MISCELLANEOUS CORRESPONDENCE

FOR THE

BRAIDWOOD STATION INITIAL EXAMINATION

DECEMBER 2007

Valos

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

November 29, 2007

Mr. Charles G. Pardee
Chief Nuclear Officer (CNO) and
Senior Vice President
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville IL 60555

SUBJECT: OPERATOR LICENSING EXAMINATION APPROVAL

Dear Mr. Crane:

The purpose of this letter is to confirm the final arrangements for the upcoming operator licensing examinations at Braidwood Station.

The U. S. Nuclear Regulatory Commission (NRC) has completed its review of the operator license applications in connection with this examination and will separately provide a list of approved applicants to Mr. Gary J. Dudek, Training Director, Braidwood Station.

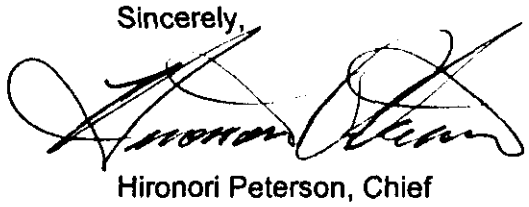
The NRC has approved the subject examinations and hereby authorizes you to administer the written examinations in accordance with Revision 9 of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," during the week of December 3 or December 10, 2007. The NRC staff will administer the operating tests during the week of December 3, 2007.

Please contact Mr. Nicholas A. Valos, at 630-829-9761, if you have any questions or identify any errors or changes in the license level (RO or SRO) or type of examination (partial or complete written examination and/or operating test) specified for each applicant.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records System (PARS) component of NRC's Agencywide Documents

Access and Management System (ADAMS), accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,



Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-456; 50-457
License Nos. NPF-72; NPF-77

cc: Site Vice President - Braidwood Station
Plant Manager - Braidwood Station
Regulatory Assurance Manager - Braidwood Station
Chief Operating Officer and Senior Vice President
Senior Vice President – Midwest Operations
Senior Vice President – Operations Support
Vice President – Licensing and Regulatory Affairs
Director Licensing Braidwood Operating Group
Manager Licensing – Braidwood, Byron and LaSalle
Associate General Counsel
Document Control Desk - Licensing
Assistant Attorney General
Illinois Emergency Management Agency
State Liaison Officer
Chairman, Illinois Commerce Commission

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/RA/

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

August 21, 2007

Mr. Christopher M. Crane
President and Chief Nuclear Officer
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Dear Mr. Crane:

In a telephone conversation on August 14, 2007, between Mr. Scott Butler, Operations Training Manager, and Mr. Nicholas Valos, Senior Operations Engineer, arrangements were made for the administration of licensing examinations at the Braidwood Station the weeks of December 3 and December 10, 2007. In addition, the NRC will make an examination validation visit to your facility the week of October 22, 2007.

As agreed during the telephone conversation, your staff will prepare the examinations based on the guidelines in Revision 9 of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors." The NRC regional office will discuss with your staff any changes that might be necessary before the examinations are administered.

To meet the above schedule, you furnished the examination outlines on August 13, 2007. The written examinations, operating tests, and the supporting reference materials identified in Attachment 2 to ES-201 will be due by October 1, 2007. Pursuant to 10 CFR 55.40(b)(3), of the Code of Federal Regulations (10 CFR 55..40(b)(3)), an authorized representative of the facility licensee shall approve the outlines, examinations, and tests before they are submitted to the NRC for review and approval. All materials shall be complete and ready to use. We request that any personal, proprietary, sensitive unclassified, or safeguards information in your response be contained in a separate enclosure and appropriately marked. Any delay in receiving the required examination and reference materials, or the submittal of inadequate or incomplete materials, may cause the examinations to be rescheduled.

In order to conduct the requested written examinations and operating tests, it will be necessary for your staff to provide adequate space and accommodations in accordance with ES-402, and to make the simulation facility available on the dates noted above. In accordance with ES-302, your staff should retain the original simulator performance data (e.g., system pressures, temperatures, and levels) generated during the dynamic operating tests until the examination results are final.

Appendix E of NUREG-1021 contains a number of NRC policies and guidelines that will be in effect while the written examinations and operating tests are being administered.

To permit timely NRC review and evaluation, your staff should submit preliminary reactor operator and senior reactor operator license applications (Office of Management and Budget

(OMB) approval number 3150-0090), medical certifications (OMB approval number 3150-0024), and waiver requests (if any) (OMB approval number 3150-0090) at least 30 days before the first examination date. If the applications are not received at least 30 days before the examination date, a postponement may be necessary. Signed applications certifying that all training has been completed should be submitted at least 14 days before the first examination date.

This letter contains information collections that are subject to the *Paperwork Reduction Act of 1995* (44 U.S.C. 3501 et seq.). These information collections were approved by the Office of Management and Budget, approval number 3150-0018, which expires on June 30, 2009. The public reporting burden for this collection is estimated to average 500 hours per response, including the time for reviewing instructions, gathering and maintaining the data needed, and completing and reviewing the collection of information. Send comments on any aspect of this collection of information, including suggestions for reducing the burden, to the Information and Records Management Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, D.C. 20555-0001, or by Internet electronic mail at BJS1@nrc.gov; and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0018), Office of Management and Budget, Washington, D.C. 20503-0001.

The NRC may neither conduct nor sponsor, and a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

Thank you for your cooperation in this matter. Mr. Butler has been advised of the policies and guidelines referenced in this letter. If you have any questions regarding the NRC's examination procedures and guidelines, please contact Nicholas Valos at 630-829-9761, or me at 630-829-9707.

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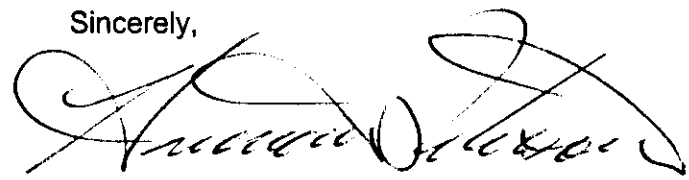

Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-456; 50-457
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Docket Nos. 50-456; 50-457
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Letter to Christopher M. Crane from Hironori Peterson dated August 21, 2007.

SUBJECT: ARRANGEMENTS FOR THE ADMINISTRATION OF LICENSING EXAMINATIONS AT THE BRAIDWOOD STATION

cc: Site Vice President - Braidwood Station
Plant Manager - Braidwood Station
Regulatory Assurance Manager - Braidwood Station
Chief Operating Officer
Senior Vice President - Nuclear Services
Vice President - Operations Support
Vice President - Licensing and Regulatory Affairs
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G. Dudek, Training Manager

Exelon Generation Company, LLC
Braidwood Station
35100 South Rt 53, Suite 84
Braceville, IL 60407-9619
Tel: 815-417-2000

www.exeloncorp.com

Exelon Nuclear

July 2, 2007
BW070046

Nicholas Valos
Lead Examiner
U. S. Nuclear Regulatory Commission
Region III
2443 Warrenville Road
Suite 210
Lisle, IL 60532-4351

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. STN 50-456 and STN 50-457

Subject: Submittal of Knowledge and Abilities (K/A) Statements That Will Be Suppressed from the Random Exam Generation Process

In accordance with NUREG 1021, Revision 9, "Operator Licensing Examination Standards for Power Reactors", Braidwood Station is submitting for your review the list of K/A statements that will be suppressed from the random exam generation process in support of our December 3, 2007 and December 10, 2007 license exam.

Should you have any questions concerning this letter, please contact James Petty, Acting Regulatory Assurance Manager, at (815) 417-2815.

