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10 CFR 50.90

December 12, 2007

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Limerick Generating Station, Units 1 and 2
Facility Operating License Nos. NPF-39 and NPF-85
NRC Docket Nos. 50-352 and 50-353

SUBJECT: License Amendment Request
Proposed Administrative Changes to Technical Specifications

Pursuant to 10 CFR 50.90, "Application for amendment of license or construction permit," Exelon Generation Company, LLC (Exelon), proposes changes to the Technical Specifications (TS), Appendix A of Operating License Nos. NPF-39 and NPF-85 for Limerick Generating Station (LGS), Units 1 and 2, respectively.

The proposed administrative changes provide an editorial cleanup of the TS. The proposed changes involve: (1) correcting the index, (2) removing cycle specific requirements or notes that have since expired and are no longer applicable, (3) deleting references to previously deleted requirements, (4) changing references to the location of previously relocated information, and (5) editorial corrections. Evaluation of the proposed changes is provided in this attachment. Markups of the proposed TS changes are provided in Attachment 2.

Exelon has concluded that the proposed changes present no significant hazards consideration under the standards set forth in 10CFR 50.92.

This amendment request contains no regulatory commitments.

Exelon requests approval of the proposed amendment by December 12, 2008. Upon NRC approval, the amendment shall be implemented within 60 days of issuance.

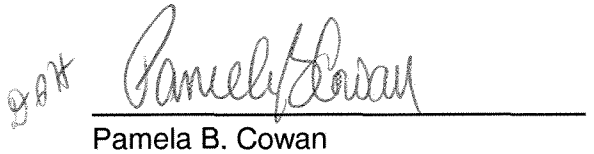
These proposed changes have been reviewed by the Plant Operations Review Committee and approved in accordance with Nuclear Safety Review Board procedures.

We are notifying the State of Pennsylvania of this application for changes to the Technical Specifications by transmitting a copy of this letter and its attachments to the designated State Official.

If you have any questions or require additional information, please contact Glenn Stewart at 610-765-5529.

I declare under penalty of perjury that the foregoing is true and correct. Executed on the 12th day of December, 2007.

Respectfully,



Pamela B. Cowan
Director, Licensing & Regulatory Affairs
Exelon Generation Company, LLC

Attachments: 1. Evaluation of Proposed Changes
2. Markup of Proposed Technical Specifications Pages

cc: Regional Administrator - NRC Region I w/ attachments
NRC Senior Resident Inspector - Limerick Generating Station "
NRC Project Manager, NRR - Limerick Generating Station "
Director, Bureau of Radiation Protection - Pennsylvania Department of Environmental Protection "

ATTACHMENT 1

License Amendment Request

Limerick Generating Station, Units 1 and 2

Docket Nos. 50-352 and 50-353

EVALUATION OF PROPOSED CHANGES

Subject: Proposed Administrative Changes to Technical Specifications

1.0 DESCRIPTION

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The proposed administrative changes provide an editorial cleanup of the TS. The proposed changes involve: (1) correcting the index, (2) removing cycle-specific requirements or notes that have since expired and are no longer applicable, (3) deleting references to previously deleted requirements, (4) changing references to the location of previously relocated information, and (5) editorial corrections. Evaluation of the proposed changes is provided in this attachment. Markups of the proposed TS changes are provided in Attachment 2.

2.0 PROPOSED CHANGES

The changes requested by this amendment application are described below.

1. Index Corrections

   a) LGS, Units 1 and 2 - Revise Index page ii to reflect the previously approved addition of TS Definition 1.35, "RECENTLY IRRADIATED FUEL" to TS page 1-6 and TS Definition 1.37a, "RESTRICTED AREA" to TS page 1-7, and the appropriate renumbering of the remaining TS Definitions.

   b) LGS, Units 1 and 2 - Change the word "Semiannual" to "Annual" in the title of the Radioactive Effluent Release Report on TS Index page xxvii to reflect the appropriate title for the report as specified under Section 6.9.1, "Routine Reports," on TS page 6-17.

2. Cycle-Specific Requirements or Notes that are No Longer Applicable

   a) LGS, Unit 1 - Delete Limiting Condition for Operation (LCO) 3.1.3.6, Action c. and associated Surveillance Requirement (SR) 4.1.3.6.d relative to repositioning uncoupled control rod 50-27. Also, delete the words "except as in 3.1.3.6.c or" from LCO 3.1.3.6, Action a.2. These requirements were limited to LGS, Unit 1, Cycle 7 only. LGS, Unit 1 is currently operating in Cycle 12. Therefore, these requirements are no longer applicable.

   b) LGS, Unit 2 - Delete the double asterisk and associated footnote from the 145^{o}F drywell average air temperature limit specified in TS LCO 3.6.1.7 and associated TS Action. This footnote was limited to LGS, Unit 2, Cycle 9 only. LGS, Unit 2 is currently operating in Cycle 10. Therefore, this footnote is no longer applicable.

