

CENTRAL FILES

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401
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MAY 29 1979

USNRC REGION
ATLANTA, GEORGIA
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Mr. James P. O'Reilly, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 3100
101 Marietta Street
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

OFFICE OF INSPECTION AND ENFORCEMENT BULLETIN 78-12 RII:JPO 50-438,
-439, -259, -260, -296, -518, -519, -520, -521, -553, -554, -327,
-328, -390, -391, -566, -567 - SEQUOYAH, WATTS BAR, BELLEFONTE,
HARTSVILLE, PHIPPS BEND, YELLOW CREEK, AND BROWNS FERRY NUCLEAR PLANTS

We are submitting the enclosed information in response to OIE Bulletin
78-12B transmitted by your letter to H. G. Parris dated March 19, 1979.

If you have any questions concerning this matter, please get in touch
with D. L. Lambert at FTS 854-2581.

Very truly yours,

J. E. Gilleland
for J. E. Gilleland
Assistant Manager of Power

Enclosure

cc: Mr. John G. Davis, Acting Director (Enclosure)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

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ENCLOSURE

RESPONSE TO OIE BULLETINS 78-12, 12A, 12B
ATYPICAL WELD MATERIAL IN REACTOR
PRESSURE VESSEL WELDS

Sequoyah and Watts Bar Nuclear Plants

Westinghouse has informed TVA that due to the large amount of data which must be researched and compiled, their generic report is not yet complete. The Westinghouse generic report responding to Bulletin 78-12 is scheduled to be submitted to the NRC in October 1979.

TVA will submit a supplemental response verifying that this generic report addresses Bulletin 78-12 for the Sequoyah and Watts Bar Nuclear Plants reactor pressure vessels within 30 days after the generic report is submitted.

Bellefonte Nuclear Plant

B&W submitted a generic report responding to Bulletin 78-12 by letter from S. H. Taylor to Wayne G. Reinmuth dated March 13, 1979. This report includes all specific vessel information required by Bulletin 78-12. TVA has reviewed the generic report and believes the exhaustive testing reported has isolated the occurrence of atypical chemistry to heat 75201. TVA is satisfied that the generic report adequately represents the Bellefonte Nuclear Plant reactor pressure vessels.

Hartsville Nuclear Plants A and B and Phipps Bend Nuclear Plant

Chicago Bridge & Iron submitted a generic report by letter from W. G. Oehlkers to W. G. Reinmuth dated May 1, 1979, responding to Bulletin 78-12 for reactor pressure vessels fabricated by them.

GE has notified us that the Hartsville and Phipps Bend reactor pressure vessels are covered by this report. TVA has not yet received this report. If, upon completion of our review, we find that the report does not provide all the information requested by Bulletin 78-12 or that the data is not representative of the Hartsville and Phipps Bend Nuclear Plants reactor pressure vessels, we will notify you.

Yellow Creek Nuclear Plant

CE has informed TVA that due to the large amount of data which must be researched and compiled their generic report is not yet complete. The CE generic report responding to Bulletin 78-12 is scheduled to be submitted to the NRC in June 1979.

TVA will submit a supplemental response verifying that this generic report addresses Bulletin 78-12 for the Yellow Creek Nuclear Plant reactor pressure vessels within 30 days after the generic report is submitted.

Browns Ferry Nuclear Plant

B&W submitted a generic report responding to Bulletin 78-12 by letter from J. H. Taylor to W. G. Reinmuth dated March 13, 1979. This report

included all specific vessel information required by the subject bulletin and adequately represents data for the Browns Ferry reactor pressure vessels.

GE reviewed the generic report submitted by B&W and concurs with its findings. However, GE did note a need for clarification of some of the data submitted. Their review identified two qualification tests for submerged arc wire/flux combination deposits which has passed the chemical analysis but failed the impact property requirements by a narrow margin. These tests had correctly been marked "failed," but the summary report did not mention a disposition of this wire/flux combination. Upon inquiry, B&W responded that the wire/flux combinations involved in these failed tests were never used on any reactor pressure vessel. Because the main concern voiced in the bulletin is a deviating chemical composition, it was concluded that the weld material was not "atypical" for the purpose of this report, even if it had been used.