

March 5, 2008

Mr. Michael W. Rencheck
Senior Vice President and
Chief Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF
AMENDMENTS TO REVISE A SPECIFICATION INADVERTENTLY OMITTED
BY PREVIOUS AMENDMENTS RE: POWER RANGE NEUTRON FLUX HIGH
NEGATIVE RATE TRIP FUNCTION (TAC NOS. MD7551 AND MD7552)

Dear Mr. Rencheck:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 302 to Renewed Facility Operating License No. DPR-58, and Amendment No. 285 to Renewed Facility Operating License No. DPR-74 for Donald C. Cook Nuclear Plant, Units 1 and 2. The amendments change the Technical Specifications in response to your application dated September 15, 2006, requesting, among other things (see footnote on page 1 of the enclosed safety evaluation), to modify Action Q of Section 3.3.1, "Reactor Trip System (RTS) Instrumentation." This change was inadvertently omitted in your application for amendment dated August 10, 2005, regarding deleting the power range neutron flux high negative rate trip function; as a result, the NRC staff issued Amendment Nos. 293 (for Unit 1) and 275 (for Unit 2) without addressing Action Q of Section 3.3.1.

The amendments revised Action Q of Section 3.3.1, "Reactor Trip System (RTS) Instrumentation," to reflect deletion of the power range neutron flux high negative rate trip function previously approved by Amendment Nos. 293 (for Unit 1) and 275 (for Unit 2).

A copy of the associated safety evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Peter S. Tam, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures: 1. Amendment No. 302 to DPR-58
2. Amendment No. 285 to DPR-74
3. Safety Evaluation

cc w/encls: See next page

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Package Accession Number: **ML080520355**

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INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 302
License No. DPR-58

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated September 15, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and Appendix B, as revised through Amendment No. 302 are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Patrick D. Milano, Acting Chief
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Renewed Operating License
and Appendix A

Date of Issuance: March 5, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 302
RENEWED FACILITY OPERATING LICENSE NO. DPR-58
DOCKET NO. 50-315

Replace the following page of Renewed Facility Operating License No. DPR-58 with the attached revised page. The change area is identified by a marginal line.

REMOVE

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Replace the following page of Appendix A, Technical Specifications, with the attached revised page. The change area is identified by a marginal line.

REMOVE

INSERT

3.3.1-5

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and radiation monitoring equipment calibration, and as fission detectors in amounts as required.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components; and
 - (5) Pursuant to the Act and 10 CFR 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not to exceed 3304 megawatts thermal in accordance with the conditions specified therein.
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A and Appendix B, as revised through Amendment No. 302, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Less Than Four Loop Operation

The licensee shall not operate the reactor at power levels above P-7 (as defined in Table 3.3.1-1 of Specification 3.3.1 of Appendix A to this renewed operating license) with less than four reactor coolant loops in operation until (a) safety analyses for less than four loop operation have been submitted, and (b) approval for less than four loop operation at power levels above P-7 has been granted by the Commission by amendment of this license.
 - (4) Indiana Michigan Power Company shall implement and maintain, in effect, all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for the facility and as approved in the SERs dated December 12, 1977, July 31, 1979, January 10, 1981, February 7, 1983, November 22, 1983, December 23, 1983, March 16, 1984, August 27, 1985

Renewed License No. DPR-58
Amendment No. 1 through 301, 302

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 285
License No. DPR-74

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated September 15, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and Appendix B, as revised through Amendment No. 285, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Patrick D. Milano, Acting Chief
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Renewed Operating License
and Appendix A

Date of Issuance: March 5, 2008

ATTACHMENT TO LICENSE AMENDMENT NO. 285

RENEWED FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Replace the following page of Renewed Facility Operating License No. DPR-74 with the attached revised page. The change area is identified by a marginal line.

REMOVE

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INSERT

3

Replace the following page of Appendix A, Technical Specifications, with the attached revised page. The change area is identified by a marginal line.

REMOVE

3.3.1-5

INSERT

3.3.1-5

radiation monitoring equipment calibration, and as fission detectors in amounts as required.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not to exceed 3468 megawatts thermal in accordance with the conditions specified therein and in attachment 1 to the renewed operating license.

