

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.1 Nuclear Safety Capability System and Equipment Selection

A comprehensive list of systems and equipment and their interrelationships to be analyzed for a fire event shall be developed. The equipment list shall contain an inventory of those critical components required to achieve the nuclear safety performance criteria of Section 1.5. Components required to achieve and maintain the nuclear safety functions and components whose fire-induced failure could prevent the operation or result in the maloperation of those components needed to meet the nuclear safety criteria shall be included. Availability and reliability of equipment selected shall be evaluated.

NEI 00-01 Ref

NEI 00-01 Guidance

3 Deterministic Methodology

This section discusses a generic deterministic methodology and criteria that licensees can use to perform a post-fire safe shutdown analysis to address regulatory requirements. The plant-specific analysis approved by NRC is reflected in the plant's licensing basis. The methodology described in this section is also an acceptable method of performing a post-fire safe shutdown analysis. This methodology is indicated in Figure 3-1. Other methods acceptable to NRC may also be used. Regardless of the method selected by an individual licensee, the criteria and assumptions provided in this guidance document may apply. The methodology described in Section 3 is based on a computer database oriented approach, which is utilized by several licensees to model Appendix R data relationships. This guidance document, however, does not require the use of a computer database oriented approach.

The requirements of Appendix R Sections III.G.1, III.G.2 and III.G.3 apply to equipment and cables required for achieving and maintaining safe shutdown in any fire area. Although equipment and cables for fire detection and suppression systems, communications systems and 8-hour emergency lighting systems are important features, this guidance document does not address them.

Additional information is provided in Appendix B to this document.

Applicability

Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	Plant USA' safe shutdown methodology was reviewed against the guidelines of NUREG-0800, so references to the requirements of specific sections of Appendix R do not apply. The corresponding sections of NUREG-0800 are C.5.b and C.5.c.			Plant USA SER initial and Supplement 4 , Rev. ,	

NEI 00-01 Ref

NEI 00-01 Guidance

3.1 [A, Intro] Safe Shutdown Systems and Path Development

This section discusses the identification of systems available and necessary to perform the required safe shutdown functions. It also provides information on the process for combining these systems into safe shutdown paths. Appendix R Section III.G.1.a requires that the capability to achieve and maintain hot shutdown be free of fire damage. It is expected that the term "free of fire damage" will be further clarified in a forthcoming Regulatory Issue Summary. Appendix R Section III.G.1.b requires that repairs to systems and equipment necessary to achieve and maintain cold shutdown be completed within 72 hours. It is the intent of the NRC that requirements related to the use of manual operator actions will be addressed in a forthcoming rulemaking.

[Refer to hard copy of NEI 00-01 for Figure 3-1]

Applicability

Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	The corresponding guidelines for Plant USA are found in NUREG-0800, BTP CMEB 9.5-1 Sections C.5.b(1) and (2).			, , Rev. , Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	C.5.b(1) and (2) Sections B.3 and B.5.1

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<u>NEI 00-01 Ref</u> 3.1 [B, Goals] Safe Shutdown Systems and Path Development	<u>NEI 00-01 Guidance</u> The goal of post-fire safe shutdown is to assure that a one train of shutdown systems, structures, and components remains free of fire damage for a single fire in any single plant fire area. This goal is accomplished by determining those functions important to achieve and maintain hot shutdown. Safe shutdown systems are selected so that the capability to perform these required functions is a part of each safe shutdown path. The functions important to post-fire safe shutdown generally include, but are not limited to the following: Reactivity control Pressure control systems Inventory control systems Decay heat removal systems Process monitoring Support systems - Electrical systems - Cooling systems These functions are of importance because they have a direct bearing on the safe shutdown goal of being able to achieve and maintain hot shutdown which ensures the integrity of the fuel, the reactor pressure vessel, and the primary containment. If these functions are preserved, then the plant will be safe because the fuel, the reactor and the primary containment will not be damaged. By assuring that this equipment is not damaged and remains functional, the protection of the health and safety of the public is assured.
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<u>Applicability</u>	<u>Comments</u>
Applicable	This is generic guidance and information that applies to all existing safe shutdown analyses.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Plant USA-E-ELEC-0001 defines the safe shutdown goals and functions for Plant USA.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 Plant USA SER initial and Supplement 4, , Rev. ,	Sections A.1 and B.2

<u>NEI 00-01 Ref</u> 3.1 [C, Spurious Operations] Safe Shutdown Systems and Path Development	<u>NEI 00-01 Guidance</u> In addition to the above listed functions, Generic Letter 81-12 specifies consideration of associated circuits with the potential for spurious equipment operation and/or loss of power source, and the common enclosure failures. Spurious operations/actuators can affect the accomplishment of the post-fire safe shutdown functions listed above. Typical examples of the effects of the spurious operations of concern are the following: - A loss of reactor pressure vessel/reactor coolant inventory in excess of the safe shutdown makeup capability - A flow loss or blockage in the inventory makeup or decay heat removal systems being used for the required safe shutdown path. Spurious operations are of concern because they have the potential to directly affect the ability to achieve and maintain hot shutdown, which could affect the fuel and cause damage to the reactor pressure vessel or the primary containment. Common power source and common enclosure concerns could also affect these and must be addressed.
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<u>Applicability</u>	<u>Comments</u>
Applicable	

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<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The Plant USA Safe Shutdown Analysis has considered the three types of associated circuits discussed in NRC Generic Letter 81-12.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.7.1, B.7.2

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.1.1 Criteria / Assumptions	The following criteria and assumptions may be considered when identifying systems available and necessary to perform the required safe shutdown functions and combining these systems into safe shutdown paths.

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	This is generic introductory information and contains no specific guidance.			, , Rev. ,	

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.1.1.1 [GE BWR Paths]	[BWR] GE Report GE-NE-T43-00002-00-01-R01 entitled "Original Safe Shutdown Paths For The BWR" addresses the systems and equipment originally designed into the GE boiling water reactors (BWRs) in the 1960s and 1970s, that can be used to achieve and maintain safe shutdown per Section III.G.1 of 10CFR 50, Appendix R. Any of the shutdown paths (methods) described in this report are considered to be acceptable methods for achieving redundant safe shutdown.

<u>Applicability</u>	<u>Comments</u>
Not Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
N/A	Plant USA is a PWR, and this guidance is specific to BWRs.			, , Rev. ,	

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.1.1.2 [SRVs / LP Systems]	[BWR] GE Report GE-NE-T43-00002-00-03-R01 provides a discussion on the BWR Owners' Group (BWROG) position regarding the use of Safety Relief Valves (SRVs) and low pressure systems (LPCI/CS) for safe shutdown. The BWROG position is that the use of SRVs and low pressure systems is an acceptable methodology for achieving redundant safe shutdown in accordance with the requirements of 10CFR50 Appendix R Sections III.G.1 and III.G.2. The NRC has accepted the BWROG position and issued an SER dated Dec. 12, 2000.

<u>Applicability</u>	<u>Comments</u>
Not Applicable	

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<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
N/A	Plant USA is a PWR, and this guidance is specific to BWRs.			, , Rev. ,	

<u>NEI 00-01 Ref</u> 3.1.1.3 [Pressurizer Heaters]	<u>NEI 00-01 Guidance</u> [PWR] Generic Letter 86-10, Enclosure 2, Section 5.3.5 specifies that hot shutdown can be maintained without the use of pressurizer heaters (i.e., pressure control is provided by controlling the makeup/charging pumps). Hot shutdown conditions can be maintained via natural circulation of the RCS through the steam generators. The cooldown rate must be controlled to prevent the formation of a bubble in the reactor head. Therefore, feedwater (either auxiliary or emergency) flow rates as well as steam release must be controlled.				
<u>Applicability</u>	<u>Comments</u>				

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The plant's safe shutdown procedures (AOP-004 and AOP-036 (series) ensure that cooldown rate is controlled, and that the cooldown process adheres to the required pressure and temperature limits.			, , Rev. ,	
				, , Rev. ,	

<u>NEI 00-01 Ref</u> 3.1.1.4 [Alternative Shutdown Capability]	<u>NEI 00-01 Guidance</u> The classification of shutdown capability as alternative shutdown is made independent of the selection of systems used for shutdown. Alternative shutdown capability is determined based on an inability to assure the availability of a redundant safe shutdown path. Compliance to the separation requirements of Sections III.G.1 and III.G.2 may be supplemented by the use of manual actions to the extent allowed by the regulations and the licensing basis of the plant, repairs (cold shutdown only), exemptions, deviations, GL 86-10 fire hazards analyses or fire protection design change evaluations, as appropriate. These may also be used in conjunction with alternative shutdown capability.				
<u>Applicability</u>	<u>Comments</u>				

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Guidelines for alternative shutdown as Plant USA are found in NUREG-0800, BTP CMEB Section 9.5-1, Sections C.5.b(3) and C.5.c.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	

<u>NEI 00-01 Ref</u> 3.1.1.5 [Initial Conditions]	<u>NEI 00-01 Guidance</u> At the onset of the postulated fire, all safe shutdown systems (including applicable redundant trains) are assumed operable and available for post-fire safe				
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Test Plant USA for Transition Report Examples Jan. 2008.mdb

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shutdown. Systems are assumed to be operational with no repairs, maintenance, testing, Limiting Conditions for Operation, etc. in progress. The units are assumed to be operating at full power under normal conditions and normal lineups.

Applicability

Comments

Applicable

Alignment Statement

Alignment Basis

Comments

Unit

Reference Document

Doc. Details

Aligns

These are basic assumptions for all safe shutdown analyses and also apply to the Plant USA SSA.

Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006

Sections A.3.2;A.3.5

NEI 00-01 Ref

NEI 00-01 Guidance

3.1.1.6 [Other Events in Conjunction with Fire]

No Final Safety Analysis Report accidents or other design basis events (e.g. loss of coolant accident, earthquake), single failures or non-fire induced transients need be considered in conjunction with the fire.

Applicability

Comments

Applicable

Alignment Statement

Alignment Basis

Comments

Unit

Reference Document

Doc. Details

Aligns

NUREG 0800 Section C.1.b states that "Worst case" fires need not be postulated to be simultaneous with nonfire-related failures in safety systems, plant accidents, or the most severe natural phenomena.worst

Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006

Section A.3.6

NEI 00-01 Ref

NEI 00-01 Guidance

3.1.1.7 [Offsite Power]

For the case of redundant shutdown, offsite power may be credited if demonstrated to be free of fire damage. Offsite power should be assumed to remain available for those cases where its availability may adversely impact safety (i.e., reliance cannot be placed on fire causing a loss of offsite power if the consequences of offsite power availability are more severe than its presumed loss). No credit should be taken for a fire causing a loss of offsite power. For areas where train separation cannot be achieved and alternative shutdown capability is necessary, shutdown must be demonstrated both where offsite power is available and where offsite power is not available for 72 hours.

Applicability

Comments

Applicable

Alignment Statement

Alignment Basis

Comments

Unit

Reference Document

Doc. Details

Aligns

The SSA credits offsite power where analysis has demonstrated that it will be available. A loss of offsite power was not assumed in areas where offsite power was not credited.

