SONGS UNIT 2 SINGLE-SIDED

Operating License & Technical Specification/Licensee Controlled Specification

Unit 2 Amendments 214/215 and Unit 2 Bases Change B07-003

Revise Unit 2 Operating License by removing pages and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and/or dates and contain marginal/lines indicating the areas of change. **Please direct any questions regarding filing of information to: (949) 368-6972 or email CDM Controlled Manuals Desk.**

REMOVE:	INSERT:
Volume 1	**Volume 1**
BEHIND: List of Effective Pages – "Blue Tab"	**BEHIND**: List of Effective Pages – "Blue Tab"
List of Effective Pages – Pages 1-12 11/29/07	List of Effective Pages – Pages 1-12 12/12/07
BEHIND: "Blue Tab" Facility Operating License	**BEHIND:** "Blue Tab" Facility Operating License
Songs Facility Operating License List of Effective Pages – Page 1-2	Songs Facility Operating License List of Effective Pages – Page 1-2
Docket No. 50-361 letter – page 3 Amd. 212	Docket No. 50-361 letter – page 3 Amd. 215
Docket No. 50-361 letter – page 7 Amd. 185	Docket No. 50-361 letter – page 7 Amd. 185
Docket No. 50-361 letter – page 8 Amd. 205	Docket No. 50-361 letter – page 8 Amd. 214 Revised by letter dated July 26, 2007
Docket No. 50-361 letter – page 9 Amd. 185	Docket No. 50-361 letter – page 9 Amd. 214 Revised by letter dated March 6, 2007
BEHIND: "Blue Tab" 3.7 Plant Systems	**BEHIND:** "Blue Tab" 3.7 Plant Systems
Page 3.7-24 Amd. 181	Page 3.7-24 Amd. 214
Page 3.7-25 Amd. 177	Page 3.7-25 Amd. 214
Page 3.7-26 Amd. 177	Page 3.7-26 Amd. 214
BEHIND: "Blue Tab" 5.0 Administrative Controls	BEHIND: **"Blue Tab" 5.0 Administrative Controls**
Page 5.0-15a Amd. 206	Page 5.0-15a Amd. 215
Page 5.0-16 Amd. 204	Page 5.0-16 Amd. 215
Page 5.0-20b Amd. 144	Page 5.0-20b Amd. 214
Insert New Page	Page 5.0-20c Amd. 214
Volume 3	**Volume 3**
BEHIND: "Blue Tab" B-3.7	**BEHIND:** "Blue Tab" B-3.7
Page B 3.7.56 Amd.127 06/28/99	Page B 3.7-56 Amd.127 12/12/07
Page B 3.7-57 Amd.127 04/25/05	Page B 3.7-57 Amd.127 12/12/07
Page B 3.7-58 Amd. 127 01/13/99	Page B 3.7-58 Amd. 127 12/12/07
Page B 3.7-59 Amd. 128 02/05/01	Page B 2.7-59 Amd. 128 12/12/07
Page B 3.7-60 Amd. 127 10/17/01 Re-issued 08/06/03	Page B 3.7-60 Amd. 127 12/12/07
Page B 3.7-61 Amd. 128 09/16/03	Page B 3.7-61 Amd. 128 12/12/07
Page B 3.7-62 Amd. 128 02/05/01	Page B 3.7-62 Amd. 128 12/12/07
Page B 3.7-62a Amd. 128 02/05/01	No Replacement
Page B 3.7-63 Amd. 208 12/05/06	Page B 3.7-63 Amd. 208 12/12/07
Page B 3.7-64 Amd. 208 12/05/06	Page B 3.7-64 Amd. 208 12/12/07
Page B 3.7-65 Amd. 208 12/05/06	Page B 3.7-65 Amd. 208 12/12/07
Page B 3.7-66 Amd. 208 12/05/06	Page B 3.7-66 Amd. 208 12/12/07
Page B 3.7-67 Amd. 208 12/05/06	Page B 3.7-67 Amd. 208 12/12/07

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SONGS UNIT 2 SINGLE-SIDED

Operating License & Technical Specification/Licensee Controlled Specification

Unit 2 Amendments 214/215 and Unit 2 Bases Change B07-003

Revise Unit 2 Operating License by removing pages and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and/or dates and contain marginal/lines indicating the areas of change. **Please direct any questions regarding filing of information to: (949) 368-6972 or email CDM Controlled Manuals Desk.**

Filing Instructions Reviewed & Approved:

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APPENDIX A TO LICENSE NO. NPF-10

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
LOEP-1	1	215 12/12/07	2.0-1	1	207
LOEP-2	1	208 01/25/07	2.0-2	1	207
LOEP-3	1	214 10/31/07	3.0-1	1	127
LOEP-4	1	215 11/29/07	3.0-2	1	127
LOEP-4a	1	168	3.0-3	1	127
LOEP-5	1	207 12/21/06	3.0-4	1	186
LOEP-6	1	200 11/18/05	3.0-5	1	127
LOEP-7	1	197 07/19/05	3.1-1	1	200
LOEP-8	1	208 12/05/06	3.1-2	1	200
LOEP-9	1	204 09/19/06	3.1-3	1	200
LOEP-10	1	212 12/12/07	3.1-4	1	127
LOEP-11	1	211 12/12/07	3.1-5	1	127
LOEP-12	1	193 03/07/07	3.1-6	1	127
ii	1	127	3.1-7	1	200
iii	1	208	3.1-8	1	200
iv	1	208	3.1-9	1	127
v	1	197	3.1-10	1	127
vi	1	127	3.1-11	1	127
vii	1	208	3.1-12	1	200
viii	1	208	3.1-13	1	127
1.1-1	1	127	3.1-14	1	200
1.1-2	1	127	3.1-15	1	200
1.1-3	1	188	3.1-16	1	200
1.1-4	1	204	3.1-17	1	127
1.1-5	1	203	3.1-18	1	127
1.1-6	1	200	3.1-19	1	127
1.1-7	1	127	3.1-20	1	127
1.2-1	1	127	3.1-21	1	163
1.2-2	1	127	3.1-22	1	175
1.2-3	1	127	3.1-23	1	127
1.3-1	1	127	3.1-24	1	127
1.3-2	1	127	3.1-25	1	127
1.3-3	1	127	3.1-26	1	127
1.3-4	1	127	3.1-27	1	127
1.3-5	1	127	3.1-28	1	127
1.3-6	1	127	3.1-29	1	127
1.3-7	1	127	3.2-1	1	127
1.3-8	1	127	3.2-2	1	127
1.3-9	1	127	3.2-3	1	127
1.3-10	1	127	3.2-4	1	127
1.3-11	1	127	3.2-5	1	127
1.3-12	1	127	3.2-6	1	127
1.3-13	1	127	3.2-7	1	127
1.4-1	1	127	3.2-8	1	127
1.4-2	1	127	3.2-9	1	200
1.4-3	1	127	3.2-10	1	127
1.4-4	1	127	3.2-11	1	127
			3.2-12	1	127

APPENDIX A TO LICENSE NO. NPF-10

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date		Page No.	Volume	Amendment/Date
3.3-1	1	127		3.3-50	1	127
3.3-2	1	127		3.3-51	1	175
3.3-3	1	127		3.3-52	1	127
3.3-4	1	127		3.4-1	1	149
3.3-5	1	127		3.4-2	1	149
3.3-6	1	147		3.4-3	1	149
3.3-7	1	133		3.4-4	1	127
3.3-8	1	150		3.4-5	1	203
3.3-9	1	150		3.4-6	1	203
3.3-10	1	127		3.4-7	1	203
3.3-11	1	127		3.4-8	1	203
3.3-12	1	133		3.4-9	1	203
3.3-13	1	147		3.4-10	1	203
3.3-14	1	127		3.4-11	1	203
3.3-15	1	127		3.4-12	1	203
3.3-16	1	133		3.4-13	1	127
3.3-17	1	127		3.4-14	1	197
3.3-18	1	127		3.4-15	1	127
3.3-19	1	127		3.4-16	1	175
3.3-20	1	133		3.4-17	1	175
3.3-21	1	127		3.4-18	1	203
3.3-22	1	157		3.4-19	1	175
3.3-23	1	157		3.4-20	1	127
3.3-24	1	157		3.4-21	1	175
3.3-25	1	157		3.4-22	1	203
3.3-26	1	127		3.4-23	1	175
3.3-27	1	127		3.4-24	1	175
3.3-28	1	127		3.4-25	1	175
3.3-29	1	133		3.4-26	1	161
3.3-30	1	127		3.4-27	1	155
3.3-31	1	127		3.4-28	1	156
3.3-32	1	127		3.4-29	1	156
3.3-33	1	127		3.4-30	1	203
3.3-34	1	196		3.4-31	1	203
3.3-35	1	132		3.4-32	1	127
3.3-36	1	132		3.4-33	1	203
3.3-37	1	132		3.4-34	1	143
3.3-38	1	132		3.4-35	1	203
3.3-39	1	132		3.4-36	1	127
3.3-40	1	175		3.4-37	1	204
3.3-41	1	160		3.4-38	1	204
3.3-42	1	208	01/25/07	3.4-39	1	127
3.3-43	1	208	01/25/07	3.4-40	1	127
3.3-44	1	127		3.4-41	1	127
3.3-45	1	194		3.4-42	1	127
3.3-46	1	194		3.4-43	1	127
3.3-47	1	194		3.4-44	1	127
3.3-48	1	127		3.4-45	1	127
3.3-49	1	127				

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
3.4-46	1	127	3.7-6	1	127
3.4-47	1	127	3.7-7	1	127
3.4-48	1	127	3.7-8	1	127
3.4-49	1	127	3.7-9	1	127
3.4-50	1	127	3.7-10	1	127
3.4-51	1	204	3.7-11	1	164
3.4-52	1	204	3.7-12	1	127
3.5-1	1	139	3.7-13	1	127
3.5-2	1	135	3.7-14	1	191
3.5-3	1	127	3.7-15	1	147
3.5-4	1	139	3.7-16	1	162
3.5-5	1	127	3.7-17	1	127
3.5-6	1	190	3.7-18	1	127
3.5-7	1	127	3.7-19	1	127
3.5-8	1	127	3.7-19a	1	129
3.5-9	1	127	3.7-19b	1	129
3.5-10	1	127	3.7-19c	1	129
3.5-11	1	127	3.7-20	1	127
3.6-1	1	127	3.7-21	1	127
3.6-2	1	144	3.7-22	1	181
3.6-3	1	127	3.7-23	1	127
3.6-4	1	127	3.7-24	1	214
3.6-5	1	127	3.7-25	1	214
3.6-6	1	144	3.7-26	1	214
3.6-7	1	127	3.7-27	1	208 01/25/07
3.6-8	1	127	3.7-28	1	208 01/25/07
3.6-9	1	127	3.7-29	1	127
3.6-10	1	127	3.7-30	1	127
3.6-11	1	127	3.7-31	1	127
3.6-12	1	165	3.7-32	1	131
3.6-13	1	201	3.7-33	1	131
3.6-14	1	201	3.7-34	1	131
3.6-15	1	127	3.7-35	1	127
3.6-16	1	127	3.8-1	1	141
3.6-17	1	127	3.8-2	1	141
3.6-18	1	171	3.8-3	1	127
3.6-19	1	127	3.8-4	1	127
3.6-20	1	127	3.8-5	1	179
3.6-21	1	127	3.8-6	1	211
3.6-22	1	127	3.8-7	1	158
3.6-23	1	159	3.8-8	1	179
3.6-24	1	159	3.8-9	1	158
3.6-25	1	127	3.8-10	1	158
3.6-26	1	127	3.8-11	1	158
3.7-1	1	212	3.8-12	1	158
3.7-2	1	127	3.8-13	1	169
3.7-3	1	212	3.8-14	1	179
3.7-4	1	212	3.8-15	1	158
3.7-5	1	127	3.8-16	1	127

APPENDIX A TO LICENSE NO. NPF-10

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
3.8-17	1	127	5.0-8	1	207
3.8-18	1	175	5.0-9	1	178
3.8-19	1	127	5.0-10	1	127
3.8-20	1	211	5.0-11	1	170
3.8-21	1	211	5.0-12	1	178
3.8-22	1	127	5.0-13	1	204
3.8-23	1	127	5.0-14	1	204
3.8-24	1	127	5.0-15	1	206
3.8-25	1	127	5.0-15a	1	215
3.8-26	1	127	5.0-16	1	215
3.8-27	1	175	5.0-17	1	208
3.8-28	1	127	5.0-18	1	204
3.8-29	1	127	5.0-19	1	204
3.8-30	1	127	5.0-20	1	204
3.8-31	1	127	5.0-20a	1	198
3.8-32	1	127	5.0-20b	1	214
3.8-33	1	127	5.0-20c	1	214
3.8-34	1	127	5.0-21	1	127
3.8-35	1	127	5.0-22	1	127
3.8-36	1	175	5.0-23	1	195
3.8-37	1	127	5.0-24	1	127
3.8-38	1	127	5.0-25	1	127
3.8-39	1	127	5.0-26	1	200
3.8-40	1	175	5.0-27	1	203
3.8-41	1	127	5.0-27a	1	203
3.9-1	1	127	5.0-28	1	204
3.9-2	1	175	5.0-29	1	204
3.9-3	1	127	5.0-30	1	197
3.9-4	1	193	5.0-31	1	207
3.9-5	1	127			
3.9-6	1	175			
3.9-6a	1	175			
3.9-7	1	153			
3.9-8	1	153			
3.9-9	1	175			
3.9-9a	1	153			
3.9-10	1	134			
4.0-1	1	199			
4.0-2	1	127			
4.0-3	1	127			
4.0-4	1	131			
4.0-5	1	131			
5.0-1	1	207			
5.0-2	1	207			
5.0-3	1	207			
5.0-4	1	207			
5.0-5	1	207			
5.0-6	1	184			
5.0-7	1	127			

APPENDIX A TO LICENSE NO. NPF-10

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date
Bibliographic Data Sheet	1	dated February 17, 1982
Appendix B	1	dated February 17, 1982
Unit 2 Environmental Protection Plan	1	dated February 17, 1982
1-1	1	dated February 17, 1982
2-1	1	dated February 17, 1982
2-2	1	dated February 17, 1982
3-1	1	dated February 17, 1982
3-2	1	168
4-1	1	168
4-2	1	dated February 17, 1982
Appendix C Anti Trust Conditions	1	
Appendix C	1	dated February 17, 1982
2	1	dated February 17, 1982
3	1	dated February 17, 1982

APPENDIX A TO LICENSE NO. NPF-10

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 2.0-1	2	192 07/19/04	B 3.1-27	2	127
B 2.0-2	2	127	B 3.1-28	2	200 11/18/05
B 2.0-3	2	207 12/21/06	B 3.1-29	2	200 11/18/05
B 2.0-3a	2	192 07/19/04	B 3.1-30	2	200 11/18/05
B 2.0-4	2	207 12/21/06	B 3.1-31	2	127
B 2.0-5	2	207 12/21/06	B 3.1-32	2	127 05/02/97
B 2.0-6	2	127	B 3.1-33	2	127
B 2.0-7	2	127	B 3.1-34	2	127 05/02/97
B 2.0-8	2	207 12/21/06	B 3.1-35	2	127
B 2.0-9	2	207 12/21/06	B 3.1-36	2	127
B 3.0-1	2	127	B 3.1-37	2	200 11/18/05
B 3.0-2	2	127	B 3.1-38	2	200 11/18/05
B 3.0-3	2	127	B 3.1-39	2	127
B 3.0-4	2	207 12/21/06	B 3.1-40	2	127
B 3.0-5	2	127	B 3.1-41	2	127
B 3.0-6	2	127	B 3.1-42	2	127
B 3.0-7	2	127	B 3.1-43	2	200 11/18/05
B 3.0-8	2	127	B 3.1-43a	2	200 11/18/05
B 3.0-9	2	127	B 3.1-44	2	200 11/18/05
B 3.0-10	2	127	B 3.1-45	2	127
B 3.0-11	2	127	B 3.1-46	2	200 11/18/05
B 3.0-12	2	127 12/01/98	B 3.1-47	2	127
B 3.0-13	2	186 10/25/02	B 3.1-48	2	127
B 3.0-14	2	186 05/08/02	B 3.1-49	2	127
B 3.0-15	2	186 05/08/02	B 3.1-50	2	127
B 3.1-1	2	200 11/18/05	B 3.1-51	2	127
B 3.1-2	2	200 11/18/05	B 3.1-52	2	127
B 3.1-3	2	200 11/18/05	B 3.1-53	2	127
B 3.1-4	2	200 11/18/05	B 3.1-54	2	127 08/03/01
B 3.1-5	2	200 11/18/05	B 3.1-55	2	127 08/03/01
B 3.1-6	2	200 11/18/05	B 3.1-56	2	127 08/03/01
B 3.1-7	2	200 11/18/05	B 3.1-57	2	127 08/03/01
B 3.1-8	2	200 11/18/05	B 3.1-58	2	127 08/03/01
B 3.1-9	2	200 11/18/05	B 3.1-58a	2	127 08/03/01
B 3.1-10	2	200 11/18/05	B 3.1-58b	2	127 08/03/01
B 3.1-11	2	200 11/18/05	B 3.1-59	2	200 11/18/05
B 3.1-12	2	200 11/18/05	B 3.1-60	2	127 11/05/96
B 3.1-13	2	127	B 3.1-61	2	127 05/03/00
B 3.1-14	2	127	B 3.1-62	2	175 12/20/00
B 3.1-15	2	200 11/18/05	B 3.1-63	2	175 05/21/02
B 3.1-16	2	127	B 3.1-63a	2	175 12/20/00
B 3.1-17	2	200 11/18/05	B 3.1-64	2	127
B 3.1-18	2	127	B 3.1-65	2	127
B 3.1-19	2	127 01/03/02	B 3.1-66	2	127
B 3.1-20	2	127	B 3.1-67	2	127
B 3.1-21	2	127 01/03/02	B 3.1-68	2	127
B 3.1-22	2	127 05/30/97	B 3.1-69	2	127
B 3.1-23	2	127	B 3.1-70	2	127
B 3.1-24	2	127	B 3.1-71	2	127
B 3.1-25	2	127 05/02/97	B 3.1-72	2	127
B 3.1-26	2	127 07/29/99			

