

Final Submittal

(Blue Paper)

Oconee 2008-301 RETAKE

Written EXAMINATION 3/20/2008

FINAL SRO

WRITTEN EXAMINATION

AND REFERENCES



**1 POINT**

**Question 76**

Unit 1 initial conditions:

- Reactor trip
- RCS pressure = 1800 psig decreasing
- Pzr level = 280 inches increasing
- 1B HPI pump trip

Current conditions:

- LOSCM tab in progress
- 1RC-4 closed
- RCS pressure = 825 psig increasing
- Pzr level = 400 inches stable
- ALL SCMs = 4°F increasing
- RC makeup flow = 10 gpm

Based on the current conditions, which ONE of the following states an acceptable SG pressure per the LOSCM tab and which EOP tab will be used to cool down the unit to decay heat removal conditions?

- A. 1010 psig / LOCA CD
- B. 1010 psig / FCD
- C. 780 psig / LOCA CD
- D. 780 psig / FCD

**1 POINT**

**Question 77**

Unit 1 initial conditions:

- Reactor power = 100%
- 1SA2/B2 HP RCP SEAL INLET HEADER FLOW HIGH/LOW actuated
  - 0 GPM stable
- 1SA2/C2 HP INJECTION PUMP DISCH HEADER PRESSURE LOW actuated
  - ≈ 0 psig stable
- 1SA2/C3 RC PRESSURIZER LEVEL HI/LO actuated
  - 199 inches decreasing slowly
- Highest RCP seal return temperature = 175 °F stable
- RC Makeup Flow = 0 gpm

Based on the above conditions, which ONE of the following describes the current plant condition and what action will be required to mitigate this condition?

- A. HPI Seal Injection header rupture / enter AP/2 (Excessive RCS Leakage) and close 1HP-31 to prevent RCP Seal damage
- B. Loss of running and Stby HPIPs / enter AP/14 (Loss of Normal HPI Makeup and/or RCP Seal Injection) and close 1HP-20/21 to prevent exceeding 1HP-21 temperature limit
- C. HPI injection header A rupture / enter AP/2 (Excessive RCS Leakage) and close 1HP-115 to split the injection headers
- D. Loss of running and Stby HPIPs / enter AP/14 (Loss of Normal HPI Makeup and/or RCP Seal Injection) close 1HP-5 to minimize RCS inventory loss

**1 POINT**

**Question 78**

Unit 1 initial conditions:

- 1B SG has a 50 gpm tube rupture
- SGTR tab in progress
- Reactor has been manually tripped at 4% FP

Current conditions:

- 1A SG pressure = 0 psig stable
- 1B SG pressure = 900 psig and increasing slowly
- RCS temperature = 532 °F increasing slowly
- RCS pressure = 1280 psig increasing slowly

Based on the current conditions, which ONE of the following describes how the EOP will be used to mitigate the event?

- A. Remain in SGTR tab  
Unisolate and feed 1A SG
- B. Remain in SGTR tab  
Feed and steam the 1B SG
- C. Transfer to Excessive Heat Transfer tab  
Unisolate and feed 1A SG
- D. Transfer to Excessive Heat Transfer tab  
Feed and steam the 1B SG

**1 POINT**

**Question 79**

Which ONE of the following describes the basis for the mitigation strategy of the Blackout tab of the EOP and why?

- A. Maintain RCS temperature ~ 532°F  
To stabilize RCS Temperature and pressure to prevent challenging the Pressurizer PORV
- B. Maintain RCS temperature ~ 532°F  
To prevent contracting RCS inventory and emptying the Pressurizer due to loss of makeup capacity
- C. Maintain RCS temperature at ~ 550°F  
To stabilize RCS Temperature and pressure to prevent challenging the Pressurizer PORV
- D. Maintain RCS temperature at ~ 550°F  
To prevent contracting RCS inventory and emptying the Pressurizer due to loss of makeup capacity

**1 POINT**

**Question 80**

Unit 1 plant conditions:

Time = 0400

- Reactor power = 100%
- AP/2 (Excessive RCS Leakage) in progress due to a 15 gpm RCS leak
- Switchyard Isolation
- CT-1, 1TA & 1TB Lockout

Time = 0401

- Power restored from CT-4

Time = 0410

- FCD tab in progress
- CETC = 555 °F Stable
- Loop A level = 573 inches stable
- Loop B level = 573 inches stable
- Rx Vessel head level = 155 inches decreasing
- Pressurizer level = 180 inches increasing
- RC Make Up Flow = 30 gpm

Which ONE of the following describes how the EOP will initially mitigate the event above and the highest priority AP that will be performed in parallel with the EOP?

- A. Open Rx Vessel High Point Vents and maintain pressurizer level > 100 inches  
AP/2 (Excessive RCS Leakage)
- B. Raise pressurizer level to 200 - 300 inches and increase RCS pressure  $\geq$  200 psig  
AP/2 (Excessive RCS Leakage)
- C. Open Rx Vessel High Point Vents and maintain pressurizer level > 100 inches  
AP/11 (Recovery from Loss of Power)
- D. Raise pressurizer level to 200 - 300 inches and increase RCS pressure  $\geq$  200 psig  
AP/11 (Recovery from Loss of Power)

**1 POINT**

**Question 81**

Unit 1 initial conditions:

- Reactor power = 100%

Current conditions:

- 1HP-5 indicating lights (UB1) are OFF
- Pressurizer Level = 230 inches increasing
- Letdown Flow = 0 gpm
- 1HP-5 will NOT open from the control room
- NEO dispatched to open 1HP-5 locally

Based on the above conditions, which ONE of the following describes what has occurred and whether 1HP-5 is operable after it is manually opened?