, , Rev. ,

Sections 9.1.5, 9.1.4

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Aligns	The SSA credits offsite power where analysis has demonstrated that it will be available. A loss of offsite power was not assumed in areas where offsite power was not credited.	Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section A.3.4
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<u>NEI 00-01 Ref</u> 3.1.1.8 [Safety-Related Equipment]	<u>NEI 00-01 Guidance</u> Post-fire safe shutdown systems and components are not required to be safety-related.
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<u>Applicability</u>	<u>Comments</u>
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Applicable

<u>Alignment Statement</u> Aligns	<u>Alignment Basis</u> NUREG-0800, C.5.c(6). The referenced SSA section clearly states that post-fire safe shutdown trains may include non-safety related equipment.	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u> , , Rev. ,	<u>Doc. Details</u> Sections A.1.1 and B.3
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Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section 9.1.2
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<u>NEI 00-01 Ref</u> 3.1.1.9 [72 Hour Coping]	<u>NEI 00-01 Guidance</u> The post-fire safe shutdown analysis assumes a 72-hour coping period starting with a reactor scram/trip. Fire-induced impacts that provide no adverse consequences to hot shutdown within this 72-hour period need not be included in the post-fire safe shutdown analysis. At least one train can be repaired or made operable within 72 hours using onsite capability to achieve cold shutdown.
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<u>Applicability</u>	<u>Comments</u>
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Applicable

<u>Alignment Statement</u> Aligns	<u>Alignment Basis</u> This is a base safe shutdown analysis assumption.	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u> Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	<u>Doc. Details</u> Section A.1.1
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<u>NEI 00-01 Ref</u> 3.1.1.10 [Manual / Automatic Initiation of Systems]	<u>NEI 00-01 Guidance</u> Manual initiation from the main control room or emergency control stations of systems required to achieve and maintain safe shutdown is acceptable where permitted by current regulations or approved by NRC; automatic initiation of systems selected for safe shutdown is not required but may be included as an option.
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<u>Applicability</u>	<u>Comments</u>
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<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The Plant USA SSA does not currently credit the manual initiation of engineered safeguards (ESFAS) systems.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016,	

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.1.1.11 [Multiple Affected Units]	Where a single fire can impact more than one unit of a multi-unit plant, the ability to achieve and maintain safe shutdown for each affected unit must be demonstrated.

<u>Applicability</u>	<u>Comments</u>
Not Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
N/A	Plant USA is a single unit site.			, , Rev. ,	

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.1.2 Shutdown Functions	The following discussion on each of these shutdown functions provides guidance for selecting the systems and equipment required for safe shutdown. For additional information on BWR system selection, refer to GE Report GE-NE-T43-00002-00-01-R01 entitled "Original Safe Shutdown Paths for the BWR."

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	This is an introductory section with no specific requirements. The GE information does not apply to Plant USA.			, , Rev. ,	

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.1.2.1 Reactivity Control	[BWR] Control Rod Drive System The safe shutdown performance and design requirements for the reactivity control function can be met without automatic scram/trip capability. Manual scram/reactor trip is credited. The post-fire safe shutdown analysis must only provide the capability to manually scram/trip the reactor.

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[PWR] Makeup/Charging

There must be a method for ensuring that adequate shutdown margin is maintained by ensuring borated water is utilized for RCS makeup/charging.

Applicability

Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The two credited sources of makeup water for post-fire safe shutdown are the boric acid tank and the RWST. The boric acids concentrations in each tank ensure that adequate shutdown margin will be maintained throughout the cooldown process.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.2.1, B.4

NEI 00-01 Ref

3.1.2.2 Pressure Control Systems

NEI 00-01 Guidance

The systems discussed in this section are examples of systems that can be used for pressure control. This does not restrict the use of other systems for this purpose.

[BWR] Safety Relief Valves (SRVs)

The SRVs are opened to maintain hot shutdown conditions or to depressurize the vessel to allow injection using low pressure systems. These are operated manually. Automatic initiation of the Automatic Depressurization System is not a required function.

[PWR] Makeup/Charging

RCS pressure is controlled by controlling the rate of charging/makeup to the RCS. Although utilization of the pressurizer heaters and/or auxiliary spray reduces operator burden, neither component is required to provide adequate pressure control. Pressure reductions are made by allowing the RCS to cool/shrink, thus reducing pressurizer level/pressure. Pressure increases are made by initiating charging/makeup to maintain pressurizer level/pressure. Manual control of the related pumps is acceptable.

Applicability

Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Plant USA does credit operation of the pressurizer heaters and pressurizer PORVs to maintain or reduce RCS pressure as necessary during the cooldown process.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.2.3, B.4

NEI 00-01 Ref

3.1.2.3 Inventory Control

NEI 00-01 Guidance

[BWR] Systems selected for the inventory control function should be capable of supplying sufficient reactor coolant to achieve and maintain hot shutdown. Manual initiation of these systems is acceptable. Automatic initiation functions are not required.

[PWR]: Systems selected for the inventory control function should be capable of maintaining level to achieve and maintain hot shutdown. Typically, the same components providing inventory control are capable of providing pressure control. Manual initiation of these systems is acceptable. Automatic initiation functions

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are not required.

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The same systems used for post reactor trip inventory control will also be used for inventory control. Specifically, the CVCS system using the boric acid tank(s) and the RWST as sources of makeup water are used to maintain pressurizer level.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.2.2, B.4

NEI 00-01 Ref NEI 00-01 Guidance
 3.1.2.4 Decay Heat Removal [BWR] Systems selected for the decay heat removal function(s) should be capable of:

- Removing sufficient decay heat from primary containment, to prevent containment over-pressurization and failure.
- Satisfying the net positive suction head requirements of any safe shutdown systems taking suction from the containment (suppression pool).
- Removing sufficient decay heat from the reactor to achieve cold shutdown.

[PWR] Systems selected for the decay heat removal function(s) should be capable of:

- Removing sufficient decay heat from the reactor to reach hot shutdown conditions. Typically, this entails utilizing natural circulation in lieu of forced circulation via the reactor coolant pumps and controlling steam release via the Atmospheric Dump valves.
- Removing sufficient decay heat from the reactor to reach cold shutdown conditions.

This does not restrict the use of other systems.

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Plant USA uses the Auxiliary Feedwater System and Steam Generator PORVs to remove decay heat while in hot standby. Once temperature is reduced to about 350F, the RHR system is placed in service to complete the cooldown of cold shutdown conditions.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.2.4, B.4

NEI 00-01 Ref NEI 00-01 Guidance
 3.1.2.5 Process Monitoring The process monitoring function is provided for all safe shutdown paths. IN 84-09, Attachment 1, Section IX "Lessons Learned from NRC Inspections of Fire Protection Safe Shutdown Systems (10CFR50 Appendix R)" provides guidance on the instrumentation acceptable to and preferred by the NRC for meeting the process monitoring function. This instrumentation is that which monitors the process variables necessary to perform and control the functions specified in Appendix R Section III.L.1. Such instrumentation must be demonstrated to remain unaffected by the fire. The IN 84-09 list of process monitoring is applied to alternative

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 3.1.2.6.1 Electrical Systems AC Distribution System

Power for the Appendix R safe shutdown equipment is typically provided by a medium voltage system such as 4.16 KV Class 1E busses either directly from the busses or through step down transformers/load centers/distribution panels for 600, 480 or 120 VAC loads. For redundant safe shutdown performed in accordance with the requirements of Appendix R Section III.G.1 and 2, power may be supplied from either offsite power sources or the emergency diesel generator depending on which has been demonstrated to be free of fire damage. No credit should be taken for a fire causing a loss of offsite power. Refer to Section 3.1.1.7.

DC Distribution System

Typically, the 125VDC distribution system supplies DC control power to various 125VDC control panels including switchgear breaker controls. The 125VDC distribution panels may also supply power to the 120VAC distribution panels via static inverters. These distribution panels typically supply power for instrumentation necessary to complete the process monitoring functions.

For fire events that result in an interruption of power to the AC electrical bus, the station batteries are necessary to supply any required control power during the interim time period required for the diesel generators to become operational. Once the diesels are operational, the 125 VDC distribution system can be powered from the diesels through the battery chargers.

[BWR] Certain plants are also designed with a 250VDC Distribution System that supplies power to Reactor Core Isolation Cooling and/or High Pressure Coolant Injection equipment.

The DC control centers may also supply power to various small horsepower Appendix R safe shutdown system valves and pumps. If the DC system is relied upon to support safe shutdown without battery chargers being available, it must be verified that sufficient battery capacity exists to support the necessary loads for sufficient time (either until power is restored, or the loads are no longer required to operate).

<u>Applicability</u>	<u>Comments</u>
Applicable	For the DC Buses, the batteries are shown in the fault tree going into an "OR" gate with the corresponding battery charger. Thus, if only the battery is free of fire damage, success will not be achieved.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The power supply for each powered component was identified and included in the SSEL. The limited capacity of the battery to supply loads for more than a few hours was considered in the analysis, and is discussed in the CAFTA text file.			Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016,	
				Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.2.6, B.4, B.5.1.2

NEI 00-01 Ref NEI 00-01 Guidance
 3.1.2.6.2 Cooling Systems HVAC Systems
 [HVAC]

HVAC Systems may be required to assure that safe shutdown equipment remains within its operating temperature range, as specified in manufacturer's literature or demonstrated by suitable test methods, and to assure protection for plant operations staff from the effects of fire (smoke, heat, toxic gases, and gaseous fire suppression agents).

HVAC systems may be required to support safe shutdown system operation, based on plant-specific configurations. Typical uses include:

- Main control room, cable spreading room, relay room
- ECCS pump compartments
- Diesel generator rooms
- Switchgear rooms

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Plant-specific evaluations are necessary to determine which HVAC systems are essential to safe shutdown equipment operation.

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	HVAC systems required for post-fire safe shutdown are included in the analysis.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.2.6, B.4

NEI 00-01 Ref
3.1.2.6.2 Cooling Systems [Main Section]

NEI 00-01 Guidance
Various cooling water systems may be required to support safe shutdown system operation, based on plant-specific considerations. Typical uses include:
- RHR/SDC/DH Heat Exchanger cooling water
- Safe shutdown pump cooling (seal coolers, oil coolers)
- Diesel generator cooling
- HVAC system cooling water

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Cooling water systems required for post-fire safe shutdown are included in the analysis.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.2.6, B.4

NEI 00-01 Ref
3.1.3 Methodology for Shutdown System Selection

NEI 00-01 Guidance
Refer to Figure 3-2 for a flowchart illustrating the various steps involved in selecting safe shutdown systems and developing the shutdown paths.