APPENDIX A TO LICENSE NO. NPF-10

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.1-73	2	127	B 3.3-2	2	127 07/17/97
B 3.1-74	2	127	B 3.3-3	2	127
B 3.1-75	2	127	B 3.3-4	2	127
B 3.1-76	2	127	B 3.3-5	2	127
B 3.1-77	2	127 05/29/97	B 3.3-6	2	127
B 3.1-78	2	127 05/29/97	B 3.3-7	2	127
B 3.1-79	2	127	B 3.3-8	2	127
B 3.1-80	2	127	B 3.3-9	2	127
B 3.2-1	2	127	B 3.3-10	2	127
B 3.2-2	2	127	B 3.3-11	2	127
B 3.2-3	2	127	B 3.3-12	2	127 03/12/99
B 3.2-4	2	192 07/19/04	B 3.3-13	2	127
B 3.2-5	2	127	B 3.3-14	2	127
B 3.2-6	2	127	B 3.3-15	2	127
B 3.2-7	2	127	B 3.3-16	2	127 07/17/97
B 3.2-8	2	127	B 3.3-17	2	127 09/16/03
B 3.2-9	2	127	B 3.3-18	2	127 03/12/99
B 3.2-10	2	127	B 3.3-19	2	127
B 3.2-11	2	127	B 3.3-20	2	127
B 3.2-12	2	192 07/19/04	B 3.3-21	2	127 03/12/99
B 3.2-13	2	127	B 3.3-22	2	127 03/12/99
B 3.2-14	2	127	B 3.3-23	2	127 03/12/99
B 3.2-15	2	127	B 3.3-24	2	127
B 3.2-16	2	127	B 3.3-25	2	127 12/30/96
B 3.2-17	2	127	B 3.3-26	2	127 12/30/96
B 3.2-18	2	127	B 3.3-26a	2	127 12/30/96
B 3.2-19	2	192 07/19/04	B 3.3-27	2	127
B 3.2-20	2	127	B 3.3-28	2	127
B 3.2-21	2	127	B 3.3-29	2	127
B 3.2-22	2	127	B 3.3-30	2	127 05/29/97
B 3.2-23	2	127	B 3.3-31	2	127 05/02/97
B 3.2-24	2	127	B 3.3-32	2	127 03/12/99
B 3.2-25	2	127	B 3.3-33	2	127 03/12/99
B 3.2-26	2	127	B 3.3-34	2	127 03/12/99
B 3.2-27	2	127	B 3.3-35	2	127 03/12/99
B 3.2-28	2	127	B 3.3-36	2	127 03/12/99
B 3.2-29	2	192 07/19/04	B 3.3-37	2	188
B 3.2-30	2	200 11/18/05	B 3.3-37a	2	188 09/16/03
B 3.2-31	2	127	B 3.3-38	2	127
B 3.2-32	2	127	B 3.3-39	2	127 07/17/97
B 3.2-33	2	127	B 3.3-40	2	127 09/25/98
B 3.2-34	2	127	B 3.3-41	2	127 09/25/98
B 3.2-35	2	127	B 3.3-42	2	127
B 3.2-36	2	127	B 3.3-43	2	127
B 3.2-37	2	127	B 3.3-44	2	127 12/30/96
B 3.2-38	2	192 07/19/04	B 3.3-45	2	127 12/30/96
B 3.2-39	2	127	B 3.3-46	2	127 12/30/96
B 3.2-40	2	127	B 3.3-47	2	127 12/30/96
B 3.2-41	2	127	B 3.3-48	2	127 09/18/98
B 3.3-1	2	127	B 3.3-49	2	127 09/18/98

APPENDIX A TO LICENSE NO. NPF-10

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.3-50	2	127 09/18/98	B 3.3-100	2	197 07/19/05
B 3.3-51	2	127 02/11/02	B 3.3-101	2	127 10/06/99
B 3.3-52	2	127	B 3.3-102	2	127 10/06/99
B 3.3-53	2	127 07/17/97	B 3.3-103	2	127 10/06/99
B 3.3-54	2	127	B 3.3-103a	2	188
B 3.3-55	2	127	B 3.3-103b	2	157 09/16/03
B 3.3-56	2	127	B 3.3-103c	2	188
B 3.3-57	2	127	B 3.3-104	2	127
B 3.3-58	2	127	B 3.3-105	2	127
B 3.3-59	2	127	B 3.3-106	2	127
B 3.3-60	2	127 09/18/98	B 3.3-107	2	127
B 3.3-61	2	127 09/18/98	B 3.3-108	2	127
B 3.3-62	2	127 09/18/98	B 3.3-109	2	127
B 3.3-63	2	127	B 3.3-110	2	127
B 3.3-64	2	127 07/17/97	B 3.3-111	2	127
B 3.3-65	2	127	B 3.3-112	2	127
B 3.3-66	2	127	B 3.3-113	2	127
B 3.3-67	2	127	B 3.3-114	2	127
B 3.3-68	2	127	B 3.3-115	2	127
B 3.3-69	2	127	B 3.3-116	2	127
B 3.3-70	2	127	B 3.3-117	2	127
B 3.3-71	2	127	B 3.3-118	2	127
B 3.3-72	2	127 12/30/96	B 3.3-119	2	127 12/30/96
B 3.3-73	2	127 12/30/96	B 3.3-120	2	127 12/30/96
B 3.3-74	2	127 12/30/96	B 3.3-121	2	127 12/30/96
B 3.3-75	2	127 05/28/97	B 3.3-122	2	127 12/30/96
B 3.3-76	2	127 12/30/96	B 3.3-123	2	127 05/28/97
B 3.3-77	2	127 07/17/97	B 3.3-124	2	127 05/28/97
B 3.3-78	2	127	B 3.3-125	2	127 05/28/97
B 3.3-79	2	127	B 3.3-126	2	127
B 3.3-80	2	127	B 3.3-127	2	127
B 3.3-81	2	127	B 3.3-128	2	127
B 3.3-82	2	127	B 3.3-129	2	127 09/06/00
B 3.3-83	2	127	B 3.3-130	2	127
B 3.3-84	2	127	B 3.3-131	2	127 09/06/00
B 3.3-85	2	127	B 3.3-132	2	127 05/29/97
B 3.3-86	2	127	B 3.3-133	2	127 07/13/99
B 3.3-87	2	127 02/24/99	B 3.3-134	2	174 12/14/00
B 3.3-88	2	127 07/17/97	B 3.3-135	2	127 05/28/97
B 3.3-89	2	127	B 3.3-136	2	127 06/09/00
B 3.3-90	2	127	B 3.3-137	2	127 05/28/97
B 3.3-91	2	127	B 3.3-138	2	127 05/28/97
B 3.3-92	2	127	B 3.3-139	2	127 05/28/97
B 3.3-93	2	127	B 3.3-140	2	127 05/28/97
B 3.3-94	2	127	B 3.3-141	2	127 05/28/97
B 3.3-95	2	127 10/06/99	B 3.3-142	2	127 05/28/97
B 3.3-96	2	127 10/06/99	B 3.3-143	2	127 05/28/97
B 3.3-97	2	127 10/06/99	B 3.3-144	2	127 05/28/97
B 3.3-98	2	127 07/18/01	B 3.3-145	2	127 06/27/97
B 3.3-99	2	127 10/06/99	B 3.3-146	2	127 03/19/98

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.3-147	2	127 06/27/97	B 3.4-12	3	127
B 3.3-148	2	175 12/20/00	B 3.4-13	3	127
B 3.3-149	2	175 12/20/00	B 3.4-14	3	127
B 3.3-150	2	127 08/20/01	B 3.4-15	3	203
B 3.3-151	2	127 06/27/97	B 3.4-16	3	203
B 3.3-152	2	208 12/05/06	B 3.4-17	3	197 07/19/05
B 3.3-153	2	208 12/05/06	B 3.4-18	3	197 07/19/05
B 3.3-154	2	208 12/05/06	B 3.4-19	3	127
B 3.3-155	2	208 12/05/06	B 3.4-20	3	197 07/19/05
B 3.3-156	2	208 12/05/06	B 3.4-21	3	197 07/19/05
B 3.3-157	2	208 12/05/06	B 3.4-22	3	127
B 3.3-158	2	127 12/17/98	B 3.4-23	3	127
B 3.3-159	2	127 06/08/00	B 3.4-24	3	204
B 3.3-160	2	127 06/23/99	B 3.4-25	3	127
B 3.3-161	2	127 05/30/97	B 3.4-26	3	127
B 3.3-162	2	127 05/30/97	B 3.4-27	3	127
B 3.3-163	2	130 05/30/97	B 3.4-28	3	204
B 3.3-164	2	127 05/30/97	B 3.4-29	3	175 12/20/00
B 3.3-165	2	127 05/30/97	B 3.4-30	3	175 12/20/00
B 3.3-166	2	194 04/25/05	B 3.4-31	3	127
B 3.3-167	2	127 05/30/97	B 3.4-32	3	204
B 3.3-168	2	127 12/02/99	B 3.4-33	3	127
B 3.3-169	2	127 05/30/97	B 3.4-34	3	175 12/20/00
B 3.3-170	2	127	B 3.4-35	3	175 12/20/00
B 3.3-171	2	194 04/25/05	B 3.4-36	3	127
B 3.3-172	2	194 04/25/05	B 3.4-37	3	175 12/20/00
B 3.3-173	2	127 05/30/97	B 3.4-38	3	175 07/15/05
B 3.3-174	2	127 05/30/97	B 3.4-38a	3	175 04/03/06
B 3.3-175	2	127 03/06/06	B 3.4-39	3	204
B 3.3-176	2	127 10/30/98	B 3.4-40	3	175 12/20/00
B 3.3-177	2	127 10/30/98	B 3.4-41	3	127
B 3.3-178	2	127	B 3.4-42	3	127
B 3.3-179	2	127 04/24/01	B 3.4-43	3	175 12/20/00
B 3.3-180	2	127 04/24/01	B 3.4-44	3	175 04/03/06
B 3.3-181	2	127 07/29/96	B 3.4-45	3	175 12/20/00
B 3.3-182	2	175 12/20/00	B 3.4-46	3	127 05/30/00
B 3.3-183	2	175 12/20/00	B 3.4-47	3	155 05/30/00
B 3.3-184	2	175 12/20/00	B 3.4-48	3	161 05/15/02
B 3.3-185	2	175 12/20/00	B 3.4-49	3	127 05/30/00
B 3.4-1	3	127 02/24/99	B 3.4-50	3	127 05/30/00
B 3.4-2	3	127 02/24/99	B 3.4-51	3	156 03/22/01
B 3.4-3	3	127	B 3.4-52	3	156 03/22/01
B 3.4-4	3	127	B 3.4-53	3	156 03/22/01
B 3.4-5	3	127 02/24/99	B 3.4-54	3	156 04/27/05
B 3.4-6	3	127	B 3.4-55	3	203
B 3.4-7	3	127	B 3.4-56	3	203
B 3.4-8	3	127	B 3.4-57	3	203
B 3.4-9	3	203	B 3.4-58	3	203
B 3.4-10	3	127	B 3.4-59	3	203
B 3.4-11	3	127	B 3.4-60	3	203

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.4-61	3	203	B 3.5-11	3	127 03/03/00
B 3.4-62	3	203	B 3.5-12	3	127 03/03/00
B 3.4-63	3	203	B 3.5-13	3	127 03/03/00
B 3.4-64	3	203	B 3.5-14	3	127 03/03/00
B 3.4-65	3	203	B 3.5-15	3	127 03/03/00
B 3.4-66	3	203	B 3.5-16	3	197 07/19/05
B 3.4-67	3	203	B 3.5-17	3	127 03/03/00
B 3.4-68	3	203	B 3.5-18	3	127 03/14/02
B 3.4-69	3	203	B 3.5-19	3	127 03/14/02
B 3.4-70	3	127	B 3.5-20	3	127 07/17/01
B 3.4-71	3	204	B 3.5-20a	3	127 03/03/00
B 3.4-72	3	204	B 3.5-21	3	127
B 3.4-73	3	204	B 3.5-22	3	127
B 3.4-74	3	204	B 3.5-23	3	127
B 3.4-75	3	204	B 3.5-24	3	127
B 3.4-76	3	127	B 3.5-25	3	127
B 3.4-77	3	127	B 3.5-26	3	127
B 3.4-78	3	127	B 3.5-27	3	127
B 3.4-79	3	127	B 3.5-28	3	127
B 3.4-80	3	127 09/06/00	B 3.5-29	3	127
B 3.4-81	3	127 09/06/00	B 3.5-30	3	127
B 3.4-82	3	127	B 3.5-31	3	127
B 3.4-83	3	127	B 3.5-32	3	127
B 3.4-84	3	127	B 3.5-33	3	127
B 3.4-85	3	127	B 3.5-34	3	127
B 3.4-86	3	127	B 3.6-1	3	127 12/01/98
B 3.4-87	3	127	B 3.6-2	3	182 04/29/03
B 3.4-88	3	127 05/13/02	B 3.6-3	3	127 12/01/98
B 3.4-89	3	127 05/13/02	B 3.6-4	3	127 04/29/03
B 3.4-90	3	127	B 3.6-5	3	127 12/01/98
B 3.4-91	3	127	B 3.6-6	3	182 02/11/02
B 3.4-92	3	127	B 3.6-7	3	127
B 3.4-93	3	204	B 3.6-8	3	127
B 3.4-94	3	204	B 3.6-9	3	127
B 3.4-95	3	204	B 3.6-10	3	127
B 3.4-96	3	204	B 3.6-11	3	127 12/01/98
B 3.4-97	3	204	B 3.6-12	3	127 04/29/03
B 3.4-98	3	204	B 3.6-13	3	127 07/29/99
B 3.4-99	3	204	B 3.6-14	3	127
B 3.4-100	3	204	B 3.6-15	3	127
B 3.5-1	3	127	B 3.6-16	3	127 04/29/03
B 3.5-2	3	127	B 3.6-17	3	127
B 3.5-3	3	127	B 3.6-18	3	127 06/09/00
B 3.5-4	3	127 05/05/03	B 3.6-19	3	127
B 3.5-5	3	127 05/30/97	B 3.6-20	3	127
B 3.5-6	3	127	B 3.6-21	3	127 04/29/03
B 3.5-7	3	127 07/15/98	B 3.6-22	3	127 10/24/01
B 3.5-8	3	197 07/19/05	B 3.6-23	3	201 12/12/05
B 3.5-9	3	127 07/15/98	B 3.6-24	3	201 12/12/05
B 3.5-10	3	127 07/15/98	B 3.6-24a	3	201 12/12/05
B 3.5-10a	3	197 07/19/05	B 3.6-25	3	127 04/29/03

APPENDIX A TO LICENSE NO. NPF-10

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date		Page No.	Volume	Amendment/Date	
B 3.6-26	3	127	04/29/03	B 3.7-20	3	127	06/02/04
B 3.6-27	3	182	07/11/02	B 3.7-21	3	127	06/02/04
B 3.6-28	3	182	02/11/02	B 3.7-22	3	127	06/02/04
B 3.6-29	3	127		B 3.7-23	3	127	07/29/99
B 3.6-30	3	127	04/29/03	B 3.7-24	3	127	12/14/00
B 3.6-31	3	182	05/02/02	B 3.7-25	3	127	
B 3.6-32	3	127		B 3.7-26	3	127	02/28/01
B 3.6-33	3	127		B 3.7-27	3	127	02/28/01
B 3.6-34	3	127		B 3.7-28	3	127	03/03/00
B 3.6-35	3	182	02/11/02	B 3.7-29	3	127	03/03/00
B 3.6-36	3	182	02/11/02	B 3.7-30	3	127	03/03/00
B 3.6-37	3	197	07/19/05	B 3.7-31	3	191	11/25/03
B 3.6-38	3	197	07/19/05	B 3.7-32	3	191	11/25/03
B 3.6-39	3	127	10/06/00	B 3.7-33	3	191	11/25/03
B 3.6-40	3	127	10/06/00	B 3.7-34	3	127	03/03/00
B 3.6-41	3	127	10/06/00	B 3.7-35	3	127	07/31/00
B 3.6-42	3	127	04/29/03	B 3.7-36	3	127	07/31/00
B 3.6-43	3	127		B 3.7-37	3	127	08/13/01
B 3.6-44	3	127		B 3.7-38	3	127	08/01/01
B 3.6-45	3	127		B 3.7-39	3	127	06/23/99
B 3.6-46	3	127		B 3.7-40	3	127	
B 3.6-47	3	127		B 3.7-41	3	127	04/03/06
B 3.6-48	3	159	10/29/99	B 3.7-42	3	127	
B 3.6-49	3	159	10/29/99	B 3.7-43	3	127	
B 3.6-50	3	159	10/29/99	B 3.7-43a	3	127	05/29/97
B 3.6-51	3	159	10/29/99	B 3.7-43b	3	127	
B 3.6-52	3	159	10/29/99	B 3.7-43c	3	129	
B 3.6-53	3	159	10/06/00	B 3.7-43d	3	129	08/09/96
B 3.6-54	3	159	10/06/00	B 3.7-43e	3	129	
B 3.6-55	3	159	10/29/99	B 3.7-43f	3	129	
B 3.6-56	3	159	10/29/99	B 3.7-43g	3	129	
B 3.6-57	3	159	10/29/99	B 3.7-43h	3	129	
B 3.7-1	3	212	06/21/07	B 3.7-43i	3	129	
B 3.7-2	3	127	08/18/98	B 3.7-43j	3	129	
B 3.7-3	3	212	06/21/07	B 3.7-44	3	127	
B 3.7-4	3	212	06/21/07	B 3.7-45	3	127	
B 3.7-5	3	212	06/21/07	B 3.7-46	3	127	
B 3.7-6	3	212	06/21/07	B 3.7-47	3	127	05/30/97
B 3.7-7	3	127	04/16/99	B 3.7-48	3	127	
B 3.7-8	3	127	06/23/99	B 3.7-49	3	127	10/17/01
B 3.7-9	3	127		B 3.7-50	3	197	03/07/07
B 3.7-10	3	127		B 3.7-50a	3	197	03/07/07
B 3.7-11	3	127		B 3.7-51	3	127	04/08/07
B 3.7-12	3	127		B 3.7-52	3	127	10/17/01
B 3.7-13	3	127		B 3.7-53	3	127	04/12/01
B 3.7-14	3	127	04/16/99	B 3.7-54	3	127	04/12/01
B 3.7-15	3	127		B 3.7-55	3	127	03/07/07
B 3.7-16	3	127		B 3.7-55a	3	127	03/07/07
B 3.7-17	3	127	06/02/04	B 3.7-56	3	127	12/12/07
B 3.7-18	3	127		B 3.7-57	3	127	12/12/07
B 3.7-19	3	127		B 3.7-58	3	127	12/12/07