Respectfully,

Thomas Coutu

Tom Coutu
Site Vice President
Braidwood Station

Enclosures: Braidwood Station Suppressed K/A statements

AUG 3 2007

Braidwood Suppressed KAs

Viewed KA	*Category Statement*	*KA Statement*	*RO Value*	*SRO Value*	*Suppress Basis*
000001	*Continuous Rod Withdrawal*				
AK1.14	Knowledge of the following theoretical concepts as they apply to the Continuous Rod Withdrawal	Interaction of ICS control stations as well as purpose, function, and modes of operation of ICS	3.4	3.7	Not applicable to Braidwood.
000003	*Dropped Control Rod*				
AA1.04	Ability to operate and / or monitor the following as they apply the Dropped Control Rod:	Control rod drive safety rod out limit bypass switch or key	3.4*	3.3	Not applicable to Braidwood.
AK1.13	Knowledge of the following theoretical concepts as they apply to the Dropped Control Rod emergency task:	Interaction of ICS control stations as well as purpose, function, and modes of operation of ICS	3.2	3.6	Not applicable to Braidwood.
AK2.03	Knowledge of the following components:	Metroscope	3.1	3.2	Not applicable to Braidwood.
AK3.01	Knowledge of the reasons for the following Responses as they apply to the Dropped Control Rod:	When ICS logic has failed on a dropped rod, the load must be reduced until flux is within specified target Bank	3.5*	3.9*	Not applicable to Braidwood.
AK3.03	Knowledge of the reasons for the following Responses as they apply to the Dropped Control Rod:	Turbine automatic runback with reactor in order to balance power output	3.4*	3.7*	Not applicable to Braidwood.
AK3.04	Knowledge of the bases or reasons for the following:	Actions contained in EOP for dropped control rod	3.8	4.1	Not applicable to Braidwood.
000005	*Inoperable/Stuck Control Rod*				
AA1.03	Ability to operate and monitor the following:	Metroscope	3.4	3.4	Not applicable to Braidwood.
AA2.02	Ability to determine or interpret:	Difference between jog and run rod speeds, effect on CRDM of stuck rod	2.5	3.0	Not applicable to Braidwood.
AK1.04	Knowledge of the following theoretical concepts as they apply to the inoperable/stuck control rod emergency task:	Definitions of axial imbalance, neutron error, power demand, actual power tracking mode, ICS	3.0	3.4	Not applicable to Braidwood.
AK2.03	Knowledge of the following components:	Metroscope	3.1	3.3	Not applicable to Braidwood.
000008	*Pressurizer (PZR) Vapor Space Accident (Relief Valve Stuck Open)*				
AA2.04	Ability to determine or interpret:	High-temperature computer alarm and alarm type	3.2	3.4	Not applicable to Braidwood.
AA2.09	Ability to determine or interpret:	PZR spray block valve controls and indicators	3.6	3.7	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
000009	*Small Break LOCA*				
EA1.03	Ability to operate and monitor the following:	Low-pressure SWS activity monitor	3.2	3.2	Not applicable to Braidwood.
EA1.18	Ability to operate and monitor the following:	Balancing of HPI loop flows	3.4	3.2	Not applicable to Braidwood.
EA2.09	Ability to determine or interpret:	Low-pressure SWS activity monitor	2.8	3.3	Not applicable to Braidwood.
EA2.22	Ability to determine or interpret:	Charging flow trend recorder	3.0*	3.3*	Not applicable to Braidwood.
EA2.35	Ability to determine or interpret:	Conditions for throttling or stopping reflux boiling spray	3.4	4.1	Not applicable to Braidwood.
EK3.25	Knowledge of the bases or reasons for the following:	Monitoring of in-core T-cold	3.6	3.9	Not applicable to Braidwood.
EK3.27	Knowledge of the bases or reasons for the following:	Manual depressurization or HPI recirculation for sustained high pressure	3.6	3.8	Not applicable to Braidwood.
000011	*Large Break LOCA*				
EA1.02	Ability to operate and monitor the following:	Reflux boiling sump level indicators	3.8	4.1	Not applicable to Braidwood.
EA1.09	Ability to operate and monitor the following:	Core flood tank initiation	4.3	4.3	Not applicable to Braidwood.
EA1.16	Ability to operate and monitor the following:	Balancing of HPI loop flows	3.5	3.5	Not applicable to Braidwood.
EA2.11	Ability to determine or interpret:	Conditions for throttling or stopping HPI	3.9	4.3	Not applicable to Braidwood.
EA2.12	Ability to determine or interpret:	Conditions for throttling or stopping reflux boiling spray	3.6	3.8	Not applicable to Braidwood.
EK3.07	Knowledge of the bases or reasons for the following:	Stopping charging pump bypass flow	3.5	3.6	Not applicable to Braidwood.
000015/17	*Reactor Coolant Pump (RCP) Malfunctions*				
AA1.04	Ability to operate and monitor the following:	RCP ventilation cooling fan run indicators	2.5	2.5	Not applicable to Braidwood.
AA1.19	Ability to operate and monitor the following:	Power transfer confirm lamp	2.9	3.0	Not applicable to Braidwood.
AA2.09	Ability to determine or interpret:	When to secure RCPs on high stator temperatures	3.4	3.5	Braidwood has no procedural requirement to stop RCPs based on Stator Temperature.
AK1.03	Knowledge of the following theoretical concepts as they apply to the RCP malfunctions emergency task:	The basis for operating at a reduced power level when one RCP is out of service	3.0	4.0	Not applicable to Braidwood.
AK3.04	Knowledge of the bases or reasons for the following:	Reduction of power to below the steady state power-to-flow limit	3.1	3.2	Not applicable to Braidwood.
AK3.05	Knowledge of the bases or reasons for the following:	Shift of T-ave sensors to the loop with the highest flow	2.8*	3.0*	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
000024	*Emergency Boration*				
AA1.01	Ability to operate and monitor the following:	Use of spent fuel pool as backup to BWST	2.7*	3.4*	Not applicable to Braidwood.
AA1.08	Ability to operate and monitor the following:	Pump speed controlled to protect pump seals	2.7	3.0	Not applicable to Braidwood.
AA1.11	Ability to operate and monitor the following:	BIT suction and recirculation valves	2.9	2.7	Not applicable to Braidwood.
AA1.24	Ability to operate and monitor the following:	BIT inlet and outlet valve switches and indicators	3.2	3.1	Not applicable to Braidwood.
000025	*Loss of Residual Heat Removal System (RHRS)*				
AA1.05	Ability to operate and monitor the following:	Raw water or sea water pumps	2.7	2.6	Not applicable to Braidwood.
AA1.19	Ability to operate and monitor the following:	Block orifice bypass valve controllers and indicators	2.6*	2.4*	Not applicable to Braidwood.
AA1.22	Ability to operate and monitor the following:	Obtaining of water from BWST for LPI system	2.9	2.8	Not applicable to Braidwood.
AA2.05	Ability to determine or interpret:	Limitations on LPI flow and temperature rates of change	3.1	3.5	Not applicable to Braidwood.
AK2.04	Knowledge of the following components:	Raw water or sea water pumps	2.4	2.4	Not applicable to Braidwood.
000026	*Loss of Component Cooling Water (CCW)*				
AA1.03	Ability to operate and monitor the following:	SWS as a backup to the CCWS	3.6*	3.6*	Not applicable to Braidwood.
AA1.04	Ability to operate and monitor the following:	CRDM high-temperature alarm system	2.7	2.8	Not applicable to Braidwood.
AK3.01	Knowledge of the bases or reasons for the following:	The conditions that will initiate the automatic opening and closing of the SWS isolation valves to the CCW/nuclear service water coolers	3.2	3.5	Not applicable to Braidwood.
000027	*Pressurizer Pressure Control System (PZR PCS) Malfunction*				
AA1.04	Ability to operate and monitor the following:	Pressure recovery, using emergency-only heaters	3.9*	3.6*	Not applicable to Braidwood.
AA1.05	Ability to operate and monitor the following:	Transfer of heaters to backup power supply	3.3*	3.2*	Not applicable to Braidwood.
AA2.13	Ability to determine or interpret:	Seal return flow	2.8	2.9	Seal Return flow not impacted by PZR pressure changes Braidwood.
000028	*Pressurizer (PZR) Level Control Malfunction*				
AK3.04	Knowledge of the reasons for the following responses as they apply to the Pressurizer Level Control Malfunctions:	Change in PZR level with power change, even though RCS T-ave. constant, due to loop size difference	2.9	3.0	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
000029	*Anticipated Transient Without Scram (ATWS)*				
EA1.04	Ability to operate and monitor the following:	BIT inlet valve switches	3.9	3.8	Not applicable to Braidwood.
EA1.05	Ability to operate and monitor the following:	BIT outlet valve switches	3.7	3.6	Not applicable to Braidwood.
EA2.10	Ability to determine or interpret:	Positive displacement charging pumps	3.1	3.4	Not applicable to Braidwood.
EK3.03	Knowledge of the bases or reasons for the following:	Opening BIT inlet and outlet valves	3.7	3.6	Not applicable to Braidwood.
EK3.04	Knowledge of the bases or reasons for the following:	Closing the normal charging header isolation valves	3.1	3.1	Not applicable to Braidwood.
EK3.05	Knowledge of the bases or reasons for the following:	Closing the centrifugal charging pump recirculation valve	3.4	3.5	Not applicable to Braidwood.
000037	*Steam Generator (S/G) Tube Leak*				
AA1.03	Ability to operate and/or monitor the following as they apply to the Steam Generator Tube Leak	Loop isolation valves	3.0*	2.9	Not applicable to Braidwood
AK3.01	Knowledge of the reasons for the following responses as they apply to the Steam Generator Tube Leak:	Collection of Condensate in air ejector monitor due to its failure	2.3	2.6	Not applicable to Braidwood
AK3.04	Knowledge of the reasons for the following responses as they apply to the Steam Generator Tube Leak:	Use of "feed and bleed" process	2.5	2.9	Not applicable to Braidwood
000038	*Steam Generator Tube Rupture (SGTR)*				
EK3.07	Knowledge of the reasons for the following responses as they apply to the SGTR	RCS loop isolation valves	3.4*	3.8	Not applicable to Braidwood
000040	*Steam Line Rupture*				
AA1.21	Ability to operate and / or monitor the following as they apply to the Steam Line Rupture:	Vibration alarm	2.3*	2.5	Not applicable to Braidwood.
AA1.22	Ability to operate and / or monitor the following as they apply to the Steam Line Rupture:	Load sequencer status lights	3.0*	3.0*	Not applicable to Braidwood.
000056	*Loss of Offsite Power*				
AA1.15	Ability to operate and monitor the following:	Service water booster pump	2.7*	2.9*	Not applicable to Braidwood.
AA1.20	Ability to operate and monitor the following:	Speed switch room ventilation fan	3.0	3.0	Not applicable to Braidwood.
AA2.02	Ability to determine or interpret:	ESF load sequencer status lights	3.5*	3.6*	Not applicable to Braidwood.
AA2.11	Ability to determine or interpret:	Operational status of service water booster pump	2.9*	2.9*	Not applicable to Braidwood.
AA2.29	Ability to determine or interpret:	Service water booster pump ammeter and flowmeter	3.0*	3.2*	Not applicable to Braidwood.
AA2.38	Ability to determine or interpret:	Load sequencer status lights	3.7*	3.8	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
000057	*Loss of Vital AC Electrical Instrument Bus*				
AA2.02	Ability to determine or interpret:	Core flood tank pressure and level indicators	3.7	3.8	Not applicable to Braidwood.
AA2.08	Ability to determine or interpret:	Reactor power digital display and remote flux meter	3.4*	3.5*	Not applicable to Braidwood.
000062	*Loss of Nuclear Service Water*				
AA1.03	Ability to operate and monitor the following:	SWS as a backup to the CCWS	2.7	2.8	Not applicable to Braidwood.
AA1.04	Ability to operate and monitor the following:	CRDM high-temperature alarm system	3.6*	3.6	Not applicable to Braidwood.
AK3.01	Knowledge of the reasons for the following responses:	The conditions that will initiate the automatic opening and closing of the SWS isolation valves to the nuclear nuclear service water coolers	3.2*	3.5*	Not applicable to Braidwood.
000065	*Loss of Instrument Air*				
AA2.02	Ability to determine or interpret:	Relationship of flow readings to system operation	2.4*	2.6*	Not applicable to Braidwood.
000067	*Plant Fire on Site*				
AK3.04	Knowledge of the reasons for the following responses:	Actions contained in EOP for plant fire on site	3.3	4.1	Not applicable to Braidwood.
AA2.10	Ability to determine or interpret:	Time limit of long-term breathing air system for control room	2.9*	3.6*	Not applicable to Braidwood.
000068	*Control Room Evacuation*				
AA1.09	Ability to operate and monitor the following:	Synchroscope key	3.1*	2.7*	Not applicable to Braidwood.
AA1.20	Ability to operate and monitor the following:	Indicators for operation of startup transformer	3.2	3.2	Not applicable to Braidwood.
AK3.04	Knowledge of the reasons for the following responses:	Filling the feedwater system and closing the AFW pump discharge valve	3.0*	3.2*	Not applicable to Braidwood.
AK3.05	Knowledge of the reasons for the following responses:	Repositioning valves to isolate and drain the AFW pump turbine and steam supply header	3.5	3.6	Not applicable to Braidwood.
AK3.16	Knowledge of the reasons for the following responses:	Fail-open of the control room doors for personnel evacuation	2.8	3.3	Not applicable to Braidwood.
000074	*Inadequate Core Cooling*				
EA1.03	Ability to operate and monitor the following:	The alternate control station for turbine bypass valve operation	3.9	3.9	Not applicable to Braidwood.
EA1.08	Ability to operate and monitor the following:	HPI System	4.2	4.2	Not applicable to Braidwood.
EA1.14	Ability to operate and monitor the following:	Alarm for loss of subcooling margin	4.1	4.2	Not applicable to Braidwood.

