



P.O. Box 63
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January 31, 2008

U. S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Attention: Document Control Desk

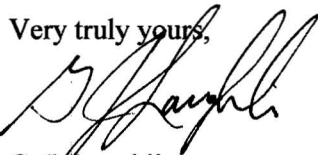
Subject: Nine Mile Point Nuclear Station, LLC
Unit No. 1; Docket No. 50-220; License No. DPR-63
Unit No. 2; Docket No. 50-410; License No. NPF-69

10 CFR 50.46 ECCS Evaluation Model Annual Reports for 2007

Pursuant to the reporting requirements of 10 CFR 50.46(a)(3)(ii), Nine Mile Point Nuclear Station, LLC (NMPNS) is submitting the emergency core cooling system (ECCS) evaluation model annual reports for Nine Mile Point Unit 1 (NMP1) and Nine Mile Point Unit 2 (NMP2).

These annual reports, provided in Attachments 1 and 2, summarize the nature of and estimated effect of any changes or errors in the ECCS models for NMP1 and NMP2 for the period January 1, 2007 through December 31, 2007.

This letter contains no NRC commitments. Should you have any questions regarding this submittal, please contact T. F. Syrell, Licensing Director, at (315) 349-5219.

Very truly yours,

G. J. Laughlin
Manager Engineering Services

GJL/MHS

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Attachments: 1. Nine Mile Point Unit 1 10 CFR 50.46 ECCS Evaluation Model Annual Report for 2007
2. Nine Mile Point Unit 2 10 CFR 50.46 ECCS Evaluation Model Annual Report for 2007

cc: S. J. Collins, NRC Regional Administrator, Region I
Senior NRC Resident Inspector
M. J. David, NRC Project Manager

ATTACHMENT 1

**Nine Mile Point Unit 1
10 CFR 50.46 ECCS Evaluation Model Annual Report for 2007**

**Nine Mile Point Nuclear Station, LLC
January 31, 2008**

Attachment 1
Nine Mile Point Unit 1
10 CFR 50.46 ECCS Evaluation Model Annual Report for 2007

BACKGROUND

In accordance with 10 CFR 50.46(a)(3)(ii), this annual report summarizes the nature of and estimated effect of any changes or errors in the emergency core cooling system (ECCS) model for the period January 1, 2007 through December 31, 2007 for Nine Mile Point Unit 1 (NMP1).

DISCUSSION

No changes or errors to the ECCS evaluation model were identified in 2007 for NMP1.

IMPACT

Not applicable

CONCLUSION

As documented in Table 1, the NMP1 Loss Of Coolant Accident analysis Peak Clad Temperature remains in compliance with 10 CFR 50.46(b)(1), which requires that the PCT shall not exceed 2200 °F.

**Attachment 1
Nine Mile Point Unit 1
10 CFR 50.46 ECCS Evaluation Model Annual Report for 2007**

Table 1

**LOCA Margin Summary Sheet
Nine Mile Point Nuclear Station, LLC
Nine Mile Point Unit 1**

Evaluation Model: General Electric SAFER / CORCL / GESTR methodology

	<u>Net PCT Effect</u>	<u>Absolute PCT Effect</u>
A. Prior 10 CFR 50.46 Changes or Error Corrections - Previous Years	$\Delta\text{PCT} = 0\text{ }^\circ\text{F}$	$0\text{ }^\circ\text{F}$
B. Prior 10 CFR 50.46 Changes or Error Corrections - This Year	$\Delta\text{PCT} = 0\text{ }^\circ\text{F}$	$0\text{ }^\circ\text{F}$
Absolute Sum of 10 CFR 50.46 Changes	$\Delta\text{PCT} =$	$0\text{ }^\circ\text{F}$

The sum of the PCT from the most recent analysis using an acceptable evaluation model and the estimates of PCT impact for changes and errors identified since this analysis is less than 2200 degrees F.

ATTACHMENT 2

**Nine Mile Point Unit 2
10 CFR 50.46 ECCS Evaluation Model Annual Report for 2007**

**Nine Mile Point Nuclear Station, LLC
January 31, 2008**

Attachment 2
Nine Mile Point Unit 2
10 CFR 50.46 ECCS Evaluation Model Annual Report for 2007

BACKGROUND

In accordance with 10 CFR 50.46(a)(3)(ii), this annual report summarizes the nature of and estimated effect of any changes or errors in the emergency core cooling system (ECCS) model for the period January 1, 2007 through December 31, 2007 for Nine Mile Point Unit 2 (NMP2).

DISCUSSION

No changes or errors to the ECCS evaluation model were identified in 2007 for NMP2.

IMPACT

Not applicable

CONCLUSION

As documented in the following Table, the NMP2 Loss Of Coolant Accident analysis Peak Clad Temperature remains in compliance with 10 CFR 50.46(b)(1), which requires that the PCT shall not exceed 2200 °F.

Attachment 2
Nine Mile Point Unit 2
10 CFR 50.46 ECCS Evaluation Model Annual Report for 2007

Table 1

LOCA Margin Summary Sheet
Nine Mile Point Nuclear Station
Nine Mile Point Unit 2

Evaluation Model: General Electric SAFER / GESTR - LOCA methodology

	<u>Net PCT Effect</u>	<u>Absolute PCT Effect</u>
A. Prior 10 CFR 50.46 Changes or Error Corrections - Previous Years	$\Delta PCT = + 90 \text{ }^\circ\text{F}$	+ 90 °F
B. Prior 10 CFR 50.46 Changes or Error Corrections - This Year	$\Delta PCT = 0 \text{ }^\circ\text{F}$	0 °F
Absolute Sum of 10 CFR 50.46 Changes	$\Delta PCT =$	+ 90 °F

The sum of the PCT from the most recent analysis using an acceptable evaluation model and the estimates of PCT impact for changes and errors identified since this analysis is less than 2200 degrees F.