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.7-59	3	128 12/12/07	B 3.8-29a	3	169 06/18/01
B 3.7-60	3	127 12/12/07	B 3.8-29b	3	169 06/18/01
B 3.7-61	3	128 12/12/07	B 3.8-29c	3	169 06/18/01
B 3.7-62	3	128 12/12/07	B 3.8-29d	3	169 06/18/01
B 3.7-63	3	208 12/12/07	B 3.8-29e	3	169 06/18/01
B 3.7-64	3	208 12/12/07	B 3.8-29f	3	169 06/18/01
B 3.7-65	3	208 12/12/07	B 3.8-30	3	127
B 3.7-66	3	208 12/12/07	B 3.8-31	3	127
B 3.7-67	3	208 12/12/07	B 3.8-32	3	127
B 3.7-68	3	127 06/23/99	B 3.8-33	3	127
B 3.7-69	3	127 10/13/98	B 3.8-34	3	175 12/20/00
B 3.7-70	3	127 10/13/98	B 3.8-35	3	175 12/20/00
B 3.7-71	3	127 04/29/03	B 3.8-36	3	211 04/05/07
B 3.7-72	3	127	B 3.8-37	3	127 01/21/98
B 3.7-73	3	127	B 3.8-38	3	211 04/05/07
B 3.7-74	3	127 04/29/03	B 3.8-39	3	211 04/05/07
B 3.7-75	3	127 04/29/03	B 3.8-40	3	127 01/21/98
B 3.7-76	3	127 05/13/02	B 3.8-41	3	211 04/05/07
B 3.7-77	3	127	B 3.8-42	3	127 03/25/02
B 3.7-78	3	127	B 3.8-43	3	127
B 3.7-79	3	127	B 3.8-44	3	127 03/25/02
B 3.8-1	3	127 10/06/99	B 3.8-45	3	127
B 3.8-2	3	127 10/06/99	B 3.8-46	3	127 05/03/00
B 3.8-3	3	127 10/06/99	B 3.8-47	3	127 01/22/99
B 3.8-4	3	127 10/06/99	B 3.8-47a	3	127 01/22/99
B 3.8-5	3	127 10/06/99	B 3.8-48	3	127
B 3.8-6	3	127 10/06/99	B 3.8-49	3	127 01/22/99
B 3.8-7	3	197 07/19/05	B 3.8-50	3	127
B 3.8-7a	3	127 12/10/99	B 3.8-51	3	127
B 3.8-8	3	127 10/06/99	B 3.8-52	3	127
B 3.8-9	3	127 11/29/07	B 3.8-53	3	127 01/22/99
B 3.8-10	3	197 07/19/05	B 3.8-54	3	127 08/05/98
B 3.8-11	3	127 10/06/99	B 3.8-55	3	127 08/30/04
B 3.8-12	3	127 10/06/99	B 3.8.55a	3	127 08/05/98
B 3.8-13	3	127 10/06/99	B 3.8-56	3	127
B 3.8-14	3	127 10/06/99	B 3.8-57	3	127
B 3.8-15	3	127 10/06/99	B 3.8-58	3	175 12/20/00
B 3.8-16	3	127 06/18/01	B 3.8-59	3	175 12/20/00
B 3.8-17	3	127 06/18/01	B 3.8-60	3	127
B 3.8-18	3	211 04/05/07	B 3.8-61	3	127
B 3.8-19	3	127 06/18/01	B 3.8-62	3	127
B 3.8-20	3	127 11/29/07	B 3.8-63	3	127
B 3.8-21	3	127 06/18/01	B 3.8-64	3	127
B 3.8-22	3	127 06/18/01	B 3.8-65	3	127
B 3.8-23	3	127 06/18/01	B 3.8-66	3	127 07/31/00
B 3.8-24	3	127 06/18/01	B 3.8-67	3	127
B 3.8-25	3	127 06/18/01	B 3.8-68	3	127
B 3.8-26	3	127 06/18/01	B 3.8-69	3	127
B 3.8-27	3	127 06/18/01	B 3.8-70	3	127
B 3.8-28	3	127 06/18/01			
B 3.8-29	3	127 06/18/01			

SAN ONOFRE UNIT 2 TECHNICAL SPECIFICATIONS (NUREG-0741)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.8-71	3	127			
B 3.8-72	3	127			
B 3.8-73	3	175 12/20/00			
B 3.8-74	3	175 12/20/00			
B 3.8-75	3	127			
B 3.8-76	3	127			
B 3.8-77	3	127			
B 3.8-78	3	127			
B 3.8-79	3	127			
B 3.8-80	3	127			
B 3.8-81	3	127			
B 3.8-82	3	127			
B 3.8-83	3	127 01/11/07			
B 3.8-84	3	127			
B 3.8-85	3	127			
B 3.8-86	3	175 12/20/00			
B 3.8-87	3	175 12/20/00			
B 3.9-1	3	127			
B 3.9-2	3	127			
B 3.9-3	3	175 12/20/00			
B 3.9-4	3	127			
B 3.9-5	3	127			
B 3.9-6	3	175 12/20/00			
B 3.9-7	3	127			
B 3.9-8	3	127			
B 3.9-9	3	127 12/01/98			
B 3.9-10	3	193 02/18/05			
B 3.9-11	3	127 10/10/97			
B 3.9-12	3	127 10/10/97			
B 3.9-13	3	193 02/18/05			
B 3.9-13a	3	193 02/18/05			
B 3.9-14	3	127 10/10/97			
B 3.9-15	3	127 10/10/97			
B 3.9-16	3	127			
B 3.9-17	3	127 07/15/05			
B 3.9-17a	3	127 04/03/06			
B 3.9-18	3	175 07/15/05			
B 3.9-19	3	175 12/20/00			
B 3.9-20	3	127			
B 3.9-21	3	127 05/30/97			
B 3.9-22	3	127 07/15/05			
B 3.9-23	3	127 04/03/06			
B 3.9-24	3	175 12/20/00			
B 3.9-24a	3	175 12/20/00			
B 3.9-25	3	127 05/30/97			
B 3.9-26	3	127 10/13/98			
B 3.9-27	3	127 10/13/98			

SAN ONOFRE UNIT 2
FACILITY OPERATING LICENSE

LIST OF EFFECTIVE PAGES

Letters Modifying the License

NRC Letter dated February 25, 2002 from Samuel J. Collins (NRC) to Harold B. Ray (SCE), SUBJECT: ISSUANCE OF ORDER FOR INTERIM SAFEGUARDS AND SECURITY COMPENSATORY MEASURES FOR - SAN ONOFRE NUCLEAR STATION, UNITS 2 & 3

NRC Letter dated January 7, 2003 from Samuel J. Collins (NRC) to Harold B. Ray (SCE), SUBJECT: ISSUANCE OF ORDER FOR COMPENSATORY MEASURES RELATED TO ACCESS AUTHORIZATION

NRC Letter dated February 11, 2003 from Samuel J. Collins (NRC) to Harold B. Ray (SCE), SUBJECT: ISSUANCE OF ORDER ESTABLISHING INTERIM INSPECTION REQUIREMENTS FOR REACTOR PRESSURE VESSEL HEADS AT PRESSURIZED WATER REACTORS

NRC Letter dated April 29, 2003 from Samuel J. Collins (NRC) to Holders of Licenses for Operating Power Reactors as listed in Attachment 1 to the Order (John Todd (SCE)), SUBJECT: ISSUANCE OF ORDER REQUIRING COMPLIANCE WITH REVISED DESIGN BASIS THREAT FOR OPERATING POWER REACTORS

NRC Letter dated April 29, 2003 from Samuel J. Collins (NRC) to Holders of Licenses for Operating Power Reactors as listed in Attachment 1 to the Order (John Todd (SCE)), SUBJECT: ISSUANCE OF ORDER FOR COMPENSATORY MEASURES RELATED TO TRAINING ENHANCEMENTS ON TACTICAL AND FIREARMS PROFICIENCY AND PHYSICAL FITNESS APPLICABLE TO ARMED NUCLEAR POWER PLANT SECURITY FORCE PERSONNEL

NRC Letter dated April 29, 2003 from Samuel J. Collins (NRC) to Holders of Licenses for Operating Power Reactors as Listed in Enclosure 2 (John Todd (SCE)), SUBJECT: ISSUANCE OF ORDER FOR COMPENSATORY MEASURES RELATED TO FITNESS-FOR-DUTY ENHANCEMENTS APPLICABLE TO NUCLEAR FACILITY SECURITY FORCE PERSONNEL

NRC Letter dated March 20, 2006 from J. E. Dyer (NRC) to Holders of Licenses for Operating Power Reactors listed in Attachment 1 to the Order (Richard M. Rosenblum (SCE)), SUBJECT: ISSUANCE OF ORDER REQUIRING COMPLIANCE WITH UPDATED ADVERSARY CHARACTERISTIC

SAN ONOFRE UNIT 2
FACILITY OPERATING LICENSE

LIST OF EFFECTIVE PAGES

Facility Operating License

Page No.	Amendment No.
1	185
2	209
3	215
4	185
5	185
6	185
7*	185
8*	214
9	214

* Revised by NRC to SCE letter dated July 26, 2007

(3) SCE, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

(4) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(5) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of San Onofre Nuclear Generating Station, Units 1 and 2 and by the decommissioning of San Onofre Nuclear Generating Station Unit 1.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Southern California Edison Company (SCE) is authorized to operate the facility at reactor core power levels not in excess of full power (3438 megawatts thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 215, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

r. Plant-Specific Calculations for Compliance with 10 CFR Section 50.46 (II.K.3.31, SSER #1)

Deleted by Amendment No. 185

s. Improving Licensee Emergency Preparedness (III.A.2, SSER#1, SSER #5)

Deleted by Amendment No. 185

(20) Surveillance Program (Section 1.12, SSER #5)

Deleted by Amendment No. 185

(21) Laboratory Instrumentation (Section 1.12, SSER #5)

Deleted by Amendment No. 185

(22) Design Verification Program (Section 3.7.4, SSER #5)

Deleted by Amendment No. 185

(23) Emergency Preparedness Conditions

Deleted by Amendment No. 185

(24) RCS Depressurization System (PORV's)

Deleted by Amendment No. 185

(25) Qualification of Auxiliary Feedwater (AFW) Pump Motor Bearings

Deleted by Amendment No. 185

(26) Mitigation Strategy License Condition

Develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

(a) Fire fighting response strategy with the following elements:
1. Pre-defined coordinated fire response strategy and guidance
2. Assessment of mutual aid fire fighting assets
3. Designated staging areas for equipment and materials
4. Command and control
5. Training of response personnel

(b) Operations to mitigate fuel damage considering the following:
1. Protection and use of personnel assets
2. Communications
3. Minimizing fire spread
4. Procedures for implementing integrated fire response strategy
5. Identification of readily-availble pre-staged equipment

6. Training on integrated fire response strategy
7. Spent fuel pool mitigation measures

(c) Actions to minimize release to include consideration of:
1. Water spray scrubbing
2. Dose to onsite responders

(27) Upon implementation of Amendment No. 214 adopting TSTF-448, Revision 3, the determination of control room envelope (CRE) unfiltered air inleakage as required by SR 3.7.11.4, in accordance with TS 5.5.2.16.c(i), the assessment of CRE habitability as required by Specification 5.5.2.16.c(ii), and the measurement of CRE pressure as required by Specification 5.5.2.16.d, shall be considered met. Following implementation:

(a) The first performance of SR 3.7.11.4, in accordance with Specification 5.5.2.16.c(i), shall be within the specified frequency of 6 years, plus the 18-month allowance of SR 3.0.2, as measured from May 18, 2004, the date of the most recent successful tracer gas test, as stated in the September 17, 2004 letter response to Generic Letter 2003-01, or within the next 18 months if the time period since the most recent successful tracer gas test is greater than 6 years.

(b) The first performance of the periodic assessment of CRE habitability, Specification 5.5.2.16.c(ii), shall be within 3 years, plus the 9-month allowance of SR 3.0.2, as measured from May 18, 2004, the date of the most recent successful tracer gas test, as stated in the September 17, 2004, letter response to Generic Letter 2003-01, or within the next 9 month if the time period since the most recent successful tracer gas is greater than 3 years.

(c) Yhe first performance of the periodic measurement of CRE pressure, Specification 5.5.2.16.d, shall be within 6 months.

D. Exemptions to certain requirements of Appendices G, H and J to 10 CFR Part 50 are described in the Office of Nuclear Reactor Regulation's Safety Evaluation Report. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. Therefore, these exemptions are hereby granted. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission.

E. SCE shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21 is entitled: "San Onofre Nuclear Generating Station Security, Training and Qualification, and Safeguards Contingency Plan, Revision 0" submitted by letter dated October 19, 2004.

F. This license is subject to the following additional condition for the protection of the environment:

Before engaging in activities that may result in a significant adverse environmental impact that was not evaluated or that is significantly greater than that evaluated in the Final Environmental Statement, SCE shall provide a written notification of such activities to the NRC Office of Nuclear Reactor Regulation and receive written approval from that office before proceeding with such activities.

G. DELETED

H. SCE shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.

I. SCE shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

J. This license is effective as of the date of issuance and shall expire at midnight on February 16, 2022.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by
Harold R. Denton

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosures:
1. Appendix A (Technical Specifications)
2. Appendix B (Environmental Protection Plan)
3. Appendix C (Antitrust Conditions)

Date of Issuance: FEB 16 1982

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* On September 29, 1983, the Safeguards Contingency Plan was made a separate, companion document to the Physical Security Plan pursuant to the authority of 10 CFR 50.54.

3.7 PLANT SYSTEMS

3.7.11 Control Room Emergency Air Cleanup System (CREACUS)

LCO 3.7.11 Two CREACUS trains shall be OPERABLE.

---NOTE---
The control room envelope (CRE) boundary may be opened intermittently under administrative control.

APPLICABILITY: MODES 1, 2, 3, 4, 5, and 6,
During movement of irradiated fuel assemblies.

ACTIONS

---NOTES---
1. The provisions of LCO 3.0.4 are not applicable when entering MODES 5, 6, or defueled configuration.

2. Each Unit shall enter applicable ACTIONS separately.

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One CREACUS train inoperable for reasons other than Condition B.	A.1	Restore CREACUS train to OPERABLE status.	14 days
B. One or more CREACUS trains inoperable due to inoperable CRE boundary in Modes 1, 2, 3, or 4.	B.1	Initiate action to implement mitigating actions.	Immediately
	AND		
	B.2	Verify mitigating actions ensure CRE occupant exposures to radiological, chemical, and smoke hazards will not exceed limits.	24 hours
	AND		
	B.3	Restore CRE boundary to OPERABLE status.	90 days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
C. Required Action and associated Completion Time of Condition A or B not met in MODE 1, 2, 3, or 4.	C.1	Be in MODE 3.	6 hours
	AND		
	C.2	Be in MODE 5.	36 hours
D. Required Action and associated Completion Time of Condition A not met in MODE 5 or 6, or during movement of irradiated fuel assemblies.	D.1	Place OPERABLE CREACUS train in emergency radiation protection mode.	Immediately
	OR		
	D.2.1	Suspend CORE ALTERATIONS.	Immediately
	AND		
	D.2.2	Suspend movement of irradiated fuel assemblies.	Immediately
E. Two CREACUS trains inoperable in MODE 5 or 6, or during movement of irradiated fuel assemblies.	E.1	Suspend CORE ALTERATIONS.	Immediately
	AND		
OR	E.2	Suspend movement of irradiated fuel assemblies.	Immediately
One or more CREACUS trains inoperable due to an inoperable CRE boundary in MODE 5 or 6, or during movement of irradiated fuel assemblies.			

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
F. Two CREACUS trains inoperable in MODE 1, 2, 3, or 4 for reasons other than Condition B.	F.1 Enter LCO 3.0.3.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.11.1	Operate each CREACUS train for ≥ 2 hours.	31 days
SR 3.7.11.2	Perform required CREACUS filter testing in accordance with the Ventilation Filter Testing Program (VFTP).	In accordance with the VFTP
SR 3.7.11.3	Verify each CREACUS train actuates on an actual or simulated actuation signal.	24 months
SR 3.7.11.4	Perform required CRE unfiltered air inleakage testing in accordance with the Control Room Envelope Habitability Program.	In accordance with the Control Room Envelope Habitability Program

5.5 Procedures, Programs, and Manuals (continued)

5.5.2.11 Steam Generator (SG) Program (continued)

c) For tubes that have been repaired in the hot leg tubesheet region: Below the bottom of the lower sleeve-to-tube joint or greater than 10.6 inches below the bottom of the hot leg expansion transition or greater than 10.6 inches below the top of the hot leg tubesheet, whichever of these three is lowest.

d) For tubes that have been repaired in the cold leg tubesheet region: Below the bottom of the lower sleeve-to-tube joint or greater than 11.0 inches below the bottom of the cold leg expansion transition or greater than 11.0 inches below the top of the cold leg tubesheet, whichever of these three is lowest.

d. Provisions for SG tube inspections. Periodic SG tube inspections shall be performed. The number and portions of the tubes inspected and methods of inspection shall be performed with the objective of detecting flaws of any type (e.g., volumetric flaws, axial and circumferential cracks) that may be present along the length of the tube, from the tube-to-tubesheet weld at the tube inlet to the tube-to-tubesheet weld at the tube outlet, and that may satisfy the applicable tube repair criteria. The tube-to-tubesheet weld is not part of the tube. In tubes repaired by sleeving, the portion of the original tube wall between the sleeve's joints is not an area requiring re-inspection.

---NOTE---
The requirement for methods of inspection with the objective of detecting flaws of any type (e.g., volumetric flaws, axial and circumferential cracks) that may be present along the length of the tube does not apply to the portion of the original tube wall adjacent to the nickel band portion (the lower half) of the lower joint for the repair process that is discussed in Technical Specification (TS) 5.5.2.11.f.1. However, the method of inspection in this area should be a rotating plus point (or equivalent) coil. The SG tube repair criterion of TS 5.5.2.11.c.3.b is applicable to flaws in this area.

In addition to meeting the requirements of d.1, d.2, d.3, and d.4 below, the inspection scope, inspection methods, and inspection intervals shall be such as to ensure that SG tube integrity is maintained until the next SG inspection. An assessment of degradation shall be performed to determine the

(continued)

5.5 Procedures, Programs, and Manuals (continued)

5.5.2.11 Steam Generator (SG) Program (continued)

type and location of flaws to which the tubes may be susceptible and, based on this assessment, to determine which inspection methods need to be employed and at what locations.

1. Inspect 100% of the tubes in each SG during the first refueling outage following SG replacement.

2. Inspect 100% of the tubes at sequential periods of 60 effective full power months. The first sequential period shall be considered to begin after the first inservice inspection of the SGs. No SG shall operate for more than 24 effective full power months or one refueling outage (whichever is less) without being inspected.

3. If crack indications are found in any SG tube, then the next inspection for each SG for the degradation mechanism that caused the crack indication shall not exceed 24 effective full power months or one refueling outage (whichever is less). If definitive information, such as from examination of a pulled tube, diagnostic non-destructive testing, or engineering evaluation indicates that a crack-like indication is not associated with a crack(s), then the indication need not be treated as a crack.

4. All sleeves shall be inspected with eddy current prior to initial operation. This includes pressure retaining portions of the parent tube in contact with the sleeve, the sleeve-to-tube weld and the pressure retaining portion of the sleeve.

e. Provisions for monitoring operational primary to secondary LEAKAGE.

f. Provisions for SG tube repair methods. Steam generator tube repair methods shall provide the means to re-establish the RCS pressure boundary integrity of SG tubes without removing the tube from service. For the purposes of these Specifications, tube plugging is not a repair. All acceptable tube repair methods are listed below.

