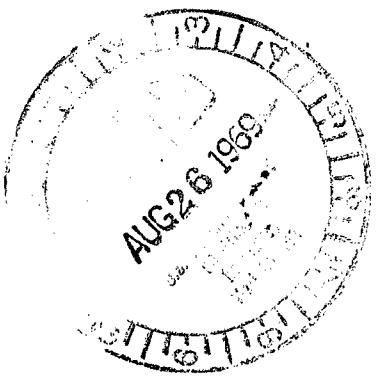
DOCKET NO. 70-58

Regulatory File Cy.




A TELEDYNE COMPANY

ISOTOPES
NUCLEAR SYSTEMS DIVISION
EASTERN BLVD AT MARTIN BLVD N. E.
P. O. BOX 4937
MIDDLE RIVER, MARYLAND 21220
(301) 682-5800 TWX (710) 239-9037

August 15, 1969

Refer to: PJK:1210:045

U. S. Atomic Energy Commission
Division of Materials Licensing
Washington, D. C. 20545

Attention: Mr. Donald A. Nussbaumer
Chief
Source and Special Nuclear Materials Branch

Subject: Notification of Isotopes' Nuclear Systems Division Move to New Facilities and Amendment and Renewal of License No. SNM-53

Reference: (a) Title 10 Code of Federal Regulations Parts 70.21, 70.22, 70.34 and 70.41(a)

Gentlemen:

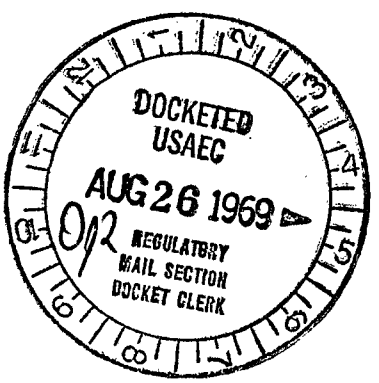
The Nuclear Systems Division of Isotopes, Inc., requests that License No. SNM-53 be amended to:

(a.) Extend its expiration date to that specified in Section I below.

(b.) Permit the possession and use of material described in Section I below for the purposes and under the conditions described therein at the company's new location at Timonium, Maryland.

(c.) Permit the possession of special nuclear material in the form of contamination on facilities and equipment as described in Section II below at the Martin Marietta Corporation installation at the address presently specified in the license during decontamination.

In the event that all of the requested amendments cannot be considered at the same time, Isotopes requests prompt action upon those amendments which can be granted in view of the fact that firm moving arrangements have been made.

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ACKNOWLEDGED

The new address will be:

110 West Timonium Road
Timonium, Maryland 21093

It is presently anticipated that Isotopes will vacate the Middle River Facility by September 30, 1969 and restore operations at the new location on or about that same date.

In view of the foregoing, a thorough review of our future special nuclear material requirements has been made and the following data is submitted in support of our application.

## SECTION I

### A. Applicant

Isotopes, Inc., a Division of Teledyne, Inc., is incorporated in the state of California with its headquarters office at 50 Van Buren Place, Westwood, New Jersey 07675. The principal officers, who are citizens of the United States, are as follows:

| | |
|---|---|
| J. Laurence Kulp | President<br>50 Van Buren Avenue<br>Westwood, New Jersey 07675 |
| Robert H. Kleiner | Vice President and Assistant Secretary<br>50 Van Buren Avenue<br>Westwood, New Jersey 07675 |
| Eric A. Willis | Vice President<br>50 Van Buren Avenue<br>Westwood, New Jersey 07675 |
| Peter J. Vogelberger, Jr. | Vice President and General Manager<br>110 West Timonium Road<br>Timonium, Maryland 21093 |
| Henry E. Singleton | Chairman of the Board<br>1901 Avenue of the Stars<br>Los Angeles, California 90067 |

Isotopes, Inc., is not owned, controlled or dominated by an alien, a foreign corporation or a foreign government.

Isotopes' Nuclear Systems Division has the responsibility for the operation of facilities at Timonium, Maryland, for the performance of programs utilizing special nuclear materials.