The following methodology may be used to define the safe shutdown systems and paths for an Appendix R analysis:

<u>Applicability</u>	<u>Comments</u>
Applicable	[Refer to hard copy of NEI 00-01 for Figure 3-2]

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	Systems are assigned to one of two (or both) safe shutdown divisions in lieu of paths. Possible combinations of systems are modeled in the CAFTA fault tree.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.5.1, B.6.1

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.1 Nuclear Safety Capability System and Equipment Selection

Aligns with intent	Systems are assigned to one of two (or both) safe shutdown divisions in lieu of paths. Possible combinations of systems are modeled in the CAFTA fault tree.	, , Rev. ,	Section 9.2
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Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016,	N/A
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<u>NEI 00-01 Ref</u> 3.1.3.1 Identify safe shutdown functions	<u>NEI 00-01 Guidance</u> Review available documentation to obtain an understanding of the available plant systems and the functions required to achieve and maintain safe shutdown. Documents such as the following may be reviewed: - Operating Procedures (Normal, Emergency, Abnormal) - System descriptions - Fire Hazard Analysis - Single-line electrical diagrams - Piping and Instrumentation Diagrams (P&IDs)
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[BWR] GE Report GE-NE-T43-00002-00-01-R02 entitled "Original Shutdown Paths for the BWR"

<u>Applicability</u>	<u>Comments</u>
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Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The general guidance provided in this section was followed in the development of the Plant USA SSA.			, , Rev. ,	Sections 4.1 and 9.1

<u>NEI 00-01 Ref</u> 3.1.3.2 Identify Combinations of Systems that Satisfy Each Safe Shutdown Function	<u>NEI 00-01 Guidance</u> Given the criteria/assumptions defined in Section 3.1.1, identify the available combinations of systems capable of achieving the safe shutdown functions of reactivity control, pressure control, inventory control, decay heat removal, process monitoring, and support systems such as electrical and cooling systems (refer to Section 3.1.2). This selection process does not restrict the use of other systems. In addition to achieving the required safe shutdown functions, consider spurious operations and power supply issues that could impact the required safe shutdown function.
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<u>Applicability</u>	<u>Comments</u>
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Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The available equipment combinations are depicted in the CAFTA fault tree, and are further explained in the associated text file and in the SSA.			, , Rev. ,	Section 9.2

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Aligns	The available equipment combinations are depicted in the CAFTA fault tree, and are further explained in the associated text file and in the SSA.	Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.3 and B.5.1
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<u>Open Item ID</u>	<u>Open Item Description</u>	<u>Disposition</u>	<u>Open/Closed</u>
40	How should we reference the CAFTA Fault Tree and the associated text file as a basis document? It is this text file that contains the safe shutdown equipment descriptions related to the modeling of the fault tree (E-5525 also contains system descriptions).	The safe shutdown system descriptions are contained in the SSA (Plant USA-E/ELEC-0001, Safe Shutdown Analysis in Case of Fire and Fire Hazards Analysis), and there is no specific reason to reference the text file separately.	Closed

Corrective Action Reference

Change Eval / Modification Candidate No

Change Eval / Modification Reference

Supporting Detail

<u>NEI 00-01 Ref</u> 3.1.3.3 Define Combinations of Systems for Each Safe Shutdown Path <u>Applicability</u>	<u>NEI 00-01 Guidance</u> Select combinations of systems with the capability of performing all of the required safe shutdown functions and designate this set of systems as a safe shutdown path. In many cases, safe shutdown paths may be defined on a divisional basis since the availability of electrical power and other support systems must be demonstrated for each path. <u>Comments</u>
Applicable	Specific safe shutdown paths need not be identified. This is an analytical tool that is more applicable to BWRs than to PWRs.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	The selected systems are not grouped together in specific safe shutdown "paths," but are depicted in an integrated fashion in the CAFTA fault tree and accompanying text file.			, , Rev. , Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016,	Section 9.2 Sections B.5.1 and B.6.1

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NFPA 805 Section: 2.4.2.1 Nuclear Safety Capability System and Equipment Selection

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>	
3.1.3.4 Assign Shutdown Paths to Each Combination of Systems	Assign a path designation to each combination of systems. The path will serve to document the combination of systems relied upon for safe shutdown in each fire area. Refer to Attachment 1 to this document (NEI 00-01) for an example of a table illustrating how to document the various combinations of systems for selected shutdown paths.	
<u>Applicability</u>	<u>Comments</u>	
Not Applicable	Safe shutdown paths are not defined at Harris. Equipment is defined as being required for Division I or Division II, and some components are required for both divisions. The component and system inter-relationships are also defined in the CAFTA fault tree.	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	The safe shutdown paths are not identified individually, but are shown in an integrated fashion in the CAFTA fault tree. The use of such fault trees is discussed in NFPA-805, Appendix B, Section B.2.2.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.3 and B.5.1
				Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, , , Rev. ,	Section 9.1

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>	
3.2 Safe Shutdown Equipment Selection	The previous section described the methodology for selecting the systems and paths necessary to achieve and maintain safe shutdown for an exposure fire event (see Section 5.0 DEFINITIONS for "Exposure Fire"). This section describes the criteria/assumptions and selection methodology for identifying the specific safe shutdown equipment necessary for the systems to perform their Appendix R function. The selected equipment should be related back to the safe shutdown systems that they support and be assigned to the same safe shutdown path as that system. The list of safe shutdown equipment will then form the basis for identifying the cables necessary for the operation or that can cause the maloperation of the safe shutdown systems.	
<u>Applicability</u>	<u>Comments</u>	
Applicable		

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	Components are assigned to one (or both) of two safe shutdown divisions rather than specific safe shutdown paths, which is more applicable to BWRs. The possible combinations of systems to meet the safe shutdown functions are shown in the fault tree.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section B.5.1.2
				, , Rev. ,	Section 9.1.2

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>	
3.2.1 Criteria / Assumptions	Consider the following criteria and assumptions when identifying equipment necessary to perform the required safe shutdown functions:	

Table B-2 Nuclear Safety Capability Assessment

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NFPA 805 Section: 2.4.2.1 Nuclear Safety Capability System and Equipment Selection

<u>Applicability</u>	<u>Comments</u>
Applicable	This is introductory guidance information, and contains no specific requirements.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	This section provides a general overview of the safe shutdown methodology suggested in NEI 00-01 and followed by Plant USA. Specific requirements or guidance discussed in NEI 00-01 is discussed in the sub-sections below.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section B.5

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.2.1.1 [Primary Secondary Components]	<p>3.2.1.1 Safe shutdown equipment can be divided into two categories. Equipment may be categorized as (1) primary components or (2) secondary components. Typically, the following types of equipment are considered to be primary components:</p> <ul style="list-style-type: none"> - Pumps, motor operated valves, solenoid valves, fans, gas bottles, dampers, unit coolers, etc. - All necessary process indicators and recorders (i.e., flow indicator, temperature indicator, turbine speed indicator, pressure indicator, level recorder) - Power supplies or other electrical components that support operation of primary components (i.e., diesel generators, switchgear, motor control centers, load centers, power supplies, distribution panels, etc.). <p>Secondary components are typically items found within the circuitry for a primary component. These provide a supporting role to the overall circuit function. Some secondary components may provide an isolation function or a signal to a primary component via either an interlock or input signal processor. Examples of secondary components include flow switches, pressure switches, temperature switches, level switches, temperature elements, speed elements, transmitters, converters, controllers, transducers, signal conditioners, hand switches, relays, fuses and various instrumentation devices.</p> <p>Determine which equipment should be included on the Safe Shutdown Equipment List (SSEL). As an option, include secondary components with a primary component(s) that would be affected by fire damage to the secondary component. By doing this, the SSEL can be kept to a manageable size and the equipment included on the SSEL can be readily related to required post-fire safe shutdown systems and functions.</p>

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	This section provides a general overview of the safe shutdown methodology suggested in NEI 00-01 and followed by Plant USA. Specific requirements or guidance outlined in NEI 00-01 is discussed below.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section B.5

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.2.1.2 [Fire Damage to Mechanical Components (not electrically supervised)]	3.2.1.2 Assume that exposure fire damage to manual valves and piping does not adversely impact their ability to perform their pressure boundary or safe shutdown function (heat sensitive piping materials, including tubing with brazed or soldered joints, are not included in this assumption). Fire damage should be evaluated with respect to the ability to manually open or close the valve should this be necessary as a part of the post-fire safe shutdown scenario.

<u>Applicability</u>	<u>Comments</u>
Plant USA	

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NFPA 805 Section: 2.4.2.1 Nuclear Safety Capability System and Equipment Selection

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Due to the substantial nature of equipment and nature and location of combustibles, fire will not impact the pressure boundary function. A fire does not cause a valve to change position unless the fire also affects the electrical equipment or circuits capable of inducing spurious operation of the valve. Manual stroking of a valve once the fire is extinguished will be evaluated as part of the Manual Action Feasibility Evaluation.			, , Rev. ,	Section 9.1.13
				Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section A.3.12

NEI 00-01 Ref
3.2.1.3 [Manual Valve Positions]

NEI 00-01 Guidance
Assume that manual valves are in their normal position as shown on P&IDs or in the plant operating procedures.

Applicability
Applicable

Comments

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	A base assumption of the SSA is that the plant is in a "normal" operating lineup.			, , Rev. ,	Section 9.1.2
				Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.5.1.2 and A.3.2

NEI 00-01 Ref
3.2.1.4 [Check Valves]

NEI 00-01 Guidance
Assume that a check valve closes in the direction of potential flow diversion and seats properly with sufficient leak tightness to prevent flow diversion. Therefore, check valves do not adversely affect the flow rate capability of the safe shutdown systems being used for inventory control, decay heat removal, equipment cooling or other related safe shutdown functions.

Applicability
Applicable

Comments

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<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	There is no clear statement concerning check valves, other than that properly oriented check valves credited as system boundaries should be included in the SSEL, and that those in the flow path need not be included. Check valves credited as boundaries are included in the SSEL, but the assumption that they are leak tight is inherent in the analysis and not clearly stated.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section B.5.1.2, Item 9
				, , Rev. ,	Sections 9.1.2.5, 9.1.2.9

<u>Open Item ID</u>	<u>Open Item Description</u>	<u>Disposition</u>	<u>Open/Closed</u>
41	Section 3.2.1.4 of NEI 00-01 suggests an assumption that check valves credited to prevent flow diversions will seat properly and are essentially leak tight. The SSEL includes check vavles that are credited as system boundaries, so the assmption that they are leak tight is inherent in the analysis but not clearly stated. Consider adding an assumption to revision 1 of Plant USA-E?ELEC-0001 that check vavles credited as system or flow diversion boundaries are assumed to be leak tight.		Open

Corrective Action Reference

Change Eval / Modification Candidate No

Change Eval / Modification Reference

Supporting Detail

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.2.1.5 [Instrument Failures]	Instruments (e.g., resistance temperature detectors, thermocouples, pressure transmitters, and flow transmitters) are assumed to fail upscale, midscale, or downscale as a result of fire damage, whichever is worse. An instrument performing a control function is assumed to provide an undesired signal to the control circuit.