- A. 1HP-5 has failed closed due to a loss of DC power to the valve solenoid  
1HP-5 is operable provided an operator in constant communication with the control room will close the valve on ES actuation.
- B. 1HP-5 has failed closed due to a loss of DC power to the valve solenoid  
1HP-5 is inoperable because it will not automatically close on a subsequent ES actuation.
- C. 1HP-5 has closed due to an inadvertent ES Signal to the valve  
1HP-5 is operable provided an operator in constant communication with the control room will close the valve on ES actuation.
- D. 1HP-5 has closed due to an inadvertent ES Signal to the valve  
1HP-5 is inoperable because it will not automatically close on a subsequent ES actuation.

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**1 POINT**

**Question 82**

Unit 1 initial plant conditions:

- Rx startup in progress
- Source Range (SR) NIs 1 and 3 are out of service for repairs
- Reactor is critical
- WR NI-2 = 1.0 E-2%
- WR NI-4 = 1.0 E-2%
- SR NI-2 = 1.0 E5 cps
- SR NI-4 = 1.0 E5 cps

Subsequently, SR NI-2 fails LOW

Which ONE of the following describes the technical specification required action and the surveillance requirement for returning NI-2 to service?

- A. Initiate action to restore SR NI -2 to operable status within 1 hour.  
NI-2 must agree with NI-4 within 1 decade
- B. Initiate action to restore SR NI -2 to operable status within 1 hour.  
NI-2 must agree with NI-4 within 2 decades
- C. Immediately suspend positive reactivity changes and initiate control rod insertion.  
Open control rod drive trip breakers within 1 hour.  
NI-2 must agree with NI-4 within 1 decade
- D. Immediately suspend positive reactivity changes and initiate control rod insertion.  
Open control rod drive trip breakers within 1 hour.  
NI-2 must agree with NI-4 within 2 decades

**1 POINT**

**Question 83**

Unit 1 initial conditions:

- Fire occurred at the Unit 1 TD EFDW pump area
- Deluge system activated

Current conditions:

- Fire extinguished
- Deluge system isolated
- ALL fire detection equipment in area is inoperable

Based on the above conditions, which ONE of the following states SLC 16.9.2 “Sprinkler and Spray System” requirements regarding the establishment of a fire watch?

- A. Establish a Continuous Fire Watch within one hour  
Backup fire suppression is required
- B. Establish a Continuous Fire Watch within one hour  
Backup fire suppression is NOT required once fire watch is established
- C. Establish a Hourly Fire Watch within one hour  
Backup fire suppression is required
- D. Establish a Hourly Fire Watch within one hour  
Backup fire suppression is NOT required once fire watch is established

**1 POINT**

**Question 84**

Unit 1 initial conditions:

- Reactor power = 100%

Current conditions:

- Main Turbine tripped
- TBVs failed closed
- All ICS Bailey Hand/Auto Stations lights are OFF

Based on the above conditions, which ONE of the following describes which section of AP/23 (Loss of ICS Power) will be implemented and when that section will be performed?

- A. Section 4A (Loss of ICS AUTO and HAND Power)  
When EOP Subsequent Actions have been completed and transition to a normal operating procedure is directed
- B. Section 4A (Loss of ICS AUTO and HAND Power)  
Immediately following performance of the EOP IMA's and Symptom Checks OR when directed by EOP Subsequent Actions
- C. Section 4B (Loss of ICS AUTO Power Only)  
When EOP Subsequent Actions have been completed and transition to a normal operating procedure is directed
- D. Section 4B (Loss of ICS AUTO Power Only)  
Immediately following performance of the EOP EOP IMA's and Symptom Checks OR when directed by EOP Subsequent Actions

**1 POINT**

**Question 85**

Unit 1 initial conditions:

- 1A SG has a SGTR
- 1B SG is unaffected
- Core SCM = 0°F
- LOSCM tab completed

Current conditions:

- SGTR tab in progress
- Core SCM = 0°F
- LPI Flow Train A = 1750 gpm
- LPI Flow Train B = 1725 gpm

Which ONE of the following describes the procedure routing used to mitigate this event and the SG status?

- A. Transition to HPI CD Tab and ONLY 1A SG is isolated
- B. Transition to LOCA CD Tab and ONLY 1A SG is isolated
- C. Transition to HPI CD Tab and BOTH SG's are isolated
- D. Transition to LOCA CD Tab and BOTH SG's are isolated

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**1 POINT**

**Question 86**

Unit 1 plant conditions:

- RCS temperature = 240°F
- RCS pressure = 260 psig
- No RCS Vent Paths are open
- HPI has been deactivated
- 1A and 1B CFT pressure = 500 psig
- 1CF-1 and 1CF-2 Closed and Bkrs WT Opened
- Pressurizer level indicates 235 inches
- Cooldown to Mode 5 in progress at 20°F/Hr