B. Term of License

To continue nuclear programs utilizing special nuclear materials, Isotopes' Nuclear Systems Division requests an extension of SNM License No. 53 subject to the provisions outlined herein as well as those terms and conditions set forth in 10 CFR Part 70. We request that the license remain in effect until September 30, 1972.

C. Use of Special Nuclear Material

The scope of utilization of special nuclear materials at Isotopes' Timonium, Maryland facilities will be substantially reduced from that previously authorized for the Middle River Facilities and will involve only neutron instrument calibration, foil activation and limited R & D activities.

D. License Limits

The following maximum quantity limits are established for special nuclear material at Isotopes' Timonium Facility:

| | |
|---|---|
| U-235 (any enrichment, any chemical form) for R & D activities | 100 Gms |
| Pu-239 for neutron and alpha instrument calibration sources | 171 Gms |

The plutonium is in the form of an encapsulated Pu-Be neutron source and in alpha sources used for instrument calibration. When not in use, the Pu-Be neutron source will be stored in a paraffin shield cask and locked to prevent unauthorized use. Use of all sources will be controlled by Health Physics personnel.

The present inventory of plutonium to be moved to the new location is as follows:

| | | |
|---|---|---|
| 1. | Four (4) metallic plutonium alpha counting standards | ~70 micrograms |
| *2. | Ten (10) metallic foils contained in the Nuclear Accident Dosimeters (no foil exceeding 1.08 gms) | 10.405 grams |

3. One (1) plutonium-beryllium neutron source doubly encapsulated in tantalum and stainless steel — 159.94 grams

Total Pu-239: ~ 171 grams

* Since the Nuclear Accident Dosimeters are Government-Furnished Equipment, Isotopes is presently negotiating return of the property to the appropriate accountable agency.

E. Technical Qualifications

On November 26, 1956, the Martin Company was granted Special Nuclear Material License No. 53 by the Atomic Energy Commission. The prime requirement for enriched uranium at that time was for the fabrication of fuel to be used in design and criticality studies for the medium power portable reactor concept. Since that time, the Martin Company has developed its technical efficiencies and production capabilities as a supplier of the nuclear fuel components under license for the PRO Critical Facility-CNEN and Pathfinder reactors. As a contractor of the New York Operations Office of the Atomic Energy Commission, portable medium power reactors have been delivered to the AEC for generation of electrical power at remote installations in Sundance, Wyoming (PM-1) and McMurdo Sound, Antarctica (PM-3A). During this development in the atomic energy field, the Martin Company has also built and operated a critical assembly facility for use in obtaining nuclear physics data in connection with various nuclear core designs. Slab type, as well as liquid fluidized bed experiments, have been conducted in this facility under specific class 104 Facilities Licenses issued in accordance with Title 10 Code of Federal Regulations Part 50. The Martin Company has also been the proponent and primary supplier of devices utilizing special nuclear material as well as radioisotopes for Systems for Nuclear Auxiliary Power (SNAP).

The fulfillment of the programs discussed above has necessitated the assembly of a competent and highly skilled technical staff. Included are engineering, physics, metallurgy, design, and production talents. On August 5, 1968, Isotopes, Inc., acquired the former Martin Marietta Corporation Nuclear Division. This transaction also involved acquisition of Martin's nuclear technical staff. This staff is further supplemented by Isotopes' Westwood staff thereby providing a broadly diversified technical capability.

F. Internal Procedural Control to Insure Compliance with License Conditions

1. General

Isotopes has established a system of internal checks and balances to provide reasonable assurance of complete compliance with licensing

requirements and applicable laws, rules and regulations. This system provides for internal review and inspection of all safety aspects involved in the handling of special nuclear material. The following general statements of the responsibilities of the functional groups associated with safety aspects in the utilization of special nuclear material are presented below.

Manager, Special Nuclear Material (R & D)

1. Supervises all R & D activities involving special nuclear materials to assure that individual users comply with all rules and regulations during utilization of the material.