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
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Aligns with intent	Per the basis document cited, instruments exposed to fire damage are assumed to fail. The documentation reviewed does not go to the level of detail suggested by NEI 00-01. It is a generally accepted practice (that can be verified based on a review of the fire area by fire area analyses) that instruments are assumed to fail to their worst case position unless a specific position to the contrary is taken.	, , Rev. ,	Section 9.2.7
		Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section A.3.13

<u>NEI 00-01 Ref</u> 3.2.1.6 [Spurious Components]	<u>NEI 00-01 Guidance</u> Identify equipment that could spuriously operate or mal-operate and impact the performance of equipment on a required safe shutdown path during the equipment selection phase. Consider Bin 1 of RIS 2004-03 during the equipment identification process.
<u>Applicability</u> Applicable	<u>Comments</u>

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Section 9.1.2.7 of FPIP-0104 directs that for boundaries formed by three normally closed valves or dampers in series, all three should be included in the SSEL. RIS 2004-03 is not specifically identified as the basis for identifying three series boundary valves/dampers.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section B.5.1.2
				, , Rev. ,	Section 9.1.2

<u>NEI 00-01 Ref</u> 3.2.1.7 [Instrument Tubing]	<u>NEI 00-01 Guidance</u> Identify instrument tubing that may cause subsequent effects on instrument readings or signals as a result of fire. Determine and consider the fire area location of the instrument tubing when evaluating the effects of fire damage to circuits and equipment in the fire area.
<u>Applicability</u> Applicable	<u>Comments</u>

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
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Aligns	Instrument tubing and its fire area routing is included in the FSSPMD. Instrument sensing lines exposed to fire are assumed by the SSA to result in erratic indications.	Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. , , , Rev. ,	Sections A.3.13 and B.7.1.2 Item 8 Section 9.1.14 Section 9.2.8
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NEI 00-01 Ref
3.2.2 Methodology for Equipment Selection

NEI 00-01 Guidance
Refer to Figure 3-3 for a flowchart illustrating the various steps involved in selecting safe shutdown equipment.

Use the following methodology to select the safe shutdown equipment for a post-fire safe shutdown analysis:

[Refer to hard copy of NEI 00-01 for Figure 3-3]

Applicability
Applicable

Comments

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	This introductory section contains no specific requirement, The sub-paragraphs with specific requirements are addressed separately as required.			, , Rev. ,	

NEI 00-01 Ref
3.2.2.1 Identify the System Flow Path for Each Shutdown Path

NEI 00-01 Guidance
Mark up and annotate a P&ID to highlight the specific flow paths for each system in support of each shutdown path. Refer to Attachment 2 for an example of an annotated P&ID illustrating this concept.

Applicability
Applicable

Comments
Harris maintains marked-up safe shutdown flow diagrams. Prior to the revalidation effort, these diagrams also served as the safe shutdown equipment list, as a SSEL was not specifically generated.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	Individual safe shutdown paths are not identified, but the available paths are displayed in the CAFTA fault tree.			CPL-2165-1000S Series, Safe Shutdown Flow Diagrams, Rev. Latest,	

NEI 00-01 Ref
3.2.2.2 Identify the Equipment in Each Safe Shutdown System Flow Path Including

NEI 00-01 Guidance
Review the applicable documentation (e.g. P&IDs, electrical drawings, instrument loop diagrams) to assure that all equipment in each system's flow path has been identified. Assure that any equipment that could spuriously operate and adversely affect the desired system function(s) is also identified. If additional systems are identified which are necessary for the operation of the safe shutdown system under review, include these as systems required for safe shutdown. Designate these

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NFPA 805 Section: 2.4.2.1 Nuclear Safety Capability System and Equipment Selection

Equipment That May Spuriously Operate and Affect System Operation

new systems with the same safe shutdown path as the primary safe shutdown system under review (Refer to Figure 3-1).

<u>Applicability</u>	<u>Comments</u>
Applicable	It is not necessary that systems and components be assigned to a specific safe shutdown path.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	The credited portions of the safe shutdown systems are shown on the SSD flow diagrams. The component's safe shutdown division (1 or 2) is also shown on these diagrams. The safe shutdown divisions are defined in Section B.3 of the SSA.			, , Rev. , Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 CPL-2165-1000S Series, Safe Shutdown Flow Diagrams, Rev. Latest,	Section 9.1.2 Sections B.3 and B.5.1.2

<u>Open Item ID</u>	<u>Open Item Description</u>	<u>Disposition</u>	<u>Open/Closed</u>
42	The marked up SSD flow diagrams are in the process of being updated to reflect the changes from the re-validation effort.		Open

Corrective Action Reference

Change Eval / Modification Candidate No

Change Eval / Modification Reference

Supporting Detail

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>	<u>Comments</u>
3.2.2.3 Develop a List of Safe Shutdown Equipment and Assign the Corresponding System and Safe Shutdown Path(s) Designation to Each.	Prepare a table listing the equipment identified for each system and the shutdown path that it supports. Identify any valves or other equipment that could spuriously operate and impact the operation of that safe shutdown system. Assign the safe shutdown path for the affected system to this equipment. During the cable selection phase, identify additional equipment required to support the safe shutdown function of the path (e.g., electrical distribution system equipment). Include this additional equipment in the safe shutdown equipment list. Attachment 3 to this document provides an example of a (SSEL). The SSEL identifies the list of equipment within the plant considered for safe shutdown and it documents various equipment-related attributes used in the analysis.	
<u>Applicability</u>		The Harris SSEL does not assign equipment to a specific safe shutdown path. The equipment and system inter-relationships required to meet the safe shutdown functions and goals are depicted in the CAFTA fault tree.

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NFPA 805 Section: 2.4.2.1 Nuclear Safety Capability System and Equipment Selection

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	The SSEL does not assign each component to a safe shutdown path, but it does assign components to safe shutdown divisions (SSD-1 or SSD-2) as defined in Section B.3 of the SSA (Plant USA-E/ELEC-0001).			N/A, Progress Energy Fire Safe Shutdown Program Manager Database User's Manual, Rev. 001,	Section 3.3.1
				Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections B.3 and B.5.1.2
				, , Rev. ,	Sectin 9.1.2

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.2.2.4 Identify Equipment Information Required for the Safe Shutdown Analysis	Collect additional equipment-related information necessary for performing the post-fire safe shutdown analysis for the equipment. In order to facilitate the analysis, tabulate this data for each piece of equipment on the SSEL. Refer to Attachment 3 to this document for an example of a SSEL. Examples of related equipment data should include the equipment type, equipment description, safe shutdown system, safe shutdown path, drawing reference, fire area, fire zone, and room location of equipment. Other information such as the following may be useful in performing the safe shutdown analysis: normal position, hot shutdown position, cold shutdown position, failed air position, failed electrical position, high/low pressure interface concern, and spurious operation concern.

<u>Applicability</u>	<u>Comments</u>
Applicable	The contents and specific fields of the SSEL table should me modified according to each plant's needs and existing data.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Required equipment is shown on the marked up SSD flow diagrams and in the SSEL report from FSSPMD. The SSEL is included as Attachment 2 to Plant USA-E-ELEC-0001.			, , Rev. ,	
				Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	SSEL Report Section B.5.1.3 and Appendix 2

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.2.2.5 Identify Dependencies Between Equipment, Supporting Equipment, Safe Shutdown Systems and Safe Shutdown Paths.	In the process of defining equipment and cables for safe shutdown, identify additional supporting equipment such as electrical power and interlocked equipment. As an aid in assessing identified impacts to safe shutdown, consider modeling the dependency between equipment within each safe shutdown path either in a relational database or in the form of a Safe Shutdown Logic Diagram (SSLD). Attachment 4 provides an example of a SSLD that may be developed to document these relationships.

<u>Applicability</u>	<u>Comments</u>
Applicable	The equipment and system dependencies are modeled in the CAFTA fault tree and FSSPMD.

Table B-2 Nuclear Safety Capability Assessment

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NFPA 805 Section: 2.4.2.1 Nuclear Safety Capability System and Equipment Selection

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The CAFTA fault tree captures the system and equipment inter-dependencies. Power supply and associated circuit dependencies are also captured in the FSSPMD. The text file that corresponds to the CAFTA fault tree is contained in Appendix 4 of Plant USA-E/ELEC-0001.			Plant USA FSSPMD R1600, Fire Safe Shutdown Program Manager Database, Rev. 016, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006, , Rev. ,	Section B.6.1 and Appendix 4 Section 9.1.2

Table B-2 Nuclear Safety Capability Assessment

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NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

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Section 9.3.3 of FPIP-0105 discusses when it is appropriate to code associated circuits as "B" cables and thus not required for the component being analyzed. It provides four examples of when "B" is appropriate, the third of which states

Open

"If the SSAC and other contacts in the circuit misoperate, the result can be mitigated by a control switch in the Main Control Room."

This would appear to be a non-conservative approach, since the required control room actions would not be identified by the SSA. In section 9.2, it is pretty clear that all cables that could affect the ability of a component to perform its safe shutdown function should be identified as required cables, so this may be a case where the FPIP contradicts itself.

Corrective Action Reference

Change Eval / Modification Candidate No

Change Eval / Modification Reference

Supporting Detail

NEI 00-01 Ref

3.3.1.3 [Isolation Devices]

NEI 00-01 Guidance

Electrical devices such as relays, switches and signal resistor units are considered to be acceptable isolation devices. In the case of instrument loops, review the isolation capabilities of the devices in the loop to determine that an acceptable isolation device has been installed at each point where the loop must be isolated so that a fault would not impact the performance of the safe shutdown instrument function.

Applicability

Comments

Applicable

Alignment Statement

Aligns

Alignment Basis

Isolation devices are defined in the SSA, Plant USA-E/ELEC-0001, Appendix 6, Section 2.1.

Comments

Unit

Reference Document

, , Rev. ,
Plant USA-E/ELEC-0001,
Safe Shutdown in Case of
Fire and Fire Hazards
Analysis, Rev. 0, 6/2/2006

Doc. Details

Section 9.2
Appendix 6, Section 2.1

NEI 00-01 Ref

3.3.1.4 [Identify "Not

NEI 00-01 Guidance

Screen out cables for circuits that do not impact the safe shutdown function of a component (i.e., annunciator circuits, space heater circuits and computer input

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Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	This is a discussion of hte common power supply concern, which is taken into consideration in the safe shutdown analysis.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section B.7.2

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.3.2 Associated Circuit Cables	Appendix R, Section III.G.2, requires that separation features be provided for equipment and cables, including associated nonsafety circuits that could prevent operation or cause maloperation due to hot shorts, open circuits, or shorts to ground, of redundant trains of systems necessary to achieve hot shutdown. The three types of associated circuits were identified in Reference 6.1.5 and further clarified in a NRC memorandum dated March 22, 1982 from R. Mattson to D. Eisenhut, Reference 6.1.6. They are as follows: - Spurious actuations - Common power source - Common enclosure.