3. References to Previously Deleted Requirements

   a) LGS, Units 1 and 2 - Delete the reference to TS SR 4.8.1.1.3 from TS SR 4.8.1.2 on TS page 3/4 8-9 since TS SR 4.8.1.1.3 was previously deleted and is no longer in TS.

4. References to the Location of Previously Relocated Information

   a) LGS, Units 1 and 2 - Replace the word "UFSAR" with the word "TRM" in the relocation information provided of TS pages 3/4 3-68, 3/4 3-110, 3/4 6-49, 3/4 6-51, 3/4 6-51a, and 3/4 8-21.

5. Editorial Changes

   a) LGS, Unit 2 - Capitalize the first letter "r" in the word "radioactive" in the title of the Radioactive Effluent Controls Program referenced under Item No. (1) in TS Definition 1.24 (OFFSITE DOSE CALCULATION MANUAL) on TS page 1-4.

   b) LGS, Unit 2 - Replace misspelled word "RELOCATD" with the word "RELOCATED" on TS page 3/4 3-110.

   c) LGS, Units 1 and 2 - Replace capitalized word "INOPERABLE" with lower case word "inoperable" in TS LCO 3.4.3.1, Actions B, C, D, and E on TS page 3/4 4-8.

   d) LGS, Unit 2 - Replace the word "or" with the word "of" in TS SR 4.4.3.1.b on TS page 3/4 4-8a.

   e) LGS, Units 1 and 2 - Replace the word "patch" with the word "path" in TS LCO 3.5.1.a.2 on TS page 3/4 5-1.

   f) LGS, Units 1 and 2 - Delete the "s" at the end of the word "valves" and replace the word "the" with the word "each" in TS LCO 3.6.3 on TS page 3/4 6-17.

   g) LGS, Unit 1 - Change the paragraph indent for TS SR 4.7.1.2.b.2 to line up with the paragraph indent for TS SR 4.7.1.2.b.1 on TS page 3/4 7-4.

   h) LGS, Unit 2 - Delete the duplicate word "both" from the first sentence in TS LCO 3.7.2, Action b.2 on TS page 3/4 7-6.

3.0 BACKGROUND

As a result of various changes to TS that have been requested and approved over time, a number of minor inconsistencies or editorial errors that have no safety impact have been introduced into the LGS Unit 1 and Unit 2 TS. The purpose of this amendment request is to correct those inconsistencies or errors.

4.0 TECHNICAL ANALYSIS

The Technical justification for each of the changes proposed in Section 2.0 of this request is provided below.

1. Index Corrections

   By letter dated August 23, 2006 (Reference 1), the NRC issued Amendment No. 185 to Facility Operating License No. NPF-39 and Amendment No. 146 to Facility Operating License No. NPF-85 for LGS, Units 1 and 2, respectively. These amendments revised LGS TS to support application of an alternate source term methodology. These amendments approved the addition of Definition No. 1.35 (RECENTLY IRRADIATED FUEL) to TS Section 1.0, "DEFINITIONS," on TS page 1-6. As a result of this addition, pre-existing Definition Nos. 1.35 (REFUELING FLOOR SECONDARY CONTAINMENT INTEGRITY), 1.36 (REPORTABLE EVENT), 1.37 (ROD DENSITY), 1.38 (SHUTDOWN MARGIN), and 1.39 (SITE BOUNDARY) were renumbered to 1.36 through 1.40, respectively.

   By letter dated June 29, 2007 (Reference 2), the NRC issued Amendment No. 187 to Facility Operating License No. NPF-39 and Amendment No. 148 to Facility Operating License No. NPF-85 for LGS, Units 1 and 2, respectively. These amendments revised LGS TS to incorporate revised requirements from Title 10 of the Code of Federal Regulations (CFR), Part 20. These amendments added Definition No. 1.37a (RESTRICTED AREA), to TS Section 1.0 on TS page 1-7.

   By letter dated August 10, 1994 (Reference 3), the NRC issued Amendment No. 73 to Facility Operating License No. NPF-39 and Amendment No. 35 to Facility Operating License No. NPF-85 for LGS, Units 1 and 2, respectively. These amendments revised various LGS TS sections to change the frequency for submitting the Semiannual Radioactive Effluent Release Report to the NRC from semiannually to annually. These changes involved changing the title of TS Section 6.9.1.8 on TS page 6-17 from "Semiannual Radioactive Effluent Release Report" to "Annual Radioactive Effluent Release Report."

