

2.3 Meteorology

The U.S. EPR design is based on meteorological parameters (e.g., air temperature extremes, humidity, precipitation [rainfall, snow and ice], maximum wind speeds, tornado wind speeds, and atmospheric stability characteristics) provided in Section 2.1, Table 2.1-1, Envelope of U.S. EPR Site Design Parameters. If a COL applicant that references the U.S. EPR design certification identifies site-specific meteorology values outside the range of the design parameters in Table 2.1-1, then the COL applicant will demonstrate the acceptability of the site-specific values in the appropriate sections of the Combined License application.

2.3.1 Regional Climatology

The following information is provided in Section 2.1, Table 2.1-1:

- Weight of 100-year return period snow pack and the weight of the 48-hour probable maximum winter precipitation (PMWP).
- 100-year, 3-second gust wind speed.
- Tornado parameters.
- Dry-bulb and wet-bulb temperatures.

2.3.1.1 Basis for Meteorological Parameters

The design parameters for the dry-bulb and wet-bulb temperatures are based on the EPRI ALWR Utility Requirements Document (Reference 1) and available Early Site Permit applications. The two percent annual exceedance dry and wet bulb temperature values, as recommended by RG 1.206 and SRP 2.3.1, are not provided in Table 2.1-1. However, the two percent annual exceedance dry and wet bulb temperature values are bounded by the provided zero percent annual exceedance and one percent annual exceedance dry and wet bulb temperature values.

SRP 2.3.1 and RG 1.206 also recommend that the 100-year maximum dry bulb and coincident wet bulb temperature values, the 100-year maximum non-coincident wet bulb temperature value, and the 100-year minimum dry bulb temperature values be provided. Instead, the zero percent exceedance values for these parameters have been provided. Zero percent exceedance values are based on conservative estimates of historical high and low values for potential sites..

The prescribed loads included in the combination of normal live loads are based on the weight of the 100-year snow pack or snowfall, whichever is greater, recorded at ground level. Winter precipitation loads to be included in the combination of extreme live loads is based on the addition of the weight of the 100-year snow pack at ground level plus the weight of the 48-hour PMWP at ground level for the month



corresponding to the selected snow pack. Snow pack and snowfall are adjusted for density differences and ground level values are adjusted to represent appropriate weights on roofs.

A COL applicant that references the U.S. EPR design certification will provide sitespecific characteristics for regional climatology.

2.3.1.2 Meterological Data for Evaluating the Ultimate Heat Sink

As described in Section 9.2.5, the ultimate heat sink (UHS) is designed to operate for a nominal 30 days following a LOCA without requiring any makeup water to the source or it must be demonstrated that replenishment or use of an alternate or additional water supply can provide continuous capability of the heat sink to perform its safety-related functions. The tower basin contains a minimum 72-hour supply of water.

Meteorological conditions resulting in the maximum evaporative and drift loss of water for the UHS over a 72 hour period are presented in Table 2.1-3. The UHS cooling tower basin is designed considering the air temperature data of Table 2.1-1 and maintains its cooling function for the Table 2.1-3 meteorological conditions.

Water makeup to the UHS cooling tower basin beyond 72 hours is site specific. A COL applicant that references the U.S. EPR design certification will describe the means for providing UHS makeup sufficient to meet the maximum evaporative and drift water loss after 72 hours through the remainder of the 30 day period consistent with RG 1.27.

Meteorological conditions resulting in minimum water cooling are presented in Table 2.1-4. These conditions reflect a 1 day period where evaporative cooling is at a minimum. The UHS heat loads peak and decline within the first day, such that extending the 1 day meteorological profile for 5 consecutive days does not cause the UHS cooling tower basin water temperature to exceed the maximum temperature of 95°F listed in Table 2.1-1. The potential for water freezing in the UHS water storage facility is addressed in Section 2.4.

2.3.2 Local Meteorology

A COL applicant that references the U.S. EPR design certification will provide sitespecific characteristics for local meteorology.

2.3.3 Onsite Meteorological Measurement Program

A COL applicant that references the U.S. EPR design certification will provide the sitespecific, onsite meteorological measurement program.



2.3.4 Short-Term Atmospheric Dispersion Estimates for Accident Releases

Atmospheric dispersion factors (χ /Q values) considered to be representative of potential future nuclear plant sites in the U.S. were used to calculate the consequences from postulated accidental releases of radioactive and hazardous materials.

 χ /Q values for ground-level releases were calculated at the exclusion area boundary (EAB) and at the low population zone (LPZ) for appropriate time periods up to 30 days after an accident. The accident χ /Q values were either extracted from Reference 1 or were calculated following the methodology in NRC RG 1.145. The ground-level χ /Q values used for short-term atmospheric dispersion dose analyses at the EAB and LPZ receptor locations are provided in Table 2.1-1.

In addition to the offsite accident consequences evaluated at the EAB and LPZ, onsite accident dose consequences at the main control room (MCR) and technical support center (TSC) were evaluated. MCR and TSC χ /Q values, provided in Table 2.3-1 for the main air supply and Table 2.3-2 for the unfiltered inleakage, are used for these analyses from potential post-accident release points. These multiple potential release points affecting the MCR and the TSC include:

- The vent stack.
- Main steam relief train (MSRT) releases for steam generator overpressure protection.
- Safeguard Building roofs via the Safeguard Building canopies.
- An open equipment hatch.
- Safeguard Building depressurization shaft.

The information in these tables conforms to the guidance in RG 1.23, RG 1.145, and RG 1.194. Conformance with RG 1.78 is addressed in Sections 2.2, 6.4, 9.4, and 9.5.