Viewed KA	*Category Statement*	*KA Statement*	*RO Value*	*SRO Value*	*Suppress Basis*
000076	*High Reactor Coolant Activity*				
AK3.02	Knowledge of the reasons for the following responses:	Increased CCW flow	2.4	2.6	Not applicable to Braidwood.
001000	*Control Rod Drive System*				
A1.08	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CRDS controls including:	Verification that CRDS temperatures are within limits before starting	2.6	3.0	Not applicable to Braidwood.
A1.10	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CRDS controls including:	Location and operation of controls and indications for CRDS component cooling water	2.9	2.7	Not applicable to Braidwood.
A1.11	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CRDS controls including:	Required primary system subcooling during shutdown; location of indication	3.7	3.9	Not applicable to Braidwood.
A1.13	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CRDS controls including:	"Prepower dependent insertion limit" and power dependent insertion limit, determined with metroscope	4.0?	4.2?	Not applicable to Braidwood.
A2.04	Ability to (a) predict the impacts of the following malfunction or operations on the CRDS- and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Positioning of axial shaping rods and their effect on SDM	3.2*	3.8*	Not applicable to Braidwood.
A2.08	Ability to (a) predict the impacts of the following malfunction or operations on the CRDS- and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Loss of CCW to CRDS	2.9	3.3	Not applicable to Braidwood.
A2.20	Ability to (a) predict the impacts of the following malfunction or operations on the CRDS- and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Isolation of Left coil on affected rod to prevent coil burnout	2.6	3.6	Not applicable to Braidwood.
A4.01	Ability to manually operate and/or monitor in the control room:	Controls for CCWS	3.1	2.9	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
001000	*Control Rod Drive System (Continued)*				
A4.04	Ability to manually operate and/or monitor in the control room:	Part-length rod position	3.9	3.6	Not applicable to Braidwood.
A4.07	Ability to manually operate and/or monitor in the control room:	Power source transfer check	3.3	3.3	Not applicable to Braidwood.
A4.09	Ability to manually operate and/or monitor in the control room:	CCWS	2.8	3.1	Not applicable to Braidwood.
A4.12	Ability to manually operate and/or monitor in the control room:	Stopping T/G load changes; only make minor adjustments to prevent coil burnout	2.9*	2.9	Not applicable to Braidwood.
A4.13	Ability to manually operate and/or monitor in the control room:	Stopping other changes in plant, e.g., turbine, S/G, SDBCS, boration, before adjusting rods	2.7*	2.9*	Not applicable to Braidwood.
A4.14	Ability to manually operate and/or monitor in the control room:	Resetting rod control logic while recovering from misaligned rod, using instrument Tech-Specs .	3.0	3.4	Not applicable to Braidwood.
A4.15	Ability to manually operate and/or monitor in the control room:	Stopping boration/dilution or other means of reactivity change while adjusting either rod position or T-ave .*	3.1*	3.1*	Not applicable to Braidwood.
K1.01	Knowledge of the physical connections and/or cause-effect relationships between the CRDS and the following systems:	CCW	3.0	3.2	Not applicable to Braidwood.
K1.09	Knowledge of the physical connections and/or cause-effect relationships between the CRDS and the following systems:	CCWS must be cut in before energizing CRDS	2.8	3.1	Not applicable to Braidwood.
K4.04	Knowledge of CRDS design feature(s) and/or interlock(s) which provide for the following:	Circuitry and principle of operation for LVDT or reed switch	2.5	2.8	Not applicable to Braidwood.
K5.11	Knowledge of the following theoretical concepts as they apply to the CRDS:	Reason for maintaining cross-tie breaker between between rod drive M/G sets; reliability of control rod drive trip breakers during operation of one M/G set	2.4	2.9	Not applicable to Braidwood.
K5.12	Knowledge of the following theoretical concepts as they apply to the CRDS:	Effects on power of inserting axial shaping rods	3.4	4.1	Not applicable to Braidwood.
K5.71	*Knowledge of the following theoretical concepts* as they apply to the CRDS:	*Relationship between reactivity worth of* power-shaping control rod group and other control rod groups (power-shaping, or part-length, rods have much less reactivity than full-length control	3.1	3.6	Not applicable to Braidwood.
K5.76	Knowledge of the following theoretical concepts as they apply to the CRDS:	Effects on power of inserting axial shaping rods	3.3*	3.7*	Not applicable to Braidwood.
K5.79	Knowledge of the following theoretical concepts as they apply to the CRDS:	Effects of positioning of axial shape rods on SDM	3.0*	3.6*	Not applicable to Braidwood.
K5.98	Knowledge of the following theoretical concepts *as they apply to the* CRDS:	Effect of adding high or low boron concentration to *maintain* T-ave. equal to T-ref	3.4	3.8	Not applicable to Braidwood.
K6.09	Knowledge of the effect of a loss or malfunction on the following CRDS components:	Purpose and operation of neutron flux recorder at high speed concentration	2.9*	2.9*	Not applicable to Braidwood.
002000	*Reactor Coolant System (RCS)*				
A3.02	Ability to monitor automatic operation of the RCS, including:	Containment sound-monitoring system	2.6	2.8	Not applicable to Braidwood.
A4.04	Ability to manually operate and/or monitor in the control room:	The filling/draining of LPI pumps during refueling	2.8	2.6	Not applicable to Braidwood.
A4.05	Ability to manually operate and/or monitor in the control room:	The HPI system when it is used to refill the refueling cavity	2.8	2.7	Not applicable to Braidwood.
K5.16	Knowledge of the following theoretical concepts as they apply to the RCS:	Reason for automatic features of the Feedwater control system during total loss of reactor coolant	3.5	4.0	Not applicable to Braidwood.

