

of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

2.C.(1) Maximum Power Level

Florida Power Corporation is authorized to operate the facility at a steady state reactor core power level not in excess of 2609 Megawatts (100 percent of rated core power level).

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. **228**, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

The Surveillance Requirements contained in the Appendix A Technical Specifications and listed below are not required to be performed immediately upon implementation of Amendment 149. The Surveillance Requirements shall be successfully demonstrated prior to the time and condition specified below for each.

- a) SR 3.3.8.2.b shall be successfully demonstrated prior to entering MODE 4 on the first plant start-up following Refuel Outage 9.
- b) SR 3.3.11.2, Function 2, shall be successfully demonstrated no later than 31 days following the implementation date of the ITS.
- c) SR3.3.17.1, Functions 1, 2, 6, 10, 14, & 17 shall be successfully demonstrated no later than 31 days following the implementation date of the ITS.
- d) SR3.3.17.2, Function 10 shall be successfully demonstrated prior to entering MODE 3 on the first plant start-up following Refuel Outage 9.
- e) SR 3.6.1.2 shall be successfully demonstrated prior to entering MODE 2 on the first plant start-up following Refuel Outage 9.
- f) SR 3.7.12.2 shall be successfully demonstrated prior to entering MODE 2 on the first plant start-up following Refuel Outage 9.
- g) SR 3.8.1.10 shall be successfully demonstrated prior to entering MODE 2 on the first plant start-up following Refuel Outage 9.
- h) SR 3.8.3.3 shall be successfully demonstrated prior to entering MODE 4 on the first plant start-up following Refuel Outage 9.

## 1.1 Definitions

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EFFECTIVE FULL POWER DAY (EFPD) (continued)	reactor core at RTP for one full day. (One EFPD is 2609 Mwt times 24 hours or 62616 Mwhr.)
EMERGENCY FEEDWATER INITIATION AND CONTROL (EFIC) RESPONSE TIME	The EFIC RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its EFIC actuation setpoint at the channel sensor until the emergency feedwater equipment is capable of performing its safety function (i.e., valves travel to their required positions, pump discharge pressures reach their required values, etc.) Times shall include diesel generator starting and sequence loading delays, where applicable. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured.
ENGINEERED SAFETY FEATURE (ESF) RESPONSE TIME	The ESF RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF actuation setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays, where applicable. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured.
LEAKAGE	LEAKAGE shall be:  a. <u>Identified LEAKAGE</u>  1. LEAKAGE, such as that from pump seals or valve packing, that is captured and conducted to collection systems or a sump or collecting tank; or  2. LEAKAGE into the containment atmosphere from sources that are both specifically located and quantified and known not to interfere with the operation of leakage detection systems and not to be pressure boundary LEAKAGE; or

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1.1 Definitions

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PHYSICS TESTS  
(continued)

These tests are:

- a. Described in Chapter 13, "Initial Tests and Operation" of the FSAR;
- b. Authorized under the provisions of 10 CFR 50.59; or
- c. Otherwise approved by the Nuclear Regulatory Commission.

PRESSURE AND  
TEMPERATURE LIMITS  
REPORT (PTLR)

The PTLR is the unit specific document that provides the reactor vessel pressure and temperature limits, including heatup and cooldown rates, for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 5.6.2.19. Plant operation within these operating limits is addressed in LCO 3.4.3, "RCS Pressure and Temperature Limits."

QUADRANT POWER TILT  
(QPT)

QPT shall be defined by the following equation and is expressed as a percentage.

$$QPT = 100 \left( \frac{\text{Power In Any Core Quadrant}}{\text{Average Power of all Quadrants}} - 1 \right)$$

RATED THERMAL POWER  
(RTP)

RTP shall be a total reactor core heat transfer rate to the reactor coolant of 2609 MWt.

