	1.3E-7			Note 2		
U-235	4.5E-9			Note 2		
U-238	1.9E-8			Note 2		

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 μ Ci/g, which is representative of the enrichments present during this period.

Note 2: Consistent with the guidance provided in Regulatory Guide 4.16, isotopic analysis was not performed for samples collected from the Site Dam during these reporting periods in consideration of the operational knowledge regarding radionuclide composition; and the low individual sample concentrations which did not exceed ten percent of the annual effluent limit.

B. Gaseous (Airborne) Effluents

The quantity of radioactivity released to unrestricted areas in airborne effluents from July 1, 2004, through December 31, 2004, is summarized in Tables 2 and 3 below, for the third and fourth quarters of 2004, respectively.

As indicated in Tables 2 and 3, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the activity measurements, the average quantity released in airborne effluents was less than 1 percent of the quarterly effluent limit. This data confirms that the maximum potential radiation dose to the public resulting from airborne effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 2
Stack Exhaust Air Sampling Summary Data for Third Quarter, 2004

Reporting Period	Quarter 3					
Quarter 3 July - September, 2004	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit		
Stack 054	1.8E-15	3.6E-02	2.0E-02	1.3E-04		
Stack 242	3.1E-15	6.2E-02	5.9E-02	3.9E-04		
Stack 305	1.0E-15	2.0E-02	2.0E-02	1.3E-04		
Total Quantity Released	1.0E-01μCi					
Fraction of Quantity Limit	6.6E-04					

Table 3

Stack Exhaust Air Sampling Summary Data for Fourth Quarter, 2004

Reporting Period	Quarter 4					
Quarter 4 October - December, 2004	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit		
Stack 054	8.4E-15	1.7E-01	9.6E-02	6.4E-04		
Stack 242	6.2E-15	1.2E-01	1.2E-01	8.1E-04		
Stack 305	1.0E-14	2.1E-01	2.0E-01	1.4E-03		
Stack 501	1.1E-14	2.3E-01	1.3E-01	8.8E-04		
Total Quantity Released	5.5E-01 μCi					
Fraction of Quantity Limit	3.7E-03					

III. <u>Conclusion</u>

ATTACHMENT 6 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2005, Through June 30, 2005

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2005, Through June 30, 2005

I. Introduction

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from January 1, 2005, through June 30, 2005. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from January 1, 2005, through June 30, 2005, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table notes, it is estimated that approximately 352g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 5 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 9 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1
Liquid Effluent Monitoring Summary Data

	Gross Alpha Radioactivity			Gross Beta Radioactivity		
Danautina Dania d			Period			Period
Reporting Period	Average	Average	Quantity	Average	Average	Quantity
January - June, 2005	Conc.	Fraction	Discharged	Conc.	Fraction	Discharged
	(μCi/ml)	of Limit	(Ci)	(μCi/ml)	of Limit	(Ci)
Site Dam	1.5E-09	4.9E-03	6.9E-04	4.0E-09	8.0E-04	1.8E-03
Sewage Treat. Outfall	1.6E-08	5.2E-02	1.3E-05	3.4E-08	6.8E-03	2.7E-05
Total Quantity of Gross						
Alpha/Beta Discharged			7.0E+02 μCi			1.8E+03 μCi
Total Quantity of						
Uranium Discharged Note 1	*		352 g			
Maximum Concentration	Sewa	ge Treatme				
for Sampling Period		(μCi/ml))	Si	te Dam (µ0	Ci/ml)
Gross Alpha (Dissolved)	2.6E-08	Max. Frac	ction of Limit	9.7E-09	Max. Frac	ction of Limit
Gross Alpha (Suspended)	2.8E-10	8.8E-0	02 (alpha)	3.8E-10	3.4E-0	02 (alpha)
Gross Beta (Dissolved)	3.5E-08	9.6E-	03 (beta)	1.0E-08	2.3E-	03 (beta)
Gross Beta (Suspended)	1.3E-08			1.7E-09		
U-233/234	Note 2			Note 2		
U-235	Note 2			Note 2		
U-238	Note 2			Note 2		

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 µCi/g, which is representative of the enrichments present during this period.

Note 2: Consistent with the guidance provided in Regulatory Guide 4.16, isotopic analysis was not performed for samples collected from the Sewage Treatment Outfall and the Site Dam during these reporting periods in consideration of the operational knowledge regarding radionuclide composition; and the low individual sample concentrations which did not exceed ten percent of the annual effluent limit.