Viewed KA	*Category Statement*	*KA Statement*	*RO Value*	*SRO Value*	*Suppress Basis*
		flow			
003000	*Reactor Coolant Pump System (RCPS)*				
K3.05	Knowledge of the effect that a loss of the RCPS will have on the following:	ICS	3.6	3.7	Not applicable to Braidwood.
004000	*Chemical and Volume Control System (CVCS)*				
A2.08	Ability to (a) predict the impacts of the following malfunctions or operations on the CVCS; and (b) based on those predictions, use procedures to correct, control,or mitigate the consequences of those malfunctions or operations:	Loss of heat tracing	3.0	3.7	Not applicable to Braidwood.
A2.24	Ability to (a) predict the impacts of the following malfunctions or operations on the CVCS; and (b) based on those predictions, use procedures to correct, control,or mitigate the consequences of those malfunctions or operations:	Isolation of both letdown filters at one time: down-stream relief lifts	2.8	2.8	Not applicable to Braidwood.
A2.33	Ability to (a) predict the impacts of the following malfunctions or operations on the CVCS; and (b) based on those predictions, use procedures to correct, control,or mitigate the consequences of those malfunctions or operations:	Fact that isolating cation demineralizer stops boron dilution and enables restoration of normal boron concentration	2.7	3.3	Not applicable to Braidwood.
A4.20	Ability to manually operate and/or monitor in the control room:	Deborating demineralizer selector valve and selector valve control switch	2.6	2.5	Not applicable to Braidwood.
A4.22	Ability to manually operate and/or monitor in the control room:	Boronometer chart recorder	2.5*	2.5*	Not applicable to Braidwood.
K1.09	Knowledge of the physical connections and/or cause-effect relationships between the CVCS and the following systems:	Relationship between CVCS and RPIS	2.2	2.7	Not applicable to Braidwood.
K1.25	Knowledge of the physical connections and/or cause-effect relationships between the CVCS and the following systems:	Interface between HPI flow path and excess letdown flow path	2.7*	3.2*	Not applicable to Braidwood.
K5.31	Knowledge of the operational implications of: the following concepts as they apply to the CVCS	Purpose of flow path around boric acid storage tank	3.0*	3.4	Not applicable to Braidwood.
K5.32	Knowledge of the operational implications of: the following concepts as they apply to the CVCS	Purpose and control of heat tracing (prevent crystallization)	3.1	3.4	Not applicable to Braidwood.
K5.33	Knowledge of the operational implications of: the following concepts as they apply to the CVCS	Use of a boronometer	2.3*	2.6	Not applicable to Braidwood.
K6.12	Knowledge of the effect of a loss or malfunction on the following CVCS components:	Principle of recirculation valve: (permit emergency flow even if valve is blocked by crystallized boric acid)	3.1	3.4	Not applicable to Braidwood.
005000	*Residual Heat Removal System (RHRs)*				
K4.12	Knowledge of RHRS design feature(s) and/or n interlock(s) which provide or the following:	Lineup for piggyback mode with CSS	3.1*	3.7*	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
006000	*Emergency Core Cooling System (ECCS)*				
A2.07	Ability to (a) predict the impacts of the following malfunctions or operations on the ECCS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Loss of heat tracing	2.8	3.1	Not applicable to Braidwood.
A2.09	Ability to (a) predict the impacts of the following malfunctions or operations on the ECCS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Radioactive release from venting RWST to atmosphere	2.6	3.2*	Not applicable to Braidwood.
A4.03	Ability to manually operate and/or monitor in the control room:	Transfer from boron storage tank to boron injection tank	3.5	3.5	Not applicable to Braidwood.
K1.07	Knowledge of the physical connections and/or cause-effect relationships between the ECCS and the following systems:	MFW System	2.9	3.3	Not applicable to Braidwood.
K1.10	Knowledge of the physical connections and/or cause-effect relationships between the ECCS and the following systems:	Safety injection tank heating system	2.6	2.8	Not applicable to Braidwood.
K2.03	Knowledge of bus power supplies to the	Heat tracing	2.3	2.5	Not applicable to Braidwood.
K4.19	Knowledge of ECCS design features(s) and/or interlock(s) which provide for the following:	Interlocks to storage tank makeup valve	3.0	3.1	Not applicable to Braidwood.
K4.20	Knowledge of ECCS design features(s) and/or interlock(s) which provide for the following:	Automatic closure of common drain line and fill valves to accumulator	3.2*	3.5*	Not applicable to Braidwood.
K4.29	Knowledge of ECCS design features(s) and/or interlock(s) which provide for the following:	BIT recirculation	2.5*	2.9*	Not applicable to Braidwood.
007000	*Pressurizer Relief Tank/Quench Tank System (PRTS)*				
A2.07	Knowledge of bus power supplies to the following:	Heat tracing	2.7	3.2	Not applicable to Braidwood.
A4.04	Ability to manually operate and/or monitor in the control room:	PZR vent valve	2.6	2.6	Not applicable to Braidwood.
K5.02	Knowledge of the operational implications of the following concepts as they apply to PRTS:	Method of forming a steam bubble in the PZR	3.1	3.4	Braidwood method of forming PZR bubble has no impact on PRT.
008000	*Component Cooling Water System (CCWS)*				
A4.11	Ability to manually operate and/or monitor in the control room:	CCW pump recirculation valve and its three-way control switch	3.0*	2.9*	Not applicable to Braidwood.
K3.02	Knowledge of the effect that a loss of malfunction of the CCWS will have on the following	CRDS	2.9	3.1	Not applicable to Braidwood.
010000	*Pressurizer Pressure Control System (PZR PCS)*				
K4.02	Knowledge of PZR PCS design feature(s) and/or interlock(s) which provide for the following:	Prevention of uncovering PZR heaters	3.0	3.4	prevention of uncovering pressurizer heaters is covered by level control circuitry and is addressed by K/A 011000K4.01

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
011000	*Pressurizer Level Control System (PZR LCS)*				
A2.08	Ability to (a) predict the impacts of the following malfunctions or operations on the PZR LCS and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Loss of level compensation	2.6	2.8	Not applicable to Braidwood.
K1.05	Knowledge of the physical connections and/or cause-effect relationships between the PZR LCS and the following systems:	Reactor regulating system	3.4	3.5	Not applicable to Braidwood.
K4.03	Knowledge of the PZR LCS design feature(s) and/or interlock(s) which provide for the following:	Density compensation of PZR level	2.6	2.9	Not applicable to Braidwood.
K6.01	Knowledge of the effect of a loss or malfunction on: the following will have on the PZR LCS:	Reasons for starting charging pump while increasing letdown flow rate	2.8*	3.2*	Not applicable to Braidwood.
012000	*Reactor Protection System*				
K6.07	Knowledge of the applicable performance and design attributes of the following RPS components:	Core protection calculator	2.9	3.2	Not applicable to Braidwood.
K6.08	Knowledge of the applicable performance and design attributes of the following RPS components:	COLSS	3.6*	3.7*	Not applicable to Braidwood.
K6.09	Knowledge of the applicable performance and design attributes of the following RPS components:	CEAC	3.6	3.7	Not applicable to Braidwood.
013000	*Engineered Safety Features Actuation System (ESFAS)*				
A1.03	Ability to predict and/or monitor changes in Parameters (to prevent exceeding design limits) Associated with operating the ESFAS controls Including:	Feedwater header differential	2.6	2.6	Not applicable to Braidwood.
K4.05	Knowledge of ESFAS design feature(s) and/or interlock(s) which provide for the following:	Core spray actuation signal reset	4.0	4.2	Not applicable to Braidwood.
K4.14	Knowledge of ESFAS design feature(s) and/or interlock(s) which provide for the following:	Upper head injection accumulator isolation	3.7	4.0	Not applicable to Braidwood.
K4.24	Knowledge of ESFAS design feature(s) and/or interlock(s) which provide for the following:	Reason for disabling of BIT so it will not function during ESF sequencer test	3.0	3.1	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
014000	*Rod Position Indication System (RPIS)*				
A1.01	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the RPIS controls including:	Metroscope reed switch display	2.9	3.1	Not applicable to Braidwood.
A2.06	Ability to (a) predict the impacts of the following malfunctions or operations on the RPIS and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Loss of LVDT	2.6	3.0	Not applicable to Braidwood.
A2.07	Ability to (a) predict the impacts of the following malfunctions or operations on the RPIS and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Loss of reed switch	2.6	2.9	Not applicable to Braidwood.
A4.03	Ability to manual operate and/or monitor in the control room:	Primary coil voltage measurement	2.6*	2.7*	Not applicable to Braidwood.
K4.01	Knowledge of RPIS design feature(s) and/or interlock(s) which provide for the following:	Upper electrical limit	2.5	2.7	Not applicable to Braidwood.
K4.02	Knowledge of RPIS design feature(s) and/or interlock(s) which provide for the following:	Lower electrical limit	2.5	2.7	Not applicable to Braidwood.
K4.05	Knowledge of RPIS design feature(s) and/or interlock(s) which provide for the following:	Rod hold interlocks	3.1	3.3	Not applicable to Braidwood.
K6.03	Knowledge of the applicable performance and design attributes of the following RPIS components:	Metroscope	2.1	2.6	Not applicable to Braidwood.
015000	*Nuclear Instrumentation System*				
K1.05	Knowledge of the physical connections and/or cause-effect relationships between the NIS and the following systems:	ICS	3.9	3.9	Not applicable to Braidwood.
K1.06	Knowledge of the physical connections and/or cause-effect relationships between the NIS and the following systems:	Reactor regulating system	3.1	3.4	Not applicable to Braidwood.
K3.04	Knowledge of the effect that a loss of the NIS will have on the following:	ICS	3.4	4.0	Not applicable to Braidwood.
K3.06	Knowledge of the effect that a loss of the NIS will have on the following:	Reactor regulating system	2.9	3.2	Not applicable to Braidwood.
K4.04	Knowledge of NIS design feature(s) and/or interlock(s) which provide for the following:	Slow response time of SPNDs	3.4	3.6	Not applicable to Braidwood.
022000	*Containment Cooling System (CCS)*				
K1.02	Knowledge of the physical connections and/or cause-effect relationships between the CCS and the following systems:	SEC/remote monitoring systems	3.7	3.5	Not applicable to Braidwood.
K4.04	Knowledge of CCS design feature(s) and/or interlock(s) which provide for the following:	Cooling of control rod drive motors	2.8	3.1	Not applicable to Braidwood.
K4.05	Knowledge of CCS design feature(s) and/or interlock(s) which provide for the following:	Containment cooling after LOCA destroys ventilation ducts	2.6*	2.7	Not applicable to Braidwood.

