

1.6 Material Referenced

Table 1.6-1—Reports Referenced, contains a list of AREVA NP reports submitted to the NRC that are incorporated by reference in the U.S. EPR FSAR. A COL applicant that references the US EPR design certification will include any site-specific topical reports that are incorporated by reference as part of the COL application in Table 1.6-1.

**Table 1.6-1—Reports Referenced
Sheet 1 of 3**

Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
ANF-89-060P-A ANF-89-060NP-A Supplement 1	Generic Mechanical Design Report High Thermal Performance Spacer and Intermediate Flow Mixer	3/28/91	4.2
ANP-10263P-A ANP-10263NP-A	Codes and Methods Applicability Report for the U.S. EPR	11/06/07	4, 5.1, 15, 16, and 19
ANP-10264NP	U.S. EPR Piping Analysis and Pipe Support Design Topical Report	9/29/06	3.6, 3.7, 3.8, 3.9, 3.10, 3.12, App. 3A, and App. 3C
ANP-10266-A	AREVA NP Inc. Quality Assurance Plan (QAP) for Design Certification of the U.S. EPR Topical Report	06/18/07	7.1, 17.1, 17.2, 17.3, 17.5, 18.1, 18.7, and 18.11
ANP-10268P ANP-10268NP	U.S. EPR Severe Accident Evaluation Topical Report	10/31/06	6.2.5, 15.4, 19.1, and 19.2
ANP-10269P ANP-10269NP	The ACH-2 CHF Correlation for the U.S. EPR Topical Report	11/30/06	4.4, 5, 7, 15, and 19
ANP-10272	Software Program Manual TELEPERM XS™ Safety Systems Topical Report	12/21/06	7.1 and 7.6
ANP-10273P ANP-10273NP	AV42 Priority Actuation and Control Module Topical Report	11/28/06	7 and 16
ANP-10275P ANP-10275NP	U.S. EPR Instrument Setpoint Methodology Topical Report	3/26/07	7 and 16
ANP-10278P ANP-10278NP	U.S. EPR Realistic Large Break Loss of Coolant Accident Topical Report	3/26/07	6.2 and 15
ANP-10279	U.S. EPR Human Factors Engineering Program Topical Report	1/29/07	3.4, 7.1, 13.1, and 18
ANP-10281P ANP-10281NP	U.S. EPR Digital Protection System Topical Report	3/27/07	3.1.3, 4.6, 7, and 8.1
ANP-10282P ANP-10282NP	POWERTRAX/E Online Core Monitoring Software for the U.S. EPR Technical Report	11/27/07	4.4
ANP-10283P ANP-10283NP	U.S. EPR Pressure-Temperature Limits Methodology for RCS Heat-Up and Cool-Down	12/07 (Note 4)	5.3 and 16
ANP-10284	U.S. EPR Instrumentation and Controls Diversity and Defense-in-Depth Methodology Topical Report	6/20/07	7
ANP-10285P ANP-10285NP	U.S. EPR Fuel Assembly Mechanical Design Topical Report	10/02/07	4

**Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
ANP-10286P ANP-10286NP	U.S. EPR Rod Ejection Accident Methodology Topical Report	11/20/07	4.3 and 15
ANP-10287P ANP-10287NP	Incore Trip Setpoint and Transient Methodology for U.S. EPR Topical Report	11/27/07	4, 6, 7, and 15
ANP-10288P ANP-10288NP	U.S. EPR Post-LOCA Boron Precipitation and Boron Dilution Technical Report	12/07 (Note 4)	15
ANP-10290	AREVA NP Environmental Report Standard Design Certification	12/07 (Note 4)	19.2
ANP-10291P ANP-10291NP	SBLOCA Sensitivity Studies and Methodology	12/07 (Note 4)	15
ANP-10292	U.S. EPR Conformance with Standard Review Plan	12/07 (Note 4)	1.9
BAW-10132-A	Analytical Methods Description – Reactor Coolant System Hydrodynamic Loadings During a Loss-of-Coolant Accident	7/20/79	App. 3C
BAW-10133P-A BAW-10133-A Revision 1, Addendum 1 and 2	Mark-C Fuel Assembly LOCA-Seismic Analysis	10/30/00	4.2
BAW-10147P-A, BAW-10147-A Revision 1	Fuel Rod Bowing in Babcock & Wilcox Fuel Designs	6/28/83	4.2, 4.4
BAW-10156-A, Revision 1	LYNXT, Core Transient Thermal- Hydraulic Program	8/18/93	4
BAW-10163P-A BAW-10163-A	Core Operating Limit Methodology for Westinghouse Designed PWRs	6/2/89	4.3 and 16
BAW-10164P-A BAW-10164NP-A Revision 6	RELAP5/MOD2-B&W - An Advanced Computer Program for Light Water Reactor LOCA and Non-LOCA Transient Analysis	11/20/07	3.9.1, 6.2, and 8.4
BAW-10168P-A BAW-10168-A Revision 3	BWNT Loss-of-Coolant Accident Model for Recirculating Steam Generator Plants	1/31/97	6.2
BAW-10169P-A BAW-10169-A	B&W Safety Analysis Methodology for Recirculating Steam Generator Plants	11/28/89	6.2

**Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
BAW-10172P-A BAW-10172NP-A	Mark-BW Mechanical Design Report	12/19/89 (Note 5)	4.2
BAW-10183P-A BAW-10183-A	Fuel Rod Gas Pressure Criterion (FRGPC)	7/24/95	4.2 and 4.4
BAW-10186P-A BAW-10186NP-A Revision 2	Extended Burnup Evaluation	1/30/04	4.2 and 4.4
BAW-10199P-A BAW-10199-A	The BWU Critical Heat Flux Correlations	8/19/96	4.4
BAW-10221P-A BAW-10221-A	NEMO-K A Kinetics Solution in NEMO	9/17/88	4.3 and 16
BAW-10227P-A BAW-10227NP-A Revision 1	Evaluation of Advanced Cladding and Structural Material (M5) in PWR Reactor Fuel	6/18/03 (Note 5)	4.1, 4.2, and 15
BAW-10231P-A BAW-10231NP-A Revision 1	COPERNIC Fuel Rod Design Computer Code	9/30/04	4 and 16
BAW-10239P-A BAW-10239NP-A	Advanced Mark-BW Fuel Assembly Mechanical Design Topical Report	10/5/04	4.2
BAW-10240P-A BAW-10240NP-A	Incorporation of M5 TM Properties in Framatome ANP Approved Methods	8/17/04	4.2
BAW-10243P-A BAW-10243NP-A	Statistical Fuel Assembly Hold Down Methodology	12/21/05	4.4
BAW-10252P-A BAW-10252NP-A	Analysis of Containment Response to Postulated Pipe Ruptures using GOTHIC	12/21/05	6.2
BAW-2241P-A BAW-2241NP-A Revision 2	Fluence and Uncertainty Methodologies	11/20/07	4.1 and 5.3
EMF-2110(NP)(A) Revision 1	TELEPERM XS: A Digital Reactor Protection System	7/12/00	7
EMF-96-029(P)(A) EMF-96-029(NP)(A)	Reactor Analysis Systems for PWRs, Volume 1 -Methodology Description, Volume 2 -Benchmarking Results	1/13/97	4.3 and 16

Notes:

1. P – Denotes document is proprietary.

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2. NP – Denotes the document is non-proprietary. Any reports that do not have a “P” or “NP” after the numerical designation are also non-proprietary.
 3. “-A” denotes the published version of the report following issuance of the NRC Safety Evaluation Report approving the topical report.
 4. This report is being submitted concurrent with the design certification application.
 5. This is the date that the NRC approved this topical report. The letter transmitting the approved version to NRC is not publicly available