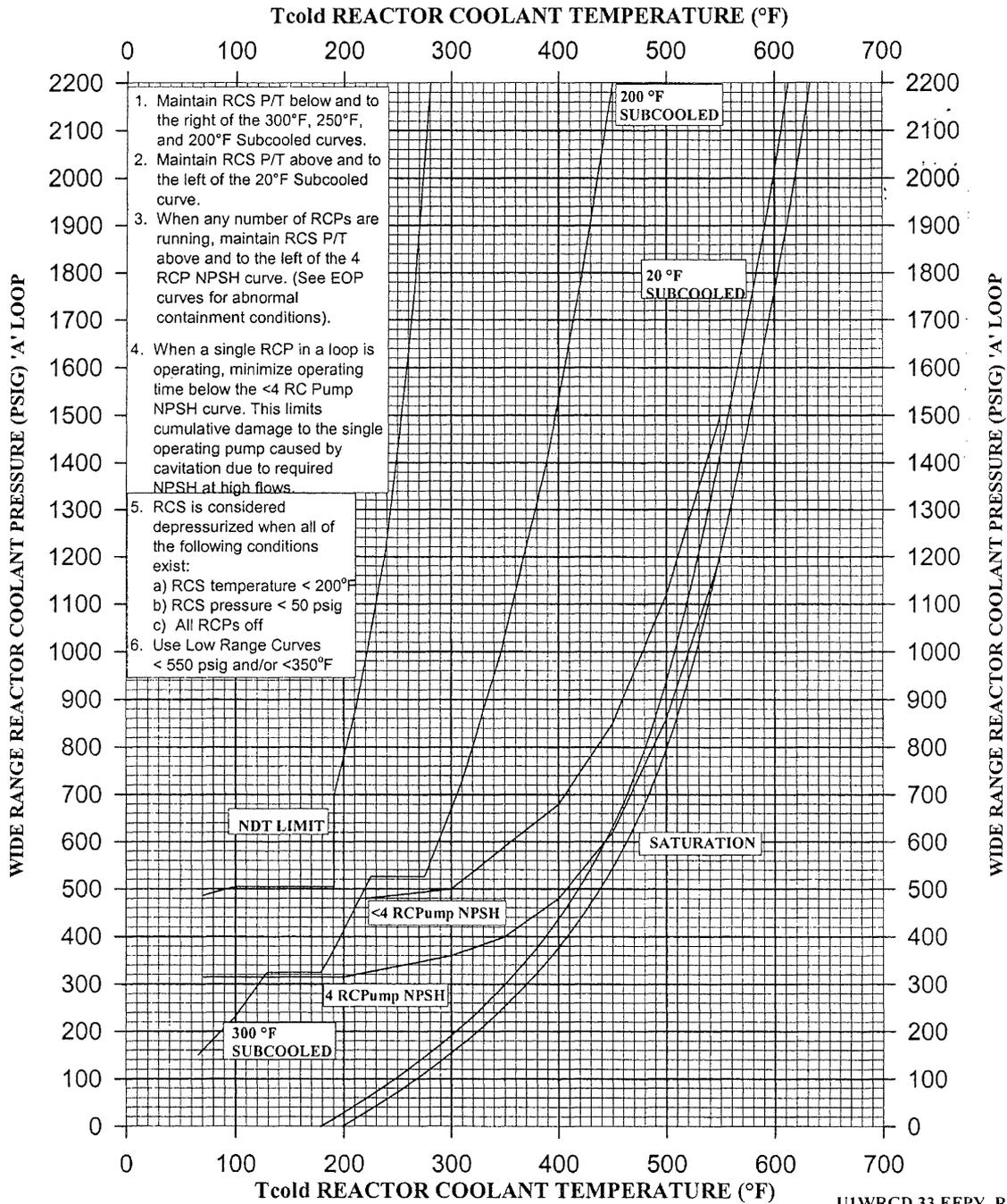
Which ONE of the following describes the impact of this condition, and the action required?

**SEE ATTACHMENT**

- A. RCS pressure and temperature exceed the limits that ensure brittle fracture prevention in accordance with TS 3.4.3 RCS Pressure and Temperature (PT) Limits; Establish an RCS Vent Path or dedicated LTOP Operator.
- B. RCS pressure and temperature exceed the limits that ensure brittle fracture prevention in accordance with TS 3.4.3 RCS Pressure and Temperature (PT) Limits; Restore RCS Pressure and Temperature to within limits in 30 minutes or less
- C. Administrative controls (Train 2) for an LTOP event are lost; Establish an RCS Vent Path or dedicated LTOP Operator in accordance with TS 3.4.12 Low Temperature Over Pressure Protection.
- D. Administrative controls (Train 2) for an LTOP event are lost; Depressurize CFTs to less than RCS pressure within 1 hour in accordance with TS 3.4.12 Low Temperature Over Pressure Protection.

**NOTE:** If changes are required to curves in this enclosure, the OAC curves must be updated at the same time or the OAC curves must be considered not valid. {1}