Viewed KA 025000	Category Statement Ice Condenser System	KA Statement	RO Value	SRO Value	Suppress Basis
A1.01	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the Ice Condenser System controls including:	Temperature chart recorders	3.0	3.0	Not applicable to do not have Ice Condensers
A1.02	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the Ice Condenser System controls including:	Glycol expansion tank level	2.5	2.2	Not applicable to Braidwood.
A1.03	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the Ice Condenser System controls including:	Glycol flow to ice condenser air handling units	2.5	2.5	Not applicable to Braidwood.
A2.01	Ability to (a) predict the impacts of the following malfunctions or operations on the Ice Condenser System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Trip of glycol circulation pumps	2.2	2.7	Not applicable to Braidwood.
A2.02	Ability to (a) predict the impacts of the following malfunctions or operations on the Ice Condenser System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	High/low floor cooling temperature	2.7	2.5	Not applicable to Braidwood.
A2.03	Ability to (a) predict the impacts of the following malfunctions or operations on the Ice Condenser System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Opening of ice condenser doors	3.0	3.2	Not applicable to Braidwood.
A2.04	Ability to (a) predict the impacts of the following malfunctions or operations on the Ice Condenser System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Containment isolation	3.0	3.2	Not applicable to Braidwood.
A2.05	Ability to (a) predict the impacts of the following malfunctions or operations on the Ice Condenser System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Abnormal glycol expansion tank level	2.5	2.7	Not applicable to Braidwood.
A2.06	Ability to (a) predict the impacts of the following malfunctions or operations on the Ice Condenser System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Decreasing ice condenser temperature	2.5	2.7	Not applicable to Braidwood.
A3.01	Ability to monitor automatic operation of the Ice Condenser System, including:	Refrigerant system	3.0	3.0	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
025000	*Ice Condenser System (Continued.)*				
A3.02	Ability to monitor automatic operation of the Ice Condenser System, including:	Isolation valves	3.4	3.4	Not applicable to Braidwood.
A4.01	Ability to manually operate and/or monitor in the control room:	Ice condenser isolation valves	3.0	2.7	Not applicable to Braidwood.
A4.02	Ability to manually operate and/or monitor in the control room:	Containment vent fans	2.7	2.5	Not applicable to Braidwood.
A4.03	Ability to manually operate and/or monitor in the control room:	Glycol circulation pumps	2.2	2.2	Not applicable to Braidwood.
K1.01	Knowledge of the physical connections and/or cause-effect relationships between the Ice Condenser System and the following systems:	Containment ventilation	2.7	2.7	Not applicable to Braidwood.
K1.02	Knowledge of the physical connections and/or cause-effect relationships between the Ice Condenser System and the following systems:	Refrigerant systems	2.7	2.7	Not applicable to Braidwood.
K1.03	Knowledge of the physical connections and/or cause-effect relationships between the Ice Condenser System and the following systems:	Containment sump system	3.2	3.0	Not applicable to Braidwood.
K2.01	Knowledge of bus power supplies to the	Containment ventilation fans and dampers	2.2	2.7	Not applicable to Braidwood.
K2.02	Knowledge of bus power supplies to the	Refrigerant systems	2.0	2.5	Not applicable to Braidwood.
K2.03	Knowledge of bus power supplies to the	Isolation valves	2.0	2.2	Not applicable to Braidwood.
K3.01	Knowledge of the effect that a loss of the Ice Condenser System will have on the following:	Containment	3.8	3.8	Not applicable to Braidwood.
K4.01	Knowledge of Ice Condenser System design feature(s) and/or interlock(s) which provide for the following:	Glycol expansion tank levels and ice condenser system containment isolation valves	2.2	2.5	Not applicable to Braidwood.
K4.02	Knowledge of Ice Condenser System design feature(s) and/or interlock(s) which provide for the following:	System control	2.8	3.0	Not applicable to Braidwood.
K5.01	Knowledge of the following theoretical concepts as they apply to the Ice Condenser System:	Relationships between pressure and temperature	3.0	3.4	Not applicable to Braidwood.
K5.02	Knowledge of the following theoretical concepts as they apply to the Ice Condenser System:	Heat transfer	2.6	2.8	Not applicable to Braidwood.
K5.03	Knowledge of the following theoretical concepts as they apply to the Ice Condenser System:	Gas laws	2.4	2.8	Not applicable to Braidwood.
K6.01	Knowledge of the applicable performance and design attributes of the following Ice Condenser	Upper and lower doors of the ice condenser	3.4	3.6	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
026000	*Containment Spray System (CSS)*				
A2.09	Ability to (a) predict the impacts of the following malfunctions or operations on the CSSS and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Radiation hazard potential of BWST	2.5*	2.9*	Not applicable to Braidwood.
K1.02	Knowledge of the physical connections and/or cause-effect relationships between the CSS and the following systems:	Cooling water	4.1	4.1	Braidwood Containment Spray pumps have no cooling water
K3.02	Knowledge of the effect that a loss or malfunction of the CSS will have on the following:	Recirculation spray system	3.9	4.1	Not applicable to Braidwood
028000	*Hydrogen Recombiner and Purge Control System (HRPS)*				
K1.01	Knowledge of the physical connections and/or cause effect relationships between the HRPS and the following systems:	Containment annulus ventilation system (including pressure limits)	2.5*	2.5	Braidwood Hydrogen Recombiners no longer in use
K1.02	Knowledge of the physical connections and/or cause effect relationships between the HRPS and the following systems.	Air supply system	2.5*	2.5	Braidwood Hydrogen Recombiners no longer in use
K2.01	Knowledge of the power supplies to the following:	Hydrogen Recombiners	2.5*	2.8*	Braidwood Hydrogen Recombiners no longer in use
K3.01	Knowledge of the effect that a loss or malfunction of the HRPS will have on the following:	Hydrogen concentration in containment	3.3	4.0	Braidwood Hydrogen Recombiners no longer in use
K5.01	Knowledge of the operational implications of the following concepts as they apply to the HRPS:	Explosive hydrogen concentration	3.4	3.9	Braidwood Hydrogen Recombiners no longer in use
K5.02	Knowledge of the operational implications of the following concepts as they apply to the HRPS:	Flammable hydrogen concentration	3.4	3.9	Braidwood Hydrogen Recombiners no longer in use
K5.03	Knowledge of the operational implications of the following concepts as they apply to the HRPS:	Sources of hydrogen within containment	2.9	3.6*	Braidwood Hydrogen Recombiners no longer in use
K5.04	Knowledge of the operational implications of the following concepts as they apply to the HRPS:	The selective removal of hydrogen	2.6?	3.2?	Braidwood Hydrogen Recombiners no longer in use
K6.01	Knowledge of the effect that a loss or malfunction on the following will have on the HRPS:	Hydrogen Recombiners	2.6	3.1	Braidwood Hydrogen Recombiners no longer in use
A1.01	Ability to predict and/or monitor changes in parameter (to prevent exceeding design limits) associated with operating the HRPS controls including:	Hydrogen concentration	3.4	3.8	Braidwood Hydrogen Recombiners no longer in use