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	This section provides an introduction to the requirements to analyze associated circuits. Specific requirements of NEI 00-01 are discussed below.			E-5505, Worst Case 120VAC/125VDC Panel Appendix 'R'/Non Appendix 'R' Circuits Short Circuit Levels, Rev. 004, , Rev. , EC 54865, SSD Validation - Issue Revised SSD Analysis Calculation(s), Rev. 0, 9/18/2006 NUREG-1038, Safety Evaluation Report Related to the Operation of the Plant USA Nuclear Power Plant, Units 1 and 2 - Docket Nos. STN-50-400 and STN 50-401, Rev. Original, 11/1/1983 E-5506, Appendix 'R' Coordination Study, Rev. 007, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section 3.3, 9.5.3 Section C02 SSER 3, page 9-15 Sections B.7.1, B.7.2

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Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

<p><u>NEI 00-01 Ref</u> 3.3.2 [A] Associated Circuit Cables - Cables Whose Failure May Cause Spurious Actuations</p>	<p><u>NEI 00-01 Guidance</u> Safe shutdown system spurious actuation concerns can result from fire damage to a cable whose failure could cause the spurious actuation/mal-operation of equipment whose operation could affect safe shutdown. These cables are identified in Section 3.3.3 together with the remaining safe shutdown cables required to support control and operation of the equipment.</p>
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<p><u>Applicability</u></p>	<p><u>Comments</u></p>
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Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
<p>Aligns</p>	<p>Cables that can cause an undesired spurious actuation are identified by an "S" in the FMEA code of the circuit information form in FSSPMD. They are evaluated in the SSA in the same manner as "required" cables.</p>			<p>Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006</p>	<p>Section B.7.1</p>
				<p>Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, , Rev. ,</p>	<p>Section 9.1.2</p>

<p><u>NEI 00-01 Ref</u> 3.3.2 [B] Associated Circuit Cables - Common Power Source Cables</p>	<p><u>NEI 00-01 Guidance</u> The concern for the common power source associated circuits is the loss of a safe shutdown power source due to inadequate breaker/fuse coordination. In the case of a fire-induced cable failure on a non-safe shutdown load circuit supplied from the safe shutdown power source, a lack of coordination between the upstream supply breaker/fuse feeding the safe shutdown power source and the load breaker/fuse supplying the non-safe shutdown faulted circuit can result in loss of the safe shutdown bus. This would result in the loss of power to the safe shutdown equipment supplied from that power source preventing the safe shutdown equipment from performing its required safe shutdown function. Identify these cables together with the remaining safe shutdown cables required to support control and operation of the equipment. Refer to Section 3.5.2.4 for an acceptable methodology for analyzing the impact of these cables on post-fire safe shutdown.</p>
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<p><u>Applicability</u></p>	<p><u>Comments</u></p>
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Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
<p>Aligns</p>	<p>The analysis has taken into account associated circuits by common power supply as defined by NRC Generic Letter 81-12 and its supplement.</p>			<p>Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 E-5506, Appendix 'R' Coordination Study, Rev. 007,</p>	<p>Section B.7.2</p>

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

Selection and Location Use the following methodology to define the cables required for safe shutdown including cables that may cause associated circuits concerns for a post-fire safe shutdown analysis:

[Refer to hard copy of NEI 00-01 for Figure 3-4]

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	This is an introductory paragraph with no specific criteria. Requirements are in the subsequent subsections.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Section B.7.1 Section 9.2

NEI 00-01 Ref NEI 00-01 Guidance
 3.3.3.1 Identify Circuits Required for the Operation of the Safe Shutdown Equipment
 For each piece of safe shutdown equipment defined in section 3.2, review the appropriate electrical diagrams including the following documentation to identify the circuits (power, control, instrumentation) required for operation or whose failure may impact the operation of each piece of equipment:
 - Single-line electrical diagrams
 - Elementary wiring diagrams
 - Electrical connection diagrams
 - Instrument loop diagrams.
 For electrical power distribution equipment such as power supplies, identify any circuits whose failure may cause a coordination concern for the bus under evaluation.
 If power is required for the equipment, include the closest upstream power distribution source on the safe shutdown equipment list. Through the iterative process described in Figures 3-2 and 3-3, include the additional upstream power sources up to either the offsite or the emergency power source.

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The circuit analysis procedure (FPIP-0105) directs that all cables that could adversely affect the component's ability to perform its safe shutdown function be identified. It also includes the identification of all required power supplies.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Section B.7.1 Section 9.2

NEI 00-01 Ref NEI 00-01 Guidance
 3.3.3.2 Identify Interlocked Circuits and Cables Whose Spurious Operation or
 In reviewing each control circuit, investigate interlocks that may lead to additional circuit schemes, cables and equipment. Assign to the equipment any cables for interlocked circuits that can affect the equipment.
 While investigating the interlocked circuits, additional equipment or power sources may be discovered. Include these interlocked equipment or power sources in

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Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

<u>NEI 00-01 Ref</u> 3.5 Circuit Analysis and Evaluation	<u>NEI 00-01 Guidance</u> This section on circuit analysis provides information on the potential impact of fire on circuits used to monitor, control and power safe shutdown equipment. Applying the circuit analysis criteria will lead to an understanding of how fire damage to the cables may affect the ability to achieve and maintain post-fire safe shutdown in a particular fire area. This section should be used in conjunction with Section 3.4, to evaluate the potential fire-induced impacts that require mitigation. Appendix R Section III.G.2 identifies the fire-induced circuit failure types that are to be evaluated for impact from exposure fires on safe shutdown equipment. Section III.G.2 of Appendix R requires consideration of hot shorts, shorts-to-ground and open circuits.
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<u>Applicability</u> Applicable	<u>Comments</u>
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<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	BTP CMEB 9.5-1 Section C.5.c.(7) requires consideration of hot shorts, shorts-to-ground and open circuits for NUREG-0800 plants.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Sections A.1.1, A.2.5 Section 3.6

<u>NEI 00-01 Ref</u> 3.5.1 Criteria / Assumptions	<u>NEI 00-01 Guidance</u> Apply the following criteria/assumptions when performing fire-induced circuit failure evaluations.
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<u>Applicability</u> Applicable	<u>Comments</u>
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<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Plant USA followed the general criteria that follows this introductory section, which contains no specific requirements.			, , Rev. , Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	

<u>NEI 00-01 Ref</u> 3.5.1.1 [Circuit Failure Types and Impact]	<u>NEI 00-01 Guidance</u> Consider the following circuit failure types on each conductor of each unprotected safe shutdown cable to determine the potential impact of a fire on the safe shutdown equipment associated with that conductor. - A hot short may result from a fire-induced insulation breakdown between conductors of the same cable, a different cable or from some other external source resulting in a compatible but undesired impressed voltage or signal on a specific conductor. A hot short may cause a spurious operation of safe shutdown equipment. - An open circuit may result from a fire-induced break in a conductor resulting in the loss of circuit continuity. An open circuit may prevent the ability to control or power the affected equipment. An open circuit may also result in a change of state for normally energized equipment. (e.g. [for BWRs] loss of power to the Main Steam Isolation Valve (MSIV) solenoid valves due to an open circuit will result in the closure of the MSIVs). Note that RIS 2004-03 indicates that open circuits, as
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Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

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The section in Supplement 3 to the SSER that contains the discussion of cable-to-cable faults is technically within a section titled "Alternate of Dedicated Shutdown Capability," which starts on page 9-6. The section on associated circuits which contains the cable-to-cable fault discussion begins on page 9-15 and is within sub-section (s). It seems clear from the discussion that it applies to all plant fire areas. For example, the preceding sub-section (i) "Procedures" (page 9-10) states that "the applicants have committed to provide plans for fires in all plant areas." Sub-section (o) on page 9-12 contains the discussion of high/low pressure interfaces which clearly applies to all plant areas. Thus, it is reasonable to conclude that sub-section (s) "Associated Circuits" which contains the applicable discussion of cable-to-cable faults also applies plant wide.

Open

Corrective Action Reference

Change Eval / Modification Candidate No

Change Eval / Modification Reference

Supporting Detail

<u>NEI 00-01 Ref</u> 3.5.1.2 [Circuit Contacts and Operational Modes]	<u>NEI 00-01 Guidance</u> Assume that circuit contacts are positioned (i.e., open or closed) consistent with the normal mode/position of the safe shutdown equipment as shown on the schematic drawings. The analyst must consider the position of the safe shutdown equipment for each specific shutdown scenario when determining the impact that fire damage to a particular circuit may have on the operation of the safe shutdown equipment.
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Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Per the analysis, components are assumed to be in their normal operating position.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Section B.7.1.1.2 Section 9.2.3

<u>NEI 00-01 Ref</u> 3.5.1.3 [Duration of Circuit Failures]	<u>NEI 00-01 Guidance</u> Assume that circuit failure types resulting in spurious operations exist until action has been taken to isolate the given circuit from the fire area, or other actions have been taken to negate the effects of circuit failure that is causing the spurious actuation. The fire is not assumed to eventually clear the circuit fault. Note that RIS
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Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

2004-03 indicates that fire-induced hot shorts typically self-mitigate after a limited period of time.

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The analysis takes no credit for "self-mitigating" circuit failures. EGR-NGGC-0102, Attachment 4, Section 3.4, under the heading "Issues Requiring Further Research" states in part "Duration of hot shorts...Cable test data indicates that the duration of a hot short is limited; PE general methodology is to conservatively assume the hot short is maintained until action is taken to mitigate its affects."			, , Rev. , EGR-NGGC-0102, Safe Shutdown/Fire Protection Review, Rev. 006,	Section 9.2.5 Section 9.1.12 Section 3.4

NEI 00-01 Ref NEI 00-01 Guidance
 3.5.1.4 [Cable Failure Configurations] When both trains are in the same fire area outside of primary containment, all cables that do not meet the separation requirements of Section III.G.2 are assumed to fail in their worst case configuration.

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	All cables in the area under consideration are assumed to fail in their worst case configuration.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Sections A.3.10 and A.3.11 Sections 9.1.11, 9.1.12.

NEI 00-01 Ref NEI 00-01 Guidance
 3.5.1.5 [A, Circuit Failure Risk Assessment Guidance] The following guidance provides the NRC inspection focus from Bin 1 of RIS 2004-03 in order to identify any potential combinations of spurious operations with higher risk significance. Bin 1 failures should also be the focus of the analysis; however, NRC has indicated that other types of failures required by the regulations for analysis should not be disregarded even if in Bin 2 or 3. If Bin 1 changes in subsequent revisions of RIS 2004-03, the guidelines in the revised RIS should be followed.

Applicability Comments

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Not Applicable Provides guidance on assessing the risk-significance of circuit failures based on RIS 2004-03, Rev. 1. Note that SSER 3 approved Harris' original methodology which did not postulate inter-cable hot shorts (SSER 3, pages 9-15, 9-16).