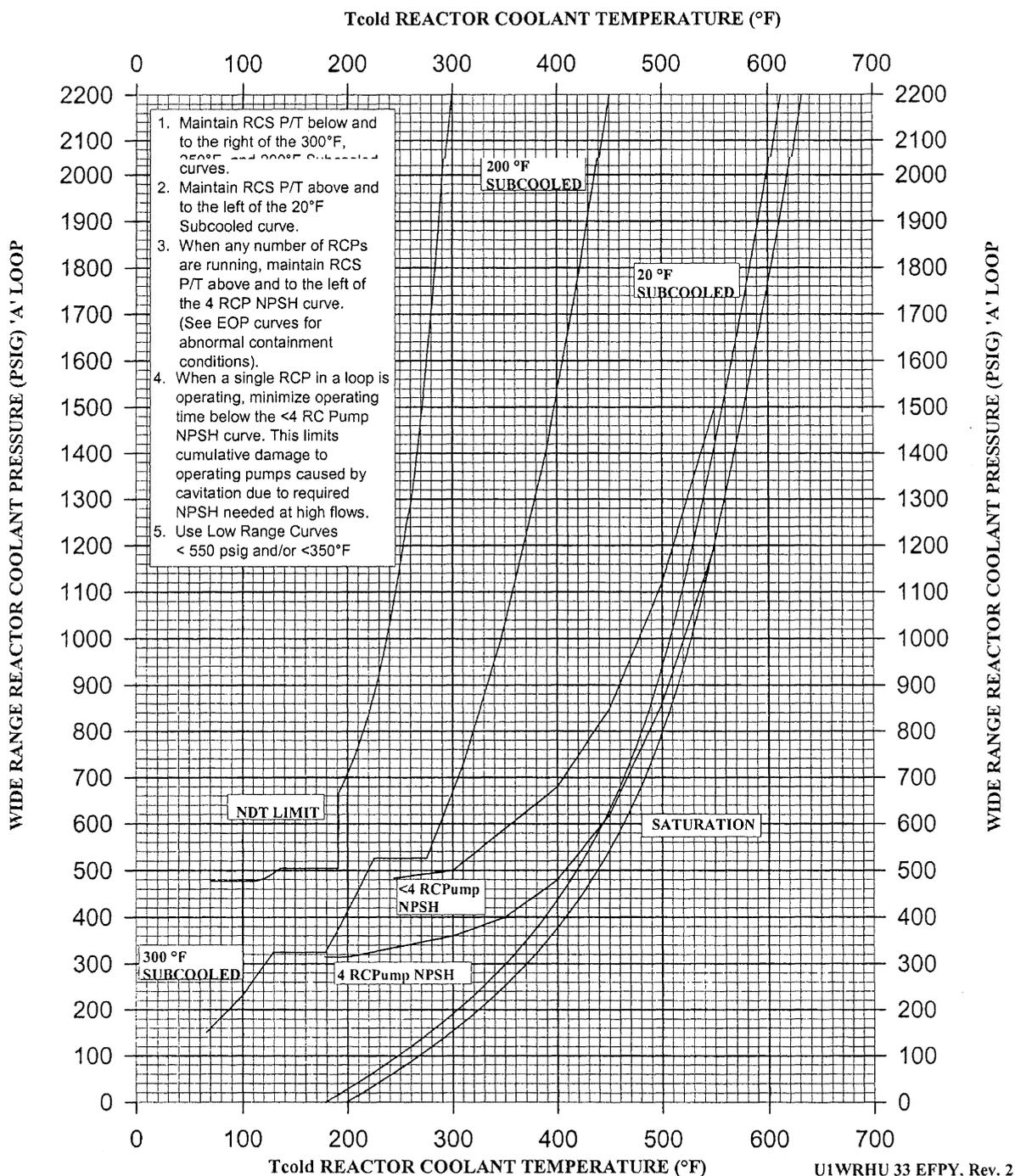
**Unit 1 Wide Range Cooldown Curve**



1. Maintain RCS P/T below and to the right of the 300°F, 250°F, and 200°F Subcooled curves.
2. Maintain RCS P/T above and to the left of the 20°F Subcooled curve.
3. When any number of RCPs are running, maintain RCS P/T above and to the left of the 4 RCP NPSH curve. (See EOP curves for abnormal containment conditions).
4. When a single RCP in a loop is operating, minimize operating time below the <4 RC Pump NPSH curve. This limits cumulative damage to the single operating pump caused by cavitation due to required NPSH at high flows.
5. RCS is considered depressurized when all of the following conditions exist:
  - a) RCS temperature < 200°F
  - b) RCS pressure < 50 psig
  - c) All RCPs off
6. Use Low Range Curves < 550 psig and/or < 350°F

