Mr. Charles G. Pardee
Chief Nuclear Officer and Senior Vice President
Exelon Generation Company, LLC
President and Chief Executive Officer
AmerGen Energy Company, LLC
4300 Winfield Road
Warrenville, IL 60555

August 28, 2008

SUBJECT: BRAIDWOOD STATION, UNITS 1 AND 2; BYRON STATION, UNIT NOS. 1 AND 2; DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3; LIMERICK GENERATING STATION, UNITS 1 AND 2; OYSTER CREEK NUCLEAR GENERATING STATION; PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3; QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2; AND THREE MILE ISLAND NUCLEAR STATION, UNIT 1 – ISSUANCE OF AMENDMENTS THAT ADOPT TECHNICAL SPECIFICATION TASK FORCE (TSTF) CHANGE TRAVELER TSTF-479 AND TSTF-497 (TAC NOS. MD6530 THRU MD6543)

Dear Mr. Pardee:

The Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 153 to Facility Operating License No. NPF-72 and Amendment No. 153 to Facility Operating License No. NPF-77 for the Braidwood Station, Units 1 and 2, respectively, and Amendment No. 157 to Facility Operating License No. NPF-37 and Amendment No. 157 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 229 to Renewed Facility Operating License No. DPR-19 and Amendment No. 222 to Renewed Facility Operating License No. DPR-25 for Dresden Nuclear Power Station, Units 2 and 3, respectively, and Amendment No. 194 to Facility Operating License No. NPF-39 and Amendment No. 155 to Facility Operating License No. NPF-85 for the Limerick Generating Station, Units 1 and 2 (LGS), respectively, and Amendment No. 268 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station (Oyster Creek), and Amendment No. 268 o Facility Operating License No. DPR-44 and Amendment No. 272 to Facility Operating License No. DPR-56 for the Peach Bottom Atomic Power Station, Units 2 and 3, respectively, and Amendment No. 241 to Renewed Facility Operating License No. DPR-29 and Amendment No. 236 to Renewed Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2, respectively, and Amendment No. 266 to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit 1 (TMI-1). The amendments are in response to Exelon Generation Corporation's, LLC, and AmerGen Energy Company's, LLC application dated August 8, 2007.

The proposed changes would replace references to Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) with references to the ASME Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code) in the applicable technical specification (TS) section for the inservice testing (IST) program for the licensee's plants that have implemented industry's improved technical specifications (ITS).

Similar changes were proposed by the licensee for its non-ITS plants Oyster Creek, LGS, and TMI-1. The proposed changes are based on TSTF 479, Revision 0, “Changes to Reflect Revision of 10 CFR 50.55a.” For all units except Oyster Creek and TMI-1, the amendments also incorporate TSTF-497, Revision 0, “Limit Inservice Testing Program SR [Surveillance Requirement] 3.0.2 Application to Frequencies of 2 Years or Less,” which adds a provision in the applicable TS section to only apply the extension allowance of SR 3.0.2 to the frequency table listed in the TSs as part of the IST program and to normal and accelerated inservice testing frequencies of two years or less, as applicable.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Christopher Gratton, Senior Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456, STN 50-457, STN 50-454,
STN 50-455, 50-237, 50-249, 50-352, 50-353, 50-219,
50-277, 50-278, 50-254, 50-265, and 50-289

Enclosures:
1. Amendment No. 153 to NPF-72
2. Amendment No. 153 to NPF-77
3. Amendment No. 157 to NPF-37
4. Amendment No. 157 to NPF-66
5. Amendment No. 229 to DPR-19
6. Amendment No. 222 to DPR-25
7. Amendment No. 194 to NPF-39
8. Amendment No. 155 to NPF-85
9. Amendment No. 268 to DPR-16
10. Amendment No. 268 to DPR-44
11. Amendment No. 272 to DPR-56
12. Amendment No. 241 to DPR-29
13. Amendment No. 236 to DPR-30
14. Amendment No. 266 to DPR-50
15. Safety Evaluation

cc w/encls: See next page

Similar changes were proposed by the licensee for its non-ITS plants Oyster Creek, LGS, and TMI-1. The proposed changes are based on TSTF 479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a." For all units except Oyster Creek and TMI-1, the amendments also incorporate TSTF-497, Revision 0, "Limit Inservice Testing Program SR [Surveillance Requirement] 3.0.2 Application to Frequencies of 2 Years or Less," which adds a provision in the applicable TS section to only apply the extension allowance of SR 3.0.2 to the frequency table listed in the TSs as part of the IST program and to normal and accelerated inservice testing frequencies of two years or less, as applicable.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,
/RA/
Christopher Gratton, Senior Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456, STN 50-457,
STN 50-454, STN 50-455, 50-237,
50-249, 50-352, 50-353, 50-219, 50-277,
50-278, 50-254, 50-265, and 50-289

Enclosures:
1. Amendment No. 153 to NPF-72
2. Amendment No. 153 to NPF-77
3. Amendment No. 157 to NPF-37
4. Amendment No. 157 to NPF-66
5. Amendment No. 229 to DPR-19
6. Amendment No. 222 to DPR-25
7. Amendment No. 194 to NPF-39
8. Amendment No. 155 to NPF-85
9. Amendment No. 268 to DPR-16
10. Amendment No. 268 to DPR-44
11. Amendment No. 272 to DPR-56
12. Amendment No. 241 to DPR-29
13. Amendment No. 236 to DPR-30
14. Amendment No. 266 to DPR-50
15. Safety Evaluation

cc w/encls: See next page

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NAME	CGratton	THarris	JMcHale	RElliott	A Gendelman	HChernoff	RGibbs
DATE	8/28/08	8/28/08	6/17/08	7/25/08	8/21/08	8/28/08	8/28/08

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EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 153
License No. NPF-72

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 153 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance:
August 28, 2008

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-457

BRAIDWOOD STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 153
License No. NPF-77

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 153 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-72, dated July 2, 1987, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance:
August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NOS. 153 AND 153

FACILITY OPERATING LICENSE NOS. NPF-72 AND NPF-77

DOCKET NOS. STN 50-456 AND STN 50-457

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License NPF-72 Page 3	License NPF-72 Page 3
License NPF-77 Page 3	License NPF-77 Page 3
TSs 5.5-6	TSs 5.5-6

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 157
License No. NPF-37

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-37 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 157 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: August 28, 2008

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-455

BYRON STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 157
License No. NPF-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, (NUREG-1113) as revised through Amendment No. 157 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-37, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance:
August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NOS. 157 AND 157

FACILITY OPERATING LICENSE NOS. NPF-37 AND NPF-66

DOCKET NOS. STN 50-454 AND STN 50-455

Replace the following pages of the Facility Operating Licenses and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License NPF-37 Page 3	License NPF-37 Page 3
License NPF-66 Page 3	License NPF-66 Page 3
TSs 5.5-6	TSs 5.5-6

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 229
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Exelon Generation Company, LLC (the licensee) dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Renewed Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 229, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/
Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance:
August 28, 2008

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 222
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Exelon Generation Company, LLC (the licensee) dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Renewed Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 222, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NOS. 229 AND 222

FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR-25

DOCKET NOS. 50-237 AND 50-249

Replace the following pages of the Facility Operating Licenses and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License DPR-19 Page 3	License DPR-19 Page 3
License DPR-25 Page 4	License DPR-25 Page 4
TSs 5.5-4 5.5-5	TSs 5.5-4 5.5-5

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 194
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee), dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 194, are hereby incorporated in the license. Exelon Generation Company, LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/
Harold K. Chernoff, Chief
Plant Licensing Branch I-2
Office of Nuclear Reactor Regulation

