

CHATTANOOGA, TENNESSEE 37401
400 Chestnut Street Tower II

Director of Nuclear Reactor Regulation
Attention: Ms. E. Adensam, Chief
Licensing Branch No. 4
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

In the Matter of the Application of) Docket Nos. 50-390
Tennessee Valley Authority) 50-391

Subsequent to the TVA review of the draft NRC evaluation report a conference call was held between TVA and NRC representatives on August 14, 1984, to discuss the NRC generated list of valves. As conveyed during the conference call, TVA concurs that some additional valves need to be included in the unit 1 Technical Specifications and agrees with the NRC generated list of valves except in two instances (charging header isolation valves 87-7 and 87-8).

TVA's bases for omitting these valves, 87-7 and 87-8, from Table 3.4-1 of the Technical Specifications were favorably received by the NRC reviewer involved in the conference call; however, at the reviewer's request we are formally documenting these bases.

TVA maintains that these valves, 87-7 and 87-8, should not be included in Table 3.4-1 since: (1) these valves do not perform a high to low pressure isolation function (the entire system has been hydrostatically pressure tested to a minimum of 2250 psi), (2) the suction side of the charging pump is protected since it is a reciprocating type pump, and (3) these valves are also containment isolation valves and are subject to the containment leak rate criteria specified by technical specification 3/4.6.3.

For use in the development of the unit 1 Technical Specifications we have enclosed a marked-up Table 3.4-1 indicating the additional pressure isolation valves and their corresponding functions.

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PDR ADCK 05000390
A PDR

Boo!

Director of Nuclear Reactor Regulation

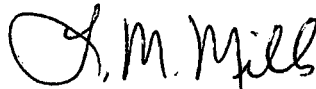
August 23, 1984

Please note that a typographical error had been included in the NRC generated list of valves, FCV-74-7 should be FCV-74-9.

If you have any questions concerning this matter, please get in touch with D. B. Ellis at FTS 858-2681.


Very truly yours,

TENNESSEE VALLEY AUTHORITY



L. M. Mills, Manager
Nuclear Licensing

Sworn to and subscribed before me
this 23rd day of Aug. 1984


Notary Public
My Commission Expires 9-5-84
Enclosure

cc: U.S. Nuclear Regulatory Commission (Enclosure)
Region II
Attn: Mr. James P. O'Reilly, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

ENCLOSURE

WATTS BAR NUCLEAR PLANT
UNIT 1 TECHNICAL SPECIFICATION
PROPOSED TABLE 3.4-1

TABLE 3.4-1

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

VALVE NUMBER

FUNCTION

63-560	Accumulator Discharge
63-561	Accumulator Discharge
63-562	Accumulator Discharge
63-563	Accumulator Discharge

FCV-74-1*	Residual Heat Removal
FCV-74-2*	Residual Heat Removal
FCV-74-8*	Residual Heat Removal
FCV-74-9*	Residual Heat Removal

87-558	Upper Head Injection
87-559	Upper Head Injection
87-560	Upper Head Injection
87-561	Upper Head Injection

63-622	Accumulator Discharge
63-623	Accumulator Discharge
63-624	Accumulator Discharge
63-625	Accumulator Discharge
63-551	Safety Injection (Cold Leg)
63-553	Safety Injection (Cold Leg)
63-555	Safety Injection (Cold Leg)
63-557	Safety Injection (Cold Leg)
63-632	Residual Heat Removal (Cold Leg)
63-633	Residual Heat Removal (Cold Leg)
63-634	Residual Heat Removal (Cold Leg)
63-635	Residual Heat Removal (Cold Leg)
63-641	RHR/Safety Injection (Hot Leg)
63-644	RHR/Safety Injection (Hot Leg)
63-558	Safety Injection (Hot Leg)
63-559	Safety Injection (Hot Leg)
63-543	Safety Injection (Hot Leg)
63-545	Safety Injection (Hot Leg)
63-547	Safety Injection (Hot Leg)
63-549	Safety Injection (Hot Leg)
63-640	Safety Injection (Hot Leg)
63-643	Residual Heat Removal (Hot Leg)
87-562	Residual Heat Removal (Hot Leg)
87-563	Upper Head Injection
	Upper Head Injection

* These valves do not have to be leak tested following manual or automatic actuation or flow through the valve.