Figure 2.3-1, U.S. EPR Release Points and Control Room Air Intakes, provides the relative locations of the release points and the control room air intakes. Section 15.0.3 addresses the dose calculation methodology for accident analyses.

A COL applicant that references the U.S. EPR design certification will confirm that site-specific χ /Q values, based on site-specific meteorological data, are bounded by those specified in Table 2.1-1 at the EAB and LPZ and by Table 2.3-1 at the control room.

For site-specific χ/Q values that exceed the bounding χ/Q values, a COL applicant that references the U.S. EPR design certification will demonstrate that the radiological consequences associated with the controlling design basis accident continue to meet



the dose reference values given in 10 CFR 50.34 and the control room operator dose limits given in GDC 19 using site-specific χ/Q values.

A COL applicant that references the U.S. EPR design certification will provide a description of the atmospheric dispersion modeling used in evaluating potential design basis events to calculate concentrations of hazardous materials (e.g., flammable or toxic clouds) outside building structures resulting from the onsite and/or offsite airborne releases of such materials.

A COL applicant that references the U.S. EPR design will provide χ/Q values for each cumulative frequency distribution which exceeds the median value (50 percent of the time) as part of the assessment of the postulated impact of an accident on the environment.

2.3.5 Long-Term Atmospheric Dispersion Estimates for Routine Releases

A COL applicant that references the U.S. EPR design certification will provide the sitespecific, long-term diffusion estimates for routine releases. In developing this information, the COL applicant should consider the guidance provided in RG 1.23, RG 1.109, RG 1.111, and RG 1.112. The maximum annual average χ /Q value at the site boundary, provided in Table 2.1-1, is used to calculate radionuclide concentrations associated with routine gaseous effluent releases, addressed in Section 11.3, for comparison with environmental release limits and dose limits given in 10 CFR Part 20. If a reactor site has an annual average χ /Q value that exceeds the reference value, then a site-specific evaluation will be performed.

A COL applicant that references the U.S EPR design certification will also provide estimates of annual average atmospheric dispersion (χ /Q values) and deposition (D/Q values) for 16 radial sectors to a distance of 50 miles (80 km) from the plant as part of its environmental assessment.

2.3.6 References

1. EPRI ALWR Utility Requirements Document, "Electric Power Research Institute Advanced Light Water Reactor Utility Requirements Document", Volume II-Revision 8, March 1999.



Table 2.3-1—Main Control Room/Technical Support Center Intake Atmospheric Dispersion Factors for Onsite Accident Dose Analysis (χ /Q)

Time Period	Vent Stack Base	Releases via Safe- guard Building Canopy #1	Releases via Safe- guard Building Canopy #2	Equip- ment Hatch Releases via Material Lock	Depressuriza -tion Shaft Releases	Main Steam Relief Train Silencer #1	Main Steam Relief Train Silencer #2	Main Steam Relief Train Silencer #3	Main Steam Relief Train Silencer #4
0 - 2 hours (sec/m ³)	1.93E-03	6.52E-03	1.66E-03	1.01E-03	4.45E-03	1.57E-03	1.88E-03	4.30E-03	3.28E-03
2-8 hours (sec/m ³)	1.73E-03	5.68E-03	1.47E-03	8.97E-04	3.95E-03	1.36E-03	1.63E-03	3.71E-03	2.83E-03
8 – 24 hours (sec/m ³)	6.74E-04	2.34E-03	6.28E-04	3.53E-04	1.70E-03	5.44E-04	6.47E-04	1.46E-03	1.12E-03
1 – 4 days (sec/m ³)	5.12E-04	1.63E-03	4.19E-04	2.66E-04	1.11E- 0 3	4.11E-04	4.93E-04	1.12E-03	8.57E-04
$\begin{array}{c} 4-30 \text{ days} \\ (\text{sec}/\text{m}^3) \end{array}$	4.72E-04	1.50E-03	3.81E-04	2.46E-04	1.00E-03	3.78E-04	4.52E-04	1.03E-03	7.86E-04



Table 2.3-2—Main Control Room/Technical Support Center Unfiltered Inleakage Atmospheric Dispersion Factors for Onsite Accident Dose Analyses

Time Period	Vent Stack Base	Releases via Safeguard Building Canopy #1	Releases via Safeguard Building Canopy #2	Equipment Hatch Releases via Material Lock	Depressuriza- tion Shaft Releases	Main Steam Relief Train Silencer #1	Main Steam Relief Train Silencer #2	Main Steam Relief Train Silencer #3	Main Steam Relief Train Silencer #4
0 - 2 hours (sec/m ³)	4.30E-03	1.67E-02	3.03E-03	1.65E-03	7.52E-03	1.93E-03	2.43E-03	1.76E-02	8.65E-03
2-8 hours (sec/m ³)	3.71E-03	1.47E-02	2.68E-03	1.47E-03	6.67E-03	1.66E-03	2.12E-03	1.48E-02	7.21E-03
8 – 24 hours (sec/m ³)	1.46E-03	5.96E-03	1.15E-03	5.74E-04	2.88E-03	6.69E-04	8.28E-04	5.88E-03	2.96E-03
1-4 days (sec/m ³)	1.12E-03	4.28E-03	7.59E-04	4.37E-04	1.89E-03	5.02E-04	6.38E-04	4.55E-03	2.22E-03
$\begin{array}{c} 4-30 \text{ days} \\ (\text{sec}/\text{m}^3) \end{array}$	1.03E-03	3.89E-03	6.89E-04	4.00E-04	1.71E-03	4.65E-04	5.85E-04	4.16E-03	2.06E-03