1. TIG welded sleeving with heat treatment, as described in ABB/CE Topical Report, CEN-630-P, Rev. 2, is currently approved by the NRC until December 2009. All sleeves shall be removed from service by December 2009.

Tube repair can be performed on certain tubes that have been previously plugged as a corrective or preventive measure. A tube inspection of the entire length of the tube shall be performed on a previously plugged tube prior to returning the tube to service.

(continued)

5.5 Procedures, Programs, and Manuals (continued)

5.5.2.15 Containment Leakage Rate Testing Program (Continued)

The provisions of Surveillance Requirement 3.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program. However, test frequencies specified in this Program may be extended consistent with the guidance provided in NEI 94-01, "Industry Guideline For Implementing Performance-Based Option Of 10CFR 50, Appendix J," as endorsed by Regulatory Guide 1.163. Specifically, NEI 94-01 has these provisions for test frequencies extension:

1. Consistent with standard scheduling practices for Technical Specifications Required Surveillances, intervals for recommended Type A testing may be extended by up to 15 months. This option should be used only in cases where refueling schedules have been changed to accommodate other factors.

2. Consistent with standard scheduling practices for Technical Specifications Required Surveillances, intervals for the recommended surveillance frequency for Type B and Type C testing may be extended by up to 25 percent of the test interval, not to exceed 15 months.

The provisions of Surveillance Requirement 3.0.3 are applicable to the Containment Leakage Rate Testing Program.

5.5.2.16 Control Room Envelope Habitability Program

A Control Room Envelope (CRE) Habitability Program shall be established and implemented to ensure that CRE habitability is maintained such that, with an OPERABLE Control Room Emergency Air Cleanup System (CREACUS), CRE occupants can control the reactor safely under normal conditions and maintain it in a safe condition following a radiological event, hazardous chemical release, or a smoke challenge. The program shall ensure that adequate radiation protection is provided to permit access and occupancy of the CRE under design basis accident (DBA) conditions without personnel receiving radiation exposures in excess of 5 rem total effective dose equivalent (TEDE) for the duration of the accident. The program shall include the following elements:

a. The definition of the CRE and the CRE boundary.

b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.

(continued)

5.5 Procedures, Programs, and Manuals (continued)

5.5.2.16 Control Room Envelope Habitability Program (Continued)

c. Requirements for (i) determining the unfiltered air inleakage past the CRE boundary into the CRE in accordance with the testing methods and at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.

The following is an exception to Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.

> Appropriate application of ASTM E-741 shall include the ability to take minor exceptions to the test methodology. These exceptions shall be documented in the test report.

d. Measurement, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one train of the CREACUS, operating at the flow rate required by the VFTP, at a Frequency of 24 months on a STAGGERED TEST BASIS. The results shall be trended and used as part of the 24 month assessment of the CRE boundary.

e. The quantitative limits on unfiltered air inleakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air inleakage measured by the testing described in paragraph c. The unfiltered air inleakage limit for radiological challenges is the inleakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air inleakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.

f. The provisions of SR 3.0.2 are applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered inleakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

B 3.7 PLANT SYSTEMS

B 3.7.11 Control Room Emergency Air Cleanup System (CREACUS)

BASES

BACKGROUND

The CREACUS provides a protected environment from which occupants can control the plant following an uncontrolled release of radioactivity, hazardous chemicals, or smoke.

The CREACUS consists of two independent, redundant trains that recirculate and filter air in the control room envelope (CRE) and a CRE boundary that limits the inleakage of unfiltered air. Each CREACUS train consists of emergency air conditioning unit, emergency ventilation air supply unit, emergency isolation dampers, and cooling coils and two cabinet coolers per Unit. Each emergency air conditioning unit includes a prefilter, a high efficiency particulate air (HEPA) filter, an activated carbon adsorber section for removal of gaseous activity (principally iodines), and a fan. A second bank of HEPA filters follows the adsorber section to collect carbon fines. Each emergency ventilation air supply unit includes prefilter, HEPA filter, carbon adsorber and fan. Ductwork, motor-operated dampers, doors, barriers, and instrumentation also form part of the system. Air and motor-operated dampers are provided for air volume control and system isolation purposes.

The CRE is the area within the confines of the CRE boundary that contains the spaces that control room occupants inhabit to control the unit during normal and accident conditions. This area encompasses the control room, and may encompass other non-critical areas to which frequent personnel access or continuous occupancy is not necessary in the event of an accident. The CRE is protected during normal operation, natural events, and accident conditions. The CRE boundary is the combination of walls, floor, roof, ducting, doors, penetrations and equipment that physically form the CRE. The OPERABILITY of the CRE boundary must be maintained to ensure that the inleakage of unfiltered air into the CRE will not exceed the inleakage assumed in the licensing basis analyses of design basis accident (DBA) consequences to CRE occupants. The CRE and its boundary are defined in the Control Room Envelope Habitability Program.

(continued)

BASES (continued)

BACKGROUND (continued)

Upon receipt of the actuating signal, normal air supply to the CRE is isolated, and the stream of ventilation air is recirculated through the system's filter trains. The prefilters remove any large particles in the air to prevent excessive loading of the HEPA filters and charcoal adsorbers. Continuous operation of each train for at least 15 minutes per month verifies proper system operability.

There are two CREACUS operational modes. Emergency mode is an operational mode when the control room is isolated to protect operational personnel from radioactive exposure through the duration of any one of the postulated limiting faults discussed in Chapter 15 UFSAR (Ref. 2). Isolation mode is an operational mode when the CRE is isolated to protect operational personnel from toxic gasses and smoke.

Actuation of the CREACUS places the system into either of two separate states of operation, depending on the initiation signal. Actuation of the system to either the emergency mode r isolation mode of CREACUS operation closes the unfiltered-outside-air intake and unfiltered exhaust dampers, and aligns the system for recirculation of air within the CRE through the redundant trains of HEPA and charcoal filters.

The emergency mode also initiates pressurization of the CRE. Outside air is added to the air being recirculated from the CRE. Pressurization of the CRE minimizes infiltration of unfiltered air through the CRE boundary from all the surrounding areas adjacent to the CRE boundary.

The CRE supply and the outside air supply of the normal control room HVAC are monitored by radiation and toxic-gas detectors respectively. One detector output above the setpoint will cause actuation of the emergency mode or isolation mode as required. The actions of the isolation mode are more restrictive, and will override the actions of the emergency mode of operation. However, toxic gas and radiation events are not considered to occur concurrently.

(continued)

BASES (continued)

BACKGROUND (continued)

Redundant recirculation trains provide the required filtration should an excessive pressure drop develop across the other filter train. Normally-open isolation dampers are arranged in series pairs so that one damper's failure to shut will not result in a breach of isolation. The CREACUS is designed in accordance with Seismic Category I requirements.

The CREACUS is designed to maintain a habitable environment in the CRE for 30 days of continuous occupancy after a Design Basis Accident (DBA) without exceeding a 5-rem total effective dose equivalent (TEDE).

APPLICABLE SAFETY ANALYSES

The CREACUS components are arranged in redundant, safety related ventilation trains. The location of components and ducting within the CRE ensures an adequate supply of filtered air to all areas requiring access.

The CREACUS provides airborne radiological protection for the CRE occupants, as demonstrated by the CRE occupant dose analyses for the most limiting design basis loss of coolant accident fission product release presented in the UFSAR, Chapter 15 (Ref. 2).

Dose calculations, as specified in Unit 2/3 UFSAR Chapter 15 (Reference 2), only take credit for the HEPA filters and charcoal adsorbers of the emergency recirculation air conditioning unit. The emergency ventilation supply unit is designed to contribute to the pressurization of the control room to minimize unfiltered inleakage as indicated in Unit 2/3 UFSAR.

(continued)

BASES (continued)

APPLICABLE SAFETY ANALYSES (continued)

The CREACUS provides protection from smoke and hazardous chemicals to the CRE occupants. The analysis of hazardous chemical releases demonstrates that the toxicity limits are not exceeded in the CRE following a hazardous chemical release (Ref. 3). The evaluation of a smoke challenge demonstrates that it will not result in the inability of the CRE occupants to control the reactor either from the control room or from the remote shutdown panels (Ref. 4).

The worst case single active failure of a component of the CREACUS, assuming a loss of offsite power, does not impair the ability of the system to perform its design function.

The CREACUS satisfies Criterion 3 of the NRC Policy Statement.

LCO

Two independent and redundant trains of the CREACUS are required to be OPERABLE to ensure that at least one is available if a single active failure disables the other train. Total system failure, such as from a loss of both ventilation trains or from an inoperable CRE boundary, could result in exceeding a dose of 5 rem TEDE to the CRE occupants in the event of a large radioactive release.

Each CREACUS train is considered OPERABLE when the individual components necessary to limit CRE occupant exposure are OPERABLE. A CREACUS train is considered OPERABLE when the associated:

a. Fan is OPERABLE;

b. HEPA filters and charcoal adsorber are not excessively restricting flow, and are capable of performing their filtration functions; and

(continued)

BASES (continued)

LCO (continued)

c. Ductwork, valves, and dampers are OPERABLE, and air circulation can be maintained. If an Emergency Isolation Damper is stuck open, the associated train of CREACUS may still be considered OPERABLE if the redundant damper in series with the inoperable damper is closed with power removed.

In order for the CREACUS trains to be considered OPERABLE, the CRE boundary must be maintained such that CRE occupant dose from a large radioactive release does not exceed the calculated dose in the licensing basis consequence analyses for DBAs, and that CRE occupants are protected from hazardous chemicals and smoke.

The LCO is modified by a Note allowing the CRE boundary to be opened intermittently under administrative controls. This note only applies to openings in the CRE boundary that can be rapidly restored to the design condition, such as doors, hatches, floor plugs, and access panels. For entry and exit through doors the administrative control of the opening is performed by the person(s) entering or exiting the area. For other openings, these controls should be proceduralized and consist of stationing a dedicated individual at the opening who is in continuous communication with the operators in the CRE. This individual will have a method to rapidly close the opening and to restore the CRE boundary to a condition equivalent to the design condition when a need for CRE isolation is indicated.

(continued)

BASES (continued)

APPLICABILITY

In MODES 1, 2, 3, 4, 5, and 6 and during movement of irradiated fuel assemblies the CREACUS must be OPERABLE to ensure that the CRE will remain habitable during and following a DBA.

In MODES 5 and 6, the CREACUS is required to cope with the release from a rupture of a waste gas tank.

During movement of irradiated fuel assemblies, the CREACUS must be OPERABLE to cope with the release from a fuel handling accident involving handling irradiated fuel.

ACTIONS

ACTION statements are modified by two NOTES. NOTE 1 says: "The provisions of LCO 3.0.4 are not applicable when entering MODES 5, 6, or defueled configuration."

Specification 3.0.4 establishes that entry into an operational mode or other specified condition shall not be made unless the conditions of the LCO are met. Applicability statement "During movement of irradiated fuel assemblies" ensures the OPERABILITY of both CREACUS trains prior to the start of movement of irradiated fuel assemblies.

NOTE 2 says: "Each Unit shall enter applicable ACTIONS separately." CREACUS is a shared system between Unit 2 and Unit 3. LCO doesn't address the operational situation when the Units are in different operational MODES. Without this NOTE it may not be clear what ACTIONS should be taken.

(continued)

BASES (continued)

ACTIONS (continued)

A.1

With one CREACUS train inoperable, for reasons other than an inoperable CRE boundary, action must be taken to restore OPERABLE status within 14 days. The 14 day AOT is based on a probabilistic risk assessment that does not require administrative controls to be implemented when a CREACUS train is taken out of service. In this Condition, the remaining OPERABLE CREACUS train is adequate to perform the CRE occupant protection function.

However, the overall reliability is reduced because a failure in the OPERABLE CREACUS train could result in loss of CREACUS function. The 14 day Completion Time is based on the low probability of a DBA occurring during this time period, and the ability of the remaining train to provide the required capability.

B.1, B.2 and B.3

If the unfiltered inleakage of potentially contaminated air past the CRE boundary and into the CRE can result in CRE occupant radiological dose greater than the calculated dose of the licensing basis analyses of DBA consequences (allowed to be up to 5 rem TEDE), or inadequate protection of CRE occupants from hazardous chemicals or smoke, the CRE boundary is inoperable. Actions must be taken to restore an OPERABLE CRE boundary within 90 days.

(continued)

BASES (continued)

ACTIONS (continued)

During the period that the CRE boundary is considered inoperable, action must be initiated to implement mitigating actions to lessen the effect on CRE occupants from the potential hazards of a radiological or chemical event or a challenge from smoke. Actions must be taken within 24 hours to verify that in the event of a DBA, the mitigating actions will ensure that CRE occupant radiological exposures will not exceed the calculated dose of the licensing basis analyses of DBA consequences, and that CRE occupants are protected from hazardous chemicals and smoke. These mitigating actions (i.e., actions that are taken to offset the consequences of the inoperable CRE boundary) should be preplanned for implementation upon entry into the condition, regardless of whether entry is intentional or unintentional. The 24 hour Completion Time is reasonable based on the low probability of a DBA occurring during this time period, and the use of mitigating actions. The 90 day Completion Time is reasonable based on the determination that the mitigating actions will ensure protection of CRE occupants within analyzed limits while limiting the probability that CRE occupants will have to implement protective measures that may adversely affect their ability to control the reactor and maintain it in a safe shutdown condition in the event of a DBA. In addition, the 90 day Completion Time is a reasonable time to diagnose, plan and possibly repair, and test most problems with the CRE boundary.

C.1 and C.2

In MODES 1, 2, 3, or 4, if the inoperable CREACUS or the CRE boundary cannot be restored to OPERABLE status within the required Completion Time, the unit must be placed in a MODE that minimizes the accident risk. To achieve this status, the unit must be placed in at least MODE 3 within 6 hours, and in MODE 5 within 36 hours. The allowed Completion Times are reasonable, based on operating experience, to reach the required unit conditions from full power conditions in an orderly manner and without challenging unit systems.

(continued)

BASES (continued)

ACTIONS (continued)

D.1, D.2.1, and D.2.2

In MODE 5 or 6, or during movement of irradiated fuel assemblies, if Required Action A.1 cannot be completed within the required Completion Time, the OPERABLE CREACUS train must be immediately placed in the emergency mode of operation. This action ensures that the remaining train is OPERABLE, that no failures preventing automatic actuation will occur, and that any active failure will be readily detected.

An alternative to Required Action D.1 is to immediately suspend activities that could result in a release of radioactivity that might require isolation of the CRE. This places the unit in a condition that minimizes the accident risk. This does not preclude the movement of fuel assemblies to a safe position.

E.1 and E.2

When in MODE 5 or 6, or during movement of irradiated fuel assemblies with two trains inoperable or with one or more CREACUS trains inoperable due to an inoperable CRE boundary, action must be taken immediately to suspend activities that could result in a release of radioactivity that might enter the CRE. This places the unit in a condition that minimizes theaccident risk. This does not preclude the movement of fuel to a safe position.

F.1

If both CREACUS trains are inoperable in MODE 1, 2, 3, or 4 for reasons other than an inoperable CRE boundary (i.e., Condition B), the CREACUS may not be capable of performing the intended function and the unit is in a condition outside the accident analyses. Therefore, LCO 3.0.3 must be entered immediately.

(continued)

BASES (continued)

SURVEILLANCE REQUIREMENTS

SR 3.7.11.1

Standby systems should be checked periodically to ensure that they function properly. Since the environment and normal operating conditions on this system are not severe, testing each train once every month provides an adequate check on this system.

Cumulative operation of the system for at least 2 hours over a 31 day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filtes. The 2 hour time frame is based on a conservative engineering evaluation which calculated the time required to evaporate the moisture contained in the air trapped inside the CREACUS duct upstream of charcoal beds. The 31 day Frequency is based on the known reliability of the equipment, and the two train redundancy available.

SR 3.7.11.2

This SR verifies that the required CREACUS testing is performed in accordance with the Ventilation Filter Testing Program (VFTP). The CREACUS filter tests are based on Regulatory Guide 1.52 (Ref. 5). The VFTP includes testing HEPA filter performance, charcoal adsorber efficiency, minimum system flow rate, and the physical properties of theactivated charcoal (general use and following specific operations). Specific test Frequencies and additional information are discussed in detail in the VFTP.

The filtration efficiency only apply to the emergency recirculation air conditioning units E418 and E419. Therefore, testing for filtration efficiency is not required for the emergency ventilation supply units A206 and A207.

However, the specified air flow from the emergency ventilation units is required during the filtration efficiency testing of the emergency recirculation air conditioning units. Also, the air flow requirements which are specified in the VFTP apply to the emergency ventilation and emergency air conditioning units.

(continued)

BASES (continued)

SURVEILLANCE REQUIREMENTS (continued)

SR 3.7.11.3

This SR verifies that each CREACUS train starts and operates on an acutual or simulated actuation signal. The Frequency of 24 months is based on industry operating experience and is consistent with the typical refueling cycle.

SR 3.7.11.4

This SR verifies the OPERABILITY of the CRE boundary by testing for unfiltered air inleakage past the CRE boundary and into the CRE. The details of the testing are specified in the Control Room Envelope Habitability Program.

The CRE is considered habitable when the radiological dose to CRE occupants calculated in the licensing basis analyses of DBA consequences is no more than 5 rem TEDE and the CRE occupants are protected from hazardous chemicals and smoke. This SR verifies that the unfiltered air inleakage into the CRE is no greater than the flow rate assumed in the licensing basis analyses of DBA consequences. When unfiltered air inleakage is greater than the assumed flow rate, Condition B must be entered. Required Action B.3 allows time to restore the CRE boundary to OPERABLE status provided mitigating actions can ensure that the CRE remains within the licensing basis habitability limits for the occupants following an accident. Compensatory measures are discussed in Regulatory Guide 1.196, Section C.2.7.3, (Ref. 6) which endorses, with exceptions, NEI 99-03, Section

(continued)

BASES (continued)

SURVEILLANCE REQUIREMENTS (continued)

SR 3.7.11.4

8.4 and Appendix F (Ref. 7). These compensatory measures may also be used as mitigating actions as required by Required Action B.2. Temporary analytical methods may also be used as compensatory measures to restore OPERABILITY (Ref. 8). Options for restoring the CRE boundary to OPERABLE status include changing the licensing basis DBA consequence analysis, repairing the CRE boundary, or a combination of these actions. Depending upon the nature of the problem and the corrective action, a full scope inleakage test may not be necessary to establish that the CRE boundary has been restored to OPERABLE status.