Attachment: Changes to the License
and Technical Specifications

Date of issuance: August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 194

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Facility Operating License and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License NPF-39 Page 3	License NPF-39 Page 3
TSs 3/4 0-2 3/4 0-3	TSs 3/4 0-2 3/4 0-3

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-353

LIMERICK GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 155
License No. NPF-85

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-85 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 155, are hereby incorporated into this license. Exelon Generation Company, LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Harold K. Chernoff, Chief
Plant Licensing Branch I-2
Office of Nuclear Reactor Regulation

Attachment: Changes to the License and
Technical Specifications

Date of Issuance:
August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 155

FACILITY OPERATING LICENSE NO. NPF-85

DOCKET NO. 50-353

Replace the following pages of the Facility Operating License and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License NPF-85 Page 3	License NPF-85 Page 3
TSs 3/4 0-2 3/4 0-3	TSs 3/4 0-2 3/4 0-3

AMERGEN ENERGY COMPANY, LLC

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 268
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by AmerGen Energy Company, LLC, et al., (the licensee), dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 268, are hereby incorporated in the license. AmerGen Energy Company, LLC, shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Harold K. Chernoff, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the License and
Technical Specifications

Date of Issuance: August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 268

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Remove the following page of the Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove	Insert
Page 3	Page 3

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
4.3-1	4.3-1

EXELON GENERATION COMPANY, LLC

PSEG NUCLEAR LLC

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 268
Renewed License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (Exelon Generation Company), and PSEG Nuclear LLC (the licensees), dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Renewed Facility Operating License No. DPR-44 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 268, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Harold K. Chernoff, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the License and
Technical Specifications

Date of Issuance:
August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 268

RENEWED FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Facility Operating License and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License DPR-44 Page 3	License DPR-44 Page 3
TSs 5.0-11	TSs 5.0-11

EXELON GENERATION COMPANY, LLC

PSEG NUCLEAR LLC

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 272
Renewed License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (Exelon Generation Company), and PSEG Nuclear LLC (the licensees), dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Renewed Facility Operating License No. DPR-56 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 272, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Harold K. Chernoff, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the License and
Technical Specifications

Date of Issuance:
August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 272

RENEWED FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Facility Operating License and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License DPR-56 Page 3	License DPR-56 Page 3
TSs 5.0-11	TSs 5.0-11

EXELON GENERATION COMPANY, LLC

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 241
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Exelon Generation Company, LLC (the licensee) August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 241, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance:
August 28, 2008

EXELON GENERATION COMPANY, LLC

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO.50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 236
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Exelon Generation Company, LLC, et al. (the licensees) dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 236, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance:
August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NOS. 241 AND 236

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

DOCKET NOS. 50-254 AND 50-265

Replace the following pages of the Facility Operating License and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License DPR-29 Page 4	License DPR-29 Page 4
License DPR-30 Page 4	License DPR-30 Page 4
TSs 5.5-4 5.5-5	TSs 5.5-4 5.5-5

AMERGEN ENERGY COMPANY, LLC

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 266
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission or NRC) has found that:

   A. The application for amendment by AmerGen Energy Company, LLC (the licensee), dated August 8, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 266, are hereby incorporated in the license. The AmerGen Energy Company, LLC, shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/
Harold K. Chernoff, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: August 28, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 266

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following pages of the Facility Operating License and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License DPR-50 Page 3	License DPR-50 Page 3
TSs 4-11 4-52	TSs 4-11 4-52

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 153 TO FACILITY OPERATING LICENSE NO. NPF-72,

AMENDMENT NO. 153 O FACILITY OPERATING LICENSE NO. NPF-77,

AMENDMENT NO. 157 TO FACILITY OPERATING LICENSE NO. NPF-37,

AMENDMENT NO. 157 TO FACILITY OPERATING LICENSE NO. NPF-66,

AMENDMENT NO. 229 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-19,

AMENDMENT NO. 222 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-25,

AMENDMENT NO. 194 TO FACILITY OPERATING LICENSE NO. NPF-39,

AMENDMENT NO. 155 TO FACILITY OPERATING LICENSE NO. NPF-85,

AMENDMENT NO. 268 TO FACILITY OPERATING LICENSE NO. DPR-16,

AMENDMENT NO. 268 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-44,

AMENDMENT NO. 272 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-56,

AMENDMENT NO. 241 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-29,

AMENDMENT NO. 236 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-30,

AMENDMENT NO. 266 TO FACILITY OPERATING LICENSE NO. DPR-50

EXELON GENERATION COMPANY, LLC,

AMERGEN ENERGY COMPANY, LLC,

BRAIDWOOD STATION, UNITS 1 AND 2

BYRON STATION, UNIT NOS. 1 AND 2

DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3

LIMERICK GENERATING STATION, UNITS 1 AND 2

OYSTER CREEK GENERATING STATION

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2

THREE MILE ISLAND NUCLEAR STATION, UNIT 1

DOCKET NOS. STN 50-456, STN 50-457, STN 50-454, STN 50-455, 50-237, 50-249,

50-352, 50-353, 50-219, 50-277, 50-278, 50-254, 50-265, AND 50-289

1.0 INTRODUCTION

By letter to the Nuclear Regulatory Commission (NRC, Commission) dated August 8, 2007, (Agencywide Documents Access and Management System (ADAMS) Accession No. ML072210958), Exelon Generation Company (EGC), LLC, requested changes to the technical specifications (TSs) for the Braidwood Station, Units 1 and 2 (Braidwood), Byron Station, Unit Nos. 1 and 2 (Byron), Dresden Nuclear Power Station, Units 2 and 3 (DNPS), Limerick Generating Station, Units 1 and 2 (LGS), Peach Bottom Atomic Power Station, Units 2 and 3 (PBAPS), and Quad Cities Nuclear Power Station, Units 1 and 2 (QCNPS), and AmerGen Energy Company (AmerGen), LLC, requested changes to the TSs for Oyster Creek Nuclear Generating Station, (Oyster Creek), and Three Mile Island Station, Unit 1 (TMI-1).

The proposed changes would replace references to Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) with references to the ASME Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code) in the applicable TS section for the inservice testing (IST) program for the EGC plants that have implemented industry's improved technical specifications (ITS). Similar changes were proposed by EGC for its non-ITS plant Limerick, and AmerGen for its non-ITS plants Oyster Creek and TMI-1. The proposed changes are based on Technical Specification Task Force (TSTF) 479-A, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a." For all units except Oyster Creek and TMI-1, the amendments also incorporate TSTF-497, Revision 0, "Limit Inservice Testing Program SR [Surveillance Requirement] 3.0.2 Application to Frequencies of 2 Years or Less," which adds a provision in the applicable TS section to only apply the extension allowance of SR 3.0.2 to the frequency table listed in the TSs as part of the IST program and to normal and accelerated IST frequencies of two years or less, as applicable.

