February 19, 2008

Mr. David A. Christian
President and Chief Nuclear Officer
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2, REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING STEAM GENERATOR TUBE INSERVICE INSPECTION REPORTS (MD7490 AND MD7491)

Dear Mr. Christian:

By letters dated November 15, 2007 (ML073200234), and October 9, 2007 (ML072820576), Virginia Electric and Power Company, Inc. submitted steam generator tube inservice inspection reports for the North Anna Unit 1, Fall 2007 refueling outage (RFO), and the North Anna Unit 2, Spring 2007 RFO. This was in accordance with the plant's technical specifications.

The NRC staff has reviewed the reports and has determined that additional information is required in order to complete their evaluation of the information provided. The NRC staff's request is enclosed.

You are asked to provide a response to the RAI within 30 days.

Sincerely,

/RA/

R. A. Jervey, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

Enclosure: RAI

cc w/encl: See next page

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NAME	RJervey	MO'Brien	MWong
DATE	02/19 /08	02/ 19/08	02/ 19/08

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North Anna Power Station, Units 1 & 2

cc:

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REQUEST FOR ADDITIONAL INFORMATION
STEAM GENERATOR TUBE INSERVICE INSPECTION REPORTS
NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2 (NORTH ANNA 1 AND 2)
DOCKET NOS. 50-338 AND 50-339

By letters dated November 15, 2007 (ML073200234), and October 9, 2007 (ML072820576), Virginia Electric and Power Company, Inc. (the licensee), submitted steam generator (SG) inspection reports for the North Anna 1 fall 2007 refueling outage (RFO) and the North Anna 2 spring 2007 RFO in accordance with the plant's technical specifications.

Question pertaining to North Anna 1:

1. One tube was plugged in Unit 1 since a permeability indication rendered a significant portion of the tube uninspectable. Please discuss how the integrity of this tube was assessed (i.e., did the tube satisfy the performance criteria) if it could not be fully inspected. For example, was an in-situ pressure test performed?

Questions pertaining to North Anna 1 and 2:

2. For each RFO and SG tube inspection since installation of the SGs, please provide the cumulative effective full power months that the SGs have operated.
3. It was indicated that the secondary-side inspections did not reveal any component degradation that would compromise tube integrity. Please discuss the results of the inspections of the secondary-side internals (e.g., any degradation/deterioration observed, any extensive deposits observed at the tube support plate openings).
4. Tube wear was listed as a potential degradation mechanism for the straight-leg and anti-vibration bar tangent points for Rows 8, 14, and 26. Please clarify why the only rows considered susceptible to this degradation mechanism are Rows 8, 14, and 26.
5. With respect to the design of your SGs, please confirm that the tubes are arranged in a square pitch/pattern and they were manufactured by Sandvik. In addition, please provide the radius of the Row 1 tubes and the tubesheet thickness (with and without clad).
6. It was indicated that the rotating coil probe was used to inspect freespan dents/bulges in Unit 1 and dents/dings/bulges in Unit 2 that measured greater than 2-volts (as determined by the bobbin coil). Please provide the information in Table 1 of the reports for these locations (e.g., these locations are potentially susceptible to primary water stress corrosion cracking and outside diameter stress corrosion cracking). Please discuss why freespan dings were not inspected in Unit 1. In addition, please discuss whether there are any dents/dings/bulges at non-freespan locations and whether these locations are susceptible to degradation (potential, relevant, existing). If so, discuss what examinations, if any, were performed at these locations.

Enclosure