Manager, Nuclear Materials

1. Serves as the Isotope contact with regulatory agencies in all matters pertaining to radioactive materials;

2. Determines the need for and obtains all necessary governmental approvals required in the utilization of radioactive materials;

3. Develops the criteria for and administers the accountability program for radioactive materials;

4. Conducts periodic operational reviews to appraise user's compliance with latest regulations;

5. Interprets and disseminates applicable regulatory information to activities concerned.

Supervisor, Health Physics

1. Develops and administers the health physics program to minimize the hazard to personnel and reduce company liability during the utilization of radioactive materials;

2. Monitors user performance to assure safety as well as compliance with rules and regulations;

3. Reviews and approves each planned material usage.

Supervisor, Security

1. Shall be responsible to establish security control areas and maintain surveillance of employee activities to assure compliance with Security requirements.

Criticality Engineer

1. Shall be responsible for establishing safety criteria as required for processing operations and monitoring operations to assure compliance with such criteria.

2. Technical Personnel Qualifications

Manager, SNM R & D

Education: B.S. in Chemistry, Chemical Engineering, or B.S. Physics or Engineering with graduate work in either chemistry or nuclear engineering.

Experience: Minimum of 5 years experience in chemical system design, experimental system operation or related scientific type work.

Manager, Nuclear Materials

Education: B.A. in Business Management, Accounting or B.S. in Engineering Chemistry, Physics, Math or related fields.

Experience: A minimum of 2 years experience in nuclear materials management and/or related fields.

Supervisor, Health Physics

Education: B.S. in Physics, Chemistry, Math, Physical Sciences or equivalent.

Experience: Four (4) years experience in Health Physics administration and control procedures with particular emphasis being placed on personnel possessing a broad spectrum of Health Physics experience as well as a knowledge of nuclear instrumentation or general electronics.

Criticality Engineer

Education: B.S. in Physics, Engineering Physics or B.S. in Engineering, Chemistry with graduate work in physics or nuclear engineering.

Experience: Minimum of 3 years experience in reactor physics design, nuclear experimental work and/or other related scientific type work.

G. Radiological Safety Controls and General Facility Requirements

Since Isotopes forecasts no need for the handling and processing of large quantities of Uranium-235, criticality control will not be the limiting factor. The major emphasis will be placed upon radiological control to assure adequate personnel safeguards and compliance with applicable rules and regulations.

Uranium compounds in forms which can readily become airborne will be handled only in contained atmospheres or adequately ventilated enclosures.

H. Uranium Material Storage

All uranium material accountable under this license will be stored in a locked cabinet when not in use or awaiting shipment in an approved shipping container. Access to stored material will be controlled by Health Physics personnel.

I. Radiological Safety Program

1. Introduction

The fundamental prerequisite for the operation of any nuclear facility is assurance that it will present no undue hazard to the health and safety of facility personnel or to the general public.

The basic radiological safety objectives for the achievement of this requirement are (1) to preclude the uncontrolled release of radioactivity to the normally occupied areas within the facility and/or to the outside environment, and (2) to limit the radiation exposure of facility personnel to the lowest practicable level and in compliance with established radiation protection standards.

The standards used in the conduct of the Isotopes radiological safety program are in conformance with the recommendations and guides contained in the following publications: Title 10, Code of Federal Regulations; Federal Radiation Council; National Committee on Radiological Protection; National Bureau of Standards; and State Radiation Protection Codes.

2. Responsibility for the Radiological Safety Program

Responsibility for radiological safety, as for any other health and safety matter, rests with management and is discharged through management's representative.

Technical direction of the radiological safety program has been delegated to the Supervisor, Health Physics Section. This responsibility includes the development and maintenance of a complete and integrated program for radiation protection. The methods by which radiation safety is achieved involve the analysis of postulated hazards resulting from both routine and accidental occurrences within the facility and evaluation of the effectiveness of the control and surveillance mechanisms instituted to cope with any unique radiation problem which might arise.

3. Scope of the Radiological Safety Program

The Isotopes' Health Physics Section provides direct support in all programs concerned with radiation and radioactive materials.

The major elements of this radiological safety program are:

a. Nuclear facilities design consultation and approval.

b. Program plan review.

c. Radiological indoctrination and training.

d. Personnel exposure surveillance and documentation including bioassay.

e. Facilities, operations and environmental monitoring.

f. Radioactive material hazard control.

g. Radioactive waste disposal surveillance.

h. Emergency planning and capability.

i. Liaison with regulatory agencies.