2.0 REGULATORY AND TECHNICAL EVALUATIONS

Attachments A thru H contain the regulatory and technical evaluations for the 14 reactor operating licenses covered by these amendments.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State officials from Illinois, New Jersey, and Pennsylvania were notified of the proposed issuance of the amendment. The State officials had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility's components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (72 FR 68213; December 4, 2007). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: G. Waig, NRR
G. Bedi, NRR

Date: August 28, 2008

Attachments:
A. Braidwood, Units 1 and 2 Regulatory and Technical Evaluation
B. Byron, Unit Nos. 1 and 2 Regulatory and Technical Evaluation
C. DNPS, Units 2 and 3 Regulatory and Technical Evaluation
D. LGS, Units 1 and 2 Regulatory and Technical Evaluation
E. Oyster Creek Regulatory and Technical Evaluation
F. PBAPS, Units 2 and 3 Regulatory and Technical Evaluation
G. QCNPS, Units 1 and 2 Regulatory and Technical Evaluation
H. TMI-1 Regulatory and Technical Evaluation

ATTACHMENT A
REGULATORY AND TECHNICAL EVALUATIONS
BRAIDWOOD STATION, UNITS 1 AND 2

1.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. These TSs are derived from the plant safety analyses.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(f)(4), requires in part, that American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components must meet the inservice testing (IST) requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants Code (OM Code). Section 50.55a(f)(4)(ii) requires that IST programs be revised every 10 years (120 months) to comply with the requirements of the latest edition and addenda of the ASME OM Code that is incorporated by reference in 10 CFR 50.55a(b)(3). Section 50.55a(f)(5)(ii) requires that, if a revised IST program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC or Commission) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application at least 6 months before the start of the period during which the provisions become applicable, in accordance with 10 CFR 50.55a(f)(4). The Braidwood Station, Units 1 and 2 (Braidwood) third 10-year interval IST programs were developed to meet the requirements of the 2001 Edition through 2003 Addenda of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii), as required by 10 CFR 50.55a(f)(4). The third 10-year IST intervals for Braidwood began on July 29, 2008.

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Code. The rules for IST of pumps and valves were deleted from the ASME Code, Section XI in the 2000 Addenda after the IST rules were placed in the ASME OM Code. The Braidwood 10-year IST programs for pumps and valves were developed to meet the requirements of the ASME OM Code. Exelon Generation Corporation, LLC (EGC), submitted this TS amendment to revise Section 5.5.8 to reference the pump and valve IST requirements in the ASME OM Code requirements and delete the reference to Section XI IST requirements.

The NRC staff reviewed the proposed changes for consistency with TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less," and to ensure the TSs continued to meet 10 CFR 50.36, "Technical specifications." The NRC staff approved the incorporation of TSTF-479 and TSTF-497 into the standard technical specifications (STS) on December 6, 2005, and October 4, 2006, respectively. In general, licensees cannot justify TS changes solely on the basis of adopting the model STS. The NRC staff independently reviews the licensee's application to determine whether the proposed changes maintain adequate safety.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take

advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 2.0 in the context of specific proposed changes.

2.0 TECHNICAL EVALUATION

2.1 Specific Changes Requested

EGC has proposed the following changes to the TSs:

For TS Section 5.5.8, "Inservice Testing Program," the reference to "ASME Boiler & Vessel Code, Section XI," for IST requirements would be replaced with "ASME OM Code," in the following TS Sections: 5.5.8.a and 5.5.8.d.

For TS Section 5.5.8, Section 5.5.8.b would be revised to apply SR 3.0.2 to other normal and accelerated frequencies specified as 2 years or less in the IST program.

2.2 Evaluation

EGC proposes to adopt the TS changes contained in TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less."

TSTF-479 revised the IST program located in Chapter 5 of the STS to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, to be consistent with the requirements of 10 CFR 50.55a. The administrative changes of TSTF-479 eliminate the inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii). The NRC staff reviewed EGC's proposed TS changes to Section 5.5.8, to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, against TSTF-479 and found them to be consistent with the TSTF-479.

To clarify the applicability of the provisions of STS SR 3.0.2, TSTF-497 revised the sentence in paragraph 'b' of STS Section 5.5.8 to state: "The provisions of SR 3.0.2 are applicable to the above required Frequencies and to other normal and accelerated Frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities." This change clearly states that the provisions of STS SR 3.0.2 are applicable to IST frequencies of 2 years or less. This requirement referred to the testing frequencies in the table in paragraph "a" of STS Section 5.5.8, which only lists test frequency intervals of 2 years or less. EGC's proposed revision to the Braidwood TS Section 5.5.8.b would apply the 25 percent time extension provided for in TS SR 3.0.2 to other normal and accelerated frequency specified as

2 years or less in the IST Program, in addition to those periods listed in the table in TS Section 5.5.8.a. This extension facilitates surveillance scheduling and considers plant operating conditions that may not be suitable for conducting the surveillance (e.g., transient conditions or other ongoing surveillance or maintenance activities). The 25 percent extension does not significantly degrade the reliability that results from performing the surveillance at its specified Frequency. The NRC staff compared the TS changes to TSTF-479 and found them to be acceptable. This is based on the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the SRs. The NRC staff acknowledges that the 25 percent extension allows for a nonconforming condition with respect to the ASME OM Code, but finds it permissible for test frequency intervals of 2 years or less, based on the NRC staff's analysis for the approval of TSTF-497. The NRC staff also reviewed the changes against the requirements of 10 CFR 50.36 and found them to be acceptable.

3.0 SUMMARY

Therefore, the NRC staff concludes that EGC's proposed changes to the Braidwood TSs requirements are consistent with the Commission's regulations and conform to the requirements of 10 CFR 50.55a. On this basis, the NRC staff concludes that the proposed TS changes are acceptable.

ATTACHMENT B
REGULATORY AND TECHNICAL EVALUATIONS
BYRON STATION, UNIT NOS. 1 AND 2

1.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. These TSs are derived from the plant safety analyses.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(f)(4), requires in part, that American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components must meet the inservice testing (IST) requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants Code (OM Code). Section 50.55a(f)(4)(ii) requires that IST programs be revised every 10 years (120 months) to comply with the requirements of the latest edition and addenda of the ASME OM Code that is incorporated by reference in 10 CFR 50.55a(b)(3). Section 50.55a(f)(5)(ii) requires that, if a revised IST program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC or Commission) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application at least 6 months before the start of the period during which the provisions become applicable, in accordance with 10 CFR 50.55a(f)(4). The Byron Station, Unit Nos. 1 and 2 (Byron) third 10-year interval IST programs were developed to meet the requirements of the 2001 Edition through 2003 Addenda of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii), as required by 10 CFR 50.55a(f)(4). The third 10-year IST intervals for Byron began on July 1, 2006.

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Code. The rules for IST of pumps and valves were deleted from the ASME Code, Section XI in the 2000 Addenda after the IST rules were placed in the ASME OM Code. The Byron 10-year IST programs for pumps and valves were developed to meet the requirements of the ASME OM Code. Exelon Generation Corporation, LLC (EGC), submitted this TS amendment to revise Section 5.5.8 to reference the pump and valve IST requirements in the ASME OM Code requirements and delete the reference to ASME Code, Section XI IST requirements.

The NRC staff reviewed the proposed changes for consistency with TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less," and to ensure the TSs continued to meet 10 CFR 50.36, "Technical specifications." The NRC staff approved the incorporation of TSTF-479 and TSTF-497 into the standard technical specifications (STS) on December 6, 2005, and October 4, 2006, respectively. In general, licensees cannot justify TS changes solely on the basis of adopting the model STS. The NRC staff independently reviews the licensee's application to determine whether the proposed changes maintain adequate safety.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take

advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 2.0 in the context of specific proposed changes.

2.0 TECHNICAL EVALUATION

2.1 Specific Changes Requested

EGC has proposed the following changes to the TSs:

For TS Section 5.5.8, "Inservice Testing Program," the reference to "ASME Boiler & Vessel Code, Section XI," for IST requirements would be replaced with "ASME OM Code," in the following TS Sections: 5.5.8.a and 5.5.8.d.