REFERENCES

1. UFSAR, Section 9.4.

2. UFSAR, Chapter 15.

3. UFSAR, Section 6.4.

4. UFSAR, Section 9.5.

5. Regulatory Guide 1.52 (Rev. 2).

6. Regulatory Guide 1.196.

7. NEI 99-03, "Control Room Habitability Assessment," June 2001.

8. Letter from Eric J. Leeds (NRC) to James W. Davis (NEI) dated January 30, 2004, "NEI Draft White Paper, Use of Generic Letter 91-18 Process and Alternative Source Terms in the Context of Control Room Habitability." (ADAMS Accession No. ML040300694).

(continued)

SONGS UNIT 3 SINGLE-SIDED
Operating License & Technical Specification/Licensee Controlled Specification

Unit 3 Amendments 206/207 and Unit 3 Bases Change B07-003

Revise Unit 3 Operating License by removing pages and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and/or dates and contain marginal/lines indicating the areas of change. **Please direct any questions regarding filing of information to: (949) 368-6972or email CDM Controlled Manuals Desk.**

REMOVE:	**INSERT:**
Volume 1	**Volume 1**
BEHIND: List of Effective Pages – "Blue Tab"	**BEHIND**: List of Effective Pages – "Blue Tab"
List of Effective Pages – Pages 1-12 11/29/07	List of Effective Pages – Pages 1-12 12/12/07
BEHIND: "Blue Tab" Facility Operating License	**BEHIND:** "Blue Tab" Facility Operating License
Songs Unit 3 – Facility Operating License List of Effective Pages – Pages 1-2	Songs Unit 3 – Facility Operating License List of Effective Pages – Pages 1-2
Docket No. 50-362 letter – page 3 Amd. 204	Docket No. 50-362 letter – page 3 Amd. 207
Docket No. 50-362 letter – page 7 Amd. 197	Docket No. 50-362 letter – page 7 Amd. 206 Revised by letter dated July 26, 207
Docket No. 50-362 letter – page 8 Amd. 176	Docket No. 50-362 letter – page 8 Amd. 206 Revised by letter dated March 6, 207
BEHIND: "Blue Tab" Plant Systems	**BEHIND:** "Blue Tab" Plant Systems
Page 3.7-24 Amd. 172	Page 3.7-24 Amd. 206
Page 3.7-25 Amd. 168	Page 3.7-25 Amd. 206
Page 3.7-26 Amd. 168	Page 3.7-26 Amd. 206
BEHIND: "Blue Tab" 5.0 Administrative Controls	**BEHIND:** "Blue Tab" 5.0 Administrative Controls
Page 5.0-15a Amd. 198	Page 5.0-15a Amd. 207
Page 5.0-16 Amd. 196	Page 5.0-16 Amd. 207
Page 5.0-20b Amd. 135	Page 5.0-20b Amd. 206
Insert New Page	Page 5.0-20c Amd. 206
Volume 3	**Volume 3**
BEHIND: "Blue Tab" B 3.7	**BEHIND:** "Blue Tab" 3.7
Page B 3.7-56 Amd. 116 06/28/99	Page B 3.7-56 Amd. 116 12/12/07
Page B 3.7-57 Amd. 116 04/25/05	Page B 3.7-57 Amd. 116 12/12/07
Page B 3.7-58 Amd. 116 03/19/98	Page B 3.7-58 Amd. 116 12/12/07
Page B 3.7-59 Amd. 117 02/05/01	Page B 3.7-59 Amd. 117 12/12/07
Page B 3.7-60 Amd. 116 10/17/01	Page B 3.7-60 Amd. 116 12/12/07
Page B 3.7-61 Amd. 117 09/16/03	Page B 3.7-61 Amd. 117 12/12/07
Page B 3.7-62 Amd. 117 02/05/01	Page B 3.7-62 Amd. 117 12/12/07
Page B 3.7-62a Amd. 117 02/05/01	No Replacement
Page B 3.7-63 Amd. 200 12/05/06	Page B 3.7-63 Amd. 200 12/12/07
Page B 3.7-64 Amd. 200 12/05/06	Page B 3.7-64 Amd 200 12/12/07
Page B 3.7-65 Amd. 200 12/05/06	Page B 3.7-65 Amd. 200 12/12/07
Page B 3.7-66 Amd. 200 12/05/06	Page B 3.7-66 Amd. 200 12/12/07
Page B 3.7-67 Amd. 200 12/05/06	Page B 3.7-67 Amd. 200 12/12/07

SONGS UNIT 3 SINGLE-SIDED
Operating License & Technical Specification/Licensee Controlled Specification

Unit 3 Amendments 206/207 and Unit 3 Bases Change B07-003

Revise Unit 3 Operating License by removing pages and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and/or dates and contain marginal/lines indicating the areas of change. **Please direct any questions regarding filing of information to: (949) 368-6972or email CDM Controlled Manuals Desk.**

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APPENDIX A TO LICENSE NO. NPF-15

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date
LOEP-1	1	207 12/12/07
LOEP-2	1	200 01/25/07
LOEP-3	1	206 10/31/07
LOEP-4	1	207 11/29/07
LOEP-4a	1	159
LOEP-5	1	199 12/21/06
LOEP-6	1	191 11/18/05
LOEP-7	1	188 07/19/05
LOEP-8	1	200 12/05/06
LOEP-9	1	196 09/19/06
LOEP-10	1	204 12/12/07
LOEP-11	1	203 12/12/07
LOEP-12	1	184 03/07/07
ii	1	116
iii	1	200
iv	1	200
v	1	188
vi	1	116
vii	1	200
viii	1	200
1.1-1	1	116
1.1-2	1	116
1.1-3	1	179
1.1-4	1	196
1.1-5	1	195
1.1-6	1	191
1.1-7	1	116
1.2-1	1	116
1.2-2	1	116
1.2-3	1	116
1.3-1	1	116
1.3-2	1	116
1.3-3	1	116
1.3-4	1	116
1.3-5	1	116
1.3-6	1	116
1.3-7	1	116
1.3-8	1	116
1.3-9	1	116
1.3-10	1	116
1.3-11	1	116
1.3-12	1	116
1.3-13	1	116
1.4-1	1	116
1.4-2	1	116
1.4-3	1	116
1.4-4	1	116
2.0-1	1	199
2.0-2	1	199
3.0-1	1	116
3.0-2	1	116
3.0-3	1	116
3.0-4	1	177
3.0-5	1	116
3.1-1	1	191
3.1-2	1	191
3.1-3	1	191
3.1-4	1	116
3.1-5	1	116
3.1-6	1	116
3.1-7	1	191
3.1-8	1	191
3.1-9	1	116
3.1-10	1	116
3.1-11	1	126
3.1-12	1	191
3.1-13	1	116
3.1-14	1	191
3.1-15	1	191
3.1-16	1	191
3.1-17	1	116
3.1-18	1	116
3.1-19	1	116
3.1-20	1	116
3.1-21	1	154
3.1-22	1	166
3.1-23	1	116
3.1-24	1	116
3.1-25	1	116
3.1-26	1	116
3.1-27	1	116
3.1-28	1	116
3.1-29	1	116
3.2-1	1	116
3.2-2	1	116
3.2-3	1	116
3.2-4	1	116
3.2-5	1	116
3.2-6	1	116
3.2-7	1	116
3.2-8	1	116
3.2-9	1	191
3.2-10	1	116
3.2-11	1	116
3.2-12	1	116
3.3-1	1	116

APPENDIX A TO LICENSE NO. NPF-15

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
3.3-2	1	116	3.3-51	1	166
3.3-3	1	116	3.3-52	1	116
3.3-4	1	116	3.4-1	1	141
3.3-5	1	116	3.4-2	1	141
3.3-6	1	139	3.4-3	1	141
3.3-7	1	122	3.4-4	1	116
3.3-8	1	142	3.4-5	1	195
3.3-9	1	142	3.4-6	1	195
3.3-10	1	116	3.4-7	1	195
3.3-11	1	116	3.4-8	1	195
3.3-12	1	122	3.4-9	1	195
3.3-13	1	139	3.4-10	1	195
3.3-14	1	116	3.4-11	1	195
3.3-15	1	116	3.4-12	1	195
3.3-16	1	122	3.4-13	1	116
3.3-17	1	116	3.4-14	1	188
3.3-18	1	116	3.4-15	1	116
3.3-19	1	116	3.4-16	1	166
3.3-20	1	122	3.4-17	1	166
3.3-21	1	116	3.4-18	1	195
3.3-22	1	148	3.4-19	1	166
3.3-23	1	148	3.4-20	1	116
3.3-24	1	148	3.4-21	1	166
3.3-25	1	148	3.4-22	1	195
3.3-26	1	116	3.4-23	1	166
3.3-27	1	116	3.4-24	1	166
3.3-28	1	116	3.4-25	1	166
3.3-29	1	122	3.4-26	1	152
3.3-30	1	116	3.4-27	1	146
3.3-31	1	116	3.4-28	1	147
3.3-32	1	116	3.4-29	1	147
3.3-33	1	116	3.4-30	1	195
3.3-34	1	187	3.4-31	1	195
3.3-35	1	121	3.4-32	1	116
3.3-36	1	121	3.4-33	1	195
3.3-37	1	121	3.4-34	1	134
3.3-38	1	121	3.4-35	1	195
3.3-39	1	121	3.4-36	1	116
3.3-40	1	166	3.4-37	1	196
3.3-41	1	151	3.4-38	1	196
3.3-42	1	200 01/25/07	3.4-39	1	116
3.3-43	1	200 01/25/07	3.4-40	1	116
3.3-44	1	116	3.4-41	1	116
3.3-45	1	185	3.4-42	1	116
3.3-46	1	185	3.4-43	1	116
3.3-47	1	185	3.4-44	1	116
3.3-48	1	116	3.4-45	1	116
3.3-49	1	116			
3.3-50	1	116			

APPENDIX A TO LICENSE NO. NPF-15

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
3.4-46	1	116	3.7-6	1	116
3.4-47	1	116	3.7-7	1	116
3.4-48	1	116	3.7-8	1	116
3.4-49	1	116	3.7-9	1	116
3.4-50	1	116	3.7-10	1	116
3.4-51	1	196	3.7-11	1	155
3.4-52	1	196	3.7-12	1	116
3.5-1	1	131	3.7-13	1	116
3.5-2	1	124	3.7-14	1	182
3.5-3	1	116	3.7-15	1	139
3.5-4	1	131	3.7-16	1	153
3.5-5	1	116	3.7-17	1	116
3.5-6	1	181	3.7-18	1	116
3.5-7	1	116	3.7-19	1	116
3.5-8	1	116	3.7-19a	1	118
3.5-9	1	116	3.7-19b	1	118
3.5-10	1	116	3.7-19c	1	118
3.5-11	1	116	3.7-20	1	116
3.6-1	1	116	3.7-21	1	116
3.6-2	1	135	3.7-22	1	172
3.6-3	1	116	3.7-23	1	116
3.6-4	1	116	3.7-24	1	206
3.6-5	1	116	3.7-25	1	206
3.6-6	1	135	3.7-26	1	206
3.6-7	1	116	3.7-27	1	200 01/25/07
3.6-8	1	116	3.7-28	1	200 01/25/07
3.6-9	1	116	3.7-29	1	116
3.6-10	1	116	3.7-30	1	116
3.6-11	1	116	3.7-31	1	116
3.6-12	1	156	3.7-32	1	120
3.6-13	1	192	3.7-33	1	120
3.6-14	1	192	3.7-34	1	120
3.6-15	1	116	3.7-35	1	116
3.6-16	1	116	3.8-1	1	133
3.6-17	1	116	3.8-2	1	133
3.6-18	1	162	3.8-3	1	116
3.6-19	1	116	3.8-4	1	149
3.6-20	1	116	3.8-5	1	170
3.6-21	1	116	3.8-6	1	203
3.6-22	1	116	3.8-7	1	149
3.6-23	1	150	3.8-8	1	170
3.6-24	1	150	3.8-9	1	149
3.6-25	1	116	3.8-10	1	149
3.6-26	1	116	3.8-11	1	149
3.7-1	1	204	3.8-12	1	149
3.7-2	1	116	3.8-13	1	160
3.7-3	1	204	3.8-14	1	170
3.7-4	1	204	3.8-15	1	149
3.7-5	1	116	3.8-16	1	116

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date
3.8-17	1	116
3.8-18	1	166
3.8-19	1	116
3.8-20	1	203
3.8-21	1	203
3.8-22	1	116
3.8-23	1	116
3.8-24	1	116
3.8-25	1	116
3.8-26	1	116
3.8-27	1	166
3.8-28	1	116
3.8-29	1	116
3.8-30	1	116
3.8-31	1	116
3.8-32	1	116
3.8-33	1	116
3.8-34	1	116
3.8-35	1	116
3.8-36	1	166
3.8-37	1	116
3.8-38	1	116
3.8-39	1	116
3.8-40	1	166
3.8-41	1	116
3.9-1	1	116
3.9-2	1	166
3.9-3	1	116
3.9-4	1	184
3.9-5	1	116
3.9-6	1	166
3.9-6a	1	166
3.9-7	1	144
3.9-8	1	144
3.9-9	1	166
3.9-9a	1	144
3.9-10	1	123
4.0-1	1	190
4.0-2	1	116
4.0-3	1	116
4.0-4	1	120
4.0-5	1	120
5.0-1	1	199
5.0-2	1	199
5.0-3	1	199
5.0-4	1	199
5.0-5	1	199
5.0-6	1	175
5.0-7	1	116
5.0-8	1	199
5.0-9	1	169
5.0-10	1	116
5.0-11	1	161
5.0-12	1	169
5.0-13	1	196
5.0-14	1	196
5.0-15	1	198
5.0-15a	1	207
5.0-16	1	207
5.0-17	1	200
5.0-18	1	196
5.0-19	1	196
5.0-20	1	196
5.0-20a	1	189
5.0-20b	1	206
5.0-20c	1	206
5.0-21	1	116
5.0-22	1	116
5.0-23	1	186
5.0-24	1	116
5.0-25	1	116
5.0-26	1	191
5.0-27	1	195
5.0-27a	1	195
5.0-28	1	196
5.0-29	1	196
5.0-30	1	188
5.0-31	1	199

APPENDIX A TO LICENSE NO. NPF-15

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date
Bibliographic Data Sheet		
Appendix B		dated Nov. 15, 1982
Environmental Protection Plan		dated Nov. 15, 1982
1-1	1	dated Nov. 15, 1982
2-1	1	dated Nov. 15, 1982
2-2	1	dated Nov. 15, 1982
3-1	1	dated Nov. 15, 1982
3-2	1	159
4-1	1	159
4-2	1	dated Nov. 15, 1982
Appendix C	1	dated Nov. 15, 1982
Page 2	1	dated Nov. 15, 1982
Page 3	1	dated Nov. 15, 1982

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 2.0-1	2	183 07/19/04	B 3.1-27	2	116
B 2.0-2	2	116	B 3.1-28	2	191 11/18/05
B 2.0-3	2	199 12/21/06	B 3.1-29	2	191 11/18/05
B 2.0-3a	2	183 07/19/04	B 3.1-30	2	191 11/18/05
B 2.0-4	2	199 12/21/06	B 3.1-31	2	116
B 2.0-5	2	199 12/21/06	B 3.1-32	2	116 05/02/97
B 2.0-6	2	116	B 3.1-33	2	116
B 2.0-7	2	116	B 3.1-34	2	116 05/02/97
B 2.0-8	2	199 12/21/06	B 3.1-35	2	116
B 2.0-9	2	199 12/21/06	B 3.1-36	2	116
B 3.0-1	2	116	B 3.1-37	2	191 11/18/05
B 3.0-2	2	116	B 3.1-38	2	191 11/18/05
B 3.0-3	2	116	B 3.1-39	2	116
B 3.0-4	2	199 12/21/06	B 3.1-40	2	116
B 3.0-5	2	116	B 3.1-41	2	116
B 3.0-6	2	116	B 3.1-42	2	116
B 3.0-7	2	116	B 3.1-43	2	191 11/18/05
B 3.0-8	2	116	B 3.1-43a	2	191 11/18/05
B 3.0-9	2	116	B 3.1-44	2	191 11/18/05
B 3.0-10	2	116	B 3.1-45	2	116
B 3.0-11	2	116	B 3.1-46	2	191 11/18/05
B 3.0-12	2	116 12/01/98	B 3.1-47	2	116
B 3.0-13	2	177 10/25/02	B 3.1-48	2	116
B 3.0-14	2	177 05/08/02	B 3.1-49	2	116
B 3.0-15	2	177 05/08/02	B 3.1-50	2	116
B 3.1-1	2	191 11/18/05	B 3.1-51	2	116
B 3.1-2	2	191 11/18/05	B 3.1-52	2	116
B 3.1-3	2	191 11/18/05	B 3.1-53	2	116
B 3.1-4	2	191 11/18/05	B 3.1-54	2	116 08/03/01
B 3.1-5	2	191 11/18/05	B 3.1-55	2	116 08/03/01
B 3.1-6	2	191 11/18/05	B 3.1-56	2	116 08/03/01
B 3.1-7	2	191 11/18/05	B 3.1-57	2	116 08/03/01
B 3.1-8	2	191 11/18/05	B 3.1-58	2	116 08/03/01
B 3.1-9	2	191 11/18/05	B 3.1-58a	2	116 08/03/01
B 3.1-10	2	191 11/18/05	B 3.1-58b	2	116 08/03/01
B 3.1-11	2	191 11/18/05	B 3.1-59	2	191 11/18/05
B 3.1-12	2	191 11/18/05	B 3.1-60	2	116 11/05/96
B 3.1-13	2	116	B 3.1-61	2	116 05/03/00
B 3.1-14	2	116	B 3.1-62	2	166 12/20/00
B 3.1-15	2	191 11/18/05	B 3.1-63	2	166 05/21/02
B 3.1-16	2	116	B 3.1-63a	2	166 12/20/00
B 3.1-17	2	191 11/18/05	B 3.1-64	2	116
B 3.1-18	2	116	B 3.1-65	2	116
B 3.1-19	2	116 01/03/02	B 3.1-66	2	116
B 3.1-20	2	116	B 3.1-67	2	116
B 3.1-21	2	116 01/03/02	B 3.1-68	2	116
B 3.1-22	2	116 05/30/97	B 3.1-69	2	116
B 3.1-23	2	116	B 3.1-70	2	116
B 3.1-24	2	116	B 3.1-71	2	116
B 3.1-25	2	116 05/02/97	B 3.1-72	2	116
B 3.1-26	2	116 07/29/99			