   All of these amendments were approved as requested by Exelon; however, the Exelon requests did not include a revised TS Index to reflect the associated changes to TS Sections 1.0 or 6.9.1. The proposed changes described in this amendment request revise the TS Index to appropriately reflect the changes to these sections of TS that were previously approved by the amendments referenced above. The proposed changes are administrative in nature and do not involve any physical changes to structures, systems, or components (SSCs) in the plant, or the way SSCs are operated or controlled.

2. Cycle-Specific Requirements or Notes that are No Longer Applicable

   By letter dated January 16, 1998 (Reference 4), the NRC issued Amendment No. 124 to Facility Operating License No. NPF-39 for LGS, Unit 1. This amendment revised TS LCO 3.1.3.6 and associated TS SR 4.1.3.6 to allow operation of the unit with control rod 50-27 uncoupled from its drive for the remainder of operating Cycle 7. This amendment specified the conditions under which control rod 50-27 could be manipulated and modified the surveillance requirements to verify control rod position using neutron instrumentation. LGS, Unit 1 completed operating Cycle 7 in April, 1998, when the unit was shutdown for refueling outage 1R07, during which time the control rod drive was replaced and control rod 50-27 was re-coupled. LGS, Unit 1 is currently operating in Cycle 12. As a result, these requirements are no longer applicable. Therefore, the proposed change deletes LCO 3.1.3.6.c and SR 4.1.3.6.d from the LGS, Unit 1 TS, and deletes the words "except as in 3.1.3.6.c or" from LGS, Unit 1 LCO 3.1.3.6, Action a.2.

   By letter dated July 7, 2006 (Reference 5), the NRC issued Amendment No. 145 to Facility Operating License No. NPF-85 for LGS, Unit 2. This amendment approved a one-time change to TS LCO 3.6.1.7 by adding a footnote (**) to the TS limit for drywell average air temperature of 145 degrees Fahrenheit (^{o}F) to allow continued operation of LGS, Unit 2, with drywell average air temperature no greater than 148^{o}F for the remainder of operating Cycle 9, or until the next shutdown of sufficient duration to allow for unit cooler fan repairs, whichever came first. LGS, Unit 2 completed operating Cycle 9 in March, 2007, when the unit was shutdown for refueling outage 2R09, during which the unit cooler fans were repaired. LGS, Unit 2 is currently in operating Cycle 10. As a result, this footnote is no longer applicable. Therefore, the proposed change deletes this footnote from LGS, Unit 2, LCO 3.6.1.7.

   The proposed changes are administrative in nature and do not involve any physical changes to SSCs in the plant, or the way SSCs are operated or controlled.

3. References to Previously Deleted Requirements

   By letter dated November 6, 2007 (Reference 6), the NRC issued Amendment No. 189 to Facility Operating License No. NPF-39 and Amendment No. 150 to Facility Operating License No. NPF-85 for LGS, Units 1 and 2, respectively. These amendments modified emergency diesel generator (EDG) testing requirements to improve EDG reliability by reducing potential equipment degradation due to excessive testing. These amendments approved the deletion of SR 4.8.1.1.3 which required a special report to be submitted to the NRC within 30 days of all EDG failures.

   TS Section 3.8.1 specifies the LCO and SR requirements for both A.C. Sources - Operating (LCO 3.8.1.1) and A.C. Sources - Shutdown (LCO 3.8.1.2). SR 4.8.1.1.3 was previously specified under A.C. Sources - Operating. However, the operability of A.C. Sources - Shutdown is demonstrated by performing SR 4.8.1.2, which references the surveillance requirements under A.C. Sources - Operating, including

SR 4.8.1.1.3. SR 4.8.1.1.3 was an administrative requirement for reporting to the NRC and did not, in and of itself, help to demonstrate the operability of the A.C. Sources. Deleting this reporting requirement did not impact the safe operation of the plant and was approved by the NRC in the amendment referenced above. Therefore, the proposed change deletes the reference to previously deleted SR 4.8.1.1.3 from SR 4.8.1.2, and makes an associated editorial change to SR 4.8.1.2 regarding the reference to the remaining SRs 4.8.1.1.1 and 4.8.1.1.2.

The proposed change is administrative in nature and does not involve any physical changes to SSCs in the plant, or the way SSCs are operated or controlled.

4. References to the Location of Previously Relocated Information

The information provided in TS Sections 3/4.3.7.2, 3/4.3.8, 3.6.5.2.1, 3.6.5.2.2 and 3/4.8.4.1 was previously relocated to licensee documents that are controlled under the requirements of 10 CFR 50.59 based on NRC approved amendments.

Specifically, by letter dated August 29, 1994 (Reference 7), the NRC issued Amendment No. 75 to Facility Operating License No. NPF-39 and Amendment No. 36 to Facility Operating License No. NPF-85 for LGS, Units 1 and 2, respectively. These amendments relocated the seismic monitoring instrumentation requirements from TS Section 3/4.3.7.2 to the Updated Final Safety Analysis Report (UFSAR).