For TS Section 5.5.8, Section 5.5.8.b would be revised to apply SR 3.0.2 to other normal and accelerated frequencies specified as 2 years or less in the IST program.

2.2 Evaluation

EGC proposes to adopt the TS changes contained in TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less."

TSTF-479 revised the IST program located in Chapter 5 of the STS to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, to be consistent with the requirements of 10 CFR 50.55a. The administrative changes of TSTF-479 eliminate the inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii). The NRC staff reviewed EGC's proposed TS changes to Section 5.5.8, to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, against TSTF-479 and found them to be consistent with the TSTF-479.

To clarify the applicability of the provisions of STS SR 3.0.2, TSTF-497 revised the sentence in paragraph 'b' of STS Section 5.5.8 to state: "The provisions of SR 3.0.2 are applicable to the above required Frequencies and to other normal and accelerated Frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities." This change clearly states that the provisions of STS SR 3.0.2 are applicable to IST frequencies of 2 years or less. This requirement referred to the testing frequencies in the table in paragraph "a" of STS Section 5.5.8, which only lists test frequency intervals of 2 years or less. EGC's proposed revision to the Byron TS Section 5.5.8.b would apply the 25 percent time extension provided for in TS SR 3.0.2 to other normal and accelerated frequency specified as 2 years or

less in the IST Program, in addition to those periods listed in the table in TS Section 5.5.8.a. This extension facilitates surveillance scheduling and considers plant operating conditions that may not be suitable for conducting the surveillance (e.g., transient conditions or other ongoing surveillance or maintenance activities). The 25 percent extension does not significantly degrade the reliability that results from performing the surveillance at its specified Frequency. The NRC staff compared the TS changes to TSTF-479 and found them to be acceptable. This is based on the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the SRs. The NRC staff acknowledges that the 25 percent extension allows for a nonconforming condition with respect to the ASME OM Code, but finds it permissible for test frequency intervals of 2 years or less, based on the NRC staff's analysis for the approval of TSTF-497. The NRC staff also reviewed the changes against the requirements of 10 CFR 50.36 and found them to be acceptable.

3.0 SUMMARY

Therefore, the NRC staff concludes that EGC's proposed changes to the Byron TSs requirements are consistent with the Commission's regulations and conform to the requirements of 10 CFR 50.55a. On this basis, the NRC staff concludes that the proposed TS changes are acceptable.

ATTACHMENT C
REGULATORY AND TECHNICAL EVALUATIONS
DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3

1.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. These TSs are derived from the plant safety analyses.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(f)(4), requires in part, that American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components must meet the inservice testing (IST) requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants Code (OM Code). Section 50.55a(f)(4)(ii) requires that IST programs be revised every 10 years (120 months) to comply with the requirements of the latest edition and addenda of the ASME OM Code that is incorporated by reference in 10 CFR 50.55a(b)(3). Section 50.55a(f)(5)(ii) requires that, if a revised IST program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC or Commission) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application at least 6 months before the start of the period during which the provisions become applicable, in accordance with 10 CFR 50.55a(f)(4). The Dresden Nuclear Power Station, Units 2 and 3 (DNPS) third 10-year interval IST programs were developed to meet the requirements of the 1998 Edition through 2000 Addenda of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii), as required by 10 CFR 50.55a(f)(4). The fourth 10-year IST intervals for DNPS began on November 1, 2003.

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Code. The rules for IST of pumps and valves were deleted from the ASME Code, Section XI in the 2000 Addenda after the IST rules were placed in the ASME OM Code. The DNPS 10-year IST programs for pumps and valves were developed to meet the requirements of the ASME OM Code. Exelon Generation Corporation, LLC (EGC), submitted this TS amendment to revise Section 5.5.6 to reference the pump and valve IST requirements in the ASME OM Code requirements and delete the reference to Section XI IST requirements.

The NRC staff reviewed the proposed changes for consistency with TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less," and to ensure the TSs continued to meet 10 CFR 50.36, "Technical specifications." The NRC staff approved the incorporation of TSTF-479 and TSTF-497 into the standard technical specifications (STS) on December 6, 2005, and October 4, 2006, respectively. In general, licensees cannot justify TS changes solely on the basis of adopting the model STS. The NRC staff independently reviews the licensee's application to determine whether the proposed changes maintain adequate safety.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take

advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 2.0 in the context of specific proposed changes.

2.0 TECHNICAL EVALUATION

2.1 Specific Changes Requested

EGC has proposed the following changes to the TSs:

For TS Section 5.5.6, "Inservice Testing Program," the reference to "ASME Boiler and Pressure Vessel Code, Section XI," for IST requirements would be replaced with "ASME OM Code," in the following TS Sections: 5.5.6.a and 5.5.6.d.

For TS Section 5.5.6, Section 5.5.6.b would be revised to apply SR 3.0.2 to other normal and accelerated frequencies specified as 2 years or less in the IST program.

2.2 Evaluation

EGC proposes to adopt the TS changes contained in TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less."

TSTF-479 revised the IST program located in Chapter 5 of the STS to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, to be consistent with the requirements of 10 CFR 50.55a. The administrative changes of TSTF-479 eliminate the inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii). The NRC staff reviewed EGC's proposed TS changes to Section 5.5.8, to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, against TSTF-479 and found them to be consistent with the TSTF-479.

To clarify the applicability of the provisions of STS SR 3.0.2, TSTF-497 revised the sentence in paragraph 'b' of STS Section 5.5.7 to state: "The provisions of SR 3.0.2 are applicable to the above required Frequencies and to other normal and accelerated Frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities." This change clearly states that the provisions of STS SR 3.0.2 are applicable to IST frequencies of 2 years or less. This requirement referred to the testing frequencies in the table in paragraph "a" of STS Section 5.5.7, which only lists test frequency intervals of 2 years or less. EGC's proposed revision to the DNPS TS Section 5.5.6.b would apply the 25 percent time extension provided for in TS SR 3.0.2 to other normal and accelerated frequency specified as 2 years or

less in the IST Program, in addition to those periods listed in the table in TS Section 5.5.6.a. This extension facilitates surveillance scheduling and considers plant operating conditions that may not be suitable for conducting the surveillance (e.g., transient conditions or other ongoing surveillance or maintenance activities). The 25 percent extension does not significantly degrade the reliability that results from performing the surveillance at its specified Frequency. The NRC staff compared the TS changes to TSTF-479 and found them to be acceptable. This is based on the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the SRs. The NRC staff acknowledges that the 25 percent extension allows for a nonconforming condition with respect to the ASME OM Code, but finds it permissible for test frequency intervals of 2 years or less, based on the NRC staff's analysis for the approval of TSTF-497. The NRC staff also reviewed the changes against the requirements of 10 CFR 50.36 and found them to be acceptable.

3.0 SUMMARY

Therefore, the NRC staff concludes that EGC's proposed changes to the DNPS TSs requirements are consistent with the Commission's regulations and conform to the requirements of 10 CFR 50.55a. On this basis, the NRC staff concludes that the proposed TS changes are acceptable.

