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December 13, 2007

U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

ATTENTION: Document Control Desk

Subject: Duke Power Company LLC d/b/a
Duke Energy Carolinas, LLC (Duke)
McGuire Nuclear Station, Unit 2
Docket Nos. 50-370

License Amendment Request (LAR) for Technical
Specification 3.7.5, Auxiliary Feedwater System, Response
to Request for Additional Information

Reference 1: Duke letter to NRC dated November 7, 2007


This letter provides the additional information requested by the NRC staff via electronic mail from John F. Stang on December 4, 2007. The NRC staff's questions and Duke's responses are provided in Attachment 1.

The conclusions reached in the original determination that the LAR contains No Significant Hazards Considerations and the basis for the categorical exclusion from performing an Environmental/Impact Statement have not changed as a result of this request for additional information.

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Please contact Robert P. Boyer at 704-382-6830 if questions arise regarding the responses to this LAR request for additional information.

Sincerely,



Gary R. Peterson

Attachment

cc: w/attachment

W. D. Travers
Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, GA 30303

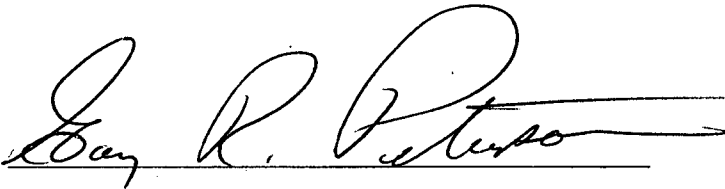
J. F. Stang, Jr. (addressee only)
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U.S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Mail Stop O-8-G9A
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J. B. Brady
NRC Senior Resident Inspector
McGuire Nuclear Station

B. O. Hall
Section Chief
Division of Radiation Section
1645 Mail Service Center
Raleigh, NC 27699

OATH AND AFFIRMATION

Gary R. Peterson affirms that he is the person who subscribed his name to the foregoing statement, and that all the matters and facts set forth herein are true and correct to the best of his knowledge.



Gary R. Peterson, Site Vice President

Subscribed and sworn to me: December 13, 2007

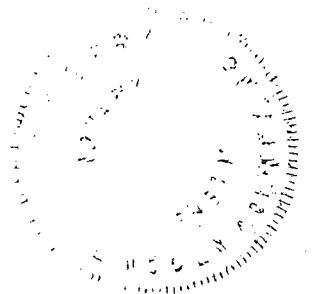
Date



Notary Public

My commission expires: July 1, 2012

Date



ATTACHMENT 1

Additional Information for the Office of Nuclear Reactor Regulation, Division of Risk Assessment, regarding the Technical Specification 3.7.5 License Amendment Request submitted by McGuire.

Question 1

The delta-CDF is reported as $2E-7$ /year, with an ICCDP of $1.8E-9$. Since this is a one-time only change, how can the annual delta risk exceed the ICCDP? Similarly for the delta-LERF and ICLERP. The licensee should further explain the basis for its numerical results.

Duke Response

The delta CDF was calculated with the plant in the configuration of interest (i.e., with the assured source from Nuclear Service Water Header A to Auxiliary Feedwater unavailable), without any adjustment for the duration of this configuration. Using a capacity factor of 0.9, the delta CDF of $2.0E-7$ /yr equates a risk change of $2.2E-7$ /rx-yr, or (multiplying by $3/365$) a one-time change of $1.8E-9$. The delta LERF and ICLERP were calculated in a similar fashion.

Question 2

The truncation is reported as "lowered to zero" for this application - the licensee should confirm the PRA model is being quantified with no limit to truncation.

Duke Response

To perform the PRA analysis, the model was revised to render the assured source to Auxiliary Feedwater from Nuclear Service Water Header A unavailable (by setting the failure probability of a valve in the flowpath to 1) and then resolved using the base case truncation limits ($1E-9$ for CDF and $1E-10$ for LERF). The resulting cut sets were recovered, the basic event of interest was set to a large value ($1E20$) and the CDF and LERF modules were compressed to delete cut sets below a truncation limit of 1. This left only those cut sets containing the event of interest. The event of interest was then set to TRUE. However, since many of the cut sets dropped below the base case truncation limits after recovery (primarily due to the loading of the final human error probabilities) the cut set file truncation limits were lowered to 0 in order to retain all of the cut sets in the calculation of delta CDF and delta LERF.

ATTACHMENT 1

Question 3

Confirm that any outstanding plant modifications, procedure changes, or PRA model deficiencies, not yet incorporated into the PRA model used for this application, have been evaluated to not impact the conclusions of the risk analysis.

Duke Response

Outstanding PRA changes due to plant modifications, procedure changes or model deficiencies are tracked in a PRA database. Consistent with our workplace procedure, these changes were reviewed and were either accounted for in the analysis or determined not to impact the conclusions of the risk analysis.