By letter dated August 24, 1995 (Reference 8), the NRC issued Amendment No. 100 to Facility Operating License No. NPF-39 and Amendment No. 64 to Facility Operating License No. NPF-85 for LGS, Units 1 and 2, respectively. These amendments relocated the turbine overspeed protection system requirements from TS Section 3/4.3.8 to the UFSAR.

By letter dated November 20, 1995 (Reference 9), the NRC issued Amendment No. 105 to Facility Operating License No. NPF-39 and Amendment No. 69 to Facility Operating License No. NPF-85 for LGS, Units 1 and 2, respectively. These amendments relocated the Reactor Enclosure and Refueling Area Secondary Containment Automatic Isolation Valves from TS Sections 3.6.5.2.1 and 3.6.5.2.2, respectively, to the UFSAR.

By letter dated June 22, 1995 (Reference 10), the NRC issued Amendment No. 93 to Facility Operating License No. NPF-39 and Amendment No. 57 to Facility Operating License No. NPF-85 for LGS, Units 1 and 2, respectively. These amendments relocated the primary containment conductor protection device requirements from TS Section 3/4.4.8.1 to the UFSAR.

TS pages 3/4 3-68, 3/4 3-110, 3/4 6-49, 3/4 6-51, 3/4 6-51a, and 3/4 8-21 each indicate that the information from their respective TS sections was relocated to the UFSAR. Subsequent to implementation of the above license amendments, LGS implemented a Technical Requirements Manual (TRM), which is incorporated into the

LGS UFSAR by reference and is controlled under the requirements of 10 CFR 50.59. The TRM contains the technical requirements that are relocated from TS as approved by the NRC via license amendments. Once the TRM was established, the technical requirements that had previously been removed from TS and relocated to the UFSAR via the license amendments referenced above were transferred from the UFSAR to the TRM under the requirements of 10 CFR 50.59. Although the reference to the UFSAR on these TS pages is technically correct, since the TRM is incorporated into the UFSAR by reference, the proposed change replaces the word "UFSAR" with the word "TRM" to more explicitly state the current location of the technical requirements relocated from these TS pages. The proposed change is administrative in nature and does not involve any physical changes to SSCs in the plant, or the way SSCs are operated or controlled.

5. Editorial Changes

   The changes described under Item 5 of Section 2.0 strictly involve editorial changes to correct typographical errors, grammar, and paragraph indentation, and delete duplicate wording. These proposed changes are non-substantive changes that have no impact on safe operation of the plant in that they do not involve any physical changes to SSCs in the plant, or the way SSCs are operated or controlled.

5.0 REGULATORY ANALYSIS

5.1 No Significant Hazards Consideration

Exelon has evaluated whether or not a significant hazards consideration is involved with the proposed amendment by focusing on the three standards set forth in 10 CFR 50.92, “Issuance of amendment,” as discussed below:

1. Do the proposed changes involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No. The proposed changes are administrative in nature and do not impact the physical configuration or function of plant structures, systems, or components (SSCs) or the manner in which SSCs are operated, maintained, modified, tested, or inspected. The proposed changes do not impact the initiators or assumptions of analyzed events, nor do they impact mitigation of accidents or transient events. Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Do the proposed changes create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No. The proposed changes are administrative in nature and do not alter plant configuration, require that new plant equipment be installed, alter assumptions made about accidents previously evaluated, or impact the function of plant SSCs or the manner in which SSCs are operated, maintained, modified, tested, or inspected. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Do the proposed changes involve a significant reduction in a margin of safety?

Response: No. The proposed changes are administrative in nature and do not involve any physical changes to plant SSCs or the manner in which SSCs are operated, maintained, modified, tested, or inspected. The proposed changes do not involve a change to any safety limits, limiting safety system settings, limiting conditions of operation, or design parameters for any SSC. The proposed changes do not impact any safety analysis assumptions and do not involve a change in initial conditions, system response times, or other parameters affecting an accident analysis. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Based on the above, Exelon concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of “no significant hazards consideration” is justified.

5.2 Applicable Regulatory Requirements/Criteria

In Title 10 of the Code of Federal Regulations (10 CFR) Section 50.36, the Nuclear Regulatory Commission (NRC) established its regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in a plant’s TSs.

The proposed changes are administrative in nature and do not involve any physical changes to plant SSCs or the manner in which SSCs are operated, maintained, modified, tested, or inspected. The proposed changes do not involve a change to any safety limits, limiting safety system settings, limiting control settings, limiting conditions of operation, surveillance requirements, design features, or administrative controls required by 10 CFR 50.36.