APPENDIX D
REGULATORY AND TECHNICAL EVALUATIONS
LIMERICK GENERATING STATION, UNITS 1 AND 2

1.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. These TSs are derived from the plant safety analyses.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(f)(4), requires in part, that American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components must meet the inservice testing (IST) requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants Code (OM Code). Section 50.55a(f)(4)(ii) requires that IST programs be revised every 10 years (120 months) to comply with the requirements of the latest edition and addenda of the ASME OM Code that is incorporated by reference in 10 CFR 50.55a(b)(3). Section 50.55a(f)(5)(ii) requires that, if a revised IST program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC or Commission) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application at least 6 months before the start of the period during which the provisions become applicable, in accordance with 10 CFR 50.55a(f)(4). The Limerick Generating Station, Units 1 and 2 (LGS) second 10-year interval IST programs were developed to meet the requirements of the 1990 Edition of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii), as required by 10 CFR 50.55a(f)(4). The second 10-year IST intervals for LGS began on January 8, 2000.

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Code. The rules for IST of pumps and valves were deleted from the ASME Code, Section XI in the 2000 Addenda after the IST rules were placed in the ASME OM Code. The LGS 10-year IST programs for pumps and valves were developed to meet the requirements of the ASME OM Code. Exelon Generation Corporation, LLC (EGC), submitted this TS amendment to revise Section 4 to reference the pump and valve IST requirements in the ASME OM Code requirements and delete the reference to Section XI IST requirements.

The NRC staff reviewed the proposed changes for consistency with TSTF-479, Revision 0, “Changes to Reflect Revision of 10 CFR 50.55a,” and TSTF-497, Revision 0, “Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less,” and to ensure the TSs continued to meet 10 CFR 50.36, “Technical specifications.” The NRC staff approved the incorporation of TSTF-479 and TSTF-497 into the standard technical specifications (STS) on December 6, 2005, and October 4, 2006, respectively. In general, licensees cannot justify TS changes solely on the basis of adopting the model STS. The NRC staff independently reviews the licensee’s application to determine whether the proposed changes maintain adequate safety.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take

advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 2.0 in the context of specific proposed changes.

2.0 TECHNICAL EVALUATION

2.1 Specific Changes Requested

EGC has proposed the following changes to the TSs:

For TS SR Section 4.0.5 for inservice inspection and testing of ASME Code Class 1, 2, and 3 components, the reference to "ASME Boiler & Pressure Vessel Code, Section XI," for IST requirements for pumps and valves would be replaced with "ASME OM Code," in TS Sections 4.0.5.a and 4.0.5.b. Additionally, "ASME Boiler & Pressure Vessel Code," was deleted from the table in TS 4.0.5.b and from TS 4.0.5.e. and replaced with "ASME Code."

For TS Section 4.0.5, "Inservice Testing Program," Section 4.0.5.c would be revised to apply SR 4.0.2 to other normal and accelerated frequencies specified as 2 years or less in the IST program.

2.2 Evaluation

LGS TSs are based on NUREG-0123, "Standard Technical Specifications for General Electric Boiling Water Reactors, BWR/5," and are in a format that does not coincide with the industry improved STS. EGC, therefore, proposed changes that were modeled after TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 [SR4.0.2 for LGS] Application to Frequencies of 2 Years or Less."

TSTF-479 revised the IST program located in Chapter 5 of the STS to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, to be consistent with the requirements of 10 CFR 50.55a. The administrative changes of TSTF-479 eliminate the inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii). The NRC staff reviewed EGC's proposed TS changes to Section 4.0.5, to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, against TSTF-479 and found them to be consistent with the TSTF-479.
To clarify the applicability of the provisions of STS SR 3.0.2, TSTF-497 revised the sentence in paragraph 'b' of STS Section 5.5.7 to state: "The provisions of SR 3.0.2 are applicable to the above required Frequencies and to other normal and accelerated Frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities." This

change clearly states that the provisions of STS SR 3.0.2 are applicable to IST frequencies of 2 years or less. This requirement referred to the testing frequencies in the table in paragraph “a” of STS Section 5.5.7, which only lists test frequency intervals of 2 years or less. EGC’s proposed revision to the LGS TS Section 4.0.5.c would apply the 25 percent time extension provided for in LGS TS SR 4.0.2 to other normal and accelerated frequency specified as 2 years or less in the IST Program, in addition to those periods listed in the table in TS Section 4.0.5.b. This extension facilitates surveillance scheduling and considers plant operating conditions that may not be suitable for conducting the surveillance (e.g., transient conditions or other ongoing surveillance or maintenance activities). The 25 percent extension does not significantly degrade the reliability that results from performing the surveillance at its specified Frequency. The NRC staff compared the TS changes to TSTF-479 and found them to be acceptable. This is based on the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the SRs. The NRC staff acknowledges that the 25 percent extension allows for a nonconforming condition with respect to the ASME OM Code, but finds it permissible for test frequency intervals of 2 years or less, based on the NRC staff's analysis for the approval of TSTF-497. The NRC staff also reviewed the changes against the requirements of 10 CFR 50.36 and found them to be acceptable.

3.0 SUMMARY

Therefore, the NRC staff concludes that EGC’s proposed changes to the LGS TSs requirements are consistent with the Commission’s regulations and conform to the requirements of 10 CFR 50.55a. On this basis, the NRC staff concludes that the proposed TS changes are acceptable.

APPENDIX E
REGULATORY AND TECHNICAL EVALUATIONS
OYSTER CREEK GENERATING STATION

1.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. These TSs are derived from the plant safety analyses.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(f)(4), requires in part, that American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components must meet the inservice testing (IST) requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants Code (OM Code). Section 50.55a(f)(4)(ii) requires that IST programs be revised every 10 years (120 months) to comply with the requirements of the latest edition and addenda of the ASME OM Code that is incorporated by reference in 10 CFR 50.55a(b)(3). Section 50.55a(f)(5)(ii) requires that, if a revised IST program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC or Commission) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application at least 6 months before the start of the period during which the provisions become applicable, in accordance with 10 CFR 50.55a(f)(4). The Oyster Creek Nuclear Generating Station (Oyster Creek) fourth 10-year interval IST programs were developed to meet the requirements of the 1995 Edition through 1996 Addenda of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii), as required by 10 CFR 50.55a(f)(4). The fourth 10-year IST intervals for Oyster Creek began October 14, 2002.

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Code. The rules for IST of pumps and valves were deleted from the ASME Code, Section XI in the 2000 Addenda after the IST rules were placed in the ASME OM Code. The Oyster Creek 10-year IST programs for pumps and valves were developed to meet the requirements of the ASME OM Code. AmerGen Electric Company, LLC (AmerGen), submitted this TS amendment to revise Section 4 to reference the pump and valve IST requirements in the ASME OM Code requirements and delete the reference to Section XI IST requirements.

Oyster Creek has custom TSs. AmerGen proposed changes to the TSs stated to be consistent with the intent of TSTF-479, Revision 0, “Changes to Reflect Revision of 10 CFR 50.55a.” The NRC staff reviewed the proposed changes for consistency with TSTF-479 and to ensure the TSs continued to meet 10 CFR 50.36, “Technical specifications.” The NRC staff approved the incorporation of TSTF-479 into the standard technical specifications (STS) on December 6, 2005. In general, licensees cannot justify TS changes solely on the basis of adopting the model STS. The NRC staff independently reviews the licensee’s application to determine whether the proposed changes maintain adequate safety.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take

advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 2.0 in the context of specific proposed changes.

2.0 TECHNICAL EVALUATION

2.1 Specific Changes Requested

AmerGen has proposed the following changes to the TSs:

For TS Section 4.3, "Reactor Coolant," the reference to "ASME Boiler & Vessel Code, Section XI," for IST requirements would be replaced with, "the ASME Code for Operation and Maintenance of Nuclear Power Plants," in the following TS Section: 4.3.C.