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.1-73	2	116	B 3.3-2	2	116 07/17/97
B 3.1-74	2	116	B 3.3-3	2	116
B 3.1-75	2	116	B 3.3-4	2	116
B 3.1-76	2	116	B 3.3-5	2	116
B 3.1-77	2	116 05/29/97	B 3.3-6	2	116
B 3.1-78	2	116 05/29/97	B 3.3-7	2	116
B 3.1-79	2	116	B 3.3-8	2	116
B 3.1-80	2	116	B 3.3-9	2	116
B 3.2-1	2	116	B 3.3-10	2	116
B 3.2-2	2	116	B 3.3-11	2	116
B 3.2-3	2	116	B 3.3-12	2	116 03/12/99
B 3.2-4	2	183 07/19/04	B 3.3-13	2	116
B 3.2-5	2	116	B 3.3-14	2	116
B 3.2-6	2	116	B 3.3-15	2	116
B 3.2-7	2	116	B 3.3-16	2	116 07/17/97
B 3.2-8	2	116	B 3.3-17	2	116 09/16/03
B 3.2-9	2	116	B 3.3-18	2	116 03/12/99
B 3.2-10	2	116	B 3.3-19	2	116
B 3.2-11	2	116	B 3.3-20	2	116
B 3.2-12	2	183 07/19/04	B 3.3-21	2	116 03/12/99
B 3.2-13	2	116	B 3.3-22	2	116 03/12/99
B 3.2-14	2	116	B 3.3-23	2	116 03/12/99
B 3.2-15	2	116	B 3.3-24	2	116
B 3.2-16	2	116	B 3.3-25	2	116 12/30/96
B 3.2-17	2	116	B 3.3-26	2	116 12/30/96
B 3.2-18	2	116	B 3.3-26a	2	116 12/30/96
B 3.2-19	2	183 07/19/04	B 3.3-27	2	116
B 3.2-20	2	116	B 3.3-28	2	116
B 3.2-21	2	116	B 3.3-29	2	116
B 3.2-22	2	116	B 3.3-30	2	116 05/29/97
B 3.2-23	2	116	B 3.3-31	2	116 05/02/97
B 3.2-24	2	116	B 3.3-32	2	116 03/12/99
B 3.2-25	2	116	B 3.3-33	2	116 03/12/99
B 3.2-26	2	116	B 3.3-34	2	116 03/12/99
B 3.2-27	2	116	B 3.3-35	2	116 03/12/99
B 3.2-28	2	116	B 3.3-36	2	116 03/12/99
B 3.2-29	2	183 07/19/04	B 3.3-37	2	179
B 3.2-30	2	191 11/18/05	B 3.3-37a	2	179 09/16/03
B 3.2-31	2	116	B 3.3-38	2	116
B 3.2-32	2	116	B 3.3-39	2	116 07/17/97
B 3.2-33	2	116	B 3.3-40	2	116 12/09/98
B 3.2-34	2	116	B 3.3-41	2	116 12/09/98
B 3.2-35	2	116	B 3.3-42	2	116
B 3.2-36	2	116	B 3.3-43	2	116
B 3.2-37	2	116	B 3.3-44	2	116 12/30/96
B 3.2-38	2	183 07/19/04	B 3.3-45	2	116 12/30/96
B 3.2-39	2	116	B 3.3-46	2	116 12/30/96
B 3.2-40	2	116	B 3.3-47	2	116 12/30/96
B 3.2-41	2	116	B 3.3-48	2	116 09/18/98
B 3.3-1	2	116	B 3.3-49	2	116 09/18/98

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.3-50	2	116 09/18/98	B 3.3-100	2	188 07/19/05
B 3.3-51	2	116 02/11/02	B 3.3-101	2	116 10/06/99
B 3.3-51a	2	116 09/18/98	B 3.3-102	2	116 10/06/99
B 3.3-52	2	116	B 3.3-103	2	116 10/06/99
B 3.3-53	2	116 07/17/97	B 3.3-103a	2	179
B 3.3-54	2	116	B 3.3-103b	2	148 09/16/03
B 3.3-55	2	116	B 3.3-103c	2	179
B 3.3-56	2	116	B 3.3-104	2	116
B 3.3-57	2	116	B 3.3-105	2	116
B 3.3-58	2	116	B 3.3-106	2	116
B 3.3-59	2	116	B 3.3-107	2	116
B 3.3-60	2	116 09/18/98	B 3.3-108	2	116
B 3.3-61	2	116 09/18/98	B 3.3-109	2	116
B 3.3-62	2	116 09/18/98	B 3.3-110	2	116
B 3.3-63	2	116	B 3.3-111	2	116
B 3.3-64	2	116 07/17/97	B 3.3-112	2	116
B 3.3-65	2	116	B 3.3-113	2	116
B 3.3-66	2	116	B 3.3-114	2	116
B 3.3-67	2	116	B 3.3-115	2	116
B 3.3-68	2	116	B 3.3-116	2	116
B 3.3-69	2	116	B 3.3-117	2	116
B 3.3-70	2	116	B 3.3-118	2	116
B 3.3-71	2	116	B 3.3-119	2	116 12/30/96
B 3.3-72	2	116 12/30/96	B 3.3-120	2	116 12/30/96
B 3.3-73	2	116 12/30/96	B 3.3-121	2	116 12/30/96
B 3.3-74	2	116 12/30/96	B 3.3-122	2	116 12/30/96
B 3.3-75	2	116 05/28/97	B 3.3-123	2	116 05/28/97
B 3.3-76	2	116 12/30/96	B 3.3-124	2	116 05/28/97
B 3.3-77	2	116 07/17/97	B 3.3-125	2	116 05/28/97
B 3.3-78	2	116	B 3.3-126	2	116
B 3.3-79	2	116	B 3.3-127	2	116
B 3.3-80	2	116	B 3.3-128	2	116
B 3.3-81	2	116	B 3.3-129	2	116 09/06/00
B 3.3-82	2	116	B 3.3-130	2	116
B 3.3-83	2	116	B 3.3-131	2	116 09/06/00
B 3.3-84	2	116	B 3.3-132	2	116 05/29/97
B 3.3-85	2	116	B 3.3-133	2	116 07/13/99
B 3.3-86	2	116	B 3.3-134	2	165 12/14/00
B 3.3-87	2	116 02/24/99	B 3.3-135	2	116 05/28/97
B 3.3-88	2	116 07/17/97	B 3.3-136	2	116 06/09/00
B 3.3-89	2	116	B 3.3-137	2	116 05/28/97
B 3.3-90	2	116	B 3.3-138	2	116 05/28/97
B 3.3-91	2	116	B 3.3-139	2	116 05/28/97
B 3.3-92	2	116	B 3.3-140	2	116 05/28/97
B 3.3-93	2	116	B 3.3-141	2	116 05/28/97
B 3.3-94	2	116	B 3.3-142	2	116 05/28/97
B 3.3-95	2	116 10/06/99	B 3.3-143	2	116 05/28/97
B 3.3-96	2	116 10/06/99	B 3.3-144	2	116 05/28/97
B 3.3-97	2	116 10/06/99	B 3.3-145	2	116 06/27/97
B 3.3-98	2	116 07/18/01	B 3.3-146	2	116 03/19/98
B 3.3-99	2	116 10/06/99			

APPENDIX A TO LICENSE NO. NPF-15

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.3-147	2	116 06/27/97	B 3.4-12	3	116
B 3.3-148	2	166 12/20/00	B 3.4-13	3	116
B 3.3-149	2	166 12/20/00	B 3.4-14	3	116
B 3.3-150	2	116 08/20/01	B 3.4-15	3	195
B 3.3-151	2	116 06/27/97	B 3.4-16	3	195
B 3.3-152	2	200 12/05/06	B 3.4-17	3	188 07/19/05
B 3.3-153	2	200 12/05/06	B 3.4-18	3	188 07/19/05
B 3.3-154	2	200 12/05/06	B 3.4-19	3	116
B 3.3-155	2	200 12/05/06	B 3.4-20	3	188 07/19/05
B 3.3-156	2	200 12/05/06	B 3.4-21	3	188 07/19/05
B 3.3-157	2	200 12/05/06	B 3.4-22	3	116
B 3.3-158	2	116 12/17/98	B 3.4-23	3	116
B 3.3-159	2	116 06/08/00	B 3.4-24	3	196
B 3.3-160	2	116 06/23/99	B 3.4-25	3	116
B 3.3-161	2	116 05/30/97	B 3.4-26	3	116
B 3.3-162	2	116 05/30/97	B 3.4-27	3	116
B 3.3-163	2	119 05/30/97	B 3.4-28	3	196
B 3.3-164	2	116 05/30/97	B 3.4-29	3	166 12/20/00
B 3.3-165	2	116 05/30/97	B 3.4-30	3	166 12/20/00
B 3.3-166	2	185 04/25/05	B 3.4-31	3	116
B 3.3-167	2	116 05/30/97	B 3.4-32	3	196
B 3.3-168	2	116 12/02/99	B 3.4-33	3	116
B 3.3-169	2	116 05/30/97	B 3.4-34	3	166 12/20/00
B 3.3-170	2	116	B 3.4-35	3	166 12/20/00
B 3.3-171	2	185 04/25/05	B 3.4-36	3	116
B 3.3-172	2	185 04/25/05	B 3.4-37	3	166 12/20/00
B 3.3-173	2	116 05/30/97	B 3.4-38	3	166 07/15/05
B 3.3-174	2	116 05/30/97	B 3.4-38a	3	166 04/03/06
B 3.3-175	2	116 03/06/06	B 3.4-39	3	196
B 3.3-176	2	116 10/30/98	B 3.4-40	3	166 12/20/00
B 3.3-177	2	116 10/30/98	B 3.4-41	3	116
B 3.3-178	2	116	B 3.4-42	3	116
B 3.3-179	2	116 04/24/01	B 3.4-43	3	166 12/20/00
B 3.3-180	2	116 04/24/01	B 3.4-44	3	166 04/03/06
B 3.3-181	2	116 07/29/96	B 3.4-45	3	166 12/20/00
B 3.3-182	2	166 12/20/00	B 3.4-46	3	116 05/30/00
B 3.3-183	2	166 12/20/00	B 3.4-47	3	146 05/30/00
B 3.3-184	2	166 12/20/00	B 3.4-48	3	152 05/15/02
B 3.3-185	2	166 12/20/00	B 3.4-49	3	116 05/30/00
B 3.4-1	3	116 02/24/99	B 3.4-50	3	116 05/30/00
B 3.4-2	3	116 02/24/99	B 3.4-51	3	147 03/22/01
B 3.4-3	3	116	B 3.4-52	3	147 03/22/01
B 3.4-4	3	116	B 3.4-53	3	147 03/22/01
B 3.4-5	3	116 02/24/99	B 3.4-54	3	147 04/27/05
B 3.4-6	3	116	B 3.4-55	3	195
B 3.4-7	3	116	B 3.4-56	3	195
B 3.4-8	3	116	B 3.4-57	3	195
B 3.4-9	3	195	B 3.4-58	3	195
B 3.4-10	3	116	B 3.4-59	3	195
B 3.4-11	3	116	B 3.4-60	3	195

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.4-61	3	195	B 3.5-11	3	116 03/03/00
B 3.4-62	3	195	B 3.5-12	3	116 03/03/00
B 3.4-63	3	195	B 3.5-13	3	116 03/03/00
B 3.4-64	3	195	B 3.5-14	3	116 03/03/00
B 3.4-65	3	195	B 3.5-15	3	116 03/03/00
B 3.4-66	3	195	B 3.5-16	3	188 07/19/05
B 3.4-67	3	195	B 3.5-17	3	116 03/03/00
B 3.4-68	3	195	B 3.5-18	3	116 03/14/02
B 3.4-69	3	195	B 3.5-19	3	116 03/14/02
B 3.4-70	3	116	B 3.5-20	3	116 07/17/01
B 3.4-71	3	196	B 3.5-20a	3	116 03/03/00
B 3.4-72	3	196	B 3.5-21	3	116
B 3.4-73	3	196	B 3.5-22	3	116
B 3.4-74	3	196	B 3.5-23	3	116
B 3.4-75	3	196	B 3.5-24	3	116
B 3.4-76	3	116	B 3.5-25	3	116
B 3.4-77	3	116	B 3.5-26	3	116
B 3.4-78	3	116	B 3.5-27	3	116
B 3.4-79	3	116	B 3.5-28	3	116
B 3.4-80	3	116 09/06/00	B 3.5-29	3	116
B 3.4-81	3	116 09/06/00	B 3.5-30	3	116
B 3.4-82	3	116	B 3.5-31	3	116
B 3.4-83	3	116	B 3.5-32	3	116
B 3.4-84	3	116	B 3.5-33	3	116
B 3.4-85	3	116	B 3.5-34	3	116
B 3.4-86	3	116	B 3.6-1	3	116 12/01/98
B 3.4-87	3	116	B 3.6-2	3	173 08/06/03
B 3.4-88	3	116 05/13/02	B 3.6-3	3	116 08/06/03
B 3.4-89	3	116 05/13/02	B 3.6-4	3	116 04/29/03
B 3.4-90	3	116	B 3.6-5	3	116 12/01/98
B 3.4-91	3	116	B 3.6-6	3	173 02/11/02
B 3.4-92	3	116	B 3.6-7	3	116
B 3.4-93	3	196	B 3.6-8	3	116
B 3.4-94	3	196	B 3.6-9	3	116
B 3.4-95	3	196	B 3.6-10	3	116
B 3.4-96	3	196	B 3.6-11	3	116 12/01/98
B 3.4-97	3	196	B 3.6-12	3	116 04/29/03
B 3.4-98	3	196	B 3.6-13	3	116 07/29/99
B 3.4-99	3	196	B 3.6-14	3	116
B 3.4-100	3	196	B 3.6-15	3	116
B 3.5-1	3	116	B 3.6-16	3	116 04/29/03
B 3.5-2	3	116	B 3.6-17	3	116
B 3.5-3	3	116	B 3.6-18	3	116 06/09/00
B 3.5-4	3	116 05/05/03	B 3.6-19	3	116
B 3.5-5	3	116 05/30/97	B 3.6-20	3	116
B 3.5-6	3	116	B 3.6-21	3	116 04/29/03
B 3.5-7	3	116 07/15/98	B 3.6-22	3	116 10/24/01
B 3.5-8	3	188 07/19/05	B 3.6-23	3	192 12/12/05
B 3.5-9	3	116 07/15/98	B 3.6-24	3	192 12/12/05
B 3.5-10	3	116 07/15/98	B 3.6-24a	3	192 12/12/05
B 3.5-10a	3	188 07/19/05	B 3.6-25	3	116 04/29/03

APPENDIX A TO LICENSE NO. NPF-15

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date		Page No.	Volume	Amendment/Date	
B 3.6-26	3	116	04/29/03	B 3.7-20	3	116	06/02/04
B 3.6-27	3	173	07/11/02	B 3.7-21	3	116	06/02/04
B 3.6-28	3	173	02/11/02	B 3.7-22	3	116	06/02/04
B 3.6-29	3	116		B 3.7-23	3	116	01/05/01
B 3.6-30	3	116	04/29/03	B 3.7-24	3	116	12/14/00
B 3.6-31	3	173	05/02/02	B 3.7-25	3	116	
B 3.6-32	3	116		B 3.7-26	3	116	02/28/01
B 3.6-33	3	116		B 3.7-27	3	116	02/28/01
B 3.6-34	3	116		B 3.7-28	3	116	03/03/00
B 3.6-35	3	173	02/11/02	B 3.7-29	3	116	03/03/00
B 3.6-36	3	173	02/11/02	B 3.7-30	3	116	03/03/00
B 3.6-37	3	188	07/19/05	B 3.7-31	3	182	11/25/03
B 3.6-38	3	188	07/19/05	B 3.7-32	3	182	11/25/03
B 3.6-39	3	116	10/06/00	B 3.7-33	3	182	11/25/03
B 3.6-40	3	116	10/06/00	B 3.7-34	3	116	03/03/00
B 3.6-41	3	116	10/06/00	B 3.7-35	3	116	03/02/01
B 3.6-42	3	116	04/29/03	B 3.7-36	3	116	07/31/00
B 3.6-43	3	116		B 3.7-37	3	116	08/13/01
B 3.6-44	3	116		B 3.7-38	3	116	08/01/01
B 3.6-45	3	116		B 3.7-39	3	116	06/23/99
B 3.6-46	3	116		B 3.7-40	3	116	
B 3.6-47	3	116		B 3.7-41	3	116	04/03/06
B 3.6-48	3	150	10/29/99	B 3.7-42	3	116	
B 3.6-49	3	150	10/29/99	B 3.7-43	3	116	
B 3.6-50	3	150	10/29/99	B 3.7-43a	3	116	05/29/97
B 3.6-51	3	150	10/29/99	B 3.7-43b	3	116	
B 3.6-52	3	150	10/29/99	B 3.7-43c	3	118	
B 3.6-53	3	150	10/06/00	B 3.7-43d	3	118	07/29/96
B 3.6-54	3	150	10/06/00	B 3.7-43e	3	118	
B 3.6-55	3	150	10/29/99	B 3.7-43f	3	118	
B 3.6-56	3	150	10/29/99	B 3.7-43g	3	118	
B 3.6-57	3	150	10/29/99	B 3.7-43h	3	118	
B 3.7-1	3	204	06/21/07	B 3.7-43i	3	118	
B 3.7-2	3	116	08/18/98	B 3.7-43j	3	118	
B 3.7-3	3	204	06/21/07	B 3.7-44	3	116	
B 3.7-4	3	204	06/21/07	B 3.7-45	3	116	
B 3.7-5	3	204	06/21/07	B 3.7-46	3	116	
B 3.7-6	3	204	06/21/07	B 3.7-47	3	116	05/30/97
B 3.7-7	3	116	04/16/99	B 3.7-48	3	116	
B 3.7-8	3	116	06/23/99	B 3.7-49	3	116	10/17/01
B 3.7-9	3	116		B 3.7-50	3	188	03/07/07
B 3.7-10	3	116		B 3.7-50a	3	188	03/07/07
B 3.7-11	3	116		B 3.7-51	3	116	04/08/07
B 3.7-12	3	116		B 3.7-52	3	116	10/17/01
B 3.7-13	3	116		B 3.7-53	3	116	04/12/01
B 3.7-14	3	116	04/16/99	B 3.7-54	3	116	04/12/01
B 3.7-15	3	116		B 3.7-55	3	116	03/07/07
B 3.7-16	3	116		B 3.7-55a	3	116	03/07/07
B 3.7-17	3	116	06/02/04	B 3.7-56	3	116	12/12/07
B 3.7-18	3	116		B 3.7-57	3	116	12/12/07
B 3.7-19	3	116		B 3.7-58	3	116	12/12/07