REACTOR PROTECTION  
SYSTEM (RPS) RESPONSE  
TIME

The RPS RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its RPS trip setpoint at the channel sensor until electrical power is interrupted at the control rod drive trip breakers. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured.

SHUTDOWN MARGIN (SDM)

SDM shall be the instantaneous amount of reactivity by which the reactor is subcritical or

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3.3 INSTRUMENTATION

3.3.1 Reactor Protection System (RPS) Instrumentation

LCO 3.3.1 Four channels of RPS instrumentation for each Function in Table 3.3.1-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.1-1.

ACTIONS

CONDITIONS	REQUIRED ACTION	COMPLETION TIME
A. One channel inoperable.	A.1 Place channel in bypass or trip.	1 hour
B. Two channels inoperable.	B.1 Place one channel in trip.	1 hour
	<u>AND</u> B.2 Place second channel in bypass.	1 hour
C. One or more RCPPM for one RCP inoperable.	C.1 Trip the RCPPM(s).	4 hours
D. Required Action and associated Completion Time of Condition A or B not met.	D.1 Enter the Condition referenced in Table 3.3.1-1 for the Function.	Immediately
E. Required Action and associated Completion Time of Condition C not met.	E.1.1 Verify 4 RCPS in operation.	1 hour
	<u>AND</u> E.1.2 Reduce THERMAL POWER <2475 MW <sub>th</sub>	1 hour
	<u>OR</u> E.2 Enter Condition F	1 hour

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ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
F. As required by Required Action D.1 and referenced in Table 3.3.1-1 or by Required Action E.2.	F.1 Be in MODE 3.	6 hours
	<u>AND</u> F.2 Open all CONTROL ROD drive (CRD) trip breakers.	6 hours
G. As required by Required Action D.1 and referenced in Table 3.3.1-1.	G.1 Open all CRD trip breakers.	6 hours
H. As required by Required Action D.1 and referenced in Table 3.3.1-1.	H.1 Reduce THERMAL POWER < 45% RTP.	6 hours
I. As required by Required Action D.1 and referenced in Table 3.3.1-1.	I.1 Reduce THERMAL POWER < 20% RTP.	6 hours
J. Secondary heat balance not based on required high accuracy instrumentation.	J.1 Reduce THERMAL POWER to $\leq 2568 \text{ MW}_{\text{th}}$	12 hours
	<u>AND</u> J.2 Reduce Nuclear Overpower - High Setpoint to $\leq 103.3\%$ RTP.	48 hours
K. Required Action and associated Completion Time of Condition J not met.	K.1 Be in MODE 3.	6 hours
	<u>AND</u> K.2 Open all Control Rod drive (CRD) trip breakers.	6 hours

SURVEILLANCE REQUIREMENTS

-----NOTE-----  
Refer to Table 3.3.1-1 to determine which SRs apply to each RPS Function.  
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SURVEILLANCE	FREQUENCY
SR 3.3.1.1 Perform CHANNEL CHECK.	12 hours
<p>SR 3.3.1.2 -----NOTES-----</p> <ol style="list-style-type: none"> <li>1. Not required to be performed until 24 hours after THERMAL POWER is <math>\geq</math> 15% RTP.</li> <li>2. High accuracy instrumentation is required to be utilized when performing calorimetric secondary heat balance comparison unless Condition J has been entered.</li> </ol> <p>-----</p> <p>Verify calorimetric secondary heat balance is <math>\leq</math> 2% RTP greater than power range channel output. Adjust power range channel output if calorimetric exceeds power range channel output by <math>&gt;</math> 2% RTP.</p>	24 hours
<p>SR 3.3.1.3 -----NOTE-----</p> <p>Not required to be performed until 24 hours after THERMAL POWER (TP) is <math>\geq</math> 30% RTP.</p> <p>-----</p> <p>Compare out of core measured AXIAL POWER IMBALANCE (<math>API_0</math>) to incore measured AXIAL POWER IMBALANCE (<math>API_1</math>) as follows:</p> $(RTP/TP)(API_0 - API_1) = \text{imbalance error}$ <p>Perform CHANNEL CALIBRATION if the absolute value of the imbalance error is <math>\geq</math> 2.5% RTP.</p>	31 days
SR 3.3.1.4 Perform CHANNEL FUNCTIONAL TEST.	45 days on a STAGGERED TEST BASIS