For TS Sections 4.3.B and 4.3.C, the reference to 10 CFR Sections 50.55a(g), 50.55a(g)(6)(i), 50.55a(f), and 50.55a(f)(6)(i) for inservice inspection and IST would be replaced with "10 CFR Section 50.55a."

2.2 Evaluation

AmerGen proposes to adopt the administrative TS changes contained in TSTF-479. TSTF-479 revised the IST program located in Chapter 5 of the STS to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, to be consistent with the requirements of 10 CFR 50.55a. The change will eliminate the ASME Code inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii).

The Oyster Creek TSs are in a custom format that do not align with the format in NUREG 1433, "General Electric Plants, BWR/4, Standard Technical Specifications." Therefore, AmerGen proposed TS changes are modeled after TSTF-479. AmerGen proposed to make administrative changes to TS Section 4.3, Reactor Coolant," and TS 4.8, "Isolation Condenser," to reflect the latest approved version of the ASME OM Code in lieu of the ASME Code, Section XI. The change eliminates the ASME Code inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii). Minor editorial changes were also incorporated into other TS sections to facilitate the inclusion of references to the ASME OM Code and references to 10 CFR 50.55a. The NRC staff compared the proposed changes against TSTF-479 and concluded that the changes met the intent of TSTF-479. The NRC staff also reviewed the proposed changes to the Oyster Creek TSs against the requirements of 10 CFR 50.36, and concluded that the changes were administrative in nature and continued to meet the requirements of 10 CFR 50.36.

3.0 SUMMARY

Therefore, the NRC staff concludes that AmerGen's proposed changes to the Oyster Creek TSs requirements are consistent with the Commission's regulations and conform to the requirements of 10 CFR 50.55a. On this basis, the NRC staff concludes that the proposed TS changes are acceptable.

APPENDIX F
REGULATORY AND TECHNICAL EVALUATIONS
PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

1.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. These TSs are derived from the plant safety analyses.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(f)(4), requires in part, that American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components must meet the inservice testing (IST) requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants Code (OM Code). Section 50.55a(f)(4)(ii) requires that IST programs be revised every 10 years (120 months) to comply with the requirements of the latest edition and addenda of the ASME OM Code that is incorporated by reference in 10 CFR 50.55a(b)(3). Section 50.55a(f)(5)(ii) requires that, if a revised IST program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC or Commission) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application at least 6 months before the start of the period during which the provisions become applicable, in accordance with 10 CFR 50.55a(f)(4). The Peach Bottom Atomic Power Station, Units 2 and 3 (PBAPS), fourth 10-year interval IST programs were developed to meet the requirements of the 2001 Edition through 2003 Addenda of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii), as required by 10 CFR 50.55a(f)(4). The fourth 10-year IST intervals for PBAPS began on August 15, 2007.

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Code. The rules for IST of pumps and valves were deleted from the ASME Code, Section XI in the 2000 Addenda after the IST rules were placed in the ASME OM Code. The PBAPS 10-year IST programs for pumps and valves were developed to meet the requirements of the ASME OM Code. Exelon Generation Corporation, LLC (EGC), submitted this TS amendment to revise Section 5.5.6 to reference the pump and valve IST requirements in the ASME OM Code requirements and delete the reference to Section XI IST requirements.

The NRC staff reviewed the proposed changes for consistency with TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less," and to ensure the TSs continued to meet 10 CFR 50.36, "Technical specifications." The NRC staff approved the incorporation of TSTF-479 and TSTF-497 into the standard technical specifications (STS) on December 6, 2005, and October 4, 2006, respectively. In general, licensees cannot justify TS changes solely on the basis of adopting the model STS. The NRC staff independently reviews the licensee's application to determine whether the proposed changes maintain adequate safety.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take

advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 2.0 in the context of specific proposed changes.

2.0 TECHNICAL EVALUATION

2.1 Specific Changes Requested

EGC has proposed the following changes to the TSs:

For TS Section 5.5.6, "Inservice Testing Program," the reference to "ASME Boiler & Vessel Code, Section XI," for IST requirements would be replaced with "ASME OM Code," in the following TS Sections: 5.5.6.a and 5.5.6.d.

Section 5.5.6.b would also be revised to apply SR 3.0.2 to other normal and accelerated frequencies specified as 2 years or less in the IST program.

2.2 Evaluation

EGC proposes to adopt the TS changes contained in TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less."

TSTF-479 revised the IST program located in Chapter 5 of the STS to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, to be consistent with the requirements of 10 CFR 50.55a. The administrative changes of TSTF-479 eliminate the inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii). The NRC staff reviewed EGC's proposed TS changes to Section 5.5.6, to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, against TSTF-479 and found them to be consistent with the TSTF-479.

To clarify the applicability of the provisions of STS SR 3.0.2, TSTF-497 revised the sentence in paragraph 'b' of STS Section 5.5.7 to state: "The provisions of SR 3.0.2 are applicable to the above required Frequencies and to other normal and accelerated Frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities." This change clearly states that the provisions of STS SR 3.0.2 are applicable to IST frequencies of 2 years or less. This requirement referred to the testing frequencies in the table in paragraph "a" of STS Section 5.5.7, which only lists test frequency intervals of 2 years or less. EGC's proposed revision to the PBAPS TS Section 5.5.6.b would apply the 25 percent time extension provided for in TS SR 3.0.2 to other normal and accelerated frequency specified as 2 years or

less in the IST Program, in addition to those periods listed in the table in TS Section 5.5.6.a. This extension facilitates surveillance scheduling and considers plant operating conditions that may not be suitable for conducting the surveillance (e.g., transient conditions or other ongoing surveillance or maintenance activities). The 25 percent extension does not significantly degrade the reliability that results from performing the surveillance at its specified Frequency. The NRC staff compared the TS changes to TSTF-479 and found them to be acceptable. This is based on the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the SRs. The NRC staff acknowledges that the 25 percent extension allows for a nonconforming condition with respect to the ASME OM Code, but finds it permissible for test frequency intervals of 2 years or less, based on the NRC staff's analysis for the approval of TSTF-497. The NRC staff also reviewed the changes against the requirements of 10 CFR 50.36 and found them to be acceptable.

3.0 SUMMARY

Therefore, the NRC staff concludes that EGC's proposed changes to the PBAPS TSs requirements are consistent with the Commission's regulations and conform to the requirements of 10 CFR 50.55a. On this basis, the NRC staff concludes that the proposed TS changes are acceptable.

APPENDIX G
REGULATORY AND TECHNICAL EVALUATIONS
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2

1.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. These TSs are derived from the plant safety analyses.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(f)(4), requires in part, that American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components must meet the inservice testing (IST) requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants Code (OM Code). Section 50.55a(f)(4)(ii) requires that IST programs be revised every 10 years (120 months) to comply with the requirements of the latest edition and addenda of the ASME OM Code that is incorporated by reference in 10 CFR 50.55a(b)(3). Section 50.55a(f)(5)(ii) requires that, if a revised IST program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC or Commission) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application at least 6 months before the start of the period during which the provisions become applicable, in accordance with 10 CFR 50.55a(f)(4). The Quad Cities Nuclear Power Station, Units 1 and 2 (QCNPS), fourth 10-year interval IST programs were developed to meet the requirements of the 1998 Edition through 2000 Addenda of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii), as required by 10 CFR 50.55a(f)(4). The fourth 10-year IST intervals for QCNPS began on February 19, 2004.