B. Gaseous (Airborne) Effluents

The quantity of radioactivity released to unrestricted areas in airborne effluents from January 1, 2005 through June 30, 2005, is summarized in Tables 2 and 3 below, for the first and second quarters of 2005, respectively.

As indicated in Tables 2 and 3, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the activity measurements, the average quantity released in airborne effluents was less than 1 percent of the quarterly effluent limit. This data confirms that the maximum potential radiation dose to the public resulting from airborne effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 2
Stack Exhaust Air Sampling Summary Data for First Quarter, 2005

Reporting Period	Quarter 1					
Quarter 1 January - March, 2005	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit		
Stack 054	1.2E-14	2.4E-01	1.5E-01	9.8E-04		
Stack 121	6.8E-15	1.4E-01	8.0E-02	5.3E-04		
Stack 242	8.8E-15	1.8E-01	1.0E-01	6.7E-04		
Stack 305	8.2E-15	1.6E-01	1.6E-01	1.0E-03		
Stack 501	1.0E-14	2.0E-01	1.4E-01	9.1E-04		
Total Quantity Released	0.62 μCi					
Fraction of Quantity Limit	4.1E-03					

Table 3

Stack Exhaust Air Sampling Summary Data for Second Quarter, 2005

Reporting Period	Quarter 2					
Quarter 2 April - June, 2005	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit		
Stack 054	1.1E-14	2.2E-01	1.3E-01	8.4E-04		
Stack 121	1.3E-14	2.5E-01	2.6E-01	1.7E-03		
Stack 240	4.1E-14	8.2E-01	6.9E-02	4.6E-04		
Stack 242	7.4E-15	1.5E-01	1.0E-01	6.6E-04		
Stack 303	2.7E-14	5.3E-01	2.7E-01	1.8E-03		
Stack 305	6.8E-15	1.4E-01	1.6E-01	1.1E-03		
Stack 401	2.2E-15	4.5E-02	1.6E-02	1.1E-04		
Stack 501	5.3E-15	1.1E-01	1.0E-01	6.7E-04		
Total Quantity Released	1.1 μCi					
Fraction of Quantity Limit	7.3E-03					

III. Conclusion

ATTACHMENT 7 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2005, Through December 31, 2005

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2005, Through December 31, 2005

I. Introduction

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from July 1, 2005, through December 31, 2005. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from July 1, 2005, through December 31, 2005, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table notes, it is estimated that approximately 152g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 6 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 36 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1
Liquid Effluent Monitoring Summary Data

	Gross	Alpha Rad	ioactivity	Gross Beta Radioactivity		oactivity
Reporting Period	A	•	Period			Period
	Average Conc.	Average Fraction	Quantity Discharged	Average Conc.	Average Fraction	Quantity Discharged
July - December, 2005	Cone. (μCi/ml)	of Limit	(Ci)	(μCi/ml)	of Limit	(Ci)
Site Dam	3.5E-09	1.2E-02	2.9E-04	6.6E-09	1.3E-03	5.5E-04
Sewage Treat. Outfall	1.7E-08	5.7E-02	1.4E-05	4.5E-08	9.1E-03	3.6E-05
Total Quantity of Gross						
Alpha/Beta Discharged			3.0E+02 μCi			5.9E+02 μCi
Total Quantity of						
Uranium Discharged Note 1			152 g			
Maximum Concentration	Sewa	ge Treatme	nt Outfall			
for Sampling Period		(µCi/ml))	Si	te Dam (µ0	Ci/ml)
Gross Alpha (Dissolved)	1.0E-07	Max. Frac	ction of Limit	1.3E-08	Max. Fra	ction of Limit
Gross Alpha (Suspended)	7.8E-09	3.6E-0	01 (alpha)	6.4E-10	4.5E-0	02 (alpha)
Gross Beta (Dissolved)	8.2E-08	1.7E-	02 (beta)	9.5E-09	2.5E-	-03 (beta)
Gross Beta (Suspended)	2.9E-09			2.8E-09		
U-233/234	5.4E-08			Note 2		
U-235	2.3E-09			Note 2		
U-238	6.8E-09			Note 2		

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 µCi/g, which is representative of the enrichments present during this period.