The preoperational tests, startup and other items identified in Attachment 1 to this renewed operating license shall be completed. Attachment 1 is an integral part of this renewed operating license.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and Appendix B, as revised through Amendment No. 285, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

(a) Deleted by Amendment No. 76

(b) Deleted by Amendment No. 2

(c) Leak Testing of Emergency Core cooling System Valves

Indiana Michigan Power Company shall prior to completion of the first inservice testing interval test each of the two valves in series in the

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 302 TO
RENEWED FACILITY OPERATING LICENSE NO. DPR-58
AND AMENDMENT NO. 285 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-74
INDIANA MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-315 AND 50-316

1.0 INTRODUCTION

By application dated August 10, 2005 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML052300238), Indiana Michigan Power Company (the licensee) requested amendments to the Technical Specifications (TSs) for the Donald C. Cook (D. C. Cook) Nuclear Plant, Units 1 and 2. The proposed amendments would delete the power range neutron flux high negative rate trip function from Table 3.3.1-1, "Reactor Trip System Instrumentation." The Nuclear Regulatory Commission (NRC) staff approved the change by issuing Amendment Nos. 293 (for Unit 1) and 275 (for Unit 2) on February 10, 2006. However, the licensee inadvertently failed to request a change to Action Q of Section 3.3.1, "Reactor Trip System (RTS) Instrumentation." Without a change, Action Q continues to refer to "Function 3b," the already deleted power range neutron flux high negative rate trip function.

By letter dated September 15, 2006 (ADAMS Accession No. ML062710309), the licensee proposed to, among other things, revise Action Q of Section 3.3.1 to reflect the approval previously conveyed by Amendment Nos. 293 and 275 cited above. Specifically, this would delete the specification for Function 3b, leaving Function 3a (power range neutron flux high positive rate) as Function 3.

The NRC staff had previously reviewed and approved the regulatory and technical aspects of the deletion of the power range neutron flux negative rate trip function. The revision of Action Q of Section 3.3.1 does not involve any new regulatory or technical review; accordingly, the corresponding sections of the safety evaluation supporting Amendment No. 293 and 275 are reproduced verbatim below in *italics*.

*The licensee's September 15, 2006, letter was principally an application for amendment regarding the reactor trip system (RTS) and engineered safety feature actuation system (ESFAS) bypass test capability, as well as a request to correct the inadvertent omission of Section 3.3.1 from the licensee's August 10, 2005, application. The principal issues of the September 15, 2006, application, that of RTS and ESFAS bypass testing, had been addressed by issuance of Amendment No. 300 (for Unit 1) and 283 (for Unit 2) on December 17, 2007.

2.0 REGULATORY EVALUATION

The following regulatory requirements are applicable to the proposed TS changes discussed in the license amendment application.

General design criterion (GDC) 10, "Reactor Design," in Appendix A to Title 10 of the Code of Federal Regulations (10 CFR) Part 50, requires that the reactor core and associated coolant, control and protection system be designed with appropriate margin to assure that specified acceptable fuel design limits are not exceeded during any condition of normal operation, including the effects of anticipated operational occurrences.

Also, the NRC's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36, "Technical Specifications." Specifically, 10 CFR 50.36(c)(2)(ii) specifies four screening criteria to be used in determining whether an limiting condition for operation (LCO) is required to be included in the TSs:

Criterion 1 - Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary (RCPB);

Criterion 2 - A process variable, design feature, or operating restriction that is an initial condition of a design-basis accident (DBA) or transient analysis that either assumes the failure of, or presents a challenge to, the integrity of a fission product barrier;

Criterion 3 - A structure, system or component (SSC) that is part of the primary success path, and which functions or actuates to mitigate a DBA or transient that either assumes the failure of, or presents a challenge to, the integrity of a fission product barrier; and

Criterion 4 - An SSC which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

The licensee proposed to delete the power range neutron flux high negative rate trip function from the D. C. Cook TSs. The NRC staff evaluation of the licensee's proposed change was predicated upon continued compliance with the GDC 10 requirements and the screening criteria specified in 10 CFR 50.36.

3.0 TECHNICAL EVALUATION

The power range neutron flux high negative rate trip function was designed as part of the reactor protection system (RPS) to mitigate the consequences of one or more dropped rod cluster control assemblies (RCCAs) event. The dropped RCCAs event is an anticipated operational occurrence, and is caused by a single electric or mechanical failure that results in a number and combination of RCCAs from the same group of a given bank to drop to the bottom of the core. The resulting negative reactivity insertion causes nuclear power to quickly decrease and core radial peaking factors to increase. The reduced power and continued steam generation cause the reactor coolant temperature to decrease. In the manual control mode, the positive reactivity feedback due to dropping temperature causes the reactor power to rise to initial power level at a reduced reactor vessel inlet temperature with no power overshoot. In the automatic control mode, the plant control system detects the reduction in core power and initiates control bank withdrawal in order to restore core power. As a result, power overshoot occurs, resulting in lower calculated departure from nucleate boiling ratios (DNBRs). At higher power levels, in the event of a dropped RCCA event, the RPS will detect the rapidly decreasing neutron flux due to

the dropped RCCAs and trip the reactor based on the power range neutron flux high negative rate trip function, thus ending the transient and assuring that DNBR design limits are maintained. Since the dropped RCCA event is an anticipated operational occurrence, it must be shown that to satisfy GDC 10 requirements, the DNBR design limits are met for the combination of high nuclear power, high radial peaking factor, and other system conditions that exist following the dropped RCCA event.