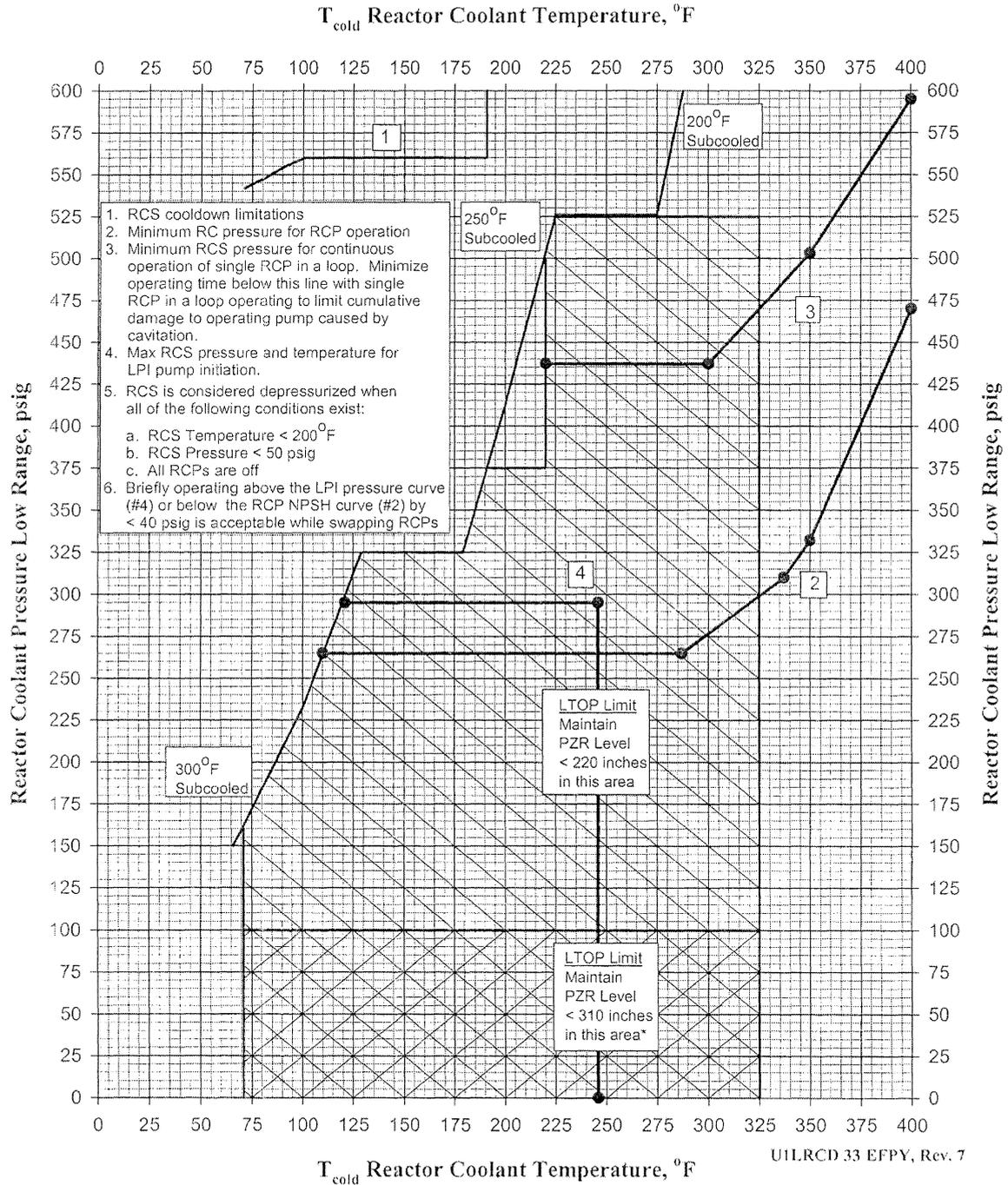
RCS TEMPERATURE	MAX COOLDOWN RATE
T ≥ 280°F	≤ 45°F in any 1/2 hour period
150°F ≤ T < 280°F	≤ 20°F in any 1/2 hour period
T < 150°F	≤ 9°F in any 1 hour period
RCS depressurized	≤ 45°F in any 1 hour period

**Unit 1 Wide Range Heatup Curve**



RCS TEMPERATURE	MAX HEATUP RATE
T < 280°F	≤ 45°F in any 1 hour period
T ≥ 280°F	≤ 90°F in any 1 hour period