4. Health Physics Experience

The individuals comprising the Health Physics Section have had wide experience in organizing and conducting Radiological Safety programs supporting the following nuclear activities:

a. Research, power and production reactors and nuclear accelerators.

b. Radiochemical processing plants, hot cell operations, and high level gamma and x-radiation facilities.

c. Large scale environmental monitoring and waste disposal programs, radioactive material transportation problems, personnel monitoring and instrumentation development, evaluation and calibration.

5. Radiological Safety Organization

The Health Physics Section provides radiation protection services on a level-of-effort basis. The Supervisor, Health Physics Section, reports to the Manager, Research and Development. In addition, he reports on a "dotted line" to the General Manager, Nuclear Systems Division, and also serves as the Radiation Protection Officer.

Surveys are usually performed by technicians under the supervision of a health physicist.

6. Radiation Surveys

Survey and control procedures are utilized to evaluate radiological hazards and establish effective control measures to limit personnel exposure. These surveys are conducted to determine radiation levels, the presence or absence of surface contamination and/or airborne radioactivity in work or occupancy areas.

As a result of these surveys, suitable control measures are imposed to insure adequate personnel protection. The frequency and extent of the surveys are governed by the work performed or by existing conditions. Documentation of all surveys are recorded in operational logs.

7. External Dose Rate Measurements

The measurement of external dose rates is conducted using portable monitoring instruments. Selection of a particular instrument is based upon the type, energy and intensity of the radiation to be monitored. Portable monitoring instruments in use are calibrated at least once every three months and after each repair.

8. Surface Contamination Measurement

Each area where radioactive materials are utilized is surveyed for the presence of fixed and loose contamination. The frequency of these surveys is governed by the nature and type of operations performed. An accelerated survey schedule is employed during the initial phases of any new process or change in process to determine any unusual radiological hazard. Thereafter when work is in progress, a general daily survey is conducted in the work areas at locations which are most likely to indicate changes in contamination levels. If this general survey indicates an increase in contamination level,

a detailed survey is initiated to determine the cause and institute measures to reduce or possibly eliminate the source of contamination.

Loose contamination is evaluated employing the wipe technique which consists of wiping an absorbent paper over an area of approximately 100 $cm^2$ and counting the wipe in a shielded detector-scaler assembly.

Alpha survey instruments are utilized to monitor floors and equipment for fixed alpha contamination.

Instrumentation used to evaluate contamination and radiation is listed in Table 1.

The contamination criteria currently in use is as follows:

| Unrestricted Area | Removable | Max. Fixed | Avg. Fixed |
|---|---|---|---|
| Plutonium and U-233 alpha | 100 dpm/100 $cm^2$ | 2500 dpm/100 $cm^2$ | 500 dpm/100 $cm^2$ |
| Uranium-235 alpha | 1000 dpm/100 $cm^2$ | 25000 dpm/100 $cm^2$ | 5000 dpm/100 $cm^2$ |

9. Surface Contamination Control

Surface contamination is controlled in the fabrication area by employing one or a combination of the following techniques:

a. Performing regular surveys to determine contamination levels.

b. Providing ventilation at the sources of dispersable contaminants.

c. Establishing isolated areas when dispersable contaminants are generated.

d. Utilizing absorbent paper, polyethylene and coatings on equipment and exchanging these during the course of the work.

e. Employing step off pads at entrances to isolated areas.

f. Requiring the use of anti-contamination clothing.

The spread of dispersable contaminants to unrestricted areas is minimized by:

a. Establishing buffer zones between the areas and providing frequent contamination surveys.

b. Monitoring personnel at the point of egress from an isolated area.

c. Monitoring equipment and utilizing an equipment check tag system.

d. Providing receptacles for contaminated wastes and used anti-contamination clothing.

e. Controlling the wearing of anti-contamination clothing outside of a restricted area.

10. Personnel Contamination Control

Anti-contamination clothing requirements are specified by Health Physics prior to starting any operation where potential contaminants may be encountered. These requirements are based upon operational experience and/or survey results.

Anti-contamination clothing will be laundered when required by an approved outside firm providing such service.

