

January 14, 2008

Mr. Michael A. Balduzzi
Senior Vice President and COO
Entergy Nuclear Operations, Inc.
Regional Operations, NE
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3, LICENSE
RENEWAL APPLICATION – SCOPING AND SCREENING METHODOLOGY

Dear Mr. Balduzzi:

By letter dated April 23, 2007, as supplemented by letters dated May 3, 2007, and June 21, 2007, Entergy Nuclear Operations, Inc., submitted an application pursuant to Title 10 of the *Code of Federal Regulations* Part 54, to renew the operating licenses for Indian Point Nuclear Generating Unit Nos. 2 and 3, for review by the U.S. Nuclear Regulatory Commission (NRC or the staff). The staff is reviewing the information contained in the license renewal application and has identified, in the enclosure, areas where additional information is needed to complete the review. Further requests for additional information may be issued in the future.

Items in the enclosure were discussed with Mr. Robert Walpole, and a mutually agreeable date for the response is within 30 days from the date of this letter. If you have any questions, please contact me at 301-415-1627, or via e-mail KJG1@nrc.gov.

Sincerely,

/RA/

Kimberly Green, Safety Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket Nos. 50-247 and 50-286

Enclosure:
As stated

cc w/encl: See next page

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OFFICE	LA/DLR	PM/DLR/RPB2	BC/DLR/RPB2
NAME	SFigueroa	KGreen	RFranovich
DATE	01/14/08	01/14/08	01/14/08

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**INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3
LICENSE RENEWAL APPLICATION (LRA)
REQUEST FOR ADDITIONAL INFORMATION (RAI)
SCOPING AND SCREENING METHODOLOGY**

RAI 2.1-1 10 CFR 54.4(a)(2) Scoping Criteria for Nonsafety-related SSCs

10 CFR 54.4 (a)(2) requires, in part, that all nonsafety-related systems, structures and components (SSCs) whose failure could prevent the satisfactory accomplishment of any of the functions identified in 10 CFR 54.4(a)(1) are to be included within the scope of license renewal.

- (a) The staff noted during the audit that the applicant had included fluid-filled, nonsafety-related pipes located within a safety-related space within the scope of license renewal based on the spaces' approach and had also included portions of nonsafety-related piping attached to safety-related SSCs within the scope of license renewal. However, the applicant did not provide sufficient information in the license renewal application nor the implementing procedures to demonstrate that when the fluid-filled pipe was also attached to a safety-related SSC, an additional portion of the pipe, beyond the safety-related space, up to and including an appropriate seismic anchor, equivalent anchor or bounding condition, was also included within the scope of license renewal.

The staff requests that the applicant provide a description of the process used to ensure that fluid-filled, nonsafety-related pipe, attached to safety-related SSCs which exit the safety-related space, was included within the scope of license renewal up to and including an appropriate seismic anchor, equivalent anchor or bounding condition.

- (b) During the NRC audit, the staff reviewed the applicant's technical evaluation and on-site documentation for nonsafety-related affecting safety-related SSCs which indicated that certain nonsafety-related SSCs potentially affecting safety-related SSCs were not included within the scope of license renewal based on the proximity of the nonsafety-related SSCs to the safety-related SSCs.

The staff requests that the applicant provide the rationale and basis for not including nonsafety-related SSCs in the vicinity of safety-related SSCs within the scope of license renewal based on their proximity to safety-related SSCs. Indicate the extent of condition by providing a description of the nonsafety-related SSCs which were not included within the scope of license renewal based on their proximity to safety-related SSCs and a description of the corresponding safety-related SSCs.

- (c) During the NRC audit, the staff reviewed the applicant's technical evaluation and on-site documentation for nonsafety-related affecting safety-related SSCs which indicated that certain similar SSCs were included within the scope of license renewal in accordance with the requirements of 10 CFR 54.4(a)(1) for one unit and 10 CFR 54.4(a)(2) for the other unit.

The staff requests that the applicant provide the rationale and basis for including similar SSCs within the scope of license renewal in accordance with the requirements of 10 CFR 54.4(a)(1) for one unit and 10 CFR 54.4(a)(2) for the other unit and how the corresponding review of the adjacent or attached nonsafety-related SSCs (for inclusion within the scope of

Enclosure

license renewal) was performed for similar systems in the two units. Indicate the extent of condition by providing a description of the SSCs which were included within the scope of license renewal in accordance with 10 CFR 54.4(a)(1) for one unit and 10 CFR 54.4(a)(2) for the other unit and corresponding descriptions of any differences in the review of adjacent or attached nonsafety-related SSCs.

In addressing these issues, indicate if your review concludes that use of the scoping methodology precluded the identification of nonsafety-related SSCs that could interact with safety-related SSCs, and which were not specifically exempted by your current licensing basis, and, therefore, should have been considered within the scope of license renewal. Describe any additional scoping evaluations to be performed to address the 10 CFR 54.4(a)(2) criteria.

As part of your response, list any additional SSCs included within the scope as a result of your efforts related to this RAI; and list those structures and components for which aging management reviews were conducted. For each structure and component, describe the aging management programs, as applicable, to be credited for managing the identified aging effects.

