

Mr. Charles G. Pardee
Chief Nuclear Officer and
Site Vice President
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

March 10,, 2008

SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2, ISSUANCE OF AMENDMENTS
RE: REVISE TECHNICAL SPECIFICATION 3.7.5, "CONTROL ROOM AREA
VENTILATION AIR CONDITIONING (AC) SYSTEM," (TAC NOS. MD5796 AND
MD5797)

Dear Mr. Pardee:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 188 to Facility Operating License No. NPF-11, and Amendment No. 175 to Facility Operating License No. NPF-18, for the LaSalle County Station, Units 1 and 2, respectively. The amendments are in response to your application dated June 18, 2007.

The amendments will revise Technical Specification (TS) 3.7.5, "Control Room Area Ventilation Air Conditioning (AC) System," to add an Action Statement for two inoperable control room area ventilation AC subsystems. This operating license improvement is part of the consolidated line item improvement process and was made available by the NRC on March 26, 2007 (72 FR 14143) as part of the NRC approved Industry/Technical Specification Task Force (TSTF) TSTF-477, Revision 3.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Stephen P. Sands, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosures: 1. Amendment No. 188 to NPF-11
2. Amendment No. 175 to NPF-18
3. Safety Evaluation

cc w/encls: See next page

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LaSalle County Station, Units 1 and 2

cc:

Corporate Distribution
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via e-mail
LaSalle Distribution
Exelon Generation Company, LLC
via e-mail

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Illinois Emergency Management Agency
Division of Disaster Assistance &
Preparedness
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EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 188
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Exelon Generation Company, LLC (the licensee), dated June 18, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 188, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: March 10, 2008

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 175
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Exelon Generation Company, LLC (the licensee), dated June 18, 2007, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 175, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Russell Gibbs, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: March 10, 2008

ATTACHMENT TO LICENSE AMENDMENT NOS.188 AND 175

FACILITY OPERATING LICENSE NOS. NPF-11 AND NPF-18

DOCKET NOS. 50-373 AND 50-374

Replace the following pages of the Facility Operating Licenses and Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment numbers and contain marginal lines indicating the areas of change.

Remove

License NPF-11

Page 3

License NPF-18

Page 3

TSs

TS 3.7.5-1

TS 3.7.5-2

TS 3.7.5-3

Insert

License NPF-11

Page 3

License NPF-18

Page 3

TSs

TS 3.7.5-1

TS 3.7.5-2

TS 3.7.5-3

- (4) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (5) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of LaSalle County Station, Units 1 and 2.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level
The licensee is authorized to operate the facility at reactor core power levels not in excess of full power (3489 megawatts thermal).
 - (2) Technical Specifications and Environmental Protection Plan
The Technical Specifications contained in Appendix A, as revised through Amendment No. 188, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
 - (3) Conduct of Work Activities During Fuel Load and Initial Startup
The licensee shall review by committee all Unit 1 Preoperational Testing and System Demonstration activities performed concurrently with Unit 1 initial fuel loading or with the Unit 1 Startup Test Program to assure that the activity will not affect the safe performance of the Unit 1 fuel loading or the portion of the Unit 1 Startup Program being performed. The review shall address, as a minimum, system interaction, span of control, staffing, security and health physics, with respect to performance of the activity concurrently with the Unit 1 fuel loading or the portion of the Unit 1 Startup Program being performed. The committee for the review shall be composed of at least three members, knowledgeable in the above areas, and who meet the qualifications for professional-technical personnel specified by

- (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70 possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of LaSalle County Station Units 1 and 2.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of full power (3489 megawatts thermal). Items in Attachment 1 shall be completed as specified. Attachment 1 is hereby incorporated into this license.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 175, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Conduct of Work Activities During Fuel Load and Initial Startup

The licensee shall review by committee all Unit 2 Preoperational Testing and System Demonstration activities performed concurrently with Unit 2 initial fuel loading or with the Unit 2 Startup Test Program to assure that the activity will not affect the safe performance of the Unit 2 fuel loading or the portion of the Unit 2 Startup Program being performed. The review shall address, as a minimum, system interaction, span of control, staffing, security and health physics, with respect to performance of the activity concurrently with the Unit 2 fuel loading or the portion of the Unit 2 Startup Program being performed. The committee for the review shall be composed of at least three members, knowledgeable in the above areas, and who meet the qualifications for professional-technical personnel specified by section 4.4 of ANSI N18.7-1971. At least one of these three shall be a senior member of the Assistant Superintendent of Operation's staff.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 188 TO FACILITY OPERATING LICENSE NO. NPF-11
AND AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. NPF-18
EXELON GENERATION COMPANY, LLC
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By letter to the Nuclear Regulatory Commission (NRC, the Commission) dated June 18, 2007 (Agencywide Documents and Access Management Systems (ADAMS) Accession No. ML071690450), Exelon Generation Company, LLC (the licensee) proposed changes to the technical specifications (TS) for LaSalle County Station, Units 1 and 2. The requested changes are the adoption of Technical Specifications Task Force (TSTF) Change Traveler TSTF-477, Revision 3, "Add Action for Two Inoperable Control Room AC Subsystems" which was proposed by the TSTF by letter on September 8, 2006. The proposed changes revising Technical Specification 3.7.5 (BWR [Boiling Water Reactor]/4) (3.7.4 for BWR/6) "Control Room Air Conditioning (AC) System" involve adding the following limiting conditions for operation (LCO):