Note 2: Consistent with the guidance provided in Regulatory Guide 4.16, isotopic analysis was not performed for samples collected from the Site Dam during these reporting periods in consideration of the operational knowledge regarding radionuclide composition; and the low individual sample concentrations which did not exceed ten percent of the annual effluent limit.

B. Gaseous (Airborne) Effluents

The quantity of radioactivity released to unrestricted areas in airborne effluents from July 1, 2005, through December 31, 2005, is summarized in Tables 2 and 3 below, for the third and fourth quarters of 2005, respectively.

As indicated in Tables 2 and 3, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the activity measurements, the average quantity released in airborne effluents was less than 1 percent of the quarterly effluent limit. This data confirms that the maximum potential radiation dose to the

public resulting from airborne effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 2

Stack Exhaust Air Sampling Summary Data for Third Quarter, 2005

Reporting Period	Quarter 3					
Reporting Period Quarter 3 July - September, 2005	Average Conc. (μCi/ml)	Fraction of Conc. Control Limit	Quantity Released (µCi)	Fraction of Quantity Limit		
Stack 054	2.7E-15	5.5E-02	3.7E-02	2.4E-04		
Stack 121	1.0E-14	2.0E-01	3.0E-01	2.0E-03		
Stack 240	8.0E-15	1.6E-01	2.1E-01	1.4E-03		
Stack 242	4.4E-15	8.8E-02	6.0E-02	4.0E-04		
Stack 303	1.8E-14	3.6E-01	6.3E-01	4.2E-03		
Stack 305	6.7E-15	1.3E-01	1.6E-01	1.0E-03		
Stack 401	1.7E-15	3.4E-02	4.3E-02	2.9E-04		
Stack 501	2.4E-15	4.7E-02	4.6E-02	3.1E-04		
Total Quantity Released	1.5 μCi					
Fraction of Quantity Limit	9.9E-03					

Table 3

Stack Exhaust Air Sampling Summary Data for Fourth Quarter, 2005

Reporting Period	Quarter 4					
Quarter 4	Average	Fraction of Conc.	Quantity	Fraction of		
October - December, 2005	Conc. (µCi/ml)	Control Limit	Released (μCi)	Quantity Limit		
Stack 054	3.7E-15	7.4E-02	2.2E-02	1.5E-04		
Stack 121	8.7E-15	1.7E-01	1.8E-01	1.2E-03		
Stack 240	5.9E-15	1.2E-01	8.5E-02	5.7E-04		
Stack 242	2.2E-14	4.3E-01	2.5E-01	1.7E-03		
Stack 303	1.2E-14	2.3E-01	3.2E-01	2.1E-03		
Stack 305	1.6E-14	3.2E-01	3.4E-01	2.3E-03		
Stack 401	2.0E-15	4.1E-02	2.1E-02	1.4E-04		
Stack 501	5.4E-15	1.1E-01	9.8E-02	6.5E-04		
Total Quantity Released	1.3 μCi					
Fraction of Quantity Limit	8.8E-03					

III. Conclusion

ATTACHMENT 8 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2006, Through June 30, 2006

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2006, Through June 30, 2006

I. Introduction

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from January 1, 2006, through June 30, 2006. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from January 1, 2006, through June 30, 2006, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table notes, it is estimated that approximately 223g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 16 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 22 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1
Liquid Effluent Monitoring Summary Data

	Gross Alpha Radioactivity			Gross Beta Radioactivity		
Reporting Period			Period			Period
Reporting refrou	Average	Average	Quantity	Average	Average	Quantity
January – June, 2006	Conc.	Fraction	Discharged	Conc.	Fraction	Discharged
	(μCi/ml)	of Limit	(Ci)	(μCi/ml)	of Limit	(Ci)
Site Dam	1.7E-09	5.7E-03	4.4E-04	5.4E-09	1.1E-03	1.4E-03
Sewage Treat. Outfall	4.7E-08	1.6E-01	5.9E-06	8.0E-08	1.6E-02	1.0E-05
Total Quantity of Gross						
Alpha/Beta Discharged			4.5E+2 μCi			1.4E+3 μCi
Total Quantity of						
Uranium Discharged Note 1			223 g			
Maximum Concentration	Sewa	ge Treatmer	nt Outfall			
for Sampling Period		(µCi/ml))	Si	te Dam (μC	Ci/ml)
Gross Alpha (Dissolved)	6.5E-08	Max. Frac	tion of Limit	4.8E-09	Max. Frac	tion of Limit
Gross Alpha (Suspended)	1.0E-09	2.2E-0)1 (alpha)	5.2E-10	1.8E-0)2 (alpha)
Gross Beta (Dissolved)	1.3E-07	2.6E-	02 (beta)	7.1E-09	1.7E-	03 (beta)
Gross Beta (Suspended)	3.1E-09		,	1.2E-09		
U-233/234	1.6E-07			Note 2		
U-235	5.4E-09			Note 2		
U-238	2.5E-08			Note 2		

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 μ Ci/g, which is representative of the enrichments present during this period.