11. Description of Decontamination Techniques

Floor contamination is normally removed by scrubbing with mops and detergent solutions. Persistent contamination is removed by employing a mechanical scrubber and steel wool pads. The scrub solution is removed with a wet type vacuum cleaner and finally damp mopped. Waste solution is discharged to the sanitary sewer after sampling and evaluation has shown that the concentration is within the limits prescribed by 10 CFR 20.303.

Equipment contamination is removed by scrubbing with detergents and water or wiping with damp cloths. Large metal machinery and hand tools are decontaminated by wiping with solvent to minimize corrosion. Contaminated scrubbing equipment, mops and wipes are discarded as solid radioactive waste.

12. Airborne Radioactivity Measurement

Breathing zone air samples are collected on an accelerated schedule during the initial phases of any new process or change in process. The sample results are used to evaluate the particulate airborne radioactivity concentrations associated with the process and provide a basis for instituting special requirements to reduce the airborne contamination levels in accordance

with the guides set forth in 10 CFR 20. After the initial breathing zone sample evaluations, general area samples are collected adjacent to any potential contamination producing operation.

Exhaust stack air samples are routinely collected and analyzed from the ventilation system exhaust whenever work is being conducted which could involve contamination dispersion of the environs. The samples are collected downstream from each exhaust system fan prior to exhausting to the environs. Air samples are collected using open face of in-line filter holders containing filter paper.

Air sample filters are counted and analyzed employing detector-scaler assemblies. The procedure can adequately determine concentrations prescribed by 10 CFR 20.103 and .106.

13. Personnel Exposure Surveillance

Personnel exposure surveillance is conducted to measure the amount of external and internal radiation exposure received by individuals whose work involves exposure to ionizing radiation.

14. Personnel Monitoring Methods

All personnel permanently assigned to work in areas where exposure to ionizing radiation is likely are required to wear film badges. Additional monitoring devices are issued as dictated by the type of work performed.

Beta-gamma film is used in all cases when personnel monitoring is required, whereas, neutron film is used only when personnel are exposed to neutron radiation. Film packets are processed and evaluated quarterly or after any suspect or unusual exposure as indicated by supplementary monitoring devices. Visitors are monitored in accordance with 10 CFR 20.202.

All personnel whose work involves the handling of loose disposable types of radioactive materials are periodically scheduled to submit bioassay samples for analysis to aid in determining exposure to concentrations of radioactive materials. In the case of suspected body intake of radioactive materials, special samples are collected from the personnel involved and analyzed.

15. Records

Results of all personnel exposures are posted in the individual's exposure file where a cumulative exposure record is maintained.

16. Instrumentation and Related Equipment

A description of the radiation survey and related equipment utilized in the conduct of the radiological safety program is presented in Table 1.

17. Method, Frequency and Standards Used in Calibrating Instruments

a. Portable Radiation Detection Instruments

All portable survey meters in use are calibrated at least once every three months or after each servicing.

Calibration of beta-gamma detection instruments includes utilization of a 30 mCi Co-60 source. The source is calibrated (employing a Roentgen "R" meter calibrated by NBS) so that at given distances the dose rate in mr/hr is known. The instrument can be remotely placed at a given distance from the source and adjusted to read the radiation intensity at that distance. Where possible, each instrument is checked at two points on each scale or range setting. A 1 mg Ra-226 source is used to calibrate low range instruments.

Neutron survey instruments are calibrated using a Pu-239 Be (10c) source of known flux. Calibration is generally performed out of doors to reduce scattering from building walls, etc. The instrument is adjusted to read the calculated neutron dose rate after adjusting the bias against gamma radiation.

Alpha survey meters are calibrated against reference sources of uranium and plutonium and adjusted to read the known value in counts per minute based upon the manufacturers recommendations concerning counter efficiency.

b. Detector/Scaler Counting Assemblies

Background and statistical accuracy checks are made on all counting room instrumentation prior to use. Calibrated alpha and beta-gamma reference sources are placed in each detector and counted for a preset time. If the instrument readings vary greater than the calculated statistical error, it is recalibrated and the counter efficiency is then determined by comparing the counter count rate with that of the standard source.

c. Instrument Maintenance and Records

Minor instrument repairs and calibrations are performed by Health Physics personnel. Major repairs are performed by the facility instrument technician.