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Plant USA performed a multiple spurious operations review in accordance with the guidelines of NRC RIS 2004-03. The results of the review are contained in Appendix 14 of the safe shutdown analysis.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Appendix 14

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.5.1.5 [B, Cable Failure Modes]	<p>For multiconductor cables testing has demonstrated that conductor-to-conductor shorting within the same cable is the most common mode of failure. This is often referred to as ‘intra-cable shorting.’ It is reasonable to assume that given damage, more than one conductor-to-conductor short will occur in a given cable. A second primary mode of cable failure is conductor-to-conductor shorting between separate cables, commonly referred to as ‘inter-cable shorting.’ Inter-cable shorting is less likely than intra-cable shorting. Consistent with the current knowledge of fire-induced cable failures, the following configurations should be considered:</p> <p>A. For any individual multiconductor cable (thermoset or thermoplastic), any and all potential spurious actuations that may result from intra-cable shorting, including any possible combination of conductors within the cable, may be postulated to occur concurrently regardless of number. However, as a practical matter, the number of combinations of potential hot shorts increases rapidly with the number of conductors within a given cable. For example, a multiconductor cable with three conductors (3C) has 3 possible combinations of two (including desired combinations), while a five conductor cable (5C) has 10 possible combinations of two (including desired combinations), and a seven conductor cable (7C) has 21 possible combinations of two (including desired combinations). To facilitate an inspection that considers most of the risk presented by postulated hot shorts within a multiconductor cable, inspectors should consider only a few (three or four) of the most critical postulated combinations.</p> <p>B. For any thermoplastic cable, any and all potential spurious actuations that may result from intra-cable and inter-cable shorting with other thermoplastic cables, including any possible combination of conductors within or between the cables, may be postulated to occur concurrently regardless of number. (The consideration of thermoset cable inter-cable shorts is deferred pending additional research.)</p> <p>C. For cases involving the potential damage of more than one multiconductor cable, a maximum of two cables should be assumed to be damaged concurrently. The spurious actuations should be evaluated as previously described. The consideration of more than two cables being damaged (and subsequent spurious actuations) is deferred pending additional research.</p> <p>D. For cases involving direct current (DC) circuits, the potential spurious operation due to failures of the associated control cables (even if the spurious operation requires two concurrent hot shorts of the proper polarity, e.g., plus-to-plus and minus-to-minus) should be considered when the required source and target conductors are each located within the same multiconductor cable.</p> <p>E. Instrumentation Circuits. Required instrumentation circuits are beyond the scope of this associated circuit approach and must meet the same requirements as required power and control circuits. There is one case where an instrument circuit could potentially be considered an associated circuit. If fire-induced damage of an instrument circuit could prevent operation (e.g., lockout permissive signal) or cause maloperation (e.g., unwanted start/stop/reposition signal) of systems necessary to achieve and maintain hot shutdown, then the instrument circuit may be considered an associated circuit and handled accordingly.</p>

<u>Applicability</u>	<u>Comments</u>
Not Applicable	Provides guidance on assessing the risk-significance of circuit failures based on RIS 2004-03, Rev. 1.

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.5.2 Types of Circuit Failures	Appendix R requires that nuclear power plants must be designed to prevent exposure fires from defeating the ability to achieve and maintain post-fire safe shutdown. Fire damage to circuits that provide control and power to equipment on the required safe shutdown path and any other equipment whose spurious

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Open circuit No. 1:
An open circuit at location No. 1 will prevent operation of the subject equipment.

Open circuit No. 2:
An open circuit at location No. 2 will prevent opening/starting of the subject equipment, but will not impact the ability to close/stop the equipment.

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The Plant USA SSA does consider open circuits. This section provides information related to the effects of an open circuit on diferent types of typical circuits.			, , Rev. , Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section 3.23 Sections A.1.1, B.7.1

<u>NEI 00-01 Ref</u> 3.5.2.2 Circuit Failures Due to a Short-to-Ground [A, General]	<u>NEI 00-01 Guidance</u> This section provides guidance for addressing the effects of a short-to-ground on circuits for safe shutdown equipment. A short-to-ground is a fire-induced breakdown of a cable insulation system resulting in the potential on the conductor being applied to ground potential. A short-to-ground can cause a loss of power to or control of required safe shutdown equipment. In addition, a short-to-ground may affect other equipment in the electrical power distribution system in the cases where proper coordination does not exist. Consider the following consequences in the post-fire safe shutdown analysis when determining the effects of circuit failures related to shorts-to-ground: - A short to ground in a power or a control circuit may result in tripping one or more isolation devices (i.e. breaker/fuse) and causing a loss of power to or control of required safe shutdown equipment. - In the case of certain energized equipment such as HVAC dampers, a loss of control power may result in loss of power to an interlocked relay or other device that may cause one or more spurious operations.
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<u>Applicability</u>	<u>Comments</u>
Applicable	This section provides specific examples of shorts to ground on a representative sample of typical control and power circuits

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The Plant USA SSA does consider shorts to ground. This section provides information related to the effects of a short to ground on diferent types of typical circuits.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Sections A.1.1, B.7.1 Section 3.17

<u>NEI 00-01 Ref</u> 3.5.2.2 Circuit Failures Due to	<u>NEI 00-01 Guidance</u> This section provides guidance for addressing the effects of a short-to-ground on circuits for safe shutdown equipment. A short-to-ground is a fire-induced
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Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

a Short-to-Ground [B, Grounded Circuits]

breakdown of a cable insulation system resulting in the potential on the conductor being applied to ground potential. A short-to-ground can cause a loss of power to or control of required safe shutdown equipment. In addition, a short-to-ground may affect other equipment in the electrical power distribution system in the cases where proper coordination does not exist.
Short-to-Ground on Grounded Circuits

Typically, in the case of a grounded circuit, a short-to-ground on any part of the circuit would present a concern for tripping the circuit isolation device thereby causing a loss of control power.

Figure 3.5.2-2 illustrates how a short-to-ground fault may impact a grounded circuit.

[Refer to hard copy of NEI 00-01 Rev. 1 for Figure 3.5.2-2]

Short-to-ground No. 1:

A short-to-ground at location No. 1 will result in the control power fuse blowing and a loss of power to the control circuit. This will result in an inability to operate the equipment using the control switch. Depending on the coordination characteristics between the protective device on this circuit and upstream circuits, the power supply to other circuits could be affected.

Short-to-ground No. 2:

A short-to-ground at location No. 2 will have no effect on the circuit until the close/stop control switch is closed. Should this occur, the effect would be identical to that for the short-to-ground at location No. 1 described above. Should the open/start control switch be closed prior to closing the close/stop control switch, the equipment will still be able to be opened/started.

Applicability

Comments

Applicable

Alignment Statement

Alignment Basis

Comments

Unit

Reference Document

Doc. Details

Aligns

The Plant USA SSA does consider shorts to ground. This section provides information related to the effects of a short to ground on typical grounded circuits.

, , Rev. ,

Section 3.17

Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006

Section A.1.1, B.7.1

NEI 00-01 Ref

NEI 00-01 Guidance

3.5.2.2 Circuit Failures Due to a Short-to-Ground [C, Ungrounded Circuits]

Short-to-Ground on Ungrounded Circuits

In the case of an ungrounded circuit, postulating only a single short-to-ground on any part of the circuit may not result in tripping the circuit isolation device. Another short-to-ground on the circuit or another circuit from the same source would need to exist to cause a loss of control power to the circuit.

Figure 3.5.2-3 illustrates how a short to ground fault may impact an ungrounded circuit.

[Refer to hard copy of NEI 00-01 Rev. 1 for Figure 3.5.2-3]

Short-to-ground No. 1: A short-to-ground at location No. 1 will result in the control power fuse blowing and a loss of power to the control circuit if short-to-ground No. 3 also exists either within the same circuit or on any other circuit fed from the same power source. This will result in an inability to operate the equipment using the control switch. Depending on the coordination characteristics between the protective device on this circuit and upstream circuits, the power supply to other circuits could be affected.

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

Short-to-ground No. 2:

A short-to-ground at location No. 2 will have no effect on the circuit until the close/stop control switch is closed. Should this occur, the effect would be identical to that for the short-to-ground at location No. 1 described above. Should the open/start control switch be closed prior to closing the close/stop control switch, the equipment will still be able to be opened/started.

Applicability

Comments

Applicable

Alignment Statement

Alignment Basis

Comments

Unit

Reference Document

Doc. Details

Aligns

The Plant USA SSA does consider shorts to ground. This section provides information related to the effects of a short to ground on typical ungrounded circuits.

, , Rev. ,

Section 3.23

Plant USA-E/ELEC-0001,
Safe Shutdown in Case of
Fire and Fire Hazards
Analysis, Rev. 0, 6/2/2006

Section A.1.1, B.7.1

NEI 00-01 Ref

NEI 00-01 Guidance

3.5.2.3 Circuit Failures Due to a Hot Short [A, General]

This section provides guidance for analyzing the effects of a hot short on circuits for required safe shutdown equipment. A hot short is defined as a fire-induced insulation breakdown between conductors of the same cable, a different cable or some other external source resulting in an undesired impressed voltage on a specific conductor. The potential effect of the undesired impressed voltage would be to cause equipment to operate or fail to operate in an undesired manner.

Consider the following specific circuit failures related to hot shorts as part of the post-fire safe shutdown analysis:

- A hot short between an energized conductor and a de-energized conductor within the same cable may cause a spurious actuation of equipment. The spuriously actuated device (e.g., relay) may be interlocked with another circuit that causes the spurious actuation of other equipment. This type of hot short is called a conductor-to-conductor hot short or an internal hot short.
- A hot short between any external energized source such as an energized conductor from another cable (thermoplastic cables only) and a de-energized conductor may also cause a spurious actuation of equipment. This is called a cable-to-cable hot short or an external hot short. Cable-to-cable hot shorts between thermoset cables are not postulated to occur pending additional research.

Applicability

Comments

Applicable

Alignment Statement

Alignment Basis

Comments

Unit

Reference Document

Doc. Details

Aligns

The Plant USA SSA does consider hot shorts. This section provides information related to the effects of a hot short on typical circuits.

, , Rev. ,

Section 3.14

Plant USA-E/ELEC-0001,
Safe Shutdown in Case of
Fire and Fire Hazards
Analysis, Rev. 0, 6/2/2006

Sections A.1.1, B.7.1

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

NEI 00-01 Ref
3.5.2.3 Circuit Failures Due to a Hot Short [B, Grounded Circuits]

NEI 00-01 Guidance
A Hot Short on Grounded Circuits

A short-to-ground is another failure mode for a grounded control circuit. A short-to-ground as described above would result in de-energizing the circuit. This would further reduce the likelihood for the circuit to change the state of the equipment either from a control switch or due to a hot short. Nevertheless, a hot short still needs to be considered. Figure 3.5.2-4 shows a typical grounded control circuit that might be used for a motor-operated valve. However, the protective devices and position indication lights that would normally be included in the control circuit for a motor-operated valve have been omitted, since these devices are not required to understand the concepts being explained in this section. In the discussion provided below, it is assumed that a single fire in a given fire area could cause any one of the hot shorts depicted. The following discussion describes how to address the impact of these individual cable faults on the operation of the equipment controlled by this circuit.

[Refer to hard copy of NEI 00-01 Rev. 1 for Figure 3.5.2-4]

Hot short No. 1:
A hot short at this location would energize the close relay and result in the undesired closure of a motor-operated valve.