Note 2: Consistent with the guidance provided in Regulatory Guide 4.16, isotopic analysis was not performed for samples collected from the Site Dam during these reporting periods in consideration of the operational knowledge regarding radionuclide composition; and the low individual sample concentrations which did not exceed ten percent of the annual effluent limit.

B. Gaseous (Airborne) Effluents

There was no measurable radioactivity released to unrestricted areas in airborne effluents from January 1, 2006, through June 30, 2006. Consistent with the then current stage of facility decommissioning, there were no activities performed during the reporting period that had any reasonable potential for significant airborne effluents. It is noted that during the reporting period the only air effluent sampling requirements remaining in the Hematite Effluent Control and Monitoring Program are for building exhaust stacks, and these stacks have been abandoned and are not in use. Thus, there was no air sampling required or performed during the reporting period as part of the Hematite Effluent Control and Monitoring Program.

Notwithstanding the above, the results of air sampling performed as part of the Hematite Environmental Monitoring Program confirm that the average air concentrations at the sampling locations were within the statistical range of background levels. As indicated in Table 2 below, the highest average concentration measured in environmental air samples was less than the minimum detectable concentration for the analysis (established at 10 percent of the annual effluent limit). Based on these results and the fact that there were no activities performed during the reporting period that had any reasonable potential for significant airborne effluents, it is concluded that there was no measurable quantity of uranium released to unrestricted areas in airborne effluents during the reporting period.

Table 2

Environmental Monitoring Program Air Sampling Summary Data

Reporting Period January – June, 2006	Average Concentration (µCi/ml)	Average Fraction of Limit
AS-1	3.1E-15	0.06
AS-2	1.9E-15	0.04
AS-3	1.4E-15	0.03
AS-4	1.2E-15	0.02

¹ By Westinghouse Electric Company LLC Letter No. HEM-08-8 dated February 15, 2008, Westinghouse submitted a license amendment request to the NRC. This request included changes to remove the requirements for exhaust stack air effluent sampling that no longer apply, and to add air effluent control and monitoring requirements that are considered conservatively appropriate for any potential air effluents resulting from remaining decommissioning activities. These include work control and discrete point effluent sampling requirements designed to ensure that once active decommissioning activities begin, the 10 CFR 20 effluent limits and site ALARA goals are satisfied.

III. Conclusion

The effluent monitoring results summarized above confirm that quantities of radioactive materials (i.e., uranium) released from Hematite in liquid and gaseous effluents during the reporting period are significantly below 10 CFR 20 limits. Thus, the maximum potential radiation dose to the public resulting from liquid and gaseous effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

ATTACHMENT 9 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2006, Through December 31, 2006

Hematite Decommissioning Project Effluent Monitoring Report For The Period July 1, 2006, Through December 31, 2006

I. <u>Introduction</u>

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from July 1, 2006, through December 31, 2006. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from July 1, 2006, through December 31, 2006, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table notes, it is estimated that approximately 254g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 22 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 27 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1
Liquid Effluent Monitoring Summary Data