Health Physics maintains permanent records of all instrument calibrations and repair records are maintained by the instrument technician. Each instrument is tagged indicating the calibration date.

18. Radioactive Waste Disposal

All radioactive waste will be disposed in accordance with 10 CFR 20.301. A record for each container of waste, indicating the type of waste, approximate quantity, radiation and contamination level is maintained in the Health Physics office.

## SECTION II

The contamination on the facilities and equipment located within the nuclear facilities shown in Figure 1B3 is in the form of uranium of various enrichments on work surfaces and contained:

1. Within the exhaust ventilation systems,

2. Within enclosures surrounding various pieces of machinery and equipment,

3. Within drain piping leading to retention tanks in the waste disposal room (Fig. 1B3).

Arrangements will be made with representatives of the Martin Marietta Corporation to control the materials by restricting access to the facilities until decontamination has been accomplished.

If you have any questions regarding this application, please contact me or Mr. R. J. Brisson.

Very truly yours,

Peter J. Knapp
Licensing and Accountability Representative

PJK:nmb

TABLE I

TYPICAL RADIATION DETECTION INSTRUMENTS AND RELATED EQUIPMENT

| Type of Instrument Make and Model Number | Number Available | Radiation Detected | Sensitivity Range (mr/hr) | Window Thickness ($mg/cm^2$) | Use: Monitoring, Surveying, Measuring |
|---|---|---|---|---|---|
| Victoreen 592 Ion Chamber | 4 | Gamma | 0-1000 | 90 $mg/cm^2$ | Monitoring |
| Eberline E-500B G.M. | 5 | Beta-Gamma | 0-2000 | 30 $mg/cm^2$ | Surveying |
| Victoreen CD 700 G.M. | 6 | Beta-Gamma | 0-50 | 30 $mg/cm^2$ | Surveying |
| Victoreen MD 740 Ion Chamber | 2 | Beta-Gamma | 0-10,000 | 7 $mg/cm^2$ | Surveying |
| Eberline PAC-1SA Scintillation | 4 | Alpha | 0-100,000 cpm | Less than 1/4 mil mylar | Surveying, measuring |
| Victoreen MD CDV 715 Ion Chamber | 6 | Gamma | 10-500,000 | 30 $mg/cm^2$ | Emergency Monitoring |
| Eberline PAC-3G Gas Proportional | 2 | Alpha | 0-100,000 cpm | Less than 1/4 mil mylar | Surveying |
| Eberline FN-1A Scintillator | 1 | Fast Neutron | 0-5000 $n/cm^2/sec$ | - - | Monitoring |
| Eberline PNC-1 $BF_3$-Tube | 3 | Fast & Thermal Neutron | 0-500,000 cpm | - - | Surveying |
| NMC Mod. PC-10 Gas Proportional Scaling Assemblies | 2 | Alpha Beta-Gamma | 1 to $10^6$ dpm | Less than 1/4 mil mylar | Measuring |

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TABLE I (Continued)



| Type of Instrument Make and Model Number | Number Available | Radiation Detected | Sensitivity Range (mr/hr) | Window Thickness ($mg/cm^2$) | Use: Monitoring, Surveying, Measuring |
|---|---|---|---|---|---|
| Eberline PC4-4 Gas Proportional Detector with MD. PC-6 Scaling Assembly | 1 | Alpha Beta-Gamma | 1 to $10^6$ dpm | Less than 1/4 mil mylar | Measuring |
| Eberline MD SAC-2 Scintillation Detector with TMC, Eberline & Tracerlab Scalers | 1 | Alpha | 10 to $10^6$ dpm | Less than 1/4 mil mylar | Measuring |
| End Window Geiger Tubes with Tracerlab Scaler | 2 | Beta-Gamma | 10 to $10^6$ dpm | Less than 1.4 $mg/cm^2$ | Measuring |
| NMC Model AM-2, AM-3 Constant Air Monitors | 2 | Beta-Gamma | $10^{-12}$ uc/ml | 1.4 $mg/cm^2$ | Air Particulate Monitoring |
| Gast Air Sampling Vacuum Pumps and Associated 2" Filter Holders | | Air sample filters counted for alpha and beta-gamma activity. | | | Collection of air samples from Nuclear work area atmosphere and stack discharge to the environs. |
| Film Badges (Service supplied by U. S. Testing Co., Inc.) | | Upper limit of film response: Gamma - 600 Rem; Beta - 400 Rem; Neutron - 10 Rem | | | Film badges are worn in accordance with applicable sections of 10 CFR 20 as part of the Personnel Monitoring Program. |