In a topical report WCAP-10297-P-A, "Dropped Rod Methodology for Negative Flux Rate Trip Plants" (see ADAMS Accession No. 8304140120) Westinghouse documented a methodology for the analysis of the dropped RCCA event and concluded that the high negative flux rate trip was required only when a dropped RCCA (or RCCA bank) exceeded the threshold value of reactivity worth. Any dropped RCCA having a worth below the threshold value would not require an automatic reactor trip to satisfy the DNBR limit. The "A" in the reference number of the topical report indicates that it had been previously approved by the NRC staff.

By letter dated April 25, 1990, the Westinghouse Owners Group submitted an updated topical report WCAP-11394-P-A, "Methodology for the Analysis of the Dropped Rod Event" (see ADAMS Accession No. 9005090249). This methodology provides a means to be used to demonstrate that DNBR limits are met during a dropped RCCA event. The analysis using this methodology takes no credit for any direct trip due to the dropped RCCAs, and assumes that no automatic power reduction features are actuated by the dropped RCCAs. The conclusion reached in WCAP-11394-P-A was that sufficient margin is available in all Westinghouse core designs and fuel types, such that the high negative flux rate trip is not required, regardless of the worth of the dropped RCCA (or bank), subject to a plant cycle-specific analysis. The NRC had previously reviewed the Westinghouse analysis and result (as is indicated by the "A" in the reference number), and concluded that the approach in WCAP-11394-P-A was acceptable for analyzing the dropped RCCAs event for which no credit is taken for any direct trip or automatic power reduction features. In the approval, the NRC noted that further review by the NRC staff for each cycle is not necessary, subject to a licensee verification that the analysis described in WCAP-11394-P-A has been performed, and the comparison specified in the topical report has been made and favorable results were obtained.

In support of the proposed amendment, the licensee reviewed the D. C. Cook safety analyses of record and confirmed that:

- (1) The power range neutron flux high negative rate trip is not credited in the loss-of-coolant accident (LOCA) and LOCA-related analyses, the containment integrity analysis, the main steamline break mass and energy release analysis, steam generator tube rupture analysis, and control system analysis. The non-LOCA analyses do not take credit for the power range neutron flux high negative rate trip function. The analysis of record for the dropped RCCA event was performed using the methods in WCAP-11394-P-A.*
- (2) The power range neutron flux high negative rate trip has no impact on the safety system setpoints, the emergency operating procedures, and the reactor coolant system component integrity.*

Based on its review of the licensee's evaluation discussed above, the NRC staff agrees with the licensee that the proposed deletion of the power range neutron flux high negative rate trip function does not affect the analyses for design basis events, nor have an impact on safety system setpoints, procedures, or reactor coolant system component integrity.

In addition, the NRC staff finds that the deletion of the power range neutron flux high negative rate trip function from the TSs satisfies the four screening criteria specified in 10 CFR 50.36(c)(2)(ii) for TS LCO, since the subject trip function is:

- (a) Not an installed instrumentation used to detect and indicate a significant abnormal degradation of the RCPB (Criterion 1);*
- (b) Not a process variable, design feature, or operating restriction that is an initial condition of a DBA or transient analysis (Criterion 2);*
- (c) Not an SSC that is part of the primary success path and which functions or actuates to mitigate a DBA or transient (Criterion 3);*
- (d) Not an SSC which operating experience or probabilistic risk assessment has shown to be significant to public health and safety (Criterion 4).*

Therefore, the NRC staff finds it acceptable to delete the power range neutron flux high negative rate trip function from Table 3.3.1-1.

3.1 Revision of Condition Q of Section 3.3.1

As explained in Section 1.0 above, the wording of Condition Q should have been revised when Amendment Nos. 293 and 275 were issued. The omission of this revision was an inadvertent error. The NRC staff re-visited the technical basis for approving deletion of the power range neutron flux negative rate trip function, and found that all technical issues had been previously addressed in the safety evaluation associated with Amendment Nos. 293 and 275, and reproduced above in *italics*. The NRC, therefore, found it acceptable to revise Condition Q as follows: “. . . Condition D not met for Function 2.b, 3-a, 3-b, 6, 7, 8.b, 14, or 15.”

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change the requirements with respect to use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued proposed findings that the amendments involve no significant hazards consideration and there has been no public comment on such finding (70 FR 72674 and 71 FR 67396). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Sun, NRR

Date: March 5, 2008

Donald C. Cook Nuclear Plant, Units 1 and
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