	Gross Alpha Radioactivity		Gross Beta Radioactivity			
Reporting Period			Period			Period
Reporting remod	Average	Average	Quantity	Average	Average	Quantity
July - December, 2006	Conc.	Fraction	Discharged	Conc.	Fraction	Discharged
a	(μCi/ml)	of Limit	(Ci)	(μCi/ml)	of Limit	(Ci)
Site Dam	3.4E-09	1.1E-02	5.0E-04	5.7E-09	1.1E-03	8.3E-04
Sewage Treat. Outfall	6.6E-08	2.2E-01	7.0E-06	1.2E-07	2.3E-02	1.2E-05
Total Quantity of Gross						
Alpha/Beta Discharged			5.1E+02 μCi			8.4E+02 μCi
Total Quantity of						
Uranium Discharged Note 1			254 g			
Maximum Concentration	Sewa	ge Treatmei	nt Outfall			
for Sampling Period	(μCi/ml)		Si	Site Dam (µCi/ml)		
Gross Alpha (Dissolved)	8.1E-08	8.1E-08 Max. Fraction of Limit		1.8E-08	Max. Frac	ction of Limit
Gross Alpha (Suspended)	1.1E-09	2.7E-0)1 (alpha)	2.6E-10	6.1E-0	02 (alpha)
Gross Beta (Dissolved)	1.6E-07	3.2E-	02 (beta)	1.2E-08	2.7E-	03 (beta)
Gross Beta (Suspended)	2.6E-09			1.6E-09		
U-233/234	1.3E-07			Note 2		
U-235	4.2E-09			Note 2		
U-238	1.6E-08	· · ·		Note 2	1 . 1 4 .	

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 μ Ci/g, which is representative of the enrichments present during this period.

Note 2: Consistent with the guidance provided in Regulatory Guide 4.16, isotopic analysis was not performed for samples collected from the Site Dam during these reporting periods in consideration of the operational knowledge regarding radionuclide composition; and the low individual sample concentrations which did not exceed ten percent of the annual effluent limit.

B. Gaseous (Airborne) Effluents

There was no measurable radioactivity released to unrestricted areas in airborne effluents from July 1, 2006, through December 31, 2006. Consistent with the then current stage of facility decommissioning, there were no activities performed during the reporting period that had any reasonable potential for significant airborne effluents. It is noted that the only air effluent sampling requirements remaining in the Hematite Effluent Control and Monitoring Program are

for building exhaust stacks, and these stacks have been abandoned and are not in use. Thus, there was no air sampling required or performed during the reporting period as part of the Hematite Effluent Control and Monitoring Program.

Notwithstanding the above, the results of air sampling performed as part of the Hematite Environmental Monitoring Program confirm that the average air concentrations at the sampling locations were within the statistical range of background levels. As indicated in Table 2 below, the highest average concentration measured in environmental air samples was less than the minimum detectable concentration for the analysis (established at 10 percent of the annual effluent limit). Based on these results and the fact that there were no activities performed during the reporting period that had any reasonable potential for significant airborne effluents, it is concluded that there was no measurable quantity of uranium released to unrestricted areas in airborne effluents during the reporting period.

Table 2

Environmental Monitoring Program Air Sampling Summary Data

Reporting Period	Average Concentration	
July - December, 2006	(μCi/ml)	Average Fraction of Limit
AS-1	2.5E-15	0.05
AS-2	1.5E-15	0.03
AS-3	9.0E-16	0.02
AS-4	1.5E-15	0.03
AS-5	1.2E-15	0.02

III. Conclusion

The effluent monitoring results summarized above confirm that quantities of radioactive materials (i.e., uranium) released from Hematite in liquid and gaseous effluents during the reporting period are significantly below 10 CFR 20 limits. Thus, the maximum potential radiation dose to the public resulting from liquid and gaseous effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

ATTACHMENT 10 TO WESTINGHOUSE LETTER NO. HEM-08-44

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2007, Through June 30, 2007

Hematite Decommissioning Project Effluent Monitoring Report For The Period January 1, 2007, Through June 30, 2007

I. Introduction

Pursuant to 10 CFR 70.59, this report summarizes the results of radiological effluent monitoring at the Hematite Decommissioning Project for the period from January 1, 2007, through June 30, 2007. This report includes the information specified in 10 CFR 70.59, which states in part:

The report must specify the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous six months of operation, and such other information as the Commission may require to estimate maximum potential annual radiation doses to the public resulting from effluent releases. If quantities of radioactive materials released during the reporting periods are significantly above the licensee's design objectives previously reviewed as part of the licensing action, the report must cover this specifically.

II. Effluent Monitoring Report

A. Liquid Effluents

The quantity of radioactivity released to unrestricted areas in liquid effluents from January 1, 2007, through June 30, 2007, is summarized in Table 1 below. Based on conservative and reasonable assumptions described in the table notes, it is estimated that approximately 346g of uranium was released to unrestricted areas in liquid effluents during the reporting period.

