WATTS BAR NUCLEAR PLANT TECHNICAL INSTRUCTIONS

TI-31.7.0

TOTAL AUXILIARY BUILDING EXTERNAL LEAKAGE

Rev. By	Rev. No.	Date	Revised Pages
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1.C	Plant Haster File
1C	Superintendent
	Assistant Superingendent
1C	Mechanical Maintenance Supervisor
1C	Results, Supervisor
1C	Operations Supervisor
1C	Quality Assurance Supervisor
1C	Health Physicist
	Administrativė Supervisor
	Chemical Laboratory
1C	Instrument Shop
	Shift Engineer's Office
2C	Unit Control Room
	Health Physics Laboratory
10	PSU Supervisor
16-	Hechanical Engineer
	Reactor Engineer
	Chemical Engineer
1C	Instrument Maintenance Supervisor
1 <u>C</u>	Asst. Director of Sac Power (Open
	Electrical Maintenance Supervisor
	Plant Industrial Engineer
1C	Outage Director
111	NRC
1 C	Nuclear Safety Review Staff
1C	P PROD Training Center - Sequoyah
1U	Plant Training Office
	strument Engineers
.U	Supv. Reactor Systems Group
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Prepared By S. L. Linginfelter

Submitted By

MSMMI

PORC Review

11/10/87

Approved By

Superintendent

Date Approved

11/10/87

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TOTAL AUXILIARY BUILDING EXTERNAL LEAKAGE

1.0 SCOPE

This technical instruction is used to compute the combined external leakage to the Auxiliary Building of all systems that may contain contaminated liquids during an accident or serious transient. Each system considered to have this possible leak problem is to be visually inspected in operation and any leakage will be identified, quantified, and repaired if the leakage is not acceptable. The individual system inspections will be performed annually in the adjoining TI-31.7 series and the results of these instructions added to determine the combined external leakage and recorded in this instruction. A current copy of this instruction will be maintained in the Shift Engineer's Office in a notebook titled "External Leak Rates" and will be available for use in an emergency situation. A copy of this TI will be sent to the NRC through the Director of Nuclear Power. This data shall be submitted to the Director's Office no later than February 1 of the following year.

2.0 REFERENCES

DPM N80M4 revised 6/12/81 NUREG 0578 Item 2.1.6.A

3.0 PREREQUISITES

The following TI's have been completed for the annual term.

TI-31.7.1 AB Containment Spray System External Leakage

TI-31.7.2 AB Safety Injection System External Leakage

TI-31.7.3 AB Residual Heat Removal System External Leakage

TI-31.7.4 AB Chemical and Volume Control System External Leakage

TI-31.7.5 AB Reactor Coolant Sample System External Leakage

4.0 INSTRUCTIONS

- 4.1 Complete data sheet 1 using the data sheets from the appropriate TI listed in 3.0.
- 4.2 Forward a copy of completed data sheet 1 to the Operations Section for filing in the "External Leak Rates" notebook.
- 4.3 Forward a copy of completed data sheet 1 to the Director of Nuclear Power for transmittal to NRC.
- 4.4 The original data package is to be transferred to the life time record storage.

5.0 ACCEPTANCE CRITERIA

The acceptance criteria for each individual system's TI shall be met.

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EXTERNAL LEAK RATES

DATA SHEET 1

DATE			rinu	7	
TI NO.	SYSTEM	DATE OF TEST		SYSTEM EXTERNAL LE	EAK RATE
31.7.1	CSS				GPM
31.7.2	SIS				GPM ·
31.7.3	RHR		*. *		GPM
31.7.4	CVCS				GPM
31.7.5	RC Sample				GRM
	TOTAL A.B. EXTERNAL LEAK	AGE		GPM	
	Data Verified By:			**************************************	
		Mechanical Engineer			
Remarks and Comments:_		•			
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