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
028000	*Hydrogen Recombiner and Purge Control System (HRPS) (Continued.)*				
A1.02	Ability to predict and/or monitor changes in parameter (to prevent exceeding design limits) associated with operating the HRPS controls including:	Containment pressure	3.4*	3.7*	Braidwood Hydrogen Recombiners no longer in use
A2.01	Ability to (a) predict the impacts of the following malfunctions or operations on the HRPS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Hydrogen recombiner power setting, determined by using plant data book	3.4*	3.6*	Braidwood Hydrogen Recombiners no longer in use
A2.02	Ability to (a) predict the impacts of the following malfunctions or operations on the HRPS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	LOCA condition and related concern over hydrogen	3.5	3.9	Braidwood Hydrogen Recombiners no longer in use
A2.03	Ability to (a) predict the impacts of the following malfunctions or operations on the HRPS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	The hydrogen air concentration in excess of limit flame propagation or detonation with resulting equipment damage in containment	3.4*	3.6*	Braidwood Hydrogen Recombiners no longer in use
A4.01	Ability to manually operate and/or monitor in the control room:	HRPS controls	4.0*	4.0*	Braidwood Hydrogen Recombiners no longer in use
033000	*Spent Fuel Pool Cooling System (SFPCS)*				
A4.02	Ability to manually operate and/or monitor in the control room:	SFPCS valves	2.4	2.8	Not applicable to Braidwood.
034000	*Fuel Handling*				
K1.05	Knowledge of the physical connections and/or cause effect relationships between the Fuel Handling System and the following systems:	Shutdown monitor	2.5*	3.4*	Not applicable to Braidwood.
035000	*Steam Generator System (S/GS)*				
K4.04	Knowledge of S/GS design feature(s) and/or interlock(s) which provide for the following:	Radiation high-level isolation while draining S/G secondary to main condenser	2.8*	3.1*	Not applicable to Braidwood.
039000	*Main and Reheat Steam System (MRSS)*				
A4.04	Ability to manually operate and/or monitor in the control room:	Emergency feedwater pump turbines	3.8	3.9	Not applicable to Braidwood.
K3.03	Knowledge of the effect that a loss of the MRSS will have on the following:	AFW pumps	3.2	3.5	Not applicable to Braidwood
K1.07	Knowledge of the physical connections and/or cause effect relationships between the MRSS and the following systems:	AFW	3.4*	3.4*	Not applicable to Braidwood.
K4.07	Knowledge of MRSS design feature(s) and/or interlock(s) which provide for the following:	Reactor building isolation	3.4	3.7	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
041000	*Steam Dump System (SDS) and Turbine Bypass Control*				
A4.01	Ability to manually operate and/or monitor in the control room:	ICS voltage inverter	2.9*	3.1*	Not applicable to Braidwood.
A4.07	Ability to manually operate and/or monitor in the control room:	Remote gagging of stuck open relief valves	2.9	3.0	Not applicable to Braidwood.
K2.01	Knowledge of bus power supplies to the following:	ICS, normal and alternate power supply	2.5	2.6	Not applicable to Braidwood.
K2.02	Knowledge of bus power supplies to the following:	ICS inverter breakers	2.8	2.9	Not applicable to Braidwood.
K4.01	Knowledge of SDS design feature(s) and/or interlock(s) which provide for the following:	RRG/ICS system	2.9*	3.3*	Not applicable to Braidwood
K4.15	Knowledge of SDS design feature(s) and/or interlock(s) which provide for the following:	"Measured variable" readings on ICS hand-automatic stations and required action if reading is out of the acceptable band	2.9*	2.9*	Not applicable to Braidwood
045000	*Main Turbine Generator (MT/G) System*				
K4.08	Knowledge of MT/G System design feature(s) and/or interlock(s) which provide for the following:	The reactor bailey station and reactor diamond station in integrated control circuitry	2.6	3.0	Not applicable to Braidwood.
K4.44	Knowledge of MT/G system design feature(s) and/or interlock(s) which provide for the following:	impulse pressure mode control of steam dumps	2.5*	2.8*	Not applicable to Braidwood
055000	*Condenser Air Removal System (CARS)*				
A3.03	Ability to monitor automatic operation of the CARS, including:	Automatic diversion of CARS exhaust	2.5*	2.7*	Not applicable to Braidwood.
059000	*Main Feedwater (MFW) System*				
A1.07	Ability to (a) predict and/or monitor changes in parameters (to prevent exceeding design Limits) associated with operating the MFW Controls including:	Feed pump speed, including normal control speed for ICS	2.5*	2.6*	Not applicable to Braidwood.
A3.07	Ability to monitor automatic operation of the MFW System, including:	ICS	3.4	3.5	Not applicable to Braidwood.
A4.10	Ability to manually operate and/or monitor in the control room:	ICS	3.9	3.8	Not applicable to Braidwood.
K1.07	Knowledge of the physical connections and/or cause-effect relationships between the MFW System and the following systems:	ICS	3.2	3.2	Not applicable to Braidwood.
K4.02	Knowledge of MFW System design feature(s) and/or interlock(s) which provide for the following:	Automatic turbine/reactor trip runback	3.3	3.5	Not applicable to Braidwood.
K4.17	Knowledge of MFW System design feature(s) and/or interlock(s) which provide for the following:	Increased feedwater flow following a reactor trip	2.5	2.8	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
061000	*Auxiliary / Emergency Feedwater (AFW) System*				
A1.03	Ability to (a) predict and/or monitor changes In parameters (to prevent exceeding design Limits) associated with operating the AFW Controls including:	Interactions when multi unit systems are cross tied	3.1*	3.6*	Not applicable to Braidwood.
A2.02	Ability to (a) predict the impacts of the following malfunctions or operations on the AFW System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Loss of air to steam supply valve	3.2	3.6	Not applicable to Braidwood.
A3.04	Ability to monitor automatic operation of the AFW System, including:	Automatic AFW isolation	4.1	4.2	Not applicable to Braidwood.
K1.09	Knowledge of the physical connections and/or cause-effect relationships between the AFW and the following systems:	PRMS	2.6*	2.8*	Not applicable to Braidwood.
K1.11	Knowledge of the physical connections and/or cause-effect relationships between the AFW and the following systems:	AFW turbine exhaust drains	2.7	2.8*	Not applicable to Braidwood.
K4.07	Knowledge of AFW System design feature(s) and/or interlock(s) which provide for the following:	Turbine trip, including overspeed	3.1	3.3	Not applicable to Braidwood.
K4.09	Knowledge of AFW System design feature(s) and/or interlock(s) which provide for the following:	Cross-ties between multi-unit station	3.7	3.3	Not applicable to Braidwood.
K4.11	Knowledge of AFW System design feature(s) and/or interlock(s) which provide for the following:	Automatic level control	2.7	2.9	Not applicable to Braidwood.
K4.14	Knowledge of AFW System design feature(s) and/or interlock(s) which provide for the following:	AFW automatic isolation	3.5	3.7	Not applicable to Braidwood.
K5.04	Knowledge of the following theoretical concepts as they apply to the AFW System:	Reason for warming up turbine prior to turbine startup	2.3	2.5	Not applicable to Braidwood.
068000	*Liquid Radwaste System (LRS)*				
A2.03	Ability to (a) predict the impacts of the following malfunctions or operations on the Liquid Radwaste System; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:	Insufficient sampling frequency of the boric acid in the evaporator bottoms	2.5*	2.6*	Not applicable to Braidwood.
A3.01	Ability to monitor automatic operation of theLiquid Radwaste System, including:	Evaporator pressure control	2.5*	2.4*	Not applicable to Braidwood.
A4.01	Ability to manually operate and/or monitor in the control room:	Control board for boron recovery	2.7*	2.4*	Not applicable to Braidwood.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
071000	Waste Gas Disposal System (WGDS)				
A4.01	Ability to manually operate and/or monitor in the control room:	Valve to put the holdup tank into service; indications of valve positions and tank pressure	2.7*	2.2*	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
A4.02	Ability to manually operate and/or monitor in the control room:	Waste-gas compressor, including control switch, unloading valve, and drain valve	2.5*	2.3*	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas compressor operations.
A4.03	Ability to manually operate and/or monitor in the control room:	Wvalves and indications for sealing water to the, gas compressor shaft	2.6*	2.2*	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas compressor operations.
A4.05	Ability to manually operate and/or monitor in the control room:	Gas decay tanks, including valves, indicators, and sample line	2.6*	2.6*	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
A4.07	Ability to manually operate and/or monitor in the control room:	Waste gas release flow meter	3.0*	3.0*	Braidwood main control rooms do not contain equipment or instrumentation to monitor waste gas release flow.
A4.10	Ability to manually operate and/or monitor in the control room:	WGDS sampling	2.5*	2.4*	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
A4.11	Ability to manually operate and/or monitor in the control room:	WGDS startup and shutdown	2.5	2.3	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
A4.13	Ability to manually operate and/or monitor in the control room:	Recovery from automatic termination of gas release due to PRM system alarm	3.0	3.1	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.