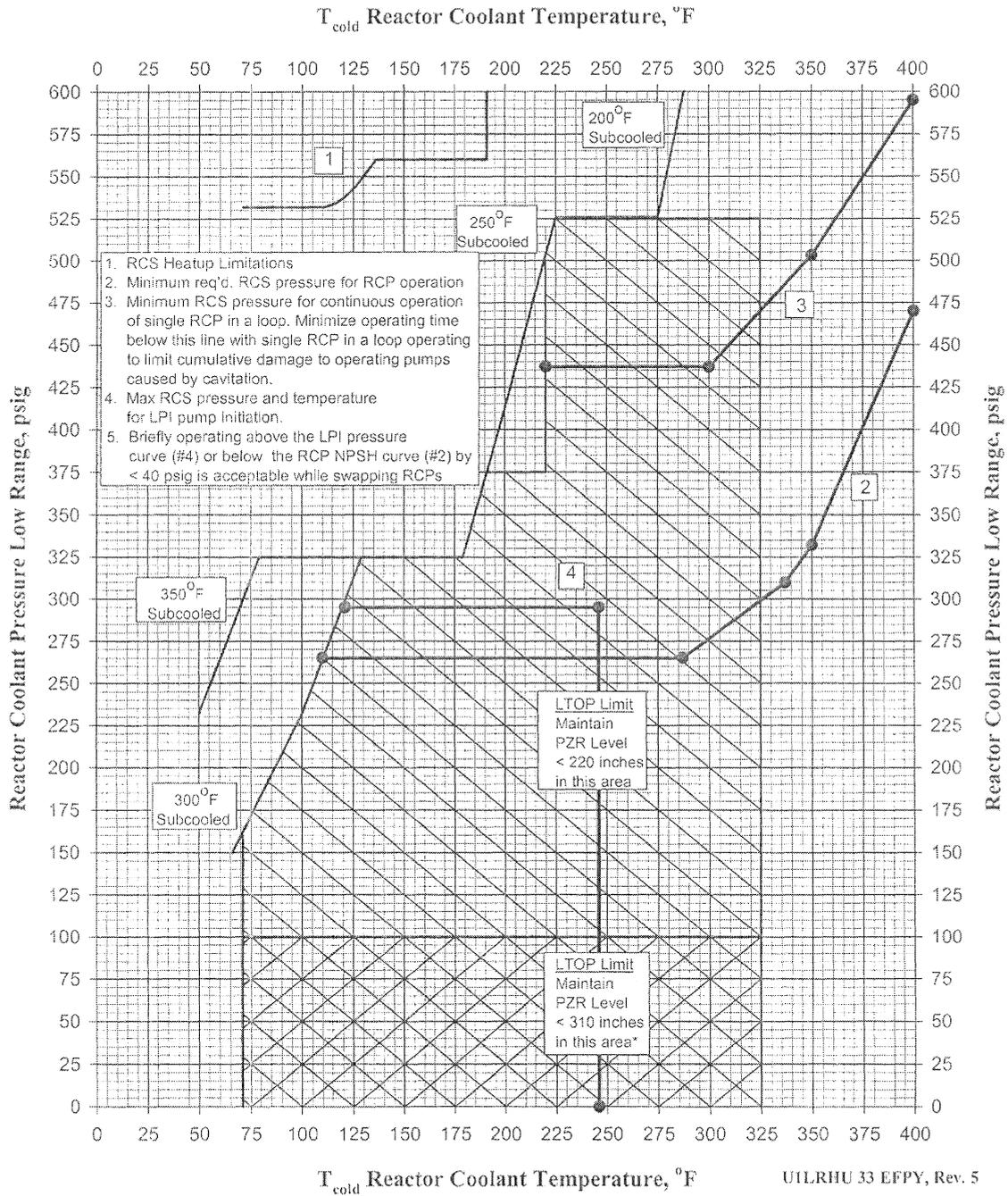
**Unit 1 Low Range Cooldown Curve**



\* PZR level restricted to ≤ 380 inches when RCS temperature is ≤ 160°F, and NO HPIPs operating.

RCS TEMPERATURE	MAX COOLDOWN RATE
T ≥ 280°F	≤ 45°F in any 1/2 hour period
150°F ≤ T < 280°F	≤ 20°F in any 1/2 hour period
T < 150°F	≤ 9°F in any 1 hour period
RCS depressurized (note 5)	≤ 45°F in any 1 hour period

**Unit 1 Low Range Heatup Curve**



UILRHU 33 EPFY, Rev. 5

\* PZR level restricted to ≤ 380 inches when RCS temperature is ≤ 160°F and NO HPIPs operating.

RCS TEMPERATURE	MAX HEATUP RATE
T < 280°F	≤ 45°F in any 1 hour period
T ≥ 280°F	≤ 90°F in any 1 hour period

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**1 POINT**

**Question 87**

Unit 1 initial conditions:

- Unit startup and heatup in progress
- OP/1102/001 Controlling procedure for Unit Startup in progress

Based on the above conditions, choose the response below that answers the following questions:

- a) Which ONE of the following describes the RCS pressure band at which the Core Flood tanks are placed in service per OP/1104/001 (Core Flood Tank Operation)?
  - A. 730 - 750 psig  
At a reduced pressure, safety injection pumps can provide sufficient flow to prevent exceeding cladding temperature limits during a Large Break LOCA.
  - B. 780 - 800 psig  
At a reduced pressure, safety injection pumps can provide sufficient flow to prevent exceeding cladding temperature limits during a Large Break LOCA.
  - C. 730 - 750 psig  
At a reduced pressure safety injection pumps can provide sufficient flow to ensure the core remains completely covered during a Large Break LOCA.
  - D. 780 - 800 psig  
At a reduced pressure safety injection pumps can provide sufficient flow to ensure the core remains completely covered during a Large Break LOCA.
- b) What is the Tech Spec Bases for not requiring CFT operability at RCS pressures below the TS limit?

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**1 POINT**

**Question 88**

Unit 1 plant conditions:

- Reactor power = 100%
- 'A' RPS Variable Low RCS Pressure Trip Bistable trips
- The crew determines that the bistable has failed
- The bistable output state cannot be reset by the crew
- SPOC has been contacted to investigate
- While performing action for the failed channel, the RO inadvertently places the 'B' RPS Channel to MANUAL BYPASS

Based on the above conditions, which ONE of the following correctly describes the effect on the unit when the "B" RPS channel is placed in MANUAL BYPASS and the operability of the RPS Variable Low RCS Pressure Trip function(s) at this time?

**ASSUME NO OPERATOR ACTION**

- A. The reactor trips  
'A' RPS Variable Low RCS Pressure Trip function is no longer required to be operable
- B. The reactor trips  
'A' RPS Variable Low RCS Pressure Trip function is operable
- C. The reactor remains at power  
RPS Variable Low RCS Pressure Trip function is operable
- D. The reactor remains at power  
RPS Variable Low RCS Pressure Trip function is NOT operable

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**1 POINT**

**Question 89**

Unit 1 initial conditions:

- Reactor power = 100%
- 1A and 1C RBCUs operating in low speed
- 1C and 1B RBCUs will be switched for run time

Current conditions:

- 1B RBS pump in recirc for testing
- 1C RBCU secured
- 1B RBCU started in low speed
- OAC alarm 1B RBCU Vibration HIGH and can not be cleared

Based on the current conditions, which ONE of the following describes actions to be taken per OP/1104/015 (Reactor Building Cooling), and whether RB pressure will be maintained below the design pressure during a subsequent LOCA?