APPENDIX A TO LICENSE NO. NPF-15

SAN ONOFRE UNIT 3 TECHNICAL SPECIFICATIONS (NUREG-0952)
LIST OF EFFECTIVE PAGES

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.7-59	3	117 12/12/07	B 3.8-29a	3	160 06/18/01
B 3.7-60	3	116 12/12/07	B 3.8-29b	3	160 06/18/01
B 3.7-61	3	117 12/12/07	B 3.8-29c	3	160 06/18/01
B 3.7-62	3	117 12/12/07	B 3.8-29d	3	160 06/18/01
B 3.7-63	3	200 12/12/07	B 3.8-29e	3	160 06/18/01
B 3.7-64	3	200 12/12/07	B 3.8-29f	3	160 06/18/01
B 3.7-65	3	200 12/12/07	B 3.8-30	3	116
B 3.7-66	3	200 12/12/07	B 3.8-31	3	116
B 3.7-67	3	200 12/12/07	B 3.8-32	3	116
B 3.7-68	3	116 10/13/98	B 3.8-33	3	116
B 3.7-69	3	116 10/13/98	B 3.8-34	3	166 12/20/00
B 3.7-70	3	116 10/13/98	B 3.8-35	3	166 12/20/00
B 3.7-71	3	116 04/29/03	B 3.8-36	3	203 04/05/07
B 3.7-72	3	116	B 3.8-37	3	116 01/21/98
B 3.7-73	3	116	B 3.8-38	3	203 04/05/07
B 3.7-74	3	116 04/29/03	B 3.8-39	3	203 04/05/07
B 3.7-75	3	116 04/29/03	B 3.8-40	3	116 01/21/98
B 3.7-76	3	116 05/13/02	B 3.8-41	3	203 04/05/07
B 3.7-77	3	116	B 3.8-42	3	116 03/25/02
B 3.7-78	3	116	B 3.8-43	3	116
B 3.7-79	3	116	B 3.8-44	3	116 03/25/02
B 3.8-1	3	116 10/06/99	B 3.8-45	3	116
B 3.8-2	3	116 10/06/99	B 3.8-46	3	116 05/03/00
B 3.8-3	3	116 10/06/99	B 3.8-47	3	116 01/22/99
B 3.8-4	3	116 10/06/99	B 3.8-47a	3	116 01/22/99
B 3.8-5	3	116 10/06/99	B 3.8-48	3	116
B 3.8-6	3	116 10/06/99	B 3.8-49	3	116 01/22/99
B 3.8-7	3	188 07/19/05	B 3.8-50	3	116
B 3.8-7a	3	116 12/10/99	B 3.8-51	3	116
B 3.8-8	3	116 10/06/99	B 3.8-52	3	116
B 3.8-9	3	116 11/29/07	B 3.8-53	3	116 01/22/99
B 3.8-10	3	188 07/19/05	B 3.8-54	3	116 08/05/98
B 3.8-11	3	116 10/06/99	B 3.8-55	3	116 08/30/04
B 3.8-12	3	116 10/06/99	B 3.8-55a	3	116 08/05/98
B 3.8-13	3	116 10/06/99	B 3.8-56	3	116
B 3.8-14	3	116 10/06/99	B 3.8-57	3	116
B 3.8-15	3	116 10/06/99	B 3.8-58	3	166 12/20/00
B 3.8-16	3	116 06/18/01	B 3.8-59	3	166 12/20/00
B 3.8-17	3	116 06/18/01	B 3.8-60	3	116
B 3.8-18	3	203 04/05/07	B 3.8-61	3	116
B 3.8-19	3	116 06/18/01	B 3.8-62	3	116
B 3.8-20	3	116 11/29/07	B 3.8-63	3	116
B 3.8-21	3	116 06/18/01	B 3.8-64	3	116
B 3.8-22	3	116 06/18/01	B 3.8-65	3	116
B 3.8-23	3	116 06/18/01	B 3.8-66	3	116 07/31/00
B 3.8-24	3	116 06/18/01	B 3.8-67	3	116
B 3.8-25	3	116 06/18/01	B 3.8-68	3	116
B 3.8-26	3	116 06/18/01	B 3.8-69	3	116
B 3.8-27	3	116 06/18/01	B 3.8-70	3	116
B 3.8-28	3	116 06/18/01	B 3.8-71	3	116
B 3.8-29	3	116 06/18/01			

Page No.	Volume	Amendment/Date	Page No.	Volume	Amendment/Date
B 3.8-72	3	116			
B 3.8-73	3	166 12/20/00			
B 3.8-74	3	166 12/20/00			
B 3.8-75	3	116			
B 3.8-76	3	116			
B 3.8-77	3	116			
B 3.8-78	3	116			
B 3.8-79	3	116			
B 3.8-80	3	116			
B 3.8-81	3	116			
B 3.8-82	3	116			
B 3.8-83	3	116 01/11/07			
B 3.8-84	3	116			
B 3.8-85	3	116			
B 3.8-86	3	166 12/20/00			
B 3.8-87	3	166 12/20/00			
B 3.9-1	3	116			
B 3.9-2	3	116			
B 3.9-3	3	166 12/20/00			
B 3.9-4	3	116			
B 3.9-5	3	116			
B 3.9-6	3	166 12/20/00			
B 3.9-7	3	116			
B 3.9-8	3	116			
B 3.9-9	3	116 12/01/98			
B 3.9-10	3	184 02/18/05			
B 3.9-11	3	116 10/10/97			
B 3.9-12	3	116 10/10/97			
B 3.9-13	3	184 02/18/05			
B 3.9-13a	3	184 02/18/05			
B 3.9-14	3	116 10/10/97			
B 3.9-15	3	116 10/10/97			
B 3.9-16	3	116			
B 3.9-17	3	116 07/15/05			
B 3.9-17a	3	116 04/03/06			
B 3.9-18	3	166 07/15/05			
B 3.9-19	3	166 12/20/00			
B 3.9-20	3	116			
B 3.9-21	3	116 05/30/97			
B 3.9-22	3	116 07/15/05			
B 3.9-23	3	116 04/03/06			
B 3.9-24	3	166 12/20/00			
B 3.9-24a	3	166 12/20/00			
B 3.9-25	3	116 05/30/97			
B 3.9-26	3	116 10/13/98			
B 3.9-27	3	116 10/13/98			

SAN ONOFRE UNIT 3
FACILITY OPERATING LICENSE

LIST OF EFFECTIVE PAGES

Letters Modifying the License

NRC Letter dated February 25, 2002 from Samuel J. Collins (NRC) to Harold B. Ray (SCE), SUBJECT: ISSUANCE OF ORDER FOR INTERIM SAFEGUARDS AND SECURITY COMPENSATORY MEASURES FOR - SAN ONOFRE NUCLEAR STATION, UNITS 2 & 3

NRC Letter dated January 7, 2003 from Samuel J. Collins (NRC) to Harold B. Ray (SCE), SUBJECT: ISSUANCE OF ORDER FOR COMPENSATORY MEASURES RELATED TO ACCESS AUTHORIZATION

NRC Letter dated February 11, 2003 from Samuel J. Collins (NRC) to Harold B. Ray (SCE), SUBJECT: ISSUANCE OF ORDER ESTABLISHING INTERIM INSPECTION REQUIREMENTS FOR REACTOR PRESSURE VESSEL HEADS AT PRESSURIZED WATER REACTORS

NRC Letter dated April 29, 2003 from Samuel J. Collins (NRC) to Holders of Licenses for Operating Power Reactors as listed in Attachment 1 to the Order (John Todd (SCE)), SUBJECT: ISSUANCE OF ORDER REQUIRING COMPLIANCE WITH REVISED DESIGN BASIS THREAT FOR OPERATING POWER REACTORS

NRC Letter dated April 29, 2003 from Samuel J. Collins (NRC) to Holders of Licenses for Operating Power Reactors as listed in Attachment 1 to the Order (John Todd (SCE)), SUBJECT: ISSUANCE OF ORDER FOR COMPENSATORY MEASURES RELATED TO TRAINING ENHANCEMENTS ON TACTICAL AND FIREARMS PROFICIENCY AND PHYSICAL FITNESS APPLICABLE TO ARMED NUCLEAR POWER PLANT SECURITY FORCE PERSONNEL

NRC Letter dated April 29, 2003 from Samuel J. Collins (NRC) to Holders of Licenses for Operating Power Reactors as Listed in Enclosure 2 (John Todd (SCE)), SUBJECT: ISSUANCE OF ORDER FOR COMPENSATORY MEASURES RELATED TO FITNESS-FOR-DUTY ENHANCEMENTS APPLICABLE TO NUCLEAR FACILITY SECURITY FORCE PERSONNEL

NRC Letter dated March 20, 2006 from J. E. Dyer (NRC) to Holders of Licenses for Operating Power Reactors listed in Attachment 1 to the Order (Richard M. Rosenblum (SCE)), SUBJECT: ISSUANCE OF ORDER REQUIRING COMPLIANCE WITH UPDATED ADVERSARY CHARACTERISTIC

SAN ONOFRE UNIT 3
FACILITY OPERATING LICENSE

LIST OF EFFECTIVE PAGES

Facility Operating License

Page No.	Amendment No.
1	176
2	201
3	207
4	176
5	176
6*	176
7*	206
8*	206

* Revised by NRC to SCE letter dated July 26, 2007

(3) SCE, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

(4) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear materials as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(5) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of San Onofre Nuclear Generating Station, Units 1 and 3 and by the decommissioning of San Onofre Nuclear Generating Station Unit 1.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Southern California Edison Company (SCE) is authorized to operate the facility at reactor core power levels not in excess of full power (3438 megawatts thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 207, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(27) Mitigation Strategy License Condition

Develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

(a) Fire fighting response strategy with the following elements:
1. Pre-defined coordinated fire response strategy and guidance
2. Assessment of mutual aid fire fighting assets
3. Designated staging areas for equipment and materials
4. Command and control
5. Training of response personnel

(b) Operations to mitigate fuel damage considering the following:
1. Protection and use of personnel assets
2. Communications
3. Minimizing fire spread
4. Procedures for implementing integrated fire response strategy
5. Identification of readily-available pre-staged equipment
6. Training on integrated fire response strategy
7. Spent fuel pool mitigation measures

(c) Actions to minimize release to include consideration of:
1. Water spray scrubbing
2. Dose to onsite responders

(28) Upon implementation of Amendment No. 206 adopting TSTF-448, Revision 3, the determination of control room envelope (CRE) unfiltered air inleakage as required by SR 3.7.11.4, in accordance with TS 5.5.2.16.c(i), the assessment of CRE habitability as required by Specification 5.5.2.16.c(ii), and the measurement of CRE pressure as required by Specification 5.5.2.16.d, shall be considered met. Following implementation:

(a) The first performance of SR 3.7.11.4, in accordance with Specification 5.5.2.16.c(i), shall be within the specified frequency of 6 years, plus the 18-month allowance of SR 3.0.2, as measured from May 18, 2004, the date of the most recent successful tracer gas test, as stated in the September 17, 2004 letter response to Generic Letter 2003-01, or within the next 18 months if the time period since the most recent successful tracer gas test is greater than 6 years.

(b) The first performance of the periodic assessment of CRE habitability, Specification 5.5.2.16.c(ii), shall be within 3 years, plus the 9-month allowance of SR 3.0.2, as measured from May 18, 2004, the date of the most recent successful tracer gas test, as stated in the September 17, 2004, letter response to Generic letter 2003-01, or within the next 9 months if the time period since the most recent successful tracer gas is greater than 3 years.

(c) The first performance of the periodic measurement of CRE pressure, Specification 5.5.2.16.d, shall be within 6 months.

D. Exemptions to certain requirements of Appendices G, H and J to 10 CFR Part 50 are described in the Office of Nuclear Reactor Regulation's Safety Evaluation

Report. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. Therefore, these exemptions are hereby granted. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission.

E. SCE shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21 is entitled: "San Onofre Nuclear Generating Station Security, Training and Qualification, and Safeguards Contingency Plan, Revision 0" submitted by letter dated October 19, 2004.

F. This license is subject to the following additional condition for the protection of the environment:

Before engaging in activities that may result in a significant adverse environmental impact that was not evaluated or that is significantly greater than that evaluated in the Final Environmental Statement, SCE shall provide a written notification of such activities to the NRC Office of Nuclear Reactor Regulation and receive written approval from that office before proceeding with such activities.

G. DELETED

H. SCE shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.

I. SCE shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

J. This license is effective as of the date of issuance and shall expire at midnight on November 15, 2022.

K. Deleted by Amendment No. 176

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by
Harold R. Denton

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Attachments:
1. Attachment 1 - Deleted by Amendment No. 176
2. Appendix A (Technical Specifications)
3. Appendix B (Environmental Protection Plan)
4. Appendix C (Antitrust Conditions)

Date of Issuance: NOV 15 1982

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* On September 29, 1983, the Safeguards Contingency Plan was made a separate, companion document to the Physical Security Plan pursuant to the authority of 10 CFR 50.54.

3.7 PLANT SYSTEMS

3.7.11 Control Room Emergency Air Cleanup System (CREACUS)

LCO 3.7.11 Two CREACUS trains shall be OPERABLE.

---NOTE---
The control room envelope (CRE) boundary may be opened intermittently under administrative control.

APPLICABILITY: MODES 1, 2, 3, 4, 5, and 6,
During movement of irradiated fuel assemblies.

ACTIONS

---NOTES---
1. The provisions of LCO 3.0.4 are not applicable when entering MODES 5, 6, or defueled configuration.

2. Each Unit shall enter applicable ACTIONS separately.

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One CREACUS train inoperable for reasons other than Condition B..	A.1	Restore CREACUS train to OPERABLE status.	14 days
B. One or more CREACUS trains inoperable due to inoperable CRE boundary in Modes 1, 2, 3, or 4.	B.1	Initiate action to implement mitigating actions.	Immediately
	AND		
	B.2	Verify mitigating actions ensure CRE occupant exposures to radiological, chemical, and smoke hazards will not exceed limits.	24 hours
	AND		
	B.3	Restore CRE boundary to OPERABLE status.	90 days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
C. Required Action and associated Completion Time of Condition A or B not met in MODE 1, 2, 3, or 4.	C.1	Be in MODE 3.	6 hours
	AND		
	C.2	Be in MODE 5.	36 hours
D. Required Action and associated Completion Time of Condition A not met in MODE 5 or 6, or during movement of irradiated fuel assemblies.	D.1	Place OPERABLE CREACUS train in emergency radiation protection mode.	Immediately
	OR		
	D.2.1	Suspend CORE ALTERATIONS.	Immediately
	AND		
	D.2.2	Suspend movement of irradiated fuel assemblies.	Immediately
E. Two CREACUS trains inoperable in MODE 5 or 6, or during movement of irradiated fuel assemblies.	E.1	Suspend CORE ALTERATIONS.	Immediately
	AND		
	E.2	Suspend movement of irradiated fuel assemblies.	Immediately
OR			
One or more CREACUS trains inoperable due to inoperable CRE boundary in MODE 5 or 6, or during movement of irradiated fuel assemblies.			

(continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
F. Two CREACUS trains inoperable in MODE 1, 2, 3, or 4 for reasons other than Condition B).	F.1 Enter LCO 3.0.3.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.11.1	Operate each CREACUS train for ≥ 2 hours.	31 days
SR 3.7.11.2	Perform required CREACUS filter testing in accordance with the Ventilation Filter Testing Program (VFTP).	In accordance with the VFTP
SR 3.7.11.3	Verify each CREACUS train actuates on an actual or simulated actuation signal.	24 months
SR 3.7.11.4	Perform required CRE unfiltered air inleakage testing in accordance with the Control Room Envelope Habitability Program.	In accordance with the Control Room Envelope Habitability Program

5.5 Procedures, Programs, and Manuals (continued)

5.5.2.11 Steam Generator (SG) Program (continued)

c) For tubes that have been repaired in the hot leg tubesheet region: Below the bottom of the lower sleeve-to-tube joint or greater than 10.6 inches below the bottom of the hot leg expansion transition or greater than 10.6 inches below the top of the hot leg tubesheet, whichever of these three is lowest.

d) For tubes that have been repaired in the cold leg tubesheet region: Below the bottom of the lower sleeve-to-tube joint or greater than 11.0 inches below the bottom of the cold leg expansion transition or greater than 11.0 inches below the top of the cold leg tubesheet, whichever of these three is lowest.

d. Provisions for SG tube inspections. Periodic SG tube inspections shall be performed. The number and portions of the tubes inspected and methods of inspection shall be performed with the objective of detecting flaws of any type (e.g., volumetric flaws, axial and circumferential cracks) that may be present along the length of the tube, from the tube-to-tubesheet weld at the tube inlet to the tube-to-tubesheet weld at the tube outlet, and that may satisfy the applicable tube repair criteria. The tube-to-tubesheet weld is not part of the tube. In tubes repaired by sleeving, the portion of the original tube wall between the sleeve's joints is not an area requiring re-inspection.

----------NOTE----------
The requirement for methods of inspection with the objective of detecting flaws of any type (e.g., volumetric flaws, axial and circumferential cracks) that may be present along the length of the tube does not apply to the portion of the original tube wall adjacent to the nickel band portion (the lower half) of the lower joint for the repair process that is discussed in Technical Specification (TS) 5.5.2.11.f.1. However, the method of inspection in this area should be a rotating plus point (or equivalent) coil. The SG tube repair criterion of TS 5.5.2.11.c.3.b is applicable to flaws in this area.

In addition to meeting the requirements of d.1, d.2, d.3, and d.4 below, the inspection scope, inspection methods, and inspection intervals shall be such as to ensure that SG tube integrity is maintained until the next SG inspection. An assessment of degradation shall be performed to determine the

(continued)

5.5 Procedures, Programs, and Manuals (continued)

5.5.2.11 Steam Generator (SG) Program (continued)

type and location of flaws to which the tubes may be susceptible and, based on this assessment, to determine which inspection methods need to be employed and at what locations.

1. Inspect 100% of the tubes in each SG during the first refueling outage following SG replacement.

2. Inspect 100% of the tubes at sequential periods of 60 effective full power months. The first sequential period shall be considered to begin after the first inservice inspection of the SGs. No SG shall operate for more than 24 effective full power months or one refueling outage (whichever is less) without being inspected.

3. If crack indications are found in any SG tube, then the next inspection for each SG for the degradation mechanism that caused the crack indication shall not exceed 24 effective full power months or one refueling outage (whichever is less). If definitive information, such as from examination of a pulled tube, diagnostic non-destructive testing, or engineering evaluation indicates that a crack-like indication is not associated with a crack(s), then the indication need not be treated as a crack.

4. All sleeves shall be inspected with eddy current prior to initial operation. This includes pressure retaining portions of the parent tube in contact with the sleeve, the sleeve-to-tube weld and the pressure retaining portion of the sleeve.

e. Provisions for monitoring operational primary to secondary LEAKAGE.

f. Provisions for SG tube repair methods. Steam generator tube repair methods shall provide the means to re-establish the RCS pressure boundary integrity of SG tubes without removing the tube from service. For the purposes of these Specifications, tube plugging is not a repair. All acceptable tube repair methods are listed below.

1. TIG welded sleeving with heat treatment, as described in ABB/CE Topical Report, CEN-630-P, Rev. 2, is currently approved by the NRC until December 2010. All sleeves shall be removed from service by December 2010.

Tube repair can be performed on certain tubes that have been previously plugged as a corrective or preventive measure. A tube inspection of the entire length of the tube shall be performed on a previously plugged tube prior to returning the tube to service.