Viewed KA	Category Statement	KA Statement	RO Value	SRO Value	Suppress Basis
071000	*Waste Gas Disposal System (WGDS) (Continued)*				
A4.14	Ability to manually operate and/or monitor in the control room:	WGDS status alarms	2.8	3.0	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
A4.16	Ability to manually operate and/or monitor in the control room:	Waste gas decay tank shifts	2.5	2.2	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
A4.17	Ability to manually operate and/or monitor in the control room:	Stopping transfer of radioactive liquids to WGDS tank	2.6	2.5	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
A4.19	Ability to manually operate and/or monitor in the control room:	Bringing an empty WDGS tank on line and shutting down a full tank	2.5*	2.2*	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
A4.20	Ability to manually operate and/or monitor in the control room:	Placing WGDS gas compressors in automatic operation	2.5*	2.2*	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas compressor operations.
A4.30	Ability to manually operate and/or monitor in the control room:	Water drainage from WGDS decay tanks	2.9	2.6	Braidwood main control rooms do not contain equipment or instrumentation to monitor or perform gas decay tank operations.
K4.03	Knowledge of design feature(s) and/or interlocks which provide for the following:	Tank loop seals	2.5*	2.6*	Not applicable to Braidwood
073000	*Process Radiation Monitoring (PRM) System*				
K4.02	Knowledge of PRM System design feature(s) and/or interlocks which provide for the following:	Letdown isolation on high-RCS activity	3.3*	3.9*	Not applicable to Braidwood

Viewed KA	*Category Statement*	*KA Statement*	*RO Value*	*SRO Value*	*Suppress Basis*
076000	*Service Water System (SWS)*				
A1.02	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits): associated with operating the SWS controls including:	Reactor and turbine building closed cooling water temperatures.	2.5*	2.3	Not applicable to Braidwood.
K1.07	Knowledge of the physical connections and/or cause-effect relationships between the SWS and the following systems:	Secondary closed cooling water	2.6*	2.6*	Not applicable to Braidwood.
K1.09	Knowledge of the physical connections and/or cause-effect relationships between the SWS and the following systems:	Reactor building closed cooling water	3.0	3.1	Not applicable to Braidwood.
K2.04	Knowledge of bus power supplies to the	Reactor building closed cooling water	2.5	2.6	Not applicable to Braidwood.
K3.03	Knowledge of the effect that a loss of the SWS will have on the following:	Reactor building closed cooling water	3.5	3.9	Not applicable to Braidwood.
K4.01	Knowledge of SWS design feature(s) and/or interlock(s) which provide for the following:	Conditions initiating automatic closure of closed cooling water auxiliary building header supply and	2.5	2.9	Not applicable to Braidwood.
K4.03	Knowledge of SWS design feature(s) and/or interlock(s) which provide for the following:	Automatic opening features associated with SWS isolation valves to CCS heat exchangers	2.9*	3.4*	Not applicable to Braidwood.
078000	*Instrument Air System (IAS)*				
K1.05	Knowledge of the physical connections and/or cause-effect relationships between the IAS and the following systems:	MSIV air	3.4	3.5	Not applicable to Braidwood.
K2.02	Knowledge of bus power supplies to the	Emergency air compressor	3.3	3.5	Not applicable to Braidwood.
086000	*Fire Protection System (FPS)*				
A1.02	Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the FPS controls including:	Fire water storage tank level	3.0	3.2	Not applicable to Braidwood.
A4.04	Ability to manually operate and/or monitor in the control room:	Fire water storage tank makeup pumps	3.4	3.3	Not applicable to Braidwood.
K1.01	Knowledge of the physical connections and/or cause-effect relationships between the FPS and the following systems:	High-pressure service water	3.0	3.4	Not applicable to Braidwood.

Viewed KA	*Category Statement*	*KA Statement*	*RO Value*	*SRO Value*	*Suppress Basis*
103000	*Containment System*				
A4.09	Ability to manually operate and/or monitor in the control room:	Containment vacuum system	3.1	3.7	Not applicable to Braidwood.
K1.03	Knowledge of the physical connections and/or cause-effect relationships between the Containment System and the following systems:	Shield building vent system	3.1	3.5	Not applicable to Braidwood.
K1.07	Knowledge of the physical connections and/or cause-effect relationships between the Containment System and the following systems:	Containment vacuum system	3.5	3.7	Not applicable to Braidwood.
K4.01	Knowledge of Containment System design feature(s) and/or interlock(s) which provide for the following:	Vacuum breaker protection	3.0	3.7	Not applicable to Braidwood.

Braidwood Exam Author

Braidwood Facility Representative

cc: B. Palagi
P. Peterson

Exelon Nuclear

Exelon Generation Company, LLC
Braidwood Station
35100 South Rt 53, Suite 84
Braceville, IL 60407-9619
Tel. 815-417-2000

www.exeloncorp.com

July 14, 2006
BW060070

James L. Caldwell
Regional Administrator
U. S. Nuclear Regulatory Commission
Region III
2443 Warrenville Road
Suite 210
Lisle, IL 60532-4352

Attn: Operator Licensing Branch

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. 50-456 and 50-457

Subject: Updated Braidwood Future Initial License Training Schedule Dates

References:
1) Letter from D. P. Helker (Exelon Generation Company, LLC and AmerGen Energy Company, LLC) to U. S. NRC "Response to NRC Regulatory Issue Summary of Operating Licensing Examinations," dated September 30, 2005

2) Letter from J. A. Bauer (Exelon Generation Company, LLC) to U. S. NRC, "Response to NRC Regulatory Issue Summary 2005-19, 'Preparation and Scheduling of Operator Licensing Examinations,'" dated March 3, 2006

3) Letter from U. S. NRC to C. M. Crane (Exelon Generation Company, LLC) "Confirmation of Requested Initial Operator License Examinations: NRC Region III Exelon Stations," dated June 6, 2006

The purpose of this letter is to provide revised operator license examination numbers and dates in response to NRC Regulatory Issue Summary (RIS) 2005-19, "Preparation and Scheduling of Operator Licensing Examinations" for Braidwood Station. This updated information revises the proposed operator license examination numbers and dates originally provided in Reference 1 and modified in the Reference 2 submittal.

The revised operator license examination numbers and dates are provided in the attached NRC 536 Form. These updated examination numbers and dates also clarify the information listed in Reference 3 for Braidwood Station.

If you have any questions about this letter, please contact Mr. Dale Ambler at (815) 417-2800.

Respectfully,

Keith J. Polson
Site Vice President
Braidwood Nuclear Power Station.

KJP/MC/vk
Attachment: Revised NRC Form 536 - Braidwood Station

NRC FORM 536 (8-2005) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0131 EXPIRES: 08/31/2008

Estimated burden per response to comply with this voluntary information collection request: 1 hour. This information collection is used to plan budgets and resources for operator examinations. Send comments regarding burden estimate to the Records and FOIA/Privacy Services Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0131), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

OPERATOR LICENSING EXAMINATION DATA

FACILITY

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. 50-456 and 50-457

NRC REGION

III

A. PROPOSED EXAMINATION PREPARATION SCHEDULE

PROPOSED NUMBER	CY 2006	CY 2007	CY 2008	CY 2009
ESTIMATED NUMBER OF LICENSEE-PREPARED EXAMINATIONS	1	1	0	1
ESTIMATED NUMBER OF NRC-PREPARED EXAMINATIONS	0	0	0	0

B. INITIAL OPERATOR LICENSE EXAMINATIONS

PROPOSED NUMBER	CY 2006	CY2007	CY2008	CY2009
NUMBER OF REACTOR OPERATORS	0	1	0	6
NUMBER OF SENIOR REACTOR OPERATORS - INSTANT	2	11	0	8
NUMBER OF SENIOR REACTOR OPERATORS - UPGRADE	1	0	0	0
NUMBER OF SENIOR REACTOR OPERATORS - LIMITED	0	0	0	0
PROPOSED DATES				
PRIMARY DATE	05/15/2006	12/03/2007		10/05/2009
ALTERNATE DATE	05/22/2006			10/19/2009

C. PROPOSED GENERIC FUNDAMENTALS EXAMINATION (GFE) SCHEDULE

PROPOSED NUMBER	CY 2006				CY 2007			
	MARCH	JUNE	SEPTEMBER	DECEMBER	MARCH	JUNE	SEPTEMBER	DECEMBER
ESTIMATED NUMBER OF CANDIDATES	0	0	12	0	0	0	0	0

June 6, 2006

Mr. Christopher M. Crane
President and Chief Nuclear Officer
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: CONFIRMATION OF REQUESTED INITIAL OPERATOR LICENSE EXAMINATIONS: NRC REGION III EXELON STATIONS

Dear Mr. Crane:

In a letter dated September 30, 2005 (RS-05-122), your staff responded to Regulatory Issue Summary 2005-19 and requested we schedule initial license operator examinations for the Exelon Nuclear Plants. This letter confirms the requested examinations at the Region III power plants. Some dates are different than those requested in your letter. The individual station training departments have been notified by telephone of the schedule differences. We have tentatively scheduled the requested examinations to occur as follows:

Station	2006	2007	2008	2009
Braidwood	5/15/2006	12/3 & 12/10/2007	3/3 & 3/10/2008	N/R
Byron	6/19 & 6/26/2006	N/R	5/19 & 5/26/2008	8/24 & 8/31/2009
Clinton	N/R	8/13 & 8/20/2007	N/	8/31 & 9/07/2009
Dresden	02/06/2006	4/23 & 4/30/2007	7/14 & 7/21/2008	10/12 & 10/19/2009
LaSalle	11/13 & 11/20/2006	N/R	8/18 & 8/25/2008	N/R
Quad Cities	N/R	06/04 & 06/11/2007	N/R	05/25 & 06/01/2009

N/R - None Requested

On-site validation of the examinations is tentatively scheduled to occur approximately three weeks before each scheduled examination. In the unlikely event that we are unable to support the examination during the scheduled weeks, we will inform you and the affected station's training department immediately upon discovery of such conditions and make arrangements to administer the examination at a mutually acceptable date.

