February 22, 2008

Mr. Mark B. Bezilla
Site Vice President
FirstEnergy Nuclear Operating Company
Davis-Besse Nuclear Power Station
Mail Stop A-DB-3080
5501 North State Route 2
Oak Harbor, OH 43449-9760

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1 - REQUEST FOR ADDITIONAL INFORMATION RELATED TO UPDATE TO B&W PLANTS STANDARD TECHNICAL SPECIFICATIONS NUREG-1430 (TAC NOS. MD6399 AND MD6400)

Dear Mr. Bezilla:

By letter to the Nuclear Regulatory Commission (NRC) dated August 3, 2007, FirstEnergy Nuclear Operating Company submitted a request to modify the technical specifications (TSs) to be consistent with B&W Plant's standard technical specifications (STS) NUREG-1430, for the Davis-Besse Nuclear Power Station, Unit No. 1.

The NRC staff is reviewing your submittal and has determined that additional information is required to complete the review. The specific information requested is addressed in the enclosure to this letter.

The NRC staff considers that timely responses to requests for additional information help ensure sufficient time is available for staff review and contribute toward the NRC's goal of efficient and effective use of staff resources. If you have any questions regarding this request, please contact me at (301) 415-4037.

Sincerely,

/RA/
Thomas J. Wengert, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosure:
Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

In reviewing the FirstEnergy Nuclear Operating Company's (FENOC's) submittal dated August 3, 2007, related to modifying the technical specifications (TSs) to be consistent with B&W Plant's standard technical specifications (STS) NUREG-1430, for the Davis-Besse Nuclear Power Station, Unit No. 1, the NRC staff has determined that the following information is needed in order to complete its review:

Beyond Scope Item - 1 "EDG Degraded Voltage & Loss of Voltage Relay Channel Check"

1. The proposed TS change to limiting condition for operation (LCO) 3.3.8 is to delete every 12-hour channel check surveillance requirement for the loss of voltage and degraded voltage emergency diesel generator (EDG) loss of power sequencing (LOPS) instrumentation. These safety functions are performed by voltage relays. The relays can be electro-mechanical or solid state design.

   To complete its review of the proposed changes to the TSs, the NRC staff needs to review the details of these relays and, therefore, requests the licensee to provide model and make information, and instruction leaflet references for each of these two types of relays.

Beyond Scope Item - 2 "Steam and Feedwater Rupture Control System Instrumentation Trip Setpoints" (LCO 3.3.11)

The proposed change revises the current TS Table 3.3-12 trip setpoint allowable values (AV) for Steam Line pressure – Low, Steam Generator Level - Low, and Steam Generator Feedwater Differential Pressure – High, functional units instrumentation.

To determine the acceptability of the proposed TSs change involving revision of instrumentation setpoints, the NRC staff generically requests all licensees who propose revision of instrumentation setpoint and/or setpoint AVs (conservative or non-conservative), to provide the following information (I&C Branch Guideline for Setpoint-Related TSs License Amendment Request – Agencywide Documents Access and Management System (ADAMS) Accession No. ML061810132):

2. Provide documentation (including sample calculations) used for establishing the limiting setpoint (or NSP) and the limiting acceptable values for the As-Found and As-Left setpoints as measured in periodic surveillance testing. Indicate the related analytical limits and other limiting design values (and the sources of these values) for each setpoint.

3. Provide a statement as to whether or not these three setpoints are limiting safety system settings (LSSSs) for the variables on which a safety limit (SL) has been placed as discussed in Title 10 of the *Code of Federal* Regulations (10 CFR) Section 50.36(c)(1)(ii)(A). These variables provide protection against violating reactor core safety

Enclosure

limits, or reactor coolant system pressure boundary safety limits. For each setpoint that you determined not to be SL-related, explain the basis for this determination.

4. For setpoints that are determined to be SL-related, the NRC letter to the Nuclear Energy Institute Setpoint Methods Task Force (SMTF) dated September 7, 2005 (ADAMS Accession No. ML052500004), describes Setpoint-Related TS (SRTS) that are acceptable to the NRC for instrument settings associated with SL-related setpoints. Specifically, part "A" of the enclosure to the letter provides LCO notes to be added to the TS, and part "B" includes a check list of the information to be provided in the TS Bases for the proposed TS changes. In this regard, please address items a, b, and c of this section.

   a. Describe whether and how you plan to implement the SRTS suggested in the September 7, 2005, letter. If you do not plan to adopt the suggested SRTS, then explain how you will ensure compliance with 10 CFR 50.36 by addressing items b and c below.

   b. Describe how surveillance test results and the associated TS limits are used to establish operability of the safety system. Show that the As-Found Setpoint valuation is consistent with the assumptions and results of the setpoint calculation methodology.

   c. Describe the controls employed to ensure that the instrument setpoint is, upon completion of surveillance testing, consistent with the assumptions of the associated analyses. If the controls document is other than the TS (e.g. plant test procedure), explain how the requirements of 10 CFR 50.36 are met.

5. For setpoints that are *not* determined to be SL-related, describe the measures to be taken to ensure that the associated instrument channel is capable of performing its specified safety functions, in accordance with applicable design requirements and associated analyses. Include in your discussion the information on controls you employ to ensure that the as-left trip setpoint setting after completion of periodic surveillance is consistent with your setpoint methodology.