Letter to Michael Balduzzi from Kimberly Green, dated January 14, 2008

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RENEWAL APPLICATION – SCOPING AND SCREENING METHODOLOGY

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SBurnell, OPA

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MMcLaughlin, RI

NMcNamara, RI

DScrenci, RI OPA

NSheehan, RI OPA

BWelling, RI

BRogers

Indian Point Nuclear Generating
Units 2 and 3

cc:

Mr. Michael R. Kansler
President & CEO/CNO
Entergy Nuclear Operations, Inc.
1340 Echelon Parkway
Jackson, MS 39213

Mr. John T. Herron
Senior Vice President
Entergy Nuclear Operations, Inc.
1340 Echelon Parkway
Jackson, MS 39213

Sr. Vice President
Engineering & Technical Services
Entergy Nuclear Operations, Inc.
1340 Echelon Parkway
Jackson, MS 39213

Mr. Fred R. Dacimo
Site Vice President
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

Mr. Anthony Vitale - Acting
General Manager, Plant Operations
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway
P.O. Box 249
Buchanan, NY 10511-0249

Mr. Oscar Limpias
Vice President Engineering
Entergy Nuclear Operations, Inc.
1340 Echelon Parkway
Jackson, MS 39213

Mr. Joseph P. DeRoy
Vice President, Operations Support
Entergy Nuclear Operations, Inc.
1340 Echelon Parkway
Jackson, MS 39213

Mr. John A. Ventosa
GM, Engineering
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

Mr. John F. McCann
Director, Nuclear Safety & Licensing
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

Ms. Charlene D. Faison
Manager, Licensing
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

Mr. Ernest J. Harkness
Director, Oversight
Entergy Nuclear Operations, Inc.
1340 Echelon Parkway
Jackson, MS 39213

Mr. Patric W. Conroy
Director, Nuclear Safety Assurance
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

Mr. T.R. Jones - Acting
Manager, Licensing
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

Mr. William C. Dennis
Assistant General Counsel
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

Indian Point Nuclear Generating
Units 2 and 3

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cc:

Mr. Paul D. Tonko
President and CEO
New York State Energy Research and
Development Authority
17 Columbia Circle
Albany, NY 12203-6399

Mr. John P. Spath
New York State Energy, Research and
Development Authority
17 Columbia Circle
Albany, NY 12203-6399

Mr. Paul Eddy
New York State Department
of Public Service
3 Empire State Plaza
Albany, NY 12223-1350

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector's Office
Indian Point 2
U.S. Nuclear Regulatory Commission
P.O. Box 59
Buchanan, NY 10511

Senior Resident Inspector's Office
Indian Point 3
U.S. Nuclear Regulatory Commission
P.O. Box 59
Buchanan, NY 10511

Mr. Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, NY 10271

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, NY 10511

Mr. Raymond L. Albanese
Four County Coordinator
200 Bradhurst Avenue
Unit 4 Westchester County
Hawthorne, NY 10532

Mr. William DiProfio
PWR SRC Consultant
48 Bear Hill Road
Newton, NH 03858

Mr. Garry Randolph
PWR SRC Consultant
1750 Ben Franklin Drive, 7E
Sarasota, FL 34236

Mr. William T. Russell
PWR SRC Consultant
400 Plantation Lane
Stevensville, MD 21666-3232

Mr. Jim Riccio
Greenpeace
702 H Street, NW
Suite 300
Washington, DC 20001

Mr. Phillip Musegaas
Riverkeeper, Inc.
828 South Broadway
Tarrytown, NY 10591

Mr. Mark Jacobs
IPSEC
46 Highland Drive
Garrison, NY 10524

Indian Point Nuclear Generating
Units 2 and 3

- 3 -

cc:

Mr. R. M. Waters
Technical Specialist Licensing
450 Broadway
P.O. Box 0249
Buchanan, NY 10511-0249

The Honorable Nita Lowey
222 Mamaroneck Avenue, Suite 310
White Plains, NY 10605

Mr. Sherwood Martinelli
351 Dyckman
Peekskill, NY 10566

Ms. Susan Shapiro, Esq.
21 Perlman Drive
Spring Valley, NY 10977

John Sipos
Assistant Attorney General
New York State Department of Law
Environmental Protection Bureau
The Capitol
Albany, NY 12224

Robert Snook
Assistant Attorney General
Office of the Attorney General
State of Connecticut
55 Elm Street
P.O. Box 120
Hartford, CT 06141-0120

Ms. Kathryn M. Sutton, Esq.
Morgan, Lewis & Bockius, LLP
1111 Pennsylvania Avenue, NW
Washington, DC 20004

Mr. Paul M. Bessette, Esq.
Morgan, Lewis & Bockius, LLP
1111 Pennsylvania Avenue, NW
Washington, DC 20004

Mr. Martin J. O'Neill, Esq.
Morgan, Lewis & Bockius, LLP
1111 Pennsylvania Avenue, NW
Washington, DC 20004