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Code. The rules for IST of pumps and valves were deleted from the ASME Code, Section XI in the 2000 Addenda after the IST rules were placed in the ASME OM Code. The QCNPS 10-year IST programs for pumps and valves were developed to meet the requirements of the ASME OM Code. Exelon Generation Corporation, LLC (EGC), submitted this TS amendment to revise Section 5.5.6 to reference the pump and valve IST requirements in the ASME OM Code requirements and delete the reference to Section XI IST requirements.

The NRC staff reviewed the proposed changes for consistency with TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less," and to ensure the TSs continued to meet 10 CFR 50.36, "Technical specifications." The NRC staff approved the incorporation of TSTF-479 and TSTF-497 into the standard technical specifications (STS) on December 6, 2005, and October 4, 2006, respectively. In general, licensees cannot justify TS changes solely on the basis of adopting the model STS. The NRC staff independently reviews the licensee's application to determine whether the proposed changes maintain adequate safety.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take

advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 2.0 in the context of specific proposed changes.

2.0 TECHNICAL EVALUATION

2.1 Specific Changes Requested

EGC has proposed the following changes to the TSs:

For TS Section 5.5.6, "Inservice Testing Program," the reference to "ASME Boiler & Vessel Code, Section XI," for IST requirements would be replaced with "ASME OM Code," in the following TS Sections: 5.5.6.a and 5.5.6.d.

Section 5.5.6.b would also be revised to apply SR 3.0.2 to other normal and accelerated frequencies specified as 2 years or less in the IST program.

2.2 Evaluation

EGC proposes to adopt the TS changes contained in TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less."

TSTF-479 revised the IST program located in Chapter 5 of the STS to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, to be consistent with the requirements of 10 CFR 50.55a. The administrative changes of TSTF-479 eliminate the inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii). The NRC staff reviewed EGC's proposed TS changes to Section 5.5.6, to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, against TSTF-479 and found them to be consistent with the TSTF-479.

To clarify the applicability of the provisions of STS SR 3.0.2, TSTF-497 revised the sentence in paragraph 'b' of STS Section 5.5.7 to state: "The provisions of SR 3.0.2 are applicable to the above required Frequencies and to other normal and accelerated Frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities." This change clearly states that the provisions of STS SR 3.0.2 are applicable to IST frequencies of 2 years or less. This requirement referred to the testing frequencies in the table in paragraph "a" of STS Section 5.5.7, which only lists test frequency intervals of 2 years or less. EGC's proposed revision to the QCNPS TS Section 5.5.6.b would apply the 25 percent time extension provided for in TS SR 3.0.2 to other normal and accelerated frequency specified as 2 years or

less in the IST Program, in addition to those periods listed in the table in TS Section 5.5.6.a. This extension facilitates surveillance scheduling and considers plant operating conditions that may not be suitable for conducting the surveillance (e.g., transient conditions or other ongoing surveillance or maintenance activities). The 25 percent extension does not significantly degrade the reliability that results from performing the surveillance at its specified Frequency. The NRC staff compared the TS changes to TSTF-479 and found them to be acceptable. This is based on the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the SRs. The NRC staff acknowledges that the 25 percent extension allows for a nonconforming condition with respect to the ASME OM Code, but finds it permissible for test frequency intervals of 2 years or less, based on the NRC staff's analysis for the approval of TSTF-497. The NRC staff also reviewed the changes against the requirements of 10 CFR 50.36 and found them to be acceptable.

3.0 SUMMARY

Therefore, the NRC staff concludes that EGC's proposed changes to the QCNPS TSs requirements are consistent with the Commission's regulations and conform to the requirements of 10 CFR 50.55a. On this basis, the NRC staff concludes that the proposed TS changes are acceptable.

APPENDIX H
REGULATORY AND TECHNICAL EVALUATIONS
THREE MILE ISLAND NUCLEAR STATION, UNIT 1

1.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. These TSs are derived from the plant safety analyses.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(f)(4), requires in part, that American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components must meet the inservice testing (IST) requirements of the ASME Code for Operation and Maintenance of Nuclear Power Plants Code (OM Code). Section 50.55a(f)(4)(ii) requires that IST programs be revised every 10 years (120 months) to comply with the requirements of the latest edition and addenda of the ASME OM Code that is incorporated by reference in 10 CFR 50.55a(b)(3). Section 50.55a(f)(5)(ii) requires that, if a revised IST program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC or Commission) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application at least 6 months before the start of the period during which the provisions become applicable, in accordance with 10 CFR 50.55a(f)(4). The Three Mile Island Nuclear Station, Unit 1 (TMI-1) fourth 10-year interval IST programs were developed to meet the requirements of the 1998 Edition through 2000 Addenda of the ASME *Code for Operation and Maintenance of Nuclear Power Plants* (ASME OM Code) pursuant to 10 CFR 50.55a(f)(4)(ii), as required by 10 CFR 50.55a(f)(4). The fourth 10-year IST interval for TMI-1 began September 23, 2004.

The ASME OM Code was initially incorporated by reference in 10 CFR 50.55a(b)(3) in a final rule dated September 22, 1999 (64 FR 51370). Prior to the final rule, IST programs were required to meet the requirements of Section XI, Division 1, of the ASME Code. The rules for IST of pumps and valves were deleted from the ASME Code, Section XI in the 2000 Addenda after the IST rules were placed in the ASME OM Code. The TMI-1 10-year IST programs for pumps and valves were developed to meet the requirements of the ASME OM Code. AmerGen Electric Company, LLC (AmerGen), submitted this TS amendment to revise Section 4.0, "Surveillance Standards," to reference the pump and valve IST requirements in the ASME OM Code and delete the references to Section XI IST requirements.

TMI-1 uses custom TSs. AmerGen proposed changes to the TSs stated to be consistent with the intent of TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a." The NRC staff reviewed the proposed changes for consistency with TSTF-479 and to ensure the TSs continued to meet 10 CFR 50.36, "Technical specifications." The NRC staff approved the incorporation of TSTF-479 into the standard technical specifications (STS) on December 6, 2005. In general, licensees cannot justify TS changes solely on the basis of adopting the model STS. The NRC staff independently reviews the licensee's application to determine whether the proposed changes maintain adequate safety.

In general, there are two classes of changes to TSs: (1) changes needed to reflect contents of the design basis (TSs are derived from the design basis), and (2) voluntary changes to take

advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time.

Licensees may revise the TSs to adopt improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 2.0 in the context of specific proposed changes.

2.0 TECHNICAL EVALUATION

2.1 Specific Changes Requested

AmerGen has proposed the following changes to the TSs:

For TS Section 4, the reference to "ASME Boiler & Vessel Code, Section XI," for IST requirements would be replaced with "the ASME Code for Operation and Maintenance of Nuclear Power Plants (ASME Code)," in the following section: TS Section 4.2, "Reactor Coolant System Inservice Inspection and Testing." The reference to ASME Section XI for the IST Program in TS Section 4.9, "Decay Heat Removal (DHR) Capability - Periodic Testing," was removed.

For TS Sections 4.2.1 and 4.2.2, the reference to Sections 50.55a(g) and 50.55a(f) for inservice inspection and IST would be replaced with Section 50.55a.

2.2 Evaluation

AmerGen proposes to adopt the administrative TS changes contained in TSTF-479. TSTF-479 revised the IST program located in Chapter 5 of the STS to reflect the latest approved version of the ASME OM Code, in lieu of the ASME Code, Section XI requirements, to be consistent with the requirements of 10 CFR 50.55a. The change will eliminate the ASME Code inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii).