Hot short No. 2:
A hot short at this location would energize the open relay and result in the undesired opening of a motor-operated valve.

Applicability

Comments

Applicable

Alignment Statement

Alignment Basis

Comments

Unit

Reference Document

Doc. Details

Aligns	The Plant USA SSA does consider hot shorts. This section provides information related to the effects of a hot short on typical grounded circuits.			, , Rev. ,	Section 3.14
				Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Sections A.1.1, B.7.1

NEI 00-01 Ref
3.5.2.3 Circuit Failures Due to a Hot Short [C, Ungrounded Circuits]

NEI 00-01 Guidance
A Hot Short on Ungrounded Circuits

In the case of an ungrounded circuit, a single hot short may be sufficient to cause a spurious operation. A single hot short can cause a spurious operation if the hot short comes from a circuit from the positive leg of the same ungrounded source as the affected circuit.

In reviewing each of these cases, the common denominator is that in every case, the conductor in the circuit between the control switch and the start/stop coil must be involved.

Figure 3.5.2-5 depicted below shows a typical ungrounded control circuit that might be used for a motor-operated valve. However, the protective devices and position indication lights that would normally be included in the control circuit for a motor-operated valve have been omitted, since these devices are not required to understand the concepts being explained in this section.

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.2 Nuclear Safety Capability Circuit Analysis

In the discussion provided below, it is assumed that a single fire in a given fire area could cause any one of the hot shorts depicted. The discussion provided below describes how to address the impact of these cable faults on the operation of the equipment controlled by this circuit.

[Refer to hard copy of NEI 00-01 Rev. 1 for Figure 3.5.2-5]

Hot short No. 1:

A hot short at this location from the same control power source would energize the close relay and result in the undesired closure of a motor operated valve.

Hot short No. 2:

A hot short at this location from the same control power source would energize the open relay and result in the undesired opening of a motor operated valve.

Applicability

Comments

Applicable

Alignment Statement

Alignment Basis

Comments

Unit

Reference Document

Doc. Details

Aligns

The Plant USA SSA does consider hot shorts. This section provides information related to the effects of a hot short on typical ungrounded circuits.

Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,

Sections A.1.1, B.7.1

Section 3.14

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.3 Nuclear Safety Equipment and Cable Location.

Nuclear Safety Equipment and Cable Location. Physical location of equipment and cables shall be identified.

NEI 00-01 Ref NEI 00-01 Guidance
 3.3.3.4 Identify Routing of Identify the routing for each cable including all raceway and cable endpoints. Typically, this information is obtained from joining the list of safe shutdown cables with
 Cables an existing cable and raceway database

Applicability Comments
 Applicable As a minimum, the cable to fire area information must be obtained.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Cable to raceway information is contained in the Cable Information Form of the FSSPMD.			Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Cable Information Section B.7.3.1

NEI 00-01 Ref NEI 00-01 Guidance
 3.3.3.5 Identify Location of Identify the fire area location of each raceway and cable endpoint identified in the previous step and join this information with the cable routing data. In addition,
 Raceway and Cables by Fire identify the location of field-routed cable by fire area. This produces a database containing all of the cables requiring fire area analysis, their locations by fire area,
 Area and their raceway.
Applicability Comments
 Applicable The particular raceway a cable is routed in within the fire area under consideration is important in a risk-informed, performance-based approach. Such information helps the analyst determine the extent to which the cable may be damaged in a credible fire scenario.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	The fire area routing of each cable was identified and entered in the FSSPMD. Raceway to fire area information is not contained, but will be added to support the transitions to NFPA 805.			Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Cable Information Section B.7.3

<u>Open Item ID</u>	<u>Open Item Description</u>	<u>Disposition</u>	<u>Open/Closed</u>
45	The FSSPMD does not contain the raceway to fire area link required by Section 3.3.3.5 of NEI 00-01. This information is in a database that has been validated for upload into the FSSPMD.		Open

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.3 Nuclear Safety Equipment and Cable Location.

Corrective Action Reference

Change Eval / Modification Candidate No

Change Eval / Modification Reference

Supporting Detail

NEI 00-01 Ref

3.5.2.4 Circuit Failures Due to Inadequate Circuit Coordination

NEI 00-01 Guidance

The evaluation of associated circuits of a common power source consists of verifying proper coordination between the supply breaker/fuse and the load breakers/fuses for power sources that are required for safe shutdown. The concern is that, for fire damage to a single power cable, lack of coordination between the supply breaker/fuse and the load breakers/fuses can result in the loss of power to a safe shutdown power source that is required to provide power to safe shutdown equipment.

For the example shown in Figure 3.5.2-6, the circuit powered from load breaker 4 supplies power to a non-safe shutdown pump. This circuit is damaged by fire in the same fire area as the circuit providing power to from the Train B bus to the Train B pump, which is redundant to the Train A pump.

To assure safe shutdown for a fire in this fire area, the damage to the non-safe shutdown pump powered from load breaker 4 of the Train A bus cannot impact the availability of the Train A pump, which is redundant to the Train B pump. To assure that there is no impact to this Train A pump due to the associated circuits' common power source breaker coordination issue, load breaker 4 must be fully coordinated with the feeder breaker to the Train A bus.

[Refer to hard copy of NEI 00-01 Rev. 1 for Figure 3.5.2-6]

A coordination study should demonstrate the coordination status for each required common power source. For coordination to exist, the time-current curves for the breakers, fuses and/or protective relaying must demonstrate that a fault on the load circuits is isolated before tripping the upstream breaker that supplies the bus. Furthermore, the available short circuit current on the load circuit must be considered to ensure that coordination is demonstrated at the maximum fault level.

The methodology for identifying potential associated circuits of a common power source and evaluating circuit coordination cases of associated circuits on a single circuit fault basis is as follows:

- Identify the power sources required to supply power to safe shutdown equipment.
- For each power source, identify the breaker/fuse ratings, types, trip settings and coordination characteristics for the incoming source breaker supplying the bus and the breakers/fuses feeding the loads supplied by the bus.
- For each power source, demonstrate proper circuit coordination using acceptable industry methods.
- For power sources not properly coordinated, tabulate by fire area the routing of cables whose breaker/fuse is not properly coordinated with the supply breaker/fuse. Evaluate the potential for disabling power to the bus in each of the fire areas in which the associated circuit cables of concern are routed and the power source is required for safe shutdown. Prepare a list of the following information for each fire area:
 - Cables of concern.
 - Affected common power source and its path.
 - Raceway in which the cable is enclosed.
 - Sequence of the raceway in the cable route.
 - Fire zone/area in which the raceway is located.

For fire zones/areas in which the power source is disabled, the effects are mitigated by appropriate methods.

Develop analyzed safe shutdown circuit dispositions for the associated circuit of concern cables routed in an area of the same path as required by the power source. Evaluate adequate separation based upon the criteria in Appendix R, NRC staff guidance, and plant licensing bases.

Applicability

Comments

Plant USA

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.3 Nuclear Safety Equipment and Cable Location.

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Associated circuits by common power supply were identified and dispositioned during the cable selection and circuit analysis process. Where a lack of coordination created a compliance issue, the cables were dispositioned in a manner similar to other cables in the area under analysis that could adversely affect safe shutdown.			, , Rev. ,	Sections 9.2.17, 9.5.
				Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Component, Cable, and Fault Tree Logic Section B.7.2.1

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.5.2.5 Circuit Failures Due to Common Enclosure Concerns	<p>The common enclosure associated circuit concern deals with the possibility of causing secondary failures due to fire damage to a circuit either whose isolation device fails to isolate the cable fault or protect the faulted cable from reaching its ignition temperature, or the fire somehow propagates along the cable into adjoining fire areas.</p> <p>The electrical circuit design for most plants provides proper circuit protection in the form of circuit breakers, fuses and other devices that are designed to isolate cable faults before ignition temperature is reached. Adequate electrical circuit protection and cable sizing are included as part of the original plant electrical design maintained as part of the design change process. Proper protection can be verified by review of as-built drawings and change documentation. Review the fire rated barrier and penetration designs that preclude the propagation of fire from one fire area to the next to demonstrate that adequate measures are in place to alleviate fire propagation concerns.</p>

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	FPIP does not reference a particular common enclosure study. The SSER 3 approval of the plant's common enclosure response is contained within the "Alternate Shutdown" review, but clearly applies to all plant fire areas.			, , Rev. ,	Section 9.5.3

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.3 Nuclear Safety Equipment and Cable Location.

Aligns

FPIP does not reference a particular common enclosure study. The SSER 3 approval of the plant's common enclosure response is contained within the "Alternate Shutdown" review, but clearly applies to all plant fire areas.

NUREG-1038, Safety Evaluation Report Related to the Operation of the Plant USA Nuclear Power Plant, Units 1 and 2 - Docket Nos. STN-50-400 and STN 50-401, Rev. Original, 11/1/1983
Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006

SSER 3, page 9-15

Section B.7.2

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.4 Fire Area Assessment.

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	A separate fire is not postulated to occur before, during, or following the fire in accordance with NUREG-0800..			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Section A.3.6 Section 9.1.7

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.4.1.2 [Damage to Unprotected Equipment and Cables]	Assume that the fire may affect all unprotected cables and equipment within the fire area. This assumes that neither the fire size nor the fire intensity is known. This is conservative and bounds the exposure fire that is required by the regulation.

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The analysis considers all potential failures in each area analyzed.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, , , Rev. , N/A, Progress Energy Fire Safe Shutdown Program Manager Database User's Manual, Rev. 001,	Section A.3.11 Fault Tree Logic Sections 9.1.11, 9.1.12. Section 5.0.

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.4.1.3 [Assess Impacts to Required Components]	Address all cable and equipment impacts affecting the required safe shutdown path in the fire area. All potential impacts within the fire area must be addressed. The focus of this section is to determine and assess the potential impacts to the required safe shutdown path selected for achieving post-fire safe shutdown and to assure that the required safe shutdown path for a given fire area is properly protected.

<u>Applicability</u>	<u>Comments</u>
Applicable	

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
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Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.4 Fire Area Assessment.

Aligns	The use of the CAFTA Fault Tree tool does not require that all affected components be addressed. Components must be addressed until the fault tree shows success.	N/A, Progress Energy Fire Safe Shutdown Program Manager Database User's Manual, Rev. 001, , , Rev. ,	Section 5.0. Sectio 9.0 Section B.10 Fault Tree Logic
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NEI 00-01 Ref NEI 00-01 Guidance
 3.4.1.4 [Manual Actions] Use manual actions where appropriate to achieve and maintain post-fire safe shutdown conditions in accordance with NRC requirements.

Applicability Comments
 Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	The specific criteria regarding what constitutes a feasible manual action, a previously approved manual action, and an acceptable manual action are all under review within the FAQ process and other industry and NRC initiatives.			EGR-NGGC-0102, Safe Shutdown/Fire Protection Review, Rev. 006, , , Rev. , Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Attachment 3 Section 9.2.9. Section B.10.1.1

<u>Open Item ID</u>	<u>Open Item Description</u>	<u>Disposition</u>	<u>Open/Closed</u>
46	The use of manual actions (recovery actions under NFPA-805) is under review through the FAQ process and other industry initiatives. Resolution of the applicable FAQs will determine the suitability of the plant's credited manual actions.		Open

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.4 Fire Area Assessment.