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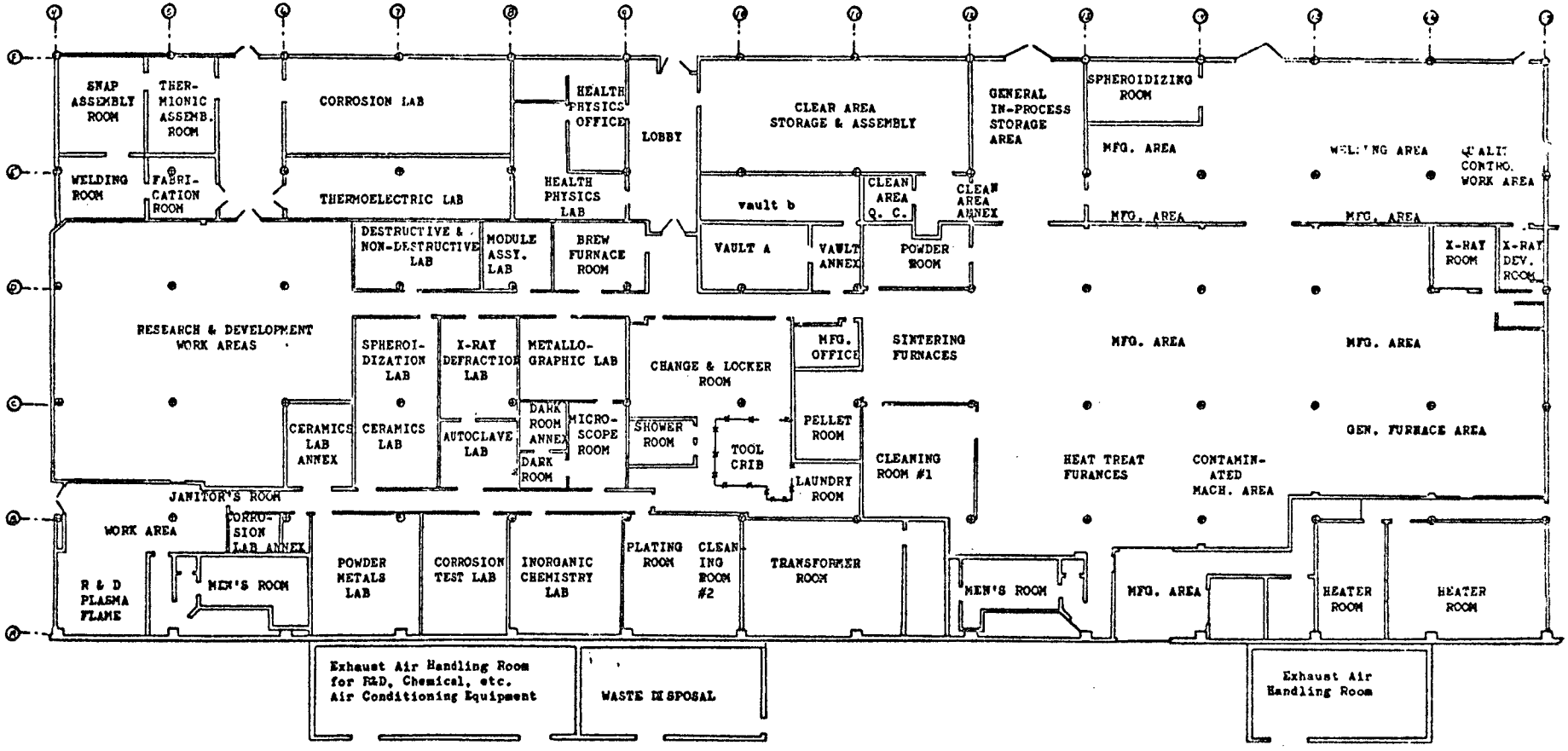


