

**From:** Kimberly Green  
**To:** <dtyner@entergy.com>,"MICHAEL D STROUD" <MSTROUD@entergy.com>  
**Date:** 1/17/2008 12:57:07 PM  
**Subject:** Draft Telecon Summary — RCS Draft RAIs  
**cc:** "Bo Pham" <BMP@nrc.gov>,<IPNonPublicHearingFile@nrc.gov>

Donna and Mike,

Attached is the draft telecon summary of 1/9/08 regarding our discussion of the draft RAIs on the reactor coolant system.

Please look over and notify me of any discrepancies or inaccuracies.

Thanks,  
Kim

**Hearing Identifier:** IndianPointUnits2and3NonPublic  
**Email Number:** 370

**Mail Envelope Properties** (479762F0.HQGWDO01.OWGWPO04.200.2000008.1.167685.1)

**Subject:** Draft Telecon Summary — RCS Draft RAIs

**Creation Date:** 1/17/2008 12:57:07 PM

**From:** Kimberly Green

**Created By:** KJG1@nrc.gov

**Recipients**

"Bo Pham" <BMP@nrc.gov>  
<IPNonPublicHearingFile@nrc.gov>  
<dtyner@entergy.com>  
"MICHAEL D STROUD" <MSTROUD@entergy.com>

**Post Office**  
OWGWPO04.HQGWDO01

**Route**  
nrc.gov

**Files** **Size**  
MESSAGE 228  
Telecon Summary 01-09-08 RCS.doc  
3:53:20 PM

**Date & Time**  
1/17/2008 12:57:07 PM  
65024 1/23/2008

**Options**

**Priority:** Standard  
**Reply Requested:** No  
**Return Notification:** None  
None

**Concealed Subject:** No  
**Security:** Standard

LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: Indian Point Nuclear Generating Unit Nos. 2 and 3

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JANUARY 9, 2008, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND ENTERGY NUCLEAR OPERATIONS, INC., CONCERNING DRAFT REQUEST FOR ADDITIONAL INFORMATION PERTAINING TO THE INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3, LICENSE RENEWAL APPLICATION – REACTOR COOLANT SYSTEM

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc. held a telephone conference call on January 9, 2008, to discuss and clarify the staff's draft request for additional information (D-RAI) concerning the Indian Point Nuclear Generating Unit Nos. 2 and 3, license renewal application. The telephone conference call was useful in clarifying the intent of the staff's D-RAI.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a listing of the D-RAI items discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

Kimberly Green, Safety Project Manager  
Projects Branch 2  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Docket Nos. 50-247 and 50-286

Enclosures:

1. List of Participants
2. List of Draft Request for Additional Information

cc w/encls: See next page

LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: Indian Point Nuclear Generating Unit Nos. 2 and 3

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JANUARY 9, 2008, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND ENTERGY NUCLEAR OPERATIONS, INC., CONCERNING DRAFT REQUEST FOR ADDITIONAL INFORMATION PERTAINING TO THE INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3, LICENSE RENEWAL APPLICATION – REACTOR COOLANT SYSTEM

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc. held a telephone conference call on January 9, 2008, to discuss and clarify the staff's draft request for additional information (D-RAI) concerning the Indian Point Nuclear Generating Unit Nos. 2 and 3, license renewal application. The telephone conference call was useful in clarifying the intent of the staff's D-RAI.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a listing of the D-RAI items discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

Kimberly Green, Safety Project Manager  
Projects Branch 2  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Docket Nos. 50-247 and 50-286

Enclosures:

1. List of Participants
2. List of Draft Request for Additional Information

cc w/encls: See next page

DISTRIBUTION: See next page

**ADAMS Accession No.: G:\ADRO\DLR\RPB2\Indian Point\Safety Review\Telecon Summaries\Telecon Summary 01-09-08 RCS.doc**

|        |         |             |             |
|--------|---------|-------------|-------------|
| OFFICE | LA:DLR  | PM:RPB2:DLR | BC:RPB2:DLR |
| NAME   |         | KGreen      | RFranovich  |
| DATE   | 01/ /08 | 01/ /08     | 01/ /08     |

OFFICIAL RECORD COPY

**TELEPHONE CONFERENCE CALL  
INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3  
LICENSE RENEWAL APPLICATION**

**LIST OF PARTICIPANTS  
JANUARY 9, 2008**

**PARTICIPANTS**

Kim Green  
Benjamin Parks  
Mike Stroud  
Ted Ivy  
Ron Finnin  
Stan Batch

**AFFILIATIONS**

U.S. Nuclear Regulatory Commission (NRC)  
NRC  
Entergy Nuclear Operations, Inc. (Entergy)  
Entergy  
Entergy  
Entergy

**DRAFT REQUEST FOR ADDITIONAL INFORMATION  
INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3  
LICENSE RENEWAL APPLICATION  
REACTOR COOLANT SYSTEM**

**JANUARY 9, 2008**

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc. held a telephone conference call on January 9, 2008, to discuss and clarify the following draft request for additional information (D-RAI) concerning the Indian Point Nuclear Generating Unit Nos. 2 and 3 license renewal application (LRA).