Corrective Action Reference

Change Eval / Modification Candidate No

Change Eval / Modification Reference

Supporting Detail

<u>NEI 00-01 Ref</u> 3.4.1.5 [Repairs]	<u>NEI 00-01 Guidance</u> Where appropriate to achieve and maintain cold shutdown within 72 hours, use repairs to equipment required in support of post fire shutdown.
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<u>Applicability</u> Applicable	<u>Comments</u>
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<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Repairs are considered recovery actions under NFPA 805. Currently, the analysis does not credit any cold shutdown repairs.			, , Rev. , Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section 9.2.4 tbl_EXCEPT_CS Section B.10.1.1, Item 4

<u>NEI 00-01 Ref</u> 3.4.1.6 [Assess Compliance with Deterministic Criteria]	<u>NEI 00-01 Guidance</u> Appendix R compliance requires that one train of systems necessary to achieve and maintain hot shutdown conditions from either the control room or emergency control station(s) is free of fire damage (III.G.1.a). When cables or equipment, including associated circuits, are within the same fire area outside primary containment and separation does not already exist, provide one of the following means of separation for the required safe shutdown path(s): <ul style="list-style-type: none"> - Separation of cables and equipment and associated nonsafety circuits of redundant trains within the same fire area by a fire barrier having a 3-hour rating (III.G.2.a) - Separation of cables and equipment and associated nonsafety circuits of redundant trains within the same fire area by a horizontal distance of more than 20 feet with no intervening combustibles or fire hazards. In addition, fire detectors and an automatic fire suppression system shall be installed in the fire area (III.G.2.b). - Enclosure of cable and equipment and associated non-safety circuits of one redundant train within a fire area in a fire barrier having a one-hour rating. In addition, fire detectors and an automatic fire suppression system shall be installed in the fire area (III.G.2.c). For fire areas inside noninerted containments, the following additional options are also available: <ul style="list-style-type: none"> - Separation of cables and equipment and associated nonsafety circuits of redundant trains by a horizontal distance of more than 20 feet with no intervening combustibles or fire hazards (III.G.2.d); - Installation of fire detectors and an automatic fire suppression system in the fire area (III.G.2.e); or - Separation of cables and equipment and associated non-safety circuits of redundant trains by a noncombustible radiant energy shield (III.G.2.f). Use exemptions, deviations and licensing change processes to satisfy the requirements mentioned above and to demonstrate equivalency depending upon the
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Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.4 Fire Area Assessment.

plant's license requirements.

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	The sections of Appendix R referenced in NEI 00-01 are mirrored in Sections C.5.b and C.7.a.(1)(b). The similar deterministic criteria of NFPA-805 are part of the acceptable compliance strategies used in the revalidation.			, , Rev. ,	Section 9.2.
				Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Section B.10.1.1

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.4.1.7 [Consider Additional Equipment]	Consider selecting other equipment that can perform the same safe shutdown function as the impacted equipment. In addressing this situation, each equipment impact, including spurious operations, is to be addressed in accordance with regulatory requirements and the NPP's current licensing basis.

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns with intent	This consideration is not clearly stated but is inherent in performing safe shutdown analyses. Proof that this was considered is the inclusion of the Normal Service Water System as a credited system in the analysis during the re-validation.			, , Rev. ,	
				, , Rev. , Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 EGR-NGGC-0102, Safe Shutdown/Fire Protection Review, Rev. 006, , , Rev. ,	

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.4.1.8 [Consider Instrument Plant USA	Consider the effects of the fire on the density of the fluid in instrument tubing and any subsequent effects on instrument readings or signals associated with the

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.4 Fire Area Assessment.

Tubing Effects] protected safe shutdown path in evaluating post-fire safe shutdown capability. This can be done systematically or via procedures such as Emergency Operating Procedures.

Applicability Comments

Applicable

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Instrument tubing and its routing is included in the FSSPMD. When necessary, it is treated in a manner similar to that in which cable damage is assessed.			, , Rev. , Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Section 9.2.8 Section B.9 and Appendix 11 Section 9.1.6

NEI 00-01 Ref NEI 00-01 Guidance
 3.4.2 Methodology for Fire Area Assessment Refer to Figure 3-5 for a flowchart illustrating the various steps involved in performing a fire area assessment. Use the following methodology to assess the impact to safe shutdown and demonstrate Appendix R compliance:

[Refer to hard copy of NEI 00-01 for Figure 3-5]

Applicability Comments

Applicable Introductory Information.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Specific requirements are detailed in the sub-paragraphs.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 , , Rev. ,	Section B.10 Section 9.0

NEI 00-01 Ref NEI 00-01 Guidance
 3.4.2.1 Identify the Affected Equipment by Fire Area Identify the safe shutdown cables, equipment and systems located in each fire area that may be potentially damaged by the fire. Provide this information in a report format. The report may be sorted by fire area and by system in order to understand the impact to each safe shutdown path within each fire area (see Attachment 5 for an example of an Affected Equipment Report).

Applicability Comments

Applicable The FSSPMD provides the affected equipment report in a Division I / Division II format.

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.4 Fire Area Assessment.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Affected equipment is sorted alpha-numerically by safe shutdown division.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, , Rev. ,	Section B.10 Fault Tree Logic Reports (SSD Report) Section 9.2.

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.4.2.2 Determine the Shutdown Paths Least Impacted By a Fire in Each Fire Area	<p>Based on a review of the systems, equipment and cables within each fire area, determine which shutdown paths are either unaffected or least impacted by a postulated fire within the fire area. Typically, the safe shutdown path with the least number of cables and equipment in the fire area would be selected as the required safe shutdown path. Consider the circuit failure criteria and the possible mitigating strategies, however, in selecting the required safe shutdown path in a particular fire area. Review support systems as a part of this assessment since their availability will be important to the ability to achieve and maintain safe shutdown. For example, impacts to the electric power distribution system for a particular safe shutdown path could present a major impediment to using a particular path for safe shutdown. By identifying this early in the assessment process, an unnecessary amount of time is not spent assessing impacts to the frontline systems that will require this power to support their operation.</p> <p>Based on an assessment as described above, designate the required safe shutdown path(s) for the fire area. Identify all equipment not in the safe shutdown path whose spurious operation or mal-operation could affect the shutdown function. Include these cables in the shutdown function list. For each of the safe shutdown cables (located in the fire area) that are part of the required safe shutdown path in the fire area, perform an evaluation to determine the impact of a fire-induced cable failure on the corresponding safe shutdown equipment and, ultimately, on the required safe shutdown path.</p> <p>When evaluating the safe shutdown mode for a particular piece of equipment, it is important to consider the equipment's position for the specific safe shutdown scenario for the full duration of the shutdown scenario. It is possible for a piece of equipment to be in two different states depending on the shutdown scenario or the stage of shutdown within a particular shutdown scenario. Document information related to the normal and shutdown positions of equipment on the safe shutdown equipment list.</p>

<u>Applicability</u>	<u>Comments</u>
Applicable	At Harris, the least affected "division" may be selected as a starting point since specific safe shutdown paths are not identified.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Specific safe shutdown paths are not designated or identified. The least affected safe shutdown division is selected and the CAFTA Fault Tree and other information in the FSSPMD is used to develop the best overall safe shutdown strategy.			, , Rev. , Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016,	Section 9.0. Sections 9.2 and 9.3. Section 9.1

Table B-2 Nuclear Safety Capability Assessment

Methodology Review

NFPA 805 Section: 2.4.2.4 Fire Area Assessment.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Compliance strategies are entered into the database as described in FPIP-0106 and the FSSPMD User's Manual. The FSSPMD Fault Tree Logic Report "Fire Area Summary Report" details the compliance strategies used in each fire area. These reports are included in Appndix 18 of Plant USA-E/ELEC-0001.			, , Rev. ,	Section 9.2
				Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006 N/A, Progress Energy Fire Safe Shutdown Program Manager Database User's Manual, Rev. 001,	SSD and Fire Area Summary Reports Section B.10.1.1 and Appendix 18 Section 5.0.

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>	<u>Comments</u>
3.4.2.5 Document the Compliance Strategy or Disposition Determined to Mitigate the Effects Due to Fire Damage to Each Required Component or Cable <u>Applicability</u>	Assign compliance strategy statements or codes to components or cables to identify the justification or mitigating actions proposed for achieving safe shutdown. The justification should address the cumulative effect of the actions relied upon by the licensee to mitigate a fire in the area. Provide each piece of safe shutdown equipment, equipment not in the path whose spurious operation or mal-operation could affect safe shutdown, and/or cable for the required safe shutdown path with a specific compliance strategy or disposition. Refer to Attachment 6 for an example of a Fire Area Assessment Report documenting each cable disposition.	
Applicable		In the CAFTA fault tree, basic events and gates are recovered until "success" is achieved. All affected equipment is not required to be addressed.

<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Resolution strategies are added to the augmented fault tree until the fault tree indicates success and that it has been demonstrated that safe shutdown can be achieved.			Plant USA FSSPMD R16 00, Fire Safe Shutdown Program Manager Database, Rev. 016, , , Rev. , N/A, Progress Energy Fire Safe Shutdown Program Manager Database User's Manual, Rev. 001, Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	SSD and Fire Area Summary Report Section 9.2 Section 5.0 Section B.10.1.1

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NFPA 805 Section: 2.4.2.4 Fire Area Assessment.

<u>NEI 00-01 Ref</u>	<u>NEI 00-01 Guidance</u>
3.5.1.5 [C, Likelihood of Undesired Consequences]	Determination of the potential consequence of the damaged associated circuits is based on the examination of specific NPP piping and instrumentation diagrams (P&IDs) and review of components that could prevent operation or cause maloperation such as flow diversions, loss of coolant, or other scenarios that could significantly impair the NPP's ability to achieve and maintain hot shutdown. When considering the potential consequence of such failures, the [analyst] should also consider the time at which the prevented operation or maloperation occurs. Failures that impede hot shutdown within the first hour of the fire tend to be most risk significant in a first-order evaluation. Consideration of cold-shutdown circuits is deferred pending additional research.

<u>Applicability</u>	<u>Comments</u>
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<u>Alignment Statement</u>	<u>Alignment Basis</u>	<u>Comments</u>	<u>Unit</u>	<u>Reference Document</u>	<u>Doc. Details</u>
Aligns	Plant USA performed a multiple spurious operations review in accordance with the guidelines of NRC RIS 2004-03. The results of the review are contained in Appendix 14 of the safe shutdown analysis.			Plant USA-E/ELEC-0001, Safe Shutdown in Case of Fire and Fire Hazards Analysis, Rev. 0, 6/2/2006	Appendix 14

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NFPA 805 Section: 2.4.2.4 Fire Area Assessment.