In conclusion, based on the considerations discussed above, (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

6.0 ENVIRONMENTAL CONSIDERATION

A review has determined that the proposed amendment would change a requirement with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR 20, or would change an inspection or surveillance requirement. However, the proposed amendment does not involve (i) a significant hazards consideration, (ii) a significant change in the types or significant increase in the amounts of any effluent that may be released offsite, or (iii) a significant increase in individual or cumulative occupational radiation exposure. Accordingly, the proposed amendment meets the eligibility criterion for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the proposed amendment.

7.0 REFERENCES

1. Letter dated August 23, 2006, from R. Guzman, USNRC to C. Crane, Exelon Nuclear, "Limerick Generating Station, Units 1 and 2 - Issuance of Amendments RE: Application of Alternate Source Term Methodology (TAC Nos. MC2295 and MC2296)."

2. Letter dated June 29, 2007, from P. Bamford, USNRC to C. Crane, AmerGen Energy Company, "Limerick Generating Station, Units 1 and 2 - Issuance of Amendment RE: Incorporation of Revised 10 CFR Part 20 Requirements into the Technical Specifications (TAC Nos. MD4946 and MD4947)."

3. Letter dated August 10, 1994, from F. Rinaldi, USNRC to G. Hunger, Philadelphia Electric Company, "Frequency for Submitting Semiannual Radioactive Effluent Release Report, Limerick Generating Station, Units 1 and 2 (TAC Nos. M89175 and M89176)."

4. Letter dated January 16, 1998, from B. Buckley, USNRC to G. Hunger, PECO Energy Company, "Revision to Technical Specifications Regarding Control Rod 50-27, Limerick Generating Station, Unit 1 (TAC Nos. M99854 and M99855)."

5. Letter dated July 7, 2006, from R. Guzman, USNRC to C. Crane, Exelon Nuclear, " Limerick Generating Station, Unit 2 - Issuance of Amendment RE: One-Time Change to the Drywell Average Air Temperature Limit (TAC No. MD2315)."

6. Letter dated November 6, 2007, from P. Bamford, USNRC to C. Crane, Exelon Generation Company, LLC, "Limerick Generating Station, Units 1 and 2 - Issuance of Amendment RE: Changes to Technical Specification Emergency Diesel Generator Testing Requirements (TAC Nos. MD3710 and MD3711)."

7. Letter dated August 29, 1994, from F. Rinaldi, USNRC to G. Hunger, Philadelphia Electric Company, "Relocation of Seismic Monitoring Equipment, Limerick Generating Station, Units 1 and 2 (TAC Nos. M88608 and M88655)."

8. Letter dated August 24, 1995, from F. Rinaldi, USNRC to G. Hunger, PECO Energy Company, "Relocation of Turbine Overspeed Protection System Requirements, Limerick Generating Station, Units 1 and 2 (TAC Nos. M90375 and M90376)."

9. Letter dated November 20, 1995, from F. Rinaldi, USNRC to G. Hunger, PECO Energy Company, "Limerick Generating Station, Units 1 and 2 (TAC Nos. M93690 and M93691)."

10. Letter dated June 22, 1995, from F. Rinaldi, USNRC to G. Hunger, PECO Energy Company, "Relocation of Primary Containment Conductor Protection Device Requirements, Limerick Generating Station, Units 1 and 2 (TAC Nos. M90375, M90376, M90512, and M90513)."

ATTACHMENT 2

License Amendment Request

Limerick Generating Station, Units 1 and 2
Docket Nos. 50-352 and 50-353

Proposed Administrative Changes to Technical Specifications

Markup of Proposed Technical Specifications Pages

Unit 1 TS Pages

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DEFINITIONS

SECTION

INDEX

ADMINISTRATIVE CONTROLS

REACTIVITY CONTROL SYSTEMS

CONTROL ROD DRIVE COUPLING

LIMITING CONDITION FOR OPERATION

3.1.3.6 All control rods shall be coupled to their drive mechanisms.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 5*.

ACTION:

a. In OPERATIONAL CONDITIONS 1 and 2 with one control rod not coupled to its associated drive mechanism, within 2 hours:

1. If permitted by the RWM, insert the control rod drive mechanism to accomplish recoupling and verify recoupling by withdrawing the control rod, and:

a) Observing any indicated response of the nuclear instrumentation, and

b) Demonstrating that the control rod will not go to the overtravel position.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.