- A. Start 1C RBCU, then stop 1B RBCU  
RB pressure will be maintained below design pressure
- B. Start 1C RBCU, then stop 1B RBCU  
RB pressure will NOT be maintained below design pressure
- C. Stop 1B RBCU, wait 30 minutes to start 1C RBCU if desired  
RB pressure will be maintained below design pressure
- D. Stop 1B RBCU, wait 30 minutes to start 1C RBCU if desired  
RB pressure will NOT be maintained below design pressure

**1 POINT**

**Question 90**

Unit 1 plant conditions:

Time = 0400:00

- Reactor power = 75% power
- AMSAC/DSS Bypassed
- TDEFDW Pump out of service
- 1A FDW Pump trips

Time = 0402:00

- 1B FDW Pump trips

Time = 0402:05

- Switchyard Isolation occurs

Which ONE of the following describes the technical specification basis for the automatic reactor trip and the EOP tab that the SRO will direct?

**ASSUME NO ADDITIONAL FAILURES OCCUR**

- A. Minimize challenges to the Pzr PORV / Subsequent Actions
- B. Minimize challenges to the Pzr PORV / Loss of Heat Transfer
- C. Prevent exceeding DNBR limits / Subsequent Actions
- D. Prevent exceeding DNBR limits / Loss of Heat Transfer

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**1 POINT**

**Question 91**

Unit 3 plant conditions:

- Shutdown for refueling in progress
- Mode 4
- Component Handling in progress in the Spent Fuel Pool
- A fuel assembly is currently in the mast and being moved
- 3RIA-6 (SFP Area Monitor) shows an observable increase, approximately  $\frac{1}{2}$  decade above background
- Spent Fuel Pool level = (-) 2.7 feet decreasing

Based on the above conditions, which ONE of the following describes the Abnormal Procedure that will provide direction for system operations to mitigate the event and a required Technical Specification entry and its bases?

- A. Enter AP/35 (Loss of SFP Cooling and/or Level)  
TS 3.10.1 (SSF) - Ensures the RC Makeup pump can maintain all three Oconee Units in MODE 3 for a minimum of 72 hours.
- B. Enter AP/35 (Loss of SFP Cooling and/or Level)  
TS 3.7.11 (Spent Fuel Pool Water Level) - Ensures adequate iodine removal during a fuel handling accident.
- C. Enter AP/18 (Abnormal Release of Radioactivity)  
TS 3.10.1 (SSF) - Ensures the RC Makeup pump can maintain all three Oconee Units in MODE 3 for a minimum of 72 hours.
- D. Enter AP/18 (Abnormal Release of Radioactivity)  
TS 3.7.11 (Spent Fuel Pool Water Level) - Ensures adequate iodine removal during a fuel handling accident.

**1 POINT**

**Question 92**

Unit 1 initial conditions:

- Reactor power = 50%
- A turbine trip occurs
- 1A SG Startup Control Valve fails open

Current conditions:

- 1A SG level = 97% Operating Range (OR) increasing

Based on the above conditions, which ONE of the following describes the actions required by the Subsequent Actions tab of the EOP and the basis?

- A. Trip BOTH Main FDW pumps and steam the 1A SG to prevent flooding the aspirating ports
- B. Trip BOTH Main FDW pumps and close the 1A TBVs to prevent water hammer in the main steam line
- C. Manually throttle Main FDW to control SG level < 96% and steam the 1A SG to prevent flooding the aspirating ports
- D. Manually throttle Main FDW to control SG level < 96% and close the 1A TBVs to prevent water hammer in the main steam line

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**1 POINT**

**Question 93**

Plant conditions:

- All 3 units are in Mode 1
- 1SA-3/B-6 (Fire Alarm) actuated
- The alarm was generated from address point 0202041
- Subsequently, a Challenging Active Fire is reported in the affected area
- The fire continues for approximately 15 minutes
- The affected control room(s) have been evacuated
- All subsequent required actions were taken within 10 minutes

Which ONE of the following describes ALL of the units affected by the fire in accordance with the alarm response, and which ONE of the following describes the minimum emergency classification for this event?

**SEE ATTACHMENT**

- A. Units 1 and 2 ONLY; Unusual Event
- B. Units 1 and 2 ONLY; Alert
- C. All 3 Units; Unusual Event
- D. All 3 Units; Alert

**1 POINT**

**Question 94**

Unit 1 initial conditions:

- Reactor power = 100%
- Loss of all Main and Emergency feedwater occurs
- All CBPs are tripped
- Rule 3 (Loss of Main or Emergency FDW) initiated
- Loss of Heat Transfer tab initiated

Current conditions:

- RCS pressure = 2305 psig increasing
- Rule 4 (Initiation of HPI Forced Cooling) has just been initiated
- 1B HPIP tripped

Based on the above conditions, which ONE of the following describes the correct action and the requirements for plant announcement?

- A. Ensure adequate HPI flow in both headers / "Emergency dose limits in effect" will be announced to plant staff per the parallel action page.
- B. Open reactor vessel head and loop vents / "Emergency dose limits in effect" will be announced to plant staff per the parallel action page.
- C. Ensure adequate HPI flow in both headers / "Emergency dose limits in effect" will NOT be announced to plant staff since Emergency Dose limits have previously been in effect.
- D. Open reactor vessel head and loop vents / "Emergency dose limits in effect" will NOT be announced to plant staff since Emergency Dose limits have previously been in effect.

**1 POINT**

**Question 95**

Unit 1 initial conditions:

- Mode 6
- Fuel Transfer Canal level = 21.5 feet
- 1C LPI pump operating

Current conditions:

- Main Fuel Bridge operator has requested temporarily securing 1C LPI pump to facilitate fuel assembly insertion

Based on the above conditions, which ONE of the following describes who approves securing the 1C LPI pump in accordance with OP/1502/007 (Operations Defueling/Refueling Responsibilities) and any additional requirements?

- A. Refueling SRO / 1C LPI pump may be stopped provided movement of irradiated fuel assemblies is suspended immediately
- B. Operations Outage Manager / 1C LPI pump may be stopped provided movement of irradiated fuel assemblies is suspended immediately
- C. Refueling SRO / 1C LPI pump may be stopped for up to 1 hour per 8 hour period
- D. Operations Outage Manager / 1C LPI pump may be stopped for up to 1 hour per 8 hour period

**1 POINT**

**Question 96**

Unit 1 plant conditions:

- MODE 2
- RCS pressure = 2755 psig

Which ONE of the following states the Technical Specification required action and its basis?

- A. Restore RCS pressure within 5 minutes  
5 minute time limit is based on the increased probability of brittle fracture
- B. Restore RCS pressure within 5 minutes  
5 minute time limit is based on the capacity of the PORV to return system pressure to within the safety limit
- C. Restore RCS pressure and be in MODE 3 within 1 hour  
1 hour time limit is based on changing to a mode where the potential for additional safety system challenges is minimized
- D. Restore RCS pressure and be in MODE 3 within 1 hour  
1 hour time limit is based on the capacity of the PORV to return system pressure to within the safety limit

**1 POINT**

**Question 97**

Unit 1 plant conditions:

- Startup in progress
- Control rod group 1 = 100% withdrawn
- Control rod group 2 = 100% withdrawn
- Control rod group 3 = 100% withdrawn
- Control rod group 4 = 0 % withdrawn
- TD EFDW pump OOS

Based on the above conditions, which ONE of the following describes the current technical specification operational MODE, and whether a change to the next higher Mode is allowed and why?

A. Mode 2

No, T.S. 3.0.4 does NOT allow a Mode change unless the applicable action statements for the next higher Mode allow continued operation without time limits.

B. Mode 2

Yes. T.S. 3.0.4 allows a Mode change as long as the applicable condition statements are entered upon entry to the next higher Mode.

C. Mode 3

No. T.S. 3.0.4 does NOT allow a Mode change unless the applicable action statements for the next higher Mode allow continued operation without time limits.

D. Mode 3

Yes. T.S. 3.0.4 allows a Mode change as long as the applicable condition statements are entered upon entry to the next higher Mode.

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**1 POINT**

**Question 98**

Which ONE of the following describes the MINIMUM level of authority for approval of ONE (1) GWR release at the 1/3 Station Limit, and the SLC bases for limiting the Curie content of the GWD Tanks?

- A. OSM approval / limits Whole Body exposure of individual at the nearest exclusion boundary to  $\leq 0.5$  Rem in the event of an GWD tank rupture.
- B. OSM approval / limits Whole Body exposure of individual at the nearest exclusion boundary to  $\leq 100$  mrem during a planned GWD tank release.
- C. SRO approval / limits Whole Body exposure of individual at the nearest exclusion boundary to  $\leq 0.5$  Rem in the event of an GWD tank rupture.
- D. SRO approval / limits Whole Body exposure of individual at the nearest exclusion boundary to  $\leq 100$  mrem during a planned GWD tank release.

**1 POINT**

**Question 99**

Unit 1 initial conditions:

- Reactor power = 100%
- RCS pressure = 1900 psig decreasing

Current conditions:

- A manual reactor trip was initiated
- Numerous control rods indicate fully withdrawn
- Reactor power indicates 4% and stable
- RCS pressure is decreasing rapidly
- SCM = 0°F
- IMAs in progress

Which ONE of the following describes how the SRO will transition through the EOP for this event?

- A. Transfer to Subsequent Actions  
Transfer to Loss of Subcooling Margin Tab using Parallel Actions page  
Transfer to the LOCA Cooldown Tab from LOSCM tab
- B. Transfer to Subsequent Actions  
Transfer to Unanticipated Nuclear Power Production tab using Parallel Actions page  
Transfer to Loss of Subcooling Margin Tab using Parallel Actions page when UNPP tab is complete
- C. Transfer to Unanticipated Nuclear Power Production Tab  
Transfer to Subsequent Actions at the end of the UNPP tab  
Transfer to Loss of Subcooling Margin Tab using Parallel Actions
- D. Transfer to Unanticipated Nuclear Power Production Tab  
Transfer to Loss of Subcooling Margin Tab using Parallel Actions page  
Transfer to the LOCA Cooldown Tab from LOSCM tab

**1 POINT**

**Question 100**

Unit 1 initial conditions:

- Reactor power = 100%
- LOHT occurs
- Rule 3 initiated

Current conditions:

- EOP LOHT tab in progress
- 1A SG pressure = 980 psig stable
- 1B SG pressure = 200 psig decreasing

Based on the above conditions, which ONE of the following describes the proper progression in the EOP and why?

- Remain in the LOHT tab and reduce heat sources in the RCS by securing all RCPs.
- Remain in the LOHT tab and attempt to re-establish a heat sink with the intact SG.
- Transfer to the EHT tab to address the steam line break while attempting to establish feedwater flow with the TD EFDWP.
- Transfer to the EHT tab to provide guidance on stabilizing RCS press / temp to prevent a Pressurized Thermal Shock condition.