NUCLEAR FACILITIES - D BUILDING

Fig. I-B3

HQ-9401
(9-66)

# INTERMEDIATE ACTION FORM

REFERENCE NUMBERS

**Source & SNM Licenses**

| 01. PROG. CODE | 03. DOCKET NO. | 09 TASK | 42. PURPOSE OF TASK | 12. CONTROL | 15. LICENSE NUMBER |
|---|---|---|---|---|---|
| 63 | 70-55 | 2179 | Amendment | N2179 | SNM-53 |

| 18. APPLICANT | 54. AM. NO. RESULTING FROM TASK |
|---|---|
| Isotopes Nuclear System Division | |

| 21. STREET & BUILDING | 45. CLASSIFICATION | 63. ASG. TO: |
|---|---|---|
| P. O. Box 4937 | | |

| 24. CITY | 27. STATE | 30. ZIP | 33. RECEIVED YR. | MO. | DAY | 36. ISSUED YR. | MO. | DAY | 39. EXPIRED YR. | MO. | DAY |
|---|---|---|---|---|---|---|---|---|---|---|---|
| Middle River, | Md. | 21220 | 69 | 07 | 10 | | | | | | |

| 57. APPLICANT'S COMMUNICATION DATED | YR. | MO. | DAY |
|---|---|---|---|
| | 69 | 07 | 07 |

59. ENCLOSURES

58. DESCRIPTION (MUST BE UNCLASSIFIED)

Ltr.trans. to possess contamination on approx. 30 pieces of heavy machine shop equipment, miscellaneous tools and work surfaces, and the area shown in fig. 3 of the original appl.....

60. DISTRIBUTION

1 — 2 CO. 1 DMP (Mi chow)

## INTERMEDIATE ACTIONS

| TYPE | ON YR. | ON MO. | ON DAY | ACTIV. 92 | RETURNED YR. | RETURNED MO. | RETURNED DAY |
|---|---|---|---|---|---|---|---|
| ADDL. INFO. REQUESTED FROM APPLICANT | 91 | | | 1 | 93 | | |
| REFERRED TO: | 94 | | | 95 2 | 96 | | |
| REFERRED TO: | | | | | | | |

## OTHER REFERRALS

| | DATE YR. | MO. | DAY |
|---|---|---|---|
| Russbauer | 69 | 07 | 10 |
| w/file cy file | | | |
| 1 DMP dispatched ~~2 extras~~ NOT NEEDED | 69 | 07 | 22 |
| 1 Luke for info dispatched DO NOT REMOVE | 69 | 07 | 22 |

DOCKET NO. 20-58
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NUCLEAR SYSTEMS DIVISION
EASTERN BLVD AT MARTIN BLVD N. E.
P. O. BOX 1937
MIDDLE RIVER, MARYLAND 21220
(301) 682-5800 · WX (710) 239-9037

A TELEDYNE COMPANY

Refer to: PJK:1210:042

July 7, 1969

U. S. Atomic Energy Commission
Division of Materials Licensing
Source and Special Nuclear Materials Branch
Washington, D. C. 20545

Subject: Application for Amendment to SNM-53

Reference: TWX from Isotopes-Westwood to Director, DML, dated June 30, 1969, 4:50 p.m.

Gentlemen:

In addition to the possession and use authorization requested in the above referenced TWX, Isotopes requests the following additional form and use with regard to the Uranium:

> To possess contamination on approximately 30 pieces of heavy machine shop equipment, miscellaneous tools and work surfaces, and in the area shown in Figure IB3 of the original application.

All the contamination is presently fixed in the sense that it is not readily removable due to the fact that it is fixed, lodged in crevices and internal parts, held within a filtered or closed off exhaust system, or otherwise contained.

The supervisor of the Health Physics Section maintains control over this contamination.

Very truly yours,

Peter J. Knapp
Licensing and Accountability
Representative

PJK:ls

DOCKETED USAEC JUL 10 1969 REGULATORY MAIL SECTION DOCKET CLERK

RECEIVED JUL 10 1969 Mail Section

3

DOCKET NO. 70-58

Regulatory File Cy.

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NUCLEAR SYSTEMS DIVISION
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