2. If recoupling is not accomplished on the first attempt or, if not permitted by the RWM, then except as in 3.1.3.6.c or until permitted by the RWM, declare the control rod inoperable, insert the control rod and disarm the associated directional control valves** either: delete

a) Electrically, or

b) Hydraulically by closing the drive water and exhaust water isolation valves.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.

b. In OPERATIONAL CONDITION 5* with a withdrawn control rod not coupled to its associated drive mechanism, within 2 hours either:

1. Insert the control rod to accomplish recoupling and verify recoupling by withdrawing the control rod and demonstrating that the control rod will not go to the overtravel position, or

2. If recoupling is not accomplished, insert the control rod and disarm the associated directional control valves** either:

a) Electrically, or

b) Hydraulically by closing the drive water and exhaust water isolation valves.

c. For control rod 50-27, for the remainder of Unit 1 Cycle 7, if coupling can not be established the uncoupled rod may be withdrawn when rated thermal power exceeds 10% only if all the following conditions are satisfied: delete

1) The uncoupled control rod may not be withdrawn past notch position 46, and

2) No other uncoupled control rod is withdrawn.

* At least each withdrawn control rod. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

** May be rearmed intermittently, under administrative control, to permit testing associated with restoring the control rod to OPERABLE status.

SURVEILLANCE REQUIREMENTS

4.1.3.6 Each affected control rod shall be demonstrated to be coupled to its drive mechanism by observing any indicated response of the nuclear instrumentation while withdrawing the control rod to the fully withdrawn position and then verifying that the control rod drive does not go to the overtravel position:

a. Prior to reactor criticality after completing CORE ALTERATIONS that could have affected the control rod drive coupling integrity,

b. Anytime the control rod is withdrawn to the "Full out" position in subsequent operation,

c. Following maintenance on or modification to the control rod or control rod drive system which could have affected the control rod drive coupling integrity, and

d. When repositioning the uncoupled control rod per Specification 3.1.3.6.d the uncoupled rod's position shall be verified to have followed the control rod drive by neutron instrumentation (LPRM or TIP). If the control blade can not be verified to have followed the drive out to its final position, then the rod shall be completely inserted and the control rod directional valves disarmed as stated in 3.1.3.6.a.2.

delete

JAN 1 6 1998

Section 3.3.7.2 (Deleted)

THE INFORMATION FROM THIS TECHNICAL SPECIFICATIONS SECTION HAS BEEN RELOCATED TO THE ~~UFSAR~~ TRM. TECHNICAL SPECIFICATIONS PAGES 3/4 3-69 THROUGH 3/4 3-72 OF THIS SECTION HAVE BEEN INTENTIONALLY OMITTED.

Section 3/4.3.8 (Deleted)

THE INFORMATION FROM THIS TECHNICAL
SPECIFICATIONS SECTION HAS BEEN
RELOCATED TO THE ~~UFSAR.~~ TRM
TECHNICAL SPECIFICATIONS PAGE 3/4 3-111
HAS BEEN INTENTIONALLY OMITTED.

SEP 21 1995

REACTOR COOLANT SYSTEM

3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.3.1 The following reactor coolant leakage detection systems shall be OPERABLE:

a. The primary containment atmosphere gaseous radioactivity monitoring system,

b. The drywell floor drain sump flow monitoring system,

c. The drywell unit coolers condensate flow rate monitoring system, and

d. The primary containment pressure and temperature monitoring system.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.*
* - The primary containment gaseous radioactivity monitor is not required to be operable until Operational Condition 2.

ACTIONS:

A. With the primary containment atmosphere gaseous radioactivity monitoring system inoperable, analyze grab samples of primary containment atmosphere at least once per 12 hours AND restore primary containment atmosphere gaseous radioactivity monitoring system to OPERABLE status within 30 days.

B. With the drywell floor drain sump flow monitoring system INOPERABLE restore the drywell floor drain sump flow monitoring system to OPERABLE status within 30 days AND increase monitoring frequency of drywell unit cooler condensate flow rate (SR 4.4.3.2.1.c) to once every 8 hours.

C. With the drywell unit coolers condensate flow rate monitoring system INOPERABLE, AND the primary containment atmosphere gaseous radioactivity monitoring system OPERABLE, perform a channel check of the primary containment atmosphere gaseous radioactivity monitoring system (SR 4.4.3.1.a) once per 8 hours.

D. With the primary containment pressure and temperature monitoring system INOPERABLE, restore the primary containment pressure and temperature monitoring system to OPERABLE status within 30 days. NOTE: All other Tech Spec Limiting Conditions For Operation and Surveillance Requirements associated with the primary containment pressure/temperature monitoring system still apply. Affected Tech Spec Sections include: 3/4.3.7.5, 4.4.3.2.1, 3/4.6.1.6, and 3/4.6.1.7.

E. With the primary containment atmosphere gaseous radioactivity monitoring system INOPERABLE AND the drywell unit coolers condensate flow rate monitoring system INOPERABLE, restore the primary containment atmosphere gaseous radioactivity monitoring system to OPERABLE status within 30 days OR restore the drywell unit coolers condensate flow rate monitoring system to OPERABLE status within 30 days. With the primary containment atmosphere gaseous radioactivity monitoring system inoperable, analyze grab samples of primary containment atmosphere at least once per 12 hours.