The TMI-1 TSs are in a custom format that do not align with the format in NUREG-1430, "Standard Technical Specifications, Babcock and Wilcox Plants." Therefore, AmerGen proposed TS changes are modeled after TSTF-479. AmerGen proposed to make administrative changes to TS Section 4.2 and TS Section 4.9 to reflect the latest approved version of the ASME OM Code in lieu of the ASME Code, Section XI. The change eliminates the ASME Code inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii). Minor editorial changes were also incorporated into other TS sections to facilitate the inclusion of references to the ASME OM Code and references to 10 CFR 50.55a. The NRC staff compared the proposed changes against TSTF-479 and concluded that the changes met the intent of TSTF-479. The NRC staff also reviewed the proposed changes to the TMI-1 TSs against the requirements of 10 CFR 50.36, and concluded that the changes were administrative in nature and continued to meet the requirements of 10 CFR 50.36.

3.0 SUMMARY

Therefore, the NRC staff concludes that AmerGen's proposed changes to the TMI-1 TSs requirements are consistent with the Commission's regulations and conform to the requirements of 10 CFR 50.55a. On this basis, the NRC staff concludes that the proposed TS changes are acceptable.

Byron/Braidwood Stations

cc:

Corporate Distribution
Exelon Generation Company, LLC
Via e-mail

Byron Distribution
Exelon Generation Company, LLC
Via e-mail

Braidwood Distribution
Exelon Generation Company, LLC
Via e-mail

Dwain W. Alexander, Project Manager
Westinghouse Electric Corporation
Via e-mail

Howard A. Learner
Environmental Law and Policy
Center of the Midwest
Via e-mail

Byron Resident Inspector
U.S. Nuclear Regulatory Commission
Via e-mail

Ms. Lorraine Creek
RR 1, Box 182
Manteno, IL 60950

Chairman, Ogle County Board
Via email

Mrs. Phillip B. Johnson
1907 Stratford Lane
Rockford, IL 61107

Attorney General
Springfield, IL 62701
Via e-mail

Illinois Emergency Management Agency
Division of Disaster Assistance &
Preparedness
Via e-mail

Mr. Barry Quigley
3512 Louisiana
Rockford, IL 61108

Braidwood Resident Inspector
U.S. Nuclear Regulatory Commission
Via e-mail

Will County Executive
Via e-mail

Ms. Bridget Little Rorem
Appleseed Coordinator
Via e-mail

Dresden and Quad Cities Nuclear Power Stations

cc:
Corporate Distribution
Exelon Generation Company, LLC
Via e-mail

Dresden Distribution
Exelon Generation Company, LLC
Via e-mail

Quad Cities Distribution
Exelon Generation Company, LLC
Via e-mail

Dresden Resident Inspector
U.S. Nuclear Regulatory Commission
Via e-mail

Chairman
Grundy County Board
Via e-mail

Quad Cities Resident Inspector
U.S. Nuclear Regulatory Commission
Via e-mail

David C. Tubbs
MidAmerican Energy Company
Via e-mail

Managing Senior Attorney
MidAmerican Energy Company
Via e-mail

Chairman
Rock Island County Board of Supervisors
Via e-mail

Limerick Generating Station, Unit Nos. 1 and 2

cc:

Site Vice President
Limerick Generating Station
Exelon Generation Company, LLC
P.O. Box 2300
Sanatoga, PA 19464

Plant Manager
Limerick Generating Station
Exelon Generation Company, LLC
P.O. Box 2300
Sanatoga, PA 19464

Regulatory Assurance Manager - Limerick
Exelon Generation Company, LLC
P.O. Box 2300
Sanatoga, PA 19464

Vice President - Operations, Mid-Atlantic
Exelon Generation Company, LLC
200 Exelon Way, KSA 3-N
Kennett Square, PA 19348

Vice President
Licensing and Regulatory Affairs
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Director
Licensing and Regulatory Affairs
Exelon Generation Company, LLC
200 Exelon Way, KSA 3-E
Kennett Square, PA 19348

Manager Licensing
Exelon Generation Company, LLC
200 Exelon Way, KSA 3-E
Kennett Square, PA 19348

Correspondence Control Desk
Exelon Generation Company, LLC
P.O. Box 160
Kennett Square, PA 19348

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Limerick Generating Station
P.O. Box 596
Pottstown, PA 19464

Library
U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Director, Bureau of Radiation Protection
Pennsylvania Dept. of Environmental Protection
Rachel Carson State Office Building
P.O. Box 8469
Harrisburg, PA 17105-8469

Chairman
Board of Supervisors of Limerick Township
646 West Ridge Pike
Linfield, PA 19468

Dr. Judith Johnsrud
National Energy Committee
Sierra Club
433 Orlando Avenue
State College, PA 16803

Associate General Counsel
Exelon Generating Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Peach Bottom Atomic Power Station, Unit Nos. 2 and 3

cc:

Site Vice President
Peach Bottom Atomic Power Station
Exelon Generation Company, LLC
1848 Lay Road
Delta, PA 17314

Plant Manager
Peach Bottom Atomic Power Station
Exelon Generation Company, LLC
1848 Lay Road
Delta, PA 17314

Regulatory Assurance Manager
Peach Bottom Atomic Power Station
Exelon Generation Company, LLC
1848 Lay Road
Delta, PA 17314

Resident Inspector
U.S. Nuclear Regulatory Commission
Peach Bottom Atomic Power Station
P.O. Box 399
Delta, PA 17314

Mr. Roland Fletcher
Department of Environment
Radiological Health Program
2400 Broening Highway
Baltimore, MD 21224

Board of Supervisors
Peach Bottom Township
545 Broad Street Ext.
Delta, PA 17314-9203

Mr. Richard McLean
Power Plant and Environmental
Review Division
Department of Natural Resources
B-3, Tawes State Office Building
Annapolis, MD 21401

Three Mile Island Nuclear Station, Unit 1

cc:

Site Vice President
AmerGen Energy Company, LLC
P. O. Box 480
Middletown, PA 17057

Chairman
Board of County Commissioners
of Dauphin County
Dauphin County Courthouse
Harrisburg, PA 17120

Chairman
Board of Supervisors
of Londonderry Township
R.D. #1, Geyers Church Road
Middletown, PA 17057

Senior Resident Inspector (TMI-1)
U.S. Nuclear Regulatory Commission
P.O. Box 219
Middletown, PA 17057

Plant Manager
AmerGen Energy Company, LLC
P. O. Box 480
Middletown, PA 17057

Regulatory Assurance Manager
AmerGen Energy Company, LLC
P.O. Box 480
Middletown, PA 17057

Ronald Bellamy, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Michael A. Schoppman
Framatome ANP, Suite 705
1911 North Ft. Myer Drive
Rosslyn, VA 22209

Eric Epstein
TMI Alert
4100 Hillsdale Road
Harrisburg, PA 17112

Oyster Creek Nuclear Generating Station

cc:

Site Vice President - Oyster Creek
Nuclear Generating Station
AmerGen Energy Company, LLC
P.O. Box 388
Forked River, NJ 08731

Kathryn M. Sutton, Esquire
Morgan, Lewis, & Bockius LLP
1111 Pennsylvania Avenue, NW
Washington, DC 20004

Kent Tosch, Chief
New Jersey Department of
Environmental Protection
Bureau of Nuclear Engineering CN 415
Trenton, NJ 08625

Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 445
Forked River, NJ 08731

Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 445
Forked River, NJ 08731

Oyster Creek Nuclear Generating Station
Plant Manager

AmerGen Energy Company, LLC
P.O. Box 388
Forked River, NJ 08731