**D-RAI 2.3A.1.2-1 Reactor Vessel Internals**

If failure of the following reactor vessel internals could potentially inhibit core coolability in an accident scenario, they would be subject to the requirement in 10 CFR 54.4(a)(2). Justify the exclusion of these components from the scope of license renewal:

- a) rectangular sample tubing
- b) sample tubing springs
- c) core energy absorber

**Discussion:** The applicant stated that the components identified in (a) and (b) are referred to as holder and holder closures (ref. WCAP-14577-A), and that the component is (c) is referred to as a secondary core support (ref. LRA, pp 3.1-80 and 3.1-99). Upon further review by the staff, component (c) will be eliminated from the question. The revised question will be sent as a formal RAI.

If failure of the following reactor vessel internals could potentially inhibit core coolability in an accident scenario, they would be subject to the requirement in 10 CFR 54.4(a)(2). Justify the exclusion of these components from the scope of license renewal:

- a) rectangular sample tubing
- b) sample tubing springs

**D-RAI 2.3A.1.3-2 Reactor Coolant Pressure Boundary**

Manway covers and insert plates (see LRA Table 2.3.1-3-IP2 and 2.3.1-3-IP3) have been identified as within the scope of license renewal and subject to an aging management review (AMR) in the LRA. Please clarify whether the manways themselves are within the scope of license renewal and subject to an AMR.

**Discussion:** Based on the discussion with the applicant, the staff agreed to revise this question as follows. The revised question will be sent as a formal RAI.

Pressurizer manway covers and insert plates (see LRA Table 2.3.1-3-IP2 and 2.3.1-3-IP3) have been identified as within the scope of license renewal and subject to an aging management review (AMR) in the LRA. Please clarify whether the pressurizer manways themselves are within the scope of license renewal and subject to an AMR.

#### **D-RAI 2.3A.1.3-3**

Level sensor housing/vents in the reactor vessel level indication system are not highlighted as components that are subject to an AMR (ref: license renewal drawing LRA-208798). The sensor housing/vents appear to provide a reactor coolant pressure boundary. Please clarify whether these components are subject to an AMR, and if not, justify their exclusion.

**Discussion:** The applicant stated that the sensors are active components; therefore, they are not subject to an AMR. The staff clarified its question to say that it is specifically asking about the vent portion of the sensor shown on the drawing and questions whether the vent has a pressure boundary function. The staff will revise this question as follows. The revised question will be sent as a formal RAI.

Level sensor vents in the reactor vessel level indication system are not highlighted as components that are subject to an AMR (ref: license renewal drawing LRA-208798-0). The sensor vents appear to provide a reactor coolant pressure boundary. Please state whether the vents associated with the level sensors, as shown on license renewal drawing LRA-208798-0, have a pressure boundary function, and therefore should be subject to an AMR. If not, justify their exclusion.

#### **D-RAI 2.3B.1.3-1**

High volume sensor housings in the reactor vessel level indication system are not highlighted as components that are subject to an AMR (ref: license renewal drawing LRA-9321-72043, Sht. 5). The sensor housings appear to provide a reactor coolant pressure boundary. Please clarify whether these components are subject to an AMR, and if not, justify their exclusion.

**Discussion:** The applicant stated that sensors are active; therefore, the sensors are not subject to an AMR. Therefore, this question is withdrawn and will not be sent as a formal RAI.

#### **D-RAI 2.3B.1.3-2**

The auxiliary pressurizer spray has a function identified within the chemical volume and control system for 10 CFR 54.4(a)(3) to provide RCS reactivity, inventory, and pressure control during an Appendix R event. Please explain whether the auxiliary pressurizer spray components that perform the above intended function are within the scope of license renewal.

**Discussion:** The applicant referred the staff to license renewal drawings LRA-9321-27363 and LRA-9321-37473 which show the auxiliary pressurizer spray components that have an Appendix R intended function. After further review by the staff, it determined that the auxiliary pressurizer spray components are highlighted on the stated drawings as subject to an AMR. Therefore, this question is withdrawn and will not be sent as a formal RAI.

#### **D-RAI 2.3A.2.4-1 Safety Injection System**

The safety injection accumulators are highlighted on license renewal drawings LRA-235296-0 and LRA-27353-0 as subject to an AMR; however, LRA Tables 2.3.2-4-IP2 and 2.3.2-4-IP3 do not include them as component types subject to an AMR. Please clarify whether the accumulators are subject to an AMR.

**Discussion:** The applicant stated that the accumulators are referred to as component type "tank." The staff confirmed that LRA Tables 2.3.2-4-IP2 and 2.3.2-4-IP3 include the component type "tank" as subject to an AMR. Therefore, this question is withdrawn and will not be sent as a formal RAI.

**D-RAI 2.3A.4.5-2** IP2 AFW Pump Room Fire Event

Please provide a separate listing of those auxiliary steam system components subject to an AMR that support the refueling water storage tank pressure boundary.

**Discussion:** The applicant referred the staff to license renewal drawing LRA-9321-2735. After further review by the staff, it determined that the auxiliary steam system components are highlighted on the stated drawing as subject to an AMR. Therefore, this question is withdrawn and will not be sent as a formal RAI.