3/4.5 EMERGENCY CORE COOLING SYSTEMS

3/4.5.1 ECCS - OPERATING

LIMITING CONDITION FOR OPERATION

3.5.1 The emergency core cooling systems shall be OPERABLE with:

a. The core spray system (CSS) consisting of two subsystems with each subsystem comprised of:

1. Two OPERABLE CSS pumps, and

2. An OPERABLE flow ~~path~~ path capable of taking suction from the suppression chamber and transferring the water through the spray sparger to the reactor vessel.

b. The low pressure coolant injection (LPCI) system of the residual heat removal system consisting of four subsystems with each subsystem comprised of:

1. One OPERABLE LPCI pump, and

2. An OPERABLE flow path capable of taking suction from the suppression chamber and transferring the water to the reactor vessel.

c. The high pressure coolant injection (HPCI) system consisting of:

1. One OPERABLE HPCI pump, and

2. An OPERABLE flow path capable of taking suction from the suppression chamber and transferring the water to the reactor vessel.

d. The automatic depressurization system (ADS) with at least five OPERABLE ADS valves.

APPLICABILITY: OPERATIONAL CONDITION 1, 2* ** #, and 3* ** ##.

*The HPCI system is not required to be OPERABLE when reactor steam dome pressure is less than or equal to 200 psig.

**The ADS is not required to be OPERABLE when the reactor steam dome pressure is less that or equal to 100 psig.

#See Special Test Exception 3.10.6.

##Two LPCI subsystems of the RHR system may be inoperable in that they are aligned in the shutdown cooling mode when reactor vessel pressure is less than the RHR Shutdown cooling permissive setpoint.

CONTAINMENT SYSTEMS

3/4.6.3 PRIMARY CONTAINMENT ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.6.3 Each primary containment isolation valve and ~~the~~ instrumentation line excess flow check valve shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

a. With one or more of the primary containment isolation valves inoperable,** maintain at least one isolation valve OPERABLE in each affected penetration that is open and within 4 hours either:

1. Restore the inoperable valve(s) to OPERABLE status, or

2. Isolate each affected penetration by use of at least one de-activated automatic valve secured in the isolated position,* or

3. Isolate each affected penetration by use of at least one closed manual valve or blind flange.*

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. With one or more of the instrumentation line excess flow check valves inoperable, operation may continue and the provisions of Specification 3.0.3 are not applicable provided that within 4 hours either:

1. The inoperable valve is returned to OPERABLE status, or

2. The instrument line is isolated and the associated instrument is declared inoperable.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

c. With one or more scram discharge volume vent or drain valves inoperable, perform the applicable actions specified in Specification 3.1.3.1.

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* Isolation valves closed to satisfy these requirements may be reopened on an intermittent basis under administrative control.

** Except for the scram discharge volume vent and drain valves.

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LIMITING CONDITION FOR OPERATION (Continued)

ACTION: (Continued)

4. With three ESW pump/diesel generator pairs** inoperable, restore at least one inoperable ESW pump/diesel generator pair** to OPERABLE status within 72 hours, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

5. With four ESW pump/diesel generator pairs** inoperable, restore at least one inoperable ESW pump/diesel generator pair** to OPERABLE status within 8 hours, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. In OPERATIONAL CONDITION 4 or 5:

1. With only one emergency service water pump and its associated flowpath OPERABLE, restore at least two pumps with at least one flow path to OPERABLE status within 72 hours or declare the associated safety related equipment inoperable and take the ACTION required by Specifications 3.5.2 and 3.8.1.2.

c. In OPERATIONAL CONDITION *

1. With only one emergency service water pump and its associated flow path OPERABLE, restore at least two pumps with at least one flow path to OPERABLE status within 72 hours or verify adequate cooling remains available for the diesel generators required to be OPERABLE or declare the associated diesel generator(s) inoperable and take the ACTION required by Specification 3.8.1.2. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENT

4.7.1.2 At least the above required emergency service water system loop(s) shall be demonstrated OPERABLE:

a. In accordance with the Surveillance Frequency Control Program by verifying that each valve (manual, power-operated, or automatic) that is not locked, sealed, or otherwise secured in position, is in its correct position.

b. In accordance with the Surveillance Frequency Control Program by verifying that:

1. Each automatic valve actuates to its correct position on its appropriate ESW pump start signal.

2. Each pump starts automatically when its associated diesel generator starts.

*When handling irradiated fuel in the secondary containment.
**An ESW pump/diesel generator pair consists of an ESW pump and its associated diesel generator. If either an ESW pump or its associated diesel generator becomes inoperable, then the ESW pump/diesel generator pair is inoperable.

