

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

400 Chestnut Street Tower II

January 27, 1982

Director of Nuclear Reactor Regulation
Attention: Ms. E. Adensam, Chief
Licensing Branch No. 4
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, DC 20555



Dear Ms. Adensam:

In the Matter of the Application of) Docket Nos. 50-390
Tennessee Valley Authority) 50-391

Enclosed for NRC information is a revised response to NRC question 031.119 on Watts Bar Nuclear Plant. This revised response will be included in Amendment 47 to the Watts Bar Final Safety Analysis Report.

If you have any questions concerning this matter, please get in touch with D. P. Ormsby at FTS 858-2682.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

L. M. Mills

L. M. Mills, Manager
Nuclear Regulation and Safety

Sworn to and subscribed before me
this 27th day of Jan 1982

Bryant M. Lowery
Notary Public

My Commission Expires 4/4/82

Enclosure

Boo1
s
1/1

8202030120 820127
PDR ADOCK 05000390
A PDR

ENCLOSURE

WATTS BAR NUCLEAR PLANT UNITS 1 AND 2
REVISED RESPONSE TO NRC QUESTION 031.119

031.119 (Q31.44) (T3.11-2B) The response to Question 031.44(2) indicates that there are areas within the primary containment (e.g. steam generator enclosure and pressurizer enclosure) where accident conditions may exceed the values given in FSAR Table 3.11-2B. For each such area within the primary containment, please provide the following information:

- (1) The name of the area,
- (2) The maximum and minimum pressure and temperatures which are expected to occur before or during any design basis event,
- (3) A listing of all Class 1E equipment which is located in that area,
- (4) The extremes of temperature and pressure for which each such piece of equipment has been qualified, and
- (5) A justification for the use of each piece of equipment which is listed in part 3 above and which is not qualified for the environment described in Part 2 above.

Response

There are three areas within the containment where local conditions obtained from short-term subcompartment analyses may exceed the values given in FSAR Table 3.11-2B. These areas are the steam generator compartment, the pressurizer enclosure, and the reactor cavity area.

There is no safety-related equipment located within the steam generator compartment.

There are two RTDs (453 and 454) located in the pressurizer enclosure. However, the peak pressure in this enclosure is 14 psig (see FSAR figure 6.2.1-92) which is below the value of 14.7 psig given in FSAR Table 3.11-2B.

There are neutron detectors located in the reactor cavity area, but they are not used following a high energy line rupture. There is no other Class 1E equipment in the reactor cavity area.