We understand that your staff will develop the proposed examinations with the exception of the November 2006 LaSalle Station examination, which will be developed by Region III examiners. Examinations scheduled for two weeks indicate a class size of ten or more candidates. Please inform us if the number of candidates declines below ten as this will impact the examination schedule. Please inform us at your earliest opportunity if you discover a need to change any of the dates of the scheduled examinations.

A supplementary letter will be sent to the station's training department approximately 120 days prior to the initial examination. The letter will provide the station with the chief examiner's name, an outline of examination security expectations, a list of the materials required by the NRC to conduct the examination, and it will reconfirm the examination dates and the number of candidates you have in the training program. If you have any questions concerning this information, please contact Mr. Dell McNeil of my staff at 630-829-9737.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's Agencywide Documents Access and Management System (ADAMS). ADAMS is accessible from the Electronic Reading Room page of the NRC's public Web site at http://www.nrc.gov/reading-rm/adams.html.

Sincerely,

/RA/

Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-456; 50-457
License Nos. NPF-72; NPF-77

See Attached Distributions

We understand that your staff will develop the proposed examinations with the exception of the November 2006 LaSalle Station examination, which will be developed by Region III examiners. Examinations scheduled for two weeks indicate a class size of ten or more candidates. Please inform us if the number of candidates declines below ten as this will impact the examination schedule. Please inform us at your earliest opportunity if you discover a need to change any of the dates of the scheduled examinations.

A supplementary letter will be sent to the station's training department approximately 120 days prior to the initial examination. The letter will provide the station with the chief examiner's name, an outline of examination security expectations, a list of the materials required by the NRC to conduct the examination, and it will reconfirm the examination dates and the number of candidates you have in the training program. If you have any questions concerning this information, please contact Mr. Dell McNeil of my staff at 630-829-9737.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's Agencywide Documents Access and Management System (ADAMS). ADAMS is accessible from the Electronic Reading Room page of the NRC's public Web site at http://www.nrc.gov/reading-rm/adams.html.

Sincerely,

/RA/

Hironori Peterson, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-456; 50-457
License Nos. NPF-72; NPF-77

See Attached Distributions

ADAMS DOCUMENT TITLE: G:\Work in Progress\Exelon 2007 Exams Ltr.wpd
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Docket Nos. 50-237; 50-249
License Nos. DPR-19; DPR-25

cc: Site Vice President - Dresden Nuclear Power Station
Dresden Nuclear Power Station Plant Manager
Regulatory Assurance Manager - Dresden
C. Symonds, Training Director, Dresden Nuclear Power Station

Docket Nos. 50-373; 50-374
License Nos. NPF-11; NPF-18

cc: Site Vice President - LaSalle County Station
LaSalle County Station Plant Manager
Regulatory Assurance Manager - LaSalle County Station
Senior Vice President - Mid-West Regional
Operating Group
Manager Licensing - Clinton and LaSalle
R. Ebright, Sr., Training Director, LaSalle County Station

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

cc: Site Vice President - Quad Cities Nuclear Power Station
Plant Manager - Quad Cities Nuclear Power Station
Regulatory Assurance Manager - Quad Cities Nuclear Power Station
Manager Licensing - Dresden and Quad Cities
Vice President - Law and Regulatory Affairs
Mid American Energy Company
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
D. Tubbs, Manager of Nuclear MidAmerican Energy Company
R. Armitage, QCNPP Training Director

cc: Site Vice President - Braidwood Station
Plant Manager - Braidwood Station
Regulatory Assurance Manager - Braidwood Station
Chief Operating Officer
Senior Vice President - Nuclear Services
Vice President - Operations Support
Vice President - Licensing and Regulatory Affairs
Director Licensing
Manager Licensing - Braidwood and Byron
Senior Counsel, Nuclear, Mid-West Regional Operating Group
Document Control Desk - Licensing
Assistant Attorney General
Illinois Emergency Management Agency
State Liaison Officer - State of Illinois
Chairman, Illinois Commerce Commission
C. Dunn, Training Director, Braidwood Station

Docket Nos. 50-454; 50-455
License Nos. NPF-37; NPF-66

cc: Site Vice President - Byron Station
Plant Manager - Byron Station
Regulatory Assurance Manager - Byron Station
State Liaison Officer, State of Wisconsin
Chairman, Illinois Commerce Commission
B. Quigley, Byron Station
S. Sitmac, Training Director, Byron Station

Docket No. 50-461
License No. NPF-62

cc: Site Vice President - Clinton Power Station
Plant Manager - Clinton Power Station
Regulatory Assurance Manager - Clinton Power Station
Manager Licensing - Clinton Power Station
Senior Counsel, Nuclear, Mid-West Regional Operating Group
J. Lindsey, Training Director, Clinton Power Station

ADAMS Distribution:
GYS
DXC1
SPR
KNJ
BCD
BLB1
MXB
DRC1
DMS6
DEK
JBH1
RidsNrrDipmIipb
GEG
KGO
NXS
CAA1
DRPIII
DRSIII
PLB1
JRK1
NFO
MAB1

Exelon
Nuclear

RS-06-032

March 3, 2006

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. 50-456 and 50-457

Byron Station, Units 1 and 2
Facility Operating License Nos. NPF-37 and NPF-66
NRC Docket Nos. 50-454 and 50-455

Subject: Revised Response to NRC Regulatory Issue Summary 2005-19, "Preparation and Scheduling of Operator Licensing Examinations"

Reference: Letter from D. P. Helker (Exelon Generation Company, LLC and AmerGen Energy Company, LLC) to U. S. NRC, "Response to NRC Regulatory Issue Summary 2005-19, "Preparation and Scheduling of Operator Licensing Examinations," dated September 30, 2005

The purpose of this letter is to provide a revised response to NRC Regulatory Issue Summary 2005-19, "Preparation and Scheduling of Operator Licensing Examinations" for Braidwood Station and Byron Station. The referenced submittal provided the Exelon Generation Company, LLC (EGC) and AmerGen Energy Company, LLC (AmerGen) estimates of the number and timeframes for initial operator licensing needs. Subsequent to the referenced submittal, Braidwood Station and Byron Station revised their estimates for initial operator licensing needs.

The revised numbers and dates are provided in the attached NRC 536 Forms. If you have any questions about this letter, please contact David Chrzanowski at (630) 657-2816.

Respectfully,

Joseph A. Bauer

Joseph A. Bauer
Manager – Licensing

Attachments: (1) Revised NRC Form 536 – Braidwood Station
(2) Revised NRC Form 536 – Byron Station

MAR [illegible] 2006

NRC FORM 536 (8-2005) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO 3150-0131 EXPIRES 08/31/2008

Estimated burden per response to comply with this voluntary information collection request 1 hour. This information collection is used to plan budgets and resources for operator examinations. Send comments regarding burden estimate to the Records and FOIA/Privacy Services Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0131), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

OPERATOR LICENSING EXAMINATION DATA

FACILITY

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. 50-456 and 50-457

NRC REGION

III

A. PROPOSED EXAMINATION PREPARATION SCHEDULE

PROPOSED NUMBER	CY 2006	CY 2007	CY 2008	CY 2009
ESTIMATED NUMBER OF LICENSEE-PREPARED EXAMINATIONS	1	1	0	0
ESTIMATED NUMBER OF NRC-PREPARED EXAMINATIONS	0	0	0	0

B. INITIAL OPERATOR LICENSE EXAMINATIONS

PROPOSED NUMBER	CY 2006	CY2007	CY2008	CY2009
NUMBER OF REACTOR OPERATORS	0	0	0	0
NUMBER OF SENIOR REACTOR OPERATORS - INSTANT	3	12	0	0
NUMBER OF SENIOR REACTOR OPERATORS - UPGRADE	0	0	0	0
NUMBER OF SENIOR REACTOR OPERATORS - LIMITED	0	0	0	0
PROPOSED DATES				
PRIMARY DATE	05/15/2006	12/03/2007		
ALTERNATE DATE	05/22/2006		01/07/2008	

C. PROPOSED GENERIC FUNDAMENTALS EXAMINATION (GFE) SCHEDULE

PROPOSED NUMBER	CY 2006				CY 2007			
	MARCH	JUNE	SEPTEMBER	DECEMBER	MARCH	JUNE	SEPTEMBER	DECEMBER
ESTIMATED NUMBER OF CANDIDATES	0	0	0	12	0	0	0	0