B. Two control room area ventilation AC subsystems inoperable	B.1 Verify control room area temperature < [90] °F.	Once per 4 hours
	and	
	B.2 Restore one control room area ventilation AC subsystems to OPERABLE status.	72 hours

The TSTF-477 was announced for availability in the *Federal Register* on March 26, 2007, as part of the consolidated line item improvement process (CLIIP).

2.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TS as part of the license. The TS ensures the operational capability of structures, systems and components that are required to protect the health and safety of the public. The Commission's regulatory requirements related to the content of the TS are contained in Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36. That regulation requires that the TS include items in the following specific categories: (1) Safety limits, limiting safety systems settings, and limiting control settings (50.36(d)(1)); (2) Limiting conditions for operation (50.36(d)(2)); (3) Surveillance requirements (50.36(d)(3)); (4) Design features (50.34(d)(4)); and (5) Administrative controls (50.36(d)(5)).

In general, there are two classes of changes to TS: (1) changes needed to reflect modifications to the design basis (TS are derived from the design basis), and (2) voluntary changes to take advantage of the evolution in policy and guidance as to the required content and preferred format of TS over time. This amendment addresses changes within the second class of changes. In determining the acceptability of revising Standard Technical Specifications (STS) 3.7.5 (STS 3.7.4 for BWR/6), the NRC staff used the accumulation of generically approved guidance in NUREG-1433, "Standard Technical Specifications, Revision 3, General Electric Plants, BWR/4" dated June, 2004 and; NUREG-1434, Revision 3, "Standard Technical Specifications, General Electric Plants, BWR/6" dated June, 2004.

Licensees may revise the TS to adopt current improved STS (ISTS) format and content provided that plant-specific review supports a finding of continued adequate safety because (1) the change is editorial, administrative or provides clarification (i.e., no requirements are materially altered), (2) the change is more restrictive than the licensee's current requirement, or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 3.0 in the context of specific proposed changes.

3.0 TECHNICAL EVALUATION

The BWR/4 and BWR/6 STS for the control room AC system do not contain an Action Statement for two inoperable subsystems. During the conversion of the BWR/6 plants to the improved STS, the BWR/6 plants added Action Statements for 2 inoperable control room ventilation and AC subsystems.

The STS for numerous safety related systems also contain Action Statements for 2 inoperable subsystems. TSTF-477 added an Action Statement for 2 inoperable control room AC subsystems to the BWR/4 and BWR/6 STS to be consistent with the current licensing basis for the BWR/6 operating plants. Furthermore, the consistency of the BWR STS will be enhanced since most safety related systems presently have Action Statements in the STS to address two inoperable subsystems.

The NRC staff reviewed the proposed changes against the corresponding changes made to the STS by TSTF-477, Revision 3, which the NRC staff has found to satisfy applicable regulatory requirements, as described above in Section 2.0. LSCS, Units 1 and 2 are classified as BWR/5 plants. While the LSCS, Units 1 and 2 TS 3.7.5 are based on NUREG1434, "Standard Technical Specifications General Electric Plants, BWR/6," the proposed changes are modeled after changes made to STS 3.7.5 for BWR/4 in TSTF-477. Because TSTF-477 provides technical justification for the addition of an allowed outage time of 72 hours versus 7 days as provided for the BWR/6 plants, this is a more conservative adoption of the TSTF-477 change. The proposed changes are consistent with this design, and are therefore acceptable.

3.1 NUREG-1433, Revision 3, "Standard Technical Specifications, General Electric Plants, BWR/4"

The proposed BWR/4 Action statement allows 72 hours to restore 1 subsystem to the operable status for 2 inoperable subsystems. During the 72 hour completion time the control room temperature is verified < 90 degrees every 4 hours. If 1 control room AC can not be restored to

operable status or the control room temperature can not be maintained < 90 degrees then the unit must be placed in at least Mode 3 within 12 hours and Mode 4 within 36 hours. Maintaining the control room temperature < 90 degrees assures that the safety related equipment in the control room will remain within the original licensed design operating temperature, because the maximum allowable control room temperature is unchanged by TSTF-477. The NRC staff finds that the proposed changes in TSTF-477 are acceptable because the TSTF-477 changes provide TS requirements that the control room temperature will be maintained within the original licensed design operating temperature of the control room equipment or the plant will be placed in the Cold Shutdown Mode (Mode 4, Safe Shut Condition).

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR part 20 or surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration determination and there has been no public comment on such finding (72 FR 51860; September 11, 2007). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principle Contributor: A. J. Lising, NRR

Date: March 10, 2008