(continued)

5.5 Procedures, Programs, and Manuals (continued)

5.5.2.15 Containment Leakage Rate Testing Program (Continued)

The provisions of Surveillance Requirement 3.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program. However, test frequencies specified in this Program may be extended consistent with the guidance provided in NEI 94-01, "Industry Guideline For Implementing Performance-Based Option Of 10CFR 50, Appendix J," as endorsed by Regulatory Guide 1.163. Specifically, NEI 94-01 has these provisions for test frequencies extension:

1. Consistent with standard scheduling practices for Technical Specifications Required Surveillances, intervals for recommended Type A testing may be extended by up to 15 months. This option should be used only in cases where refueling schedules have been changed to accommodate other factors.

2. Consistent with standard scheduling practices for Technical Specifications Required Surveillances, intervals for the recommended surveillance frequency for Type B and Type C testing may be extended by up to 25 percent of the test interval, not to exceed 15 months.

The provisions of Surveillance Requirement 3.0.3 are applicable to the Containment Leakage Rate Testing Program.

5.5.2.16 Control Room Envelope Habitability Program

A Control Room Envelope Habitability Program shall be established and implemented to ensure that CRE habitability is maintained such that, with an OPERABLE Control Room Emergency Air Cleanup System (CREACUS), CRE occupants can control the reactor safely under normal conditions and maintain it in a safe condition following a radiological event, hazardous chemical release, or a smoke challenge. The program shall ensure that adequate radiation protection is provided to permit access and occupancy of the CRE under design basis accident (DBA) conditions without personnel receiving radiation exposures in excess of 5 rem total effective dose equivalent (TEDE) for the duration of the accident. The program shall include the following elements:

a. The definition of the CRE and the CRE boundary.

b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.

(continued)

5.5 Procedures, Programs, and Manuals (continued)

5.5.2.16 Control Room Envelope Habitability Program (Continued)

c. Requirements for (i) determining the unfiltered air inleakage past the CRE boundary into the CRE in accordance with the testing methods and at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.

The following is an exception to Sections C.1 and C.2 of regulatory Guide 1.197, Revision 0:

Appropriate application of ASTM E-741 shall include the ability to take minor exceptions to the test methodology. These exceptions shall be documented in the test report.

d. Measurement, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one train of the CREACUS, operating at the flow rate required by the VFTP, at a Frequency of 24 months on a STAGGERED TEST BASIS. The results shall be trended and used as part of the 24 month assessment of the CRE boundary.

e. The quantitative limits on unfiltered air inleakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air inleakage measured by the testing described in paragraph c. The unfiltered air inleakage limit for radiological challenges is the inleakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air inleakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.

f. The provisions of SR 3.0.2 are applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered inleakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

B 3.7 PLANT SYSTEMS

B 3.7.11 Control Room Emergency Air Cleanup System (CREACUS)

BASES

BACKGROUND

The CREACUS provides a protected environment from which occupants can control the plant following an uncontrolled release of radioactivity, hazardous chemicals, or smoke.

The CREACUS consists of two independent, redundant trains that recirculate and filter the air in the control room envelope (CRE) and a CRE boundary that limits the inleakage of unfiltered air. Each CREACUS train consists of emergency air conditioning unit, emergency ventilation air supply unit, emergency isolation dampers, and cooling coils and two cabinet coolers per Unit. Each emergency air conditioning unit includes a prefilter, a high efficiency particulate air (HEPA) filter, an activated carbon adsorber section for removal of gaseous activity (principally iodines), and a fan. A second bank of HEPA filters follows the adsorber section to collect carbon fines. Each emergency ventilation air supply unit includes prefilter, HEPA filter, carbon adsorber and fan. Ductwork, motor-operated dampers, doors, barriers, and instrumentation also form part of the system. Air and motor-operated dampers are provided for air volume control and system isolation purposes.

The CRE is the area within the confines of the CRE boundary that contains the spaces that control room occupants inhabit to control the unit during normal and accident conditions. This area encompasses the control room, and may encompass other non-critical areas to which frequent personnel access or continuous occupancy is not necessary in the event of an accident. The CRE is protected during normal operation, natural events, and accident conditions. The CRE boundary is the combination of walls, floor, roof, ducting, doors, penetrations and equipment that physically form the CRE. The OPERABILITY of the CRE boundary must be maintained to ensure that the inleakage of unfiltered air into the CRE will not exceed the inleakage assumed in the licensing basis analyses of design basis accident (DBA) consequences to CRE occupants. The CRE and its boundary are defined in the Control Room Envelope Habitability Program.

(continued)

BASES (continued)

BACKGROUND (continued)

Upon receipt of the actuating signal, normal air supply to the CRE is isolated, and the stream of ventilation air is recirculated through the system's filter trains. The prefilters remove any large particles in the air to prevent excessive loading of the HEPA filters and charcoal adsorbers. Continuous operation of each train for at least 15 minutes per month verifies proper system operability.

There are two CREACUS operational modes. Emergency mode is an operational mode when the control room is isolated to protect operational personnel from radioactive exposure through the duration of any one of the postulated limiting faults discussed in Chapter 15 UFSAR (Ref. 2). Isolation mode is an operational mode when the CRE is isolated to protect operational personnel from toxic gasses and smoke.

Actuation of the CREACUS places the system into either of two separate states of operation, depending on the initiation signal. Actuation of the system to either the emergency mode or isolation mode of CREACUS operation closes the unfiltered-outside-air intake and unfiltered exhaust dampers, and aligns the system for recirculation of air within the CRE through the redundant trains of HEPA and charcoal filters.

The emergency mode also initiates pressurization of the CRE. Outside air is added to the air being recirculated from the CRE. Pressurization of the CRE minimizes infiltration of unfiltered air through the CRE boundary from all the surrounding areas adjacent to the CRE boundary.

The CRE supply and the outside air supply of the normal control room HVAC are monitored by radiation and toxic-gas detectors respectively. One detector output above the setpoint will cause actuation of the emergency mode or isolation mode as required. The actions of the isolation mode are more restrictive, and will override the actions of the emergency mode of operation. However, toxic gas and radiation events are not considered to occur concurrently.

(continued)

BASES (continued)

BACKGROUND (continued)

Redundant recirculation trains provide the required filtration should an excessive pressure drop develop across the other filter train. Normally-open isolation dampers are arranged in series pairs so that one damper's failure to shut will not result in a breach of isolation. The CREACUS is designed in accordance with Seismic Category I requirements.

The CREACUS is designed to maintain a habitable environment in the CRE for 30 days of continuous occupancy after a Design Basis Accident (DBA) without exceeding a 5-rem total effective dose equivalent (TEDE).

APPLICABLE SAFETY ANALYSES

The CREACUS components are arranged in redundant, safety related ventilation trains. The location of components and ducting within the CRE ensures an adequate supply of filtered air to all areas requiring access.

The CREACUS provides airborne radiological protection for the CRE occupants, as demonstrated by the CRE occupant dose analyses for the most limiting design basis loss of coolant accident fission product release presented in the UFSAR, Chapter 15 (Ref. 2).

Dose calculations, as specified in Unit 2/3 UFSAR Chapter 15 (Reference 2), only take credit for the HEPA filters and charcoal adsorbers of the emergency recirculation air conditioning unit. The emergency ventilation supply unit is designed to contribute to the pressurization of the control room to minimize unfiltered inleakage as indicated in Unit 2/3 UFSAR.

(continued)

BASES (continued)

APPLICABLE SAFETY ANALYSES (continued)

The CREACUS provides protection from smoke and hazardous chemicals to the control room occupants. The analysis of hazardous chemical releases demonstrates that the toxicity limits are not exceeded in the CRE following a hazardous chemical release (Ref. 3). The evaluation of a smoke challenge demonstrates that it will not result in the inability of the CRE occupants to control the reactor either from the control room or from the remote shutdown panels (Ref. 4)

The worst case single active failure of a component of the CREACUS, assuming a loss of offsite power, does not impair the ability of the system to perform its design function.

The CREACUS satisfies Criterion 3 of the NRC Policy Statement.

LCO

Two independent and redundant trains of the CREACUS are required to be OPERABLE to ensure that at least one is available if a single active failure disables the other train. Total system failure, such as from a loss of both ventilation trains or from an inoperable CRE boundary, could result in exceeding a dose of 5 rem TEDE in the event of a large radioactive release.

Each CREACUS train is considered OPERABLE when the individual components necessary to limit CRE occupant exposure are OPERABLE. A CREACUS train is considered OPERABLE when the associated:

a. Fan is OPERABLE;

b. HEPA filters and charcoal adsorber are not excessively restricting flow, and are capable of performing their filtration functions; and

(continued)

BASES (continued)

LCO (continued)

c. Ductwork, valves, and dampers are OPERABLE, and air circulation can be maintained. If an Emergency Isolation Damper is stuck open, the associated train of CREACUS may still be considered OPERABLE if the redundant damper in series with the inoperable damper is closed with power removed.

In order for the CREACUS trains to be considered OPERABLE, the CRE boundary must be maintained such that CRE occupant dose from a large radioactive release does not exceed the calculated dose in the licensing basis consequence analyses for DBAs, and that CRE occupants are protected from hazardous chemicals and smoke.

The LCO is modified by a Note allowing the CRE boundary to be opened intermittently under administrative controls. This Note only applies to openings in the CRE boundary that can be rapidly restored to the design condition, such as doors, hatches, floor plugs, and access panels. For entry and exit through doors the administrative control of the opening is performed by the person(s) entering or exiting the area. For other openings, these controls should be proceduralized and consist of stationing a dedicated individual at the opening who is in continuous communication with the operators in the CRE. This individual will have a method to rapidly close the opening and to restore the CRE boundary to a condition equivalent to the design condition when a need for CRE isolation is indicated.

(continued)

BASES (continued)

APPLICABILITY

In MODES 1, 2, 3, 4, 5, and 6 and during movement of irradiated fuel assemblies the CREACUS must be OPERABLE to ensure that the CRE will remain habitable during and following a DBA.

In MODES 5 and 6, the CREACUS is required to cope with the release from a rupture of a waste gas tank.

During movement of irradiated fuel assemblies, the CREACUS must be OPERABLE to cope with the release from a fuel handling accident involving handling irradiated fuel.

ACTIONS

ACTION statements are modified by two NOTES. NOTE 1 says: "The provisions of LCO 3.0.4 are not applicable when entering MODES 5, 6, or defueled configuration."

Specification 3.0.4 establishes that entry into an operational mode or other specified condition shall not be made unless the conditions of the LCO are met. Applicability statement "During movement of irradiated fuel assemblies" ensures the OPERABILITY of both CREACUS trains prior to the start of movement of irradiated fuel assemblies.

NOTE 2 says: "Each Unit shall enter applicable ACTIONS separately." CREACUS is a shared system between Unit 2 and Unit 3. LCO doesn't address the operational situation when the Units are in different operational MODES. Without this NOTE it may not be clear what ACTIONS should be taken.

(continued)

BASES (continued)

ACTIONS (continued)

A.1

With one CREACUS train inoperable, for reasons other than an inoperable CRE boundary, action must be taken to restore OPERABLE status within 14 days. The 14 day AOT is based on a probabilistic risk assessment that does not require administrative controls to be implemented when a CREACUS train is taken out of service. In this Condition, the remaining OPERABLE CREACUS train is adequate to perform the CRE occupant protection function.

However, the overall reliability is reduced because a failure in the OPERABLE CREACUS train could result in loss of CREACUS function. The 14 day Completion Time is based on the low probability of a DBA occurring during this time period, and the ability of the remaining train to provide the required capability.

B.1, B.2, and B.3

If the unfiltered inleakage of potentially contaminated air past the CRE boundary and into the CRE can result in CRE occupant radiological dose greater than the calculated dose of the licensing basis analyses of DBA consequences (allowed to be up to 5 rem TEDE), or inadequate protection of CRE occupants from hazardous chemicals or smoke, the CRE boundary is inoperable. Actions must be taken to restore an OPERABLE CRE boundary within 90 days.

(continued)

BASES (continued)

ACTIONS (continued)

B.1, B.2, B.3

During the period that the CRE boundary is considered inoperable, action must be initiated to implement mitigating actions to lessen the effect on CRE occupants from the potential hazards of a radiological or chemical event or a challenge from smoke. Actions must be taken within 24 hours to verify that in the event of a DBA, the mitigating actions will ensure that CRE occupant radiological exposures will not exceed the calculated dose of the licensing basis analyses of DBA consequences, and the CRE occupants are protected from hazardous chemicals and smoke. These mitigating actions (i.e., actions that are taken to offset the consequences of the inoperable CRE boundary) should be preplanned for implementation upon entry into the condition, regardless of whether entry is intentional or unintentional. The 24 hour Completion Time is reasonable based on the low probability of a DBA occurring during this time period, and the use of mitigating actions. The 90 day Completion Time is reasonable based on the determination that the mitigating actions will ensure protection of CRE occupants within analyzed limits while limiting the probability that CRE occupants will have to implement protective measures that may adversely affect their ability to control the reactor and maintain it in a safe shutdown condition in the event of a DBA. In addition, the 90 day Completion Time is a reasonable time to diagnose, plan and possibly repair, and test most problems with the CRE boundary.

C.1 and C.2

In MODES 1, 2, 3, or 4, if the inoperable CREACUS or the CRE boundary cannot be restored to OPERABLE status within the required Completion Time, the unit must be placed in a MODE that minimizes the accident risk. To achieve this status, the unit must be placed in at least MODE 3 within 6 hours, and in MODE 5 within 36 hours. The allowed Completion Times are reasonable, based on operating experience, to reach the required unit conditions from full power conditions in an orderly manner and without challenging unit systems.

(continued)

BASES (continued)

ACTIONS (continued)

D.1, D.2.1, and D.2.2

In MODE 5 or 6, or during movement of irradiated fuel assemblies, if Required Action A.1 cannot be completed within the required Completion Time, the OPERABLE CREACUS train must be immediately placed in the emergency mode of operation. This action ensures that the remaining train is OPERABLE, that no failures preventing automatic actuation will occur, and that any active failure will be readily detected.

An alternative to Required Action D.1 is to immediately suspend activities that could result in a release of radioactivity that might require isolation of the CRE. This places the unit in a condition that minimizes the accident risk. This does not preclude the movement of fuel assemblies to a safe position.

E.1 and E.2

When in MODES 5 or 6, or during movement of irradiated fuel assemblies with two trains inoperable or with one or more CREACUS trains inoperable due to an inoperable CRE boundary, action must be taken immediately to suspend activities that could result in a release of radioactivity that might enter the CRE. This places the unit in a condition that minimizes the accident risk. This does not preclude the movement of fuel to a safe position.

F.1

If both CREACUS trains are inoperable in MODE 1, 2, 3, or 4 for reasons other than an inoperable CRE boundary (i.e., Condition B), the CREACUS may not be capable of performing the intended function and the unit is in a condition outside the accident analyses. Therefore, LCO 3.0.3 must be entered immediately.

(continued)

BASES (continued)

SURVEILLANCE REQUIREMENTS

SR 3.7.11.1

Standby systems should be checked periodically to ensure that they function properly. Since the environment and normal operating conditions on this system are not severe, testing each train once every month provides an adequate check on this system.

Cumulative operation of the system for at least 2 hours over a 31 day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filtes. The 2 hour time frame is based on a conservative engineering evaluation which calculated the time required to evaporate the moisture contained in the air trapped inside the CREACUS duct upstream of charcoal beds. The 31 day Frequency is based on the known reliability of the equipment, and the two train redundancy available.

SR 3.7.11.2

This SR verifies that the required CREACUS testing is performed in accordance with the Ventilation Filter Testing Program (VFTP). The CREACUS filter tests are based on Regulatory Guide 1.52 (Ref. 5). The VFTP includes testing HEPA filter performance, charcoal adsorber efficiency, minimum system flow rate, and the physical properties of the activated charcoal (general use and following specific operations). Specific test Frequencies and additional information are discussed in detail in the VFTP.

The filtration efficiency only apply to the emergency recirculation air conditioning units E418 and E419. Therefore, testing for filtration efficiency is not required for the emergency ventilation supply units A206 and A207.

However, the specified air flow from the emergency ventilation units is required during the filtration efficiency testing of the emergency recirculation air conditioning units. Also, the air flow requirements which are specified in the VFTP apply to the emergency ventilation and emergency air conditioning units.

(continued)

BASES (continued)

SURVEILLANCE REQUIREMENTS (continued)

SR 3.7.11.3

This SR verifies that each CREACUS train starts and operates on an acutual or simulated actuation signal. The Frequency of 24 months is based on industry operating experience and is consistent with the typical refueling cycle.

SR 3.7.11.4

This SR verifies the OPERABILITY of the CRE boundary by testing for unfiltered air inleakage past the CRE boundary and into the CRE. The details of the testing are specified in the Control Room Envelope Habitability Program.

The CRE is considered habitable when the radiological dose to CRE occupants calculated in the licensing basis analyses of DBA consequences is no more than 5 rem TEDE and the CRE occupants are protected from hazardous chemicals and smoke. This SR verifies that the unfiltered air inleakage into the CRE is no greater than the flow rate assumed in the licensing basis analyses of DBA consequences. When unfiltered air inleakage is greater than the assumed flow rate, Condition B must be entered. Required Action B.3 allows time to restore the CRE boundary to OPERABLE status provided mitigating actions can ensure that the CRE remains within the licensing basis habitability limits for the occupants following an accident. Compensatory measures are discussed in Regulatory Guide 1.196, Section C.2.7.3, (Ref. 6) which endorses, with exceptions, NEI 99-03, Section

(continued)

BASES (continued)

SURVEILLANCE REQUIREMENTS (continued)

SR 3.7.11.4

8.4 and Appendix F (Ref. 7). These compensatory measures may also be used as mitigating actions as required by Required Action B.2. Temporary analytical methods may also be used as compensatory measures to restore OPERABILITY (Ref. 8). Options for restoring the CRE boundary to OPERABLE status include changing the licensing basis DBA consequence analysis, repairing the CRE boundary, or a combination of these actions. Depending upon the nature of the problem and the corrective action, a full scope inleakage test may not be necessary to establish that the CRE boundary has been restored to OPERABLE status.

REFERENCES

1. UFSAR, Section 9.4.

2. UFSAR, Chapter 15.

3. UFSAR, Section 6.4.

4. UFSAR, Section 9.5.

5. Regulatory Guide 1.52 (Rev. 2).

6. Regulatory Guide 1.196.

7. NEI 99-03, "Control Room Habitability Assessment," June 2001.

8. Letter from Eric J. Leeds (NRC) to James W. Davis (NEI) dated January 30, 2004, "NEI Draft White Paper, Use of Generic Letter 91-18 Process and Alternative Source Terms in the Context of Control Room Habitability." (ADAMS Accession No. ML040300694).

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