



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

OFFICE OF THE
GENERAL COUNSEL

April 3, 2008

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Administrative Judge
Atomic Safety and Licensing Board
U.S. Nuclear Regulatory Commission
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In the Matter of
AMERGEN ENERGY COMPANY, LLC
(License Renewal for Oyster Creek Nuclear Generating Station)
Docket No. 50-219-LR

Dear Administrative Judges:

Enclosed for your information is a copy of the April 3, 2008 Notification of Information in the Matter of Oyster Creek Nuclear Generating Station License Renewal Application, which the Staff has provided to the Commission.

Sincerely,

A handwritten signature in black ink, appearing to read "James E. Adler", written over a horizontal line.

James E. Adler
Counsel for the NRC Staff

Enclosure: As Stated

cc w/enclosure: Service List



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April 3, 2008

Board Notification 2008-01

MEMORANDUM TO: Chairman Klein
Commissioner Jaczko
Commissioner Lyons
Commissioner Svinicki
Atomic Safety and Licensing Board
All Parties

FROM: Samson S. Lee, Acting Director 
Division of License Renewal
Office of Nuclear Reactor Regulation

SUBJECT: NOTIFICATION OF INFORMATION IN THE MATTER OF OYSTER
CREEK NUCLEAR GENERATING STATION LICENSE RENEWAL
APPLICATION

In conformance with the Commission's policy on notification to the Commission and the Atomic Safety Licensing Board (ASLB) regarding significant new information, this memorandum provides the following information.

The staff is reviewing the use of a simplified method to calculate cumulative usage factors (CUF) that may not be conservative. Oyster Creek Nuclear Generating Station (Oyster Creek) used this simplified fatigue calculation method for one type of nozzle, the recirculation nozzle at the plant. This type of calculation was not applicable to the drywell shell analysis, which is the subject of the appealed contention pending before the Commission. Although, this simplified calculation is not relevant to the contention in the proceeding that was before the ASLB, we are providing this information, because this may be an issue of public interest.

The staff plans to ask Oyster Creek to perform a confirmatory analysis consistent with the methodology in Section III of the ASME Code. However, the staff believes that the safety significance of using the simplified analysis method is low based on the risk assessments performed by the staff in resolving generic safety issues (GSI)-166 and GSI-190.

Docket No. 50-219

cc: See next page

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Oyster Creek Nuclear Generating Station - 3 -

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