

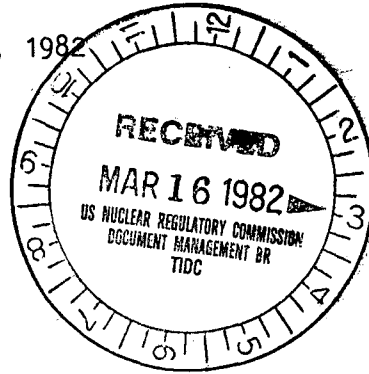
TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

400 Chestnut Street Tower II

March 12, 1982

Director of Nuclear Reactor Regulation
Attention: Ms. E. Adensam, Chief
Licensing Branch No. 4
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, DC 20555



Dear Ms. Adensam:

In the Matter of the Application of) Docket Nos. 50-390
Tennessee Valley Authority) 50-391

Enclosed for NRC review is a revised response to NRC question 260.2 on Watts Bar Nuclear Plant. This response clarifies the quality assurance program related to portable radiation monitors and expendable and consumable items necessary for the functional performance of safety-related equipment.

If you have any questions concerning this matter, please get in touch with D. P. Ormsby at FTS 858-2682.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

M. R. Wisenburg
M. R. Wisenburg
Nuclear Engineer

Sworn to and subscribed before me
this 12th day of March 1982

Bryant M. Lowery
Notary Public
My Commission Expires 4/4/82

Enclosure

cc: U.S. Nuclear Regulatory Commission
Region II
Attn: Mr. James P. O'Reilly, Regional Administrator
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

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260.2 Question

The list of items provided informally from the Watts Bar OQAM needs to be supplemented in Chapter 17 of the FSAR in accordance with the following:

- a. The following items do not appear on the list. Add the appropriate items to the list in Chapter 17 of the FSAR and provide a commitment in Chapter 17 of the FSAR that the remaining items are subject to the pertinent requirements of the FSAR operational quality assurance program or justify not doing so.
 1. Spent fuel pool and liner
 2. Fuel handling machines
 3. Spent fuel handling tool
 4. Reactor vessel head lifting rig
 5. Radioactivity monitoring (fixed and portable)
 6. Radioactivity sampling (air, surfaces, liquids)
 7. Radioactive contamination measurement and analysis
 8. Personnel monitoring - internal (e.g., whole body counter) and external (e.g., TLD system)
 9. Instrument storage, calibration, and maintenance
 10. Decontamination (facilities, personnel, and equipment)
 11. Respiratory protection, including testing
 12. Contamination control
 13. Measuring and test equipment used for CSSC
 14. Masonry walls in the control and Auxiliary Buildings
 15. PORV block valves and actuators
 16. EGTS oxygen monitors
 17. Accident-related meteorological data collection equipment

18. Expendable and consumable items necessary for the functional performance of safety-related structures, systems, and components (i.e., weld rod, fuel oil, boric acid, snubber oil, etc.).
 19. Intake channel slopes
 20. Missile protection slabs and backfill
- b. The following items from Enclosure 2 of NUREG-0737, 'Clarification of TMI Action Plan Requirements,' (November 1980) do not appear on the list. Add the appropriate items to the list in Chapter 17 of the FSAR and provide a commitment in Chapter 17 of the FSAR that the remaining items are subject to the pertinent requirements of the FSAR operational quality assurance program or justify not doing so.

NUREG-0737
Enclosure 2
Clarification Item

- | | |
|--|-----------------------|
| 1. Plant safety parameter display console | I.D.2 |
| 2. Reactor coolant system vents | II.B.1 |
| 3. Plant shielding | II.B.2 |
| 4. Post-accident sampling capabilities | II.B.3 |
| 5. Accident monitoring instrumentation | II.F.1 |
| 6. Instrumentation for detection of inadequate core cooling | II.F.2 |
| 7. Automatic PORV isolation | II.K.3(1) |
| 8. Automatic trip of reactor coolant pumps | II.K.3(5) |
| 9. Emergency plans (and related equipment) | III.A.1.1/
III.A.2 |
| 10. Equipment and other items associated with emergency support facilities | III.A.1.2 |

11. Inplant I₂ radiation
monitoring

III.D.3.3

a. Response

1. The spent fuel pool structure is included on the CSSC list. The liner will be added to the list before initial criticality.
- 2, 3. Safety-related interlocks installed on these components are included on the CSSC list.
4. The CSSC list contains only safety-related structures and equipment. The reactor vessel head lifting rig is not utilized to mitigate or aid in the recovery of any postulated accidents, nor do we consider the failure of the rig to be a nuclear safety concern.
- 5-15. TVA's position on those items was documented during the Sequoyah unit 2 license review as specified in a letter from L. M. Mills to A. Schwencer dated April 2, 1981.
16. Watts Bar does not utilize oxygen monitors in the EGTS design.
17. Accident-related meteorological data collection equipment is controlled by separate procedures provided by the Division of Natural Resources and Data Services Branch and reviewed through the Office of Power Quality Assurance Audit Program.
18. These items are subject to the controls of the operational quality assurance program as specified in the Operational Quality Assurance Manual.
- 19, 20. These items will be added to the CSSC list before fuel load.

b. See response to Items 5 through 15.