

April 15, 2008

Vice President, Operations  
Entergy Nuclear Operations  
Palisades Nuclear Plant  
27780 Blue Star Memorial Highway  
Covert, MI 49043-9530

SUBJECT: PALISADES PLANT - ISSUANCE OF AMENDMENT RE: REVISION TO  
TECHNICAL SPECIFICATION 5.5.7 INSERVICE TESTING PROGRAM  
(TAC NO. MD5713)

Dear Sir or Madam:

The Commission has issued the enclosed Amendment No. 232 to Renewed Facility Operating License No. DPR-20 for the Palisades Nuclear Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to Entergy Nuclear Operations, Inc. application dated May 22, 2007.

The amendment incorporates TS changes based on Nuclear Regulatory Commission (NRC)-approved TS Task Force (TSTF)-497-A, "Changes to Reflect Revision of 10 CFR 50.55a," Revision 0, as modified by NRC-approved TSTF-497, "Limit Inservice Testing Program [Surveillance Requirements] SR 3.0.2 Application to Frequencies of Two years or Less." Specifically, the amendment revises Palisades Nuclear Plant TS Section 5.5.7, "Inservice Testing Program," to update references to the American Society of Mechanical Engineers code and applicability of the provisions of SR 3.0.2.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

**/RA/**

Mahesh L. Chawla, Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-255

Enclosures:

1. Amendment No. 232 to DPR-20
2. Safety Evaluation

cc w/encls: See next page

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Palisades Plant

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March 3, 2008

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-255

PALISADES PLANT

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 232  
License No. DPR-20

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Nuclear Operations, Inc. (the licensee), dated May 22, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to the license amendment and Paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-20 is hereby amended to read as follows:

The Technical Specifications contained in Appendix A, as revised through Amendment No. 232 , and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. Entergy Nuclear Operations (ENO) shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

**/RA/**

Lois M. James, Chief  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility Operating License  
and Technical Specifications

Date of Issuance: April 15, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 232  
RENEWED FACILITY OPERATING LICENSE NO. DPR-20  
DOCKET NO. 50-255

Replace the following page of the Renewed Facility Operating License No. DPR-20 with the attached revised page. The changed area is identified by a marginal line.

REMOVE

INSERT

Page 3

Page 3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE  
5.0-11

INSERT  
5.0-11

- (1) Pursuant to Section 104b of the Act, as amended, and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," (a) ENP to possess and use, and (b) ENO to possess, use and operate, the facility as a utilization facility at the designated location in Van Buren County, Michigan, in accordance with the procedures and limitation set forth in this license;
  - (2) ENO, pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use source and special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report, as supplemented and amended;
  - (3) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use byproduct, source, and special nuclear material as sealed sources for reactor startup, reactor instrumentation, radiation monitoring equipment calibration, and fission detectors in amounts as required;
  - (4) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material for sample analysis or instrument calibration, or associated with radioactive apparatus or components; and
  - (5) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operations of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations in 10 CFR Chapter I and is subject to all applicable provisions of the Act; to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) ENO is authorized to operate the facility at steady-state reactor core power levels not in excess of 2565.4 Megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.
  - (2) The Technical Specifications contained in Appendix A, as revised through Amendment No. 232 and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
  - (3) ENO shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SERs dated 09/01/78, 03/19/80, 02/10/81, 05/26/83, 07/12/85, 01/29/86, 12/03/87, and 05/19/89 and subject to the following provisions:

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 232 TO RENEWED

FACILITY OPERATING LICENSE NO. DPR-20

ENTERGY NUCLEAR OPERATIONS, INC.

PALISADES PLANT

DOCKET NO. 50-255

1.0 INTRODUCTION

By application dated May 22, 2007 (Agencywide Documents and Management System No. ML071500067), the Entergy Nuclear Operations, Inc., (the licensee), requested changes to the Technical Specifications (TSs) for the Palisades Nuclear Plant. The proposed changes would incorporate TS changes based on Nuclear Regulatory Commission (NRC)-approved TS Task Force (TSTF)-497-A, "Changes to Reflect Revision of 10 CFR [*Title 10 of the Code of Federal Regulations*] 50.55a," Revision 0, as modified by NRC-approved TSTF-497, "Limit Inservice Testing Program [Surveillance Requirements] SR 3.0.2 Application to Frequencies of Two Years or Less." Specifically, the amendment would revise Palisades Nuclear Plant TS Section 5.5.7, "Inservice Testing Program," to update references to the American Society of Mechanical Engineers code and applicability of the provisions of SR 3.0.2.

2.0 REGULATORY EVALUATION

2.1 Section 50.36 of 10 CFR

The regulation at 10 CFR Part 50 includes the Nuclear Regulatory Commission's requirement that TSs shall be included by applicants for a license authorizing operation of a production or utilization facility. The NRC requires TSs to be derived from the analyses and evaluation included in the safety analysis report, and amendment thereto, submitted pursuant to 10 CFR 50.34. Moreover, 10 CFR 50.36 (d) requires that the TSs include items in five specific categories related to facility operation. These categories are (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operations (LCOs); (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls. The proposed TS change is within the administrative controls category.

The NRC staff reviewed the proposed changes for compliance with Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36 and agreement with the precedent as established in NUREG-1432, Revision 3.1, "Standard Technical Specifications, Combustion Engineering Plants," dated December 1, 2005. In general, licensees cannot justify TS changes solely on the basis of adopting the model Standard Technical Specifications (STS). To ensure this, the staff makes a determination that proposed changes maintain adequate safety. Changes that result in relaxation (less restrictive condition) of current TS requirements require detailed justification.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time. These amendments deal with the second class of change; namely, administrative changes that reflect the current configuration of the plant.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 3.0 in the context of specific proposed changes.

## 2.2 Section 50.55a of 10 CFR

Section 50.55a(f) of 10 CFR, "Inservice Testing Requirements," requires, in part, that ASME Class 1, 2, and 3 components must meet the requirements of the ASME Code. Section 50.55a(f)(4)(ii) of 10 CFR states, " Inservice tests to verify operational readiness of pumps and valves, whose function is required for safety, conducted during successive 120-month intervals must comply with the requirements of the latest edition and addenda of the Code incorporated by reference in paragraph (b) of this section 12 months before the start of the 120-month interval (or the optional ASME Code cases listed in NRC Regulatory Guide 1.147, through Revision 14, or 1.192 that are incorporated by reference in paragraph (b) of this section), subject to limitations and modifications listed in paragraph (b) of this section."

Section 50.55a(f)(5)(ii) of 10 CFR requires that, if a revised IST program for a facility conflicts with the TS for that facility, the licensee shall apply to the Commission for amendment of the TS to conform the TS to the revised program. The licensee is required to submit the application, as specified in 10 CFR 50.4, at least 6 months before the start of the period during which the provisions become applicable, as determined by 10 CFR 50.55a(f)(4).

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Boiler and Pressure Vessel Code. The ASME deleted the rules for IST of pumps and valves from Section XI in the 2000 Addenda because the rules for the IST of pumps and valves were placed in the ASME OM Code. The Palisades Nuclear Plant (PNP) fourth 10-year IST interval program for pumps and valves were developed to meet the requirements of the 2001 Edition including the 2002 and 2003 Addenda of the ASME OM Code. Entergy submitted this TS amendment to revise TS 5.5.7 to reference the pump and valve IST requirements in the ASME OM Code requirements and delete the reference to Section XI IST requirements.

The NRC staff findings with respect to authorizing the TS amendment are given below.

## 3.0 TECHNICAL EVALUATION

### 3.1 SR 3.0.2 Applicability

The licensee's TS revision proposes to make changes that are editorial, administrative or provide clarification. In order for these changes to be acceptable, the NRC staff must determine that the changes are compatible with current TS, do not result in any substantive change in operating requirements, and are consistent with the Commissions regulations.

The licensee proposes to adopt the administrative TS changes contained in TSTF-479, Revision 0 (TSTF-479), "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0 (TSTF-497), "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less." TSTF-479 was approved by the NRC staff in a letter dated December 6, 2005 (ADAMS Accession No. ML053460302). TSTF-497 was approved by the NRC staff in a letter dated October 4, 2006 (ADAMS Accession No. ML062780321). TSTF-479 revised the IST Program located in Chapter 5 of the STS to reflect the latest approved version of the American Society of Mechanical Engineers Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code) consistent with the requirements of 10 CFR 50.55a. TSTF-479 also revised paragraph b of STS Chapter 5 IST Program by adding "The provisions of SR 3.0.2 are applicable to the above required Frequencies and other normal and accelerated Frequencies specified in the IST Program for performing inservice testing activities." This requirement referred to testing frequencies specified in the table in paragraph a of STS Chapter 5, which only lists test frequencies of 2 years or less. To clarify the applicability of the provisions of TS Surveillance Requirement (SR) 3.0.2, TSTF-497 revised the sentence in paragraph b to state: "The provisions of SR 3.0.2 are applicable to the above required Frequencies and to other normal and accelerated Frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities." This is an administrative change to clearly state that the provisions of SR 3.0.2 are applicable to IST frequencies of 2 years or less.

The licensee's proposed TS revision contains a minor administrative deviation from the TSTFs. The deviation is the use of the term interval instead of frequencies in paragraph b of TS Section 5.5.7. This deviation is consistent with the current text in the TS. Other than the noted deviation, the licensee's TS revision fully adopts TSTF-479 and TSTF-497 changes to Section 5.5.7 of the Palisades Nuclear Plant TSs to reflect the latest approved version of the ASME OM Code, the applicability of the provisions of SR 3.0.2, and additional wording to clarify that the test interval extension in the IST program applies only to test intervals of 2 years or less. The staff finds no TS requirements to be materially altered by the proposed TS changes, and finds the proposed TS changes to be administrative. Therefore, the staff finds the proposed TS changes acceptable.

### 3.1 ASME Code Applicability

The ASME publishes a new edition of the ASME Code every 3 years. New addendums to the ASME Code are normally published in the years between the two editions. The PNP third interval IST program began August 21, 1995. The applicable ASME Code, Section XI, for the PNP third 10-year IST interval is the 1989 Edition. The PNP fourth interval IST program is being updated to comply with the 2001 Edition through 2003 Addenda of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii). As a consequence, the PNP TS 5.5.7 reference to Section XI of the ASME Code for IST requirements must be revised to reference the ASME OM Code.

The 2001 Edition through 2003 Addenda of the ASME OM Code was incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated October 1, 2004 (69 FR 58804) and is currently the latest edition and addenda of the ASME OM Code incorporated by reference in 10 CFR 50.55a(b)(3). On this basis, the staff finds the proposed TS change acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Michigan official was notified of the proposed issuance of the amendment. The Michigan State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (72 FR 49575). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Steven G. Tingen, CPTB  
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Date: April 15, 2008