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Table 3.3.1-1 (page 1 of 1)  
Reactor Protection System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	CONDITIONS REFERENCED FROM REQUIRED ACTION D.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Nuclear Overpower -				
a. High Setpoint	1,2 <sup>(a)</sup>	F	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.5 <sup>(f,g)</sup> SR 3.3.1.7	≤ 104.9% RTP <sup>(d)</sup> ≤ 103.3% RTP <sup>(e)</sup>
b. Low Setpoint	2 <sup>(b)</sup> , 3 <sup>(b)</sup> 4 <sup>(b)</sup> , 5 <sup>(b)</sup>	G	SR 3.3.1.1 SR 3.3.1.5	≤ 5% RTP
2. RCS High Outlet Temperature	1,2	F	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6	≤ 618°F
3. RCS High Pressure	1,2	F	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6 SR 3.3.1.7	≤ 2355 psig
4. RCS Low Pressure	1,2 <sup>(a)</sup>	F	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6 SR 3.3.1.7	≥ 1900 psig
5. RCS Variable Low Pressure	1,2 <sup>(a)</sup>	F	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6	RCS Variable Low Pressure equation in COLR
6. Reactor Building High Pressure	1,2,3 <sup>(c)</sup>	F	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6	≤ 4 psig
7. Reactor Coolant Pump Power Monitor (RCPPM)	1,2 <sup>(a)</sup>	F	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6 SR 3.3.1.7	More than one pump drawing ≤ 1152 or ≥ 14,400 kW
8. Nuclear Overpower RCS Flow and Measured AXIAL POWER IMBALANCE	1,2 <sup>(a)</sup>	F	SR 3.3.1.1 SR 3.3.1.3 SR 3.3.1.5 SR 3.3.1.6 SR 3.3.1.7	Nuclear Overpower RCS Flow and AXIAL POWER IMBALANCE setpoint envelope in COLR
9. Main Turbine Trip (Control Oil Pressure)	≥ 45% RTP	H	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6	≥ 45 psig
10. Loss of Both Main Feedwater Pumps (Control Oil Pressure)	≥ 20% RTP	I	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6	≥ 55 psig
11. Shutdown Bypass RCS High Pressure	2 <sup>(b)</sup> , 3 <sup>(b)</sup> 4 <sup>(b)</sup> , 5 <sup>(b)</sup>	G	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6	≤ 1820 psig

(a) When not in shutdown bypass operation.

(b) During shutdown bypass operation with any CRD trip breakers in the closed position and the CRD Control System (CRDCS) capable of rod withdrawal.

(c) With any CRD trip breaker in the closed position and the CRDCS capable of rod withdrawal.

(d) With secondary heat balance based on required high accuracy instrumentation.

(e) With secondary heat balance not based on required high accuracy instrumentation.

(f) If the as-found channel setpoint is conservative with respect to the Allowable Value (AV), but outside its predefined as-found acceptance criteria band, then the channel should be evaluated to verify that it is functioning as required before returning the channel to service. If the as-found instrument channel setpoint is not conservative with respect to the AV, the channel shall be declared inoperable.

(g) The instrument channel setpoint shall be reset to a value that is within the as-left tolerance of the pre-established In-Plant Setpoint, or a value that is more conservative than the pre-established In-Plant Setpoint; otherwise the channel shall not be returned to OPERABLE status. The pre-established In-Plant Setpoint and the methodology used to determine the pre-established In-Plant Setpoint, the predefined as-found acceptance criteria band, and the as-left acceptance criteria are specified in the FSAR.