ELECTRICAL POWER SYSTEMS

A.C. SOURCES - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.8.1.2 As a minimum, the following A.C. electrical power sources shall be OPERABLE:

a. One circuit between the offsite transmission network and the onsite Class 1E distribution system, and

b. Two diesel generators each with:

1. A day fuel tank containing a minimum of 200 gallons of fuel.

2. A fuel storage system containing a minimum of 33,500 gallons of fuel.

3. A fuel transfer pump.

APPLICABILITY: OPERATIONAL CONDITIONS 4, 5, and *.

ACTION:

a. With less than the above required A.C. electrical power sources OPERABLE, suspend CORE ALTERATIONS, handling of irradiated fuel in the secondary containment, operations with a potential for draining the reactor vessel and crane operations over the spent fuel storage pool when fuel assemblies are stored therein. In addition, when in OPERATIONAL CONDITION 5 with the water level less than 22 feet above the reactor pressure vessel flange, immediately initiate corrective action to restore the required power sources to OPERABLE status as soon as practical.

b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.8.1.2 At least the above required A.C. electrical power sources shall be demonstrated OPERABLE per Surveillance Requirements 4.8.1.1.1, 4.8.1.1.2, and 4.8.1.1.3. delete and

*When handling irradiated fuel in the secondary containment.

Section 3/4.8.4.1 (Deleted)

THE INFORMATION FROM THIS TECHNICAL SPECIFICATION SECTION HAS BEEN RELOCATED TO THE ~~UFSAR~~ TRM. TECHNICAL SPECIFICATIONS PAGES 3/4 8-21 THROUGH 3/4 8-26 OF THE SECTION HAVE BEEN INTENTIONALLY OMITTED.

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DEFINITIONS

SECTION

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INDEX

ADMINISTRATIVE CONTROLS

LOGIC SYSTEM FUNCTIONAL TEST

1.20 A LOGIC SYSTEM FUNCTIONAL TEST shall be a test of all logic components, i.e., all relays and contacts, all trip units, solid state logic elements, etc, of a logic circuit, from sensor through and including the actuated device, to verify OPERABILITY. The LOGIC SYSTEM FUNCTIONAL TEST may be performed by any series of sequential, overlapping or total system steps such that the entire logic system is tested.

LOW (POWER) TRIP SETPOINT (LTSP)

1.20a The low power trip setpoint associated with the Rod Block Monitor (RBM) rod block trip setting applicable between 30% and 65% reactor thermal power.

1.21 (Deleted)

MEMBER(S) OF THE PUBLIC

1.22 MEMBER OF THE PUBLIC means any individual except when that individual is receiving an occupational dose.

MAPFAC(F)-(MAPLHGR FLOW FACTOR)

1.22a A core flow dependent multiplication factor used to flow bias the standard Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) limit.

MAPFAC(P)-(POWER DEPENDENT MAPLHGR MULTIPLIER)

1.22b A core power dependent multiplication factor used to power bias the standard Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) limit.

MINIMUM CRITICAL POWER RATIO (MCPR)

1.23 The MINIMUM CRITICAL POWER RATIO (MCPR) shall be the smallest CPR which exists in the core (for each class of fuel). Associated with the minimum critical power ratio is a core flow dependent (MCPR(F)) and core power dependent (MCPR(P)) minimum critical power ratio.

OFFSITE DOSE CALCULATION MANUAL

1.24 The OFFSITE DOSE CALCULATION MANUAL (ODCM) shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm/trip setpoints, and in the conduct of the Radiological Environmental Monitoring Program. The ODCM shall also contain (1) the Radioactive Effluent Controls and Radiological Environmental Monitoring Programs required by Section 6.8.4 and (2) descriptions of the information that should be included in the Annual Radiological Environmental Operating and Annual Radioactive Effluent Release Reports required by Specifications 6.9.1.7 and 6.9.1.8.

OPERABLE - OPERABILITY

1.25 A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s) and when all necessary attendant instrumentation, controls, electrical power, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its function(s) are also capable of performing their related support function(s).

Section 3.3.7.2 (Deleted)

THE INFORMATION FROM THIS TECHNICAL SPECIFICATIONS SECTION HAS BEEN RELOCATED TO THE ~~UFSAR.~~ TRM TECHNICAL SPECIFICATIONS PAGES 3/4 3-69 THROUGH 3/4 3-72 OF THIS SECTION HAVE BEEN INTENTIONALLY OMITTED.

Section 3/4.3.8 (Deleted)

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3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.3.1 The following reactor coolant leakage detection systems shall be OPERABLE:

a. The primary containment atmosphere gaseous radioactivity monitoring system,

b. The drywell floor drain sump flow monitoring system,

c. The drywell unit coolers condensate flow rate monitoring system, and

d. The primary containment pressure and temperature monitoring