As indicated in Table 1, quantities of radioactive materials (i.e., uranium) released during the reporting period are significantly below 10 CFR 20 limits. Based on the gross activity measurements, the average activity concentration in liquid effluents was approximately 28 percent of the annual effluent limit. The maximum activity concentration, measured at the Sewage Treatment Outfall, was approximately 63 percent of the annual effluent limit based on gross activity results. This data confirms that the maximum potential radiation dose to the public resulting from liquid effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.

Table 1

Liquid Effluent Monitoring Summary Data

	Gross Alpha Radioactivity			Gross Beta Radioactivity		
Reporting Period			Period		•	Period
Reporting Ferrod	Average	Average	Quantity	Average	Average	Quantity
January - June, 2007	Conc.	Fraction	Discharged	Conc.	Fraction	Discharged
	(µCi/ml)	of Limit	(Ci)	(μCi/ml)	of Limit	(Ci)
Site Dam	1.7E-09	5.7E-03	6.6E-04	4.5E-09	9.0E-04	1.7E-03
Sewage Treat. Outfall	8.5E-08	2.8E-01	3.1E-05	1.1E-07	2.2E-02	4.0E-05
Total Quantity of Gross						
Alpha/Beta Discharged			6.9E+02 μCi			1.7E+03 μCi
Total Quantity of						
Uranium Discharged Note 1			346 g			
Maximum Concentration	Sewa	ge Treatme	nt Outfall			
for Sampling Period	(μCi/ml)			Site Dam (μCi/ml)		
Gross Alpha (Dissolved)	1.8E-07	1.8E-07 Max. Fraction of Limit		1.1E-08	Max. Frac	ction of Limit
Gross Alpha (Suspended)	6.2E-09	2E-09 6.3E-01 (alpha)		3.4E-09	4.9E-(02 (alpha)
Gross Beta (Dissolved)	2.1E-07	4.3E-	02 (beta)	7.7E-09	1.7E-	03 (beta)
Gross Beta (Suspended)	1.4E-08			9.9E-10		
U-233/234	2.3E-07			Note 2		
U-235	6.9E-09			Note 2		
U-238	3.2E-08			Note 2		

Note 1: The estimated mass of discharged uranium is conservatively based on the total gross alpha activity, and an activity concentration assumption of 2 μ Ci/g, which is representative of the enrichments present during this period.

Note 2: Consistent with the guidance provided in Regulatory Guide 4.16, isotopic analysis was not performed for samples collected from the Site Dam during these reporting periods in consideration of the operational knowledge regarding radionuclide composition; and the low individual sample concentrations which did not exceed ten percent of the annual effluent limit.

B. Gaseous (Airborne) Effluents

There was no measurable radioactivity released to unrestricted areas in airborne effluents from January 1, 2007, through June 30, 2007. Consistent with the current stage of facility decommissioning, there were no activities performed during the reporting period that had any reasonable potential for significant airborne effluents. It is noted that the only air effluent sampling requirements remaining in the Hematite Effluent Control and Monitoring Program are

for building exhaust stacks, and these stacks have been abandoned and are not in use. Thus, there was no air sampling required or performed during the reporting period as part of the Hematite Effluent Control and Monitoring Program.

Notwithstanding the above, the results of air sampling performed as part of the Hematite Environmental Monitoring Program confirm that the average air concentrations at the sampling locations were within the statistical range of background levels. As indicated in Table 2 below, the highest average concentration measured in environmental air samples was less than the minimum detectable concentration for the analysis (established at 10 percent of the annual effluent limit). Based on these results and the fact that there were no activities performed during the reporting period that had any reasonable potential for significant airborne effluents, it is concluded that there was no measurable quantity of uranium released to unrestricted areas in airborne effluents during the reporting period.

Table 2

Environmental Monitoring Program Air Sampling Summary Data

Reporting Period January – June, 2007	Average Concentration (μCi/ml)	Average Fraction of Limit
AS-1	2.1E-15	0.04
AS-2	1.0E-15	0.02
AS-3	8.5E-16	0.02
AS-4	9.7E-16	0.02
AS-5	9.0E-16	0.02

III. Conclusion

The effluent monitoring results summarized above confirm that quantities of radioactive materials (i.e., uranium) released from Hematite in liquid and gaseous effluents during the reporting period are significantly below 10 CFR 20 limits. Thus, the maximum potential radiation dose to the public resulting from liquid and gaseous effluent releases during the reporting period is well below the limits of 10 CFR 20.1301.