(1) Maximum Power Level

Nine Mile Point Nuclear Station, LLC, is authorized to operate the facility at reactor core power levels not in excess of 3467 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained In Appendix A and the Environmental Protection Plan contained In Appendix B, both of which are attached hereto, as revised through Amendment No. 124 are hereby incorporated into this license. Nine Mile Point Nuclear Station, LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Fuel Storage and Handling (Section 9.1, SSER 4)*

a. Fuel assemblies, when stored in their shipping containers, shall be stacked no more than three containers high.

b. When not in the reactor vessel, no more than three fuel assemblies shall be allowed outside of their shipping containers or storage racks in the New Fuel Vault or Spent Fuel Storage Facility.

c. The above three fuel assemblies shall maintain a minimum edge-to-edge spacing of twelve (12) inches from the shipping container array and approved storage rack locations.

d. The New Fuel Storage Vault shall have no more than ten fresh fuel assemblies uncovered at any one time.

(4) Turbine System Maintenance Program (Section 3.5,1.3.10, SER)

The operating licensee shall submit for NRC approval by October 31, 1989, a turbine system maintenance program based on the manufacturer's calculations of missile generation probabilities. (Submitted by NMPC letter dated October 30, 1989, from C.D. Terry and approved by NRC letter dated March 15, 1990, from Robert Martin to Mr. Lawrence Burkhardt, III).

* The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report (SER) and/or its supplements wherein the license condition is discussed.

(10) Additional Condition 1

The operating licensee is authorized by Amendment No. 91 to relocate certain Technical Specification requirements previously included in Appendix A to licensee-controlled documents, as described in Table R, Relocated Specifications and Removal of Details Matrix, attached to the NRC Staff's safety evaluation dated February 15, 2000, enclosed with the amendment. Implementation of Amendment No. 91 shall include the relocation of these requirements to the appropriate documents, which shall be completed no later than December 31, 2000. The relocations to the Updated Safety Analysis Report shall be reflected in updates completed in accordance with 10 CFR 50.71(e).

(11) Additional Condition 2

The schedule for performing Surveillance Requirements (SRs) that are new or revised in Amendment No. 91 shall be as follows:

For SRs that are new in this amendment, the first performance is due at the end of the first surveillance interval that begins on the date of implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of this amendment.

For SRs that existed prior to this amendment that have modified acceptance criteria, the first performance is due at the end of the first surveillance interval that began on the date the surveillance was last performed prior to the implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being extended, the first extended surveillance interval begins upon completion of the last surveillance performed prior to the implementation of this amendment.

(11a) Deleted

(12) On the closing date(s) of the transfer of the NMPC, RG&E, CHGEC, and NYSEG interests in NMP-2 to it, Nine Mile Point Nuclear Station, LLC shall: (1) obtain from the transferors then transferring their interests all of their accumulated decommissioning trust funds for NMP-2, and (2)

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(continued)

SURVEILLANCE REQUIREMENTS (continued)

| SURVEILLANCE | | FREQUENCY |
|---|---|---|
| SR 3.3.3.1.2 | Perform CHANNEL CALIBRATION. | 24 months |

Table 3.3.3.1-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

| | FUNCTION | REQUIRED CHANNELS | CONDITIONS REFERENCED FROM REQUIRED ACTION D.1 |
|---|---|---|---|
| 1. | Reactor Vessel Pressure | 2 | E |
| 2. | Reactor Vessel Water Level | | |
| | a. Fuel Zone Range | 2 | E |
| | b. Wide Range | 2 | E |
| 3. | Suppression Pool Water Level | | |
| | a. Narrow Range | 2 | E |
| | b. Wide Range | 2 | E |
| 4. | Drywell Pressure | | |
| | a. Narrow Range | 2 | E |
| | b. Wide Range | 2 | E |
| 5. | Drywell Radiation (High Range) | 2 | F |
| 6. | Drywell Air Temperature | 2 | E |
| 7. | Suppression Chamber Pressure | 2 | E |
| 8. | PCIV Position | 2 per penetration flow path[(a)(b)] | E |
| 9. | Suppression Pool Water Temperature | 2[(c)] | E |

(a) Not required for isolation valves whose associated penetration flow path is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) Monitoring each suppression pool quadrant.

## 3.6 CONTAINMENT SYSTEMS

### 3.6.3.1 Deleted