March 19, 2008

Mr. J. A. Stall
Senior Vice president, Nuclear and
Chief Nuclear Officer
Florida Power and Light Company
P.O. Box 1400
Juno Beach, Florida 33408-0420

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - ST. LUCIE PLANT, UNITS 1 AND 2 - TECHNICAL SPECIFICATION AMENDMENT FOR ALTERNATIVE SOURCE TERM (TAC NOS. MD6173 AND MD6202)

Dear Mr. Stall:

By letter dated July 16, 2007, Florida Power & Light Company (the licensee) submitted a request to amend the Operating License DPR-67 and Technical Specifications for St. Lucie, Unit 1 for Alternative Source Term as allowed by Title 10 of the *Code of Federal Regulations,* Section 50.67. We have reviewed the submittal and have determined that we need additional information before we can complete our review. Enclosed is our request for additional information.

This request was discussed with Mr. Ken Frehafer of your staff on February 12, 2008, and the NRC staff confirmed that the questions applied to both units. It was agreed that a final response would be provided by April 22, 2008. If you have any questions, please feel free to contact me at 301-415-2020.

Sincerely,

/RA/

Brenda L. Mozafari, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

Enclosure: Request for Additional Information

cc: See next page

REQUEST FOR ADDITIONAL INFORMATION

ST. LUCIE UNITS 1 AND 2

ALTERNATIVE SOURCE TERM AND CONFORMING AMENDMENT

TAC NOS. MD6173 AND MD6202

Pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.67, Florida Power & Light Company (the licensee) is requesting a license amendment for St. Lucie Unit No. 1 to adopt alternative source term (AST) as allowed in 10 CFR 50.67. The licensee stated that analyses were performed for Loss-of-Coolant Accident, Fuel-handling Accident, Main Steam Line Break, Steam Generator Tube Rupture, Reactor Coolant Pump Shaft Seizure (Locked Rotor), Control Element Assembly Ejection, and Inadvertent Opening of a Main Steam Safety Valve, based on a bounding allowable control room unfiltered air in-leakage of 500 cfm. Increased values for control room unfiltered air in-leakage were assumed. The following additional information is requested to clarify the following items.

1. In accordance with Regulatory Guide 1.183, evaluations may need to be performed regarding the ability of the damper to close against increased containment pressure or the ability of the ductwork downstream of the dampers to withstand increased stresses. Confirm whether any evaluations were performed and documented to determine the adequacy of the damper to close against any increased containment pressure. Provide a summary of the structural evaluation of the damper.

2. Provide a summary of the structural evaluations for the ductwork (and duct supports) downstream of the dampers. Include information about the duct sizes, analysis methodology, design codes utilized, load combinations, stress levels and margins.

3. Provide a list of any mechanical equipment items and or systems requiring modification as a result of the implementation of the AST term at Saint Lucie Unit 1.

   a) Indicate whether the equipment is new or existing.

   b) Describe the location of the equipment and the seismic qualification method employed such as the plant licensing basis or an NRC-endorsed industry standard.

   c) Provide the results of the seismic qualification of the equipment indicating whether any modification or re-design is necessary.

ENCLOSURE

March 19, 2008

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Chief Nuclear Officer
Florida Power and Light Company
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NAME	BMozafari	BClayton	KManoly By memo dated	TBoyce
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cc:
Mr. Mano Nazar
Senior Vice President
and Nuclear Chief Operating Officer
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408

Senior Resident Inspector
St. Lucie Plant
U.S. Nuclear Regulatory Commission
P.O. Box 6090
Jensen Beach, Florida 34957

Craig Fugate, Director
Division of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, Florida 32399-2100

M. S. Ross, Managing Attorney
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

Marjan Mashhadi, Senior Attorney
Florida Power & Light Company
801 Pennsylvania Avenue, NW.
Suite 220
Washington, DC 20004

Mr. Douglas Anderson
County Administrator
St. Lucie County
2300 Virginia Avenue
Fort Pierce, Florida 34982

Mr. William A. Passetti, Chief
Department of Health
Bureau of Radiation Control
2020 Capital Circle, SE, Bin #C21
Tallahassee, Florida 32399-1741

Mr. Gordon L. Johnston
Site Vice President
St. Lucie Nuclear Plant
6351 South Ocean Drive
Jensen Beach, Florida 34957-2000

Mr. Christopher R. Costanzo
Plant General Manager
St. Lucie Nuclear Plant
6351 South Ocean Drive
Jensen Beach, Florida 34957

Mr. Terry Patterson
Licensing Manager
St. Lucie Nuclear Plant
6351 South Ocean Drive
Jensen Beach, Florida 34957

Don E. Grissette
Vice President, Nuclear Training
and Performance Improvement
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

Mr. Rajiv S. Kundalkar
Vice President - Nuclear Technical Services
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

Mr. J. Kammel
Radiological Emergency
Planning Administrator
Department of Public Safety
6000 Southeast Tower Drive
Stuart, Florida 34997

Mr. Bill Parks
Operations Manager
St. Lucie Nuclear Plant
6351 South Ocean Drive
Jensen Beach, Florida 34957-2000

Mr. Seth B. Duston
Training Manager
St. Lucie Nuclear Plant
6351 South Ocean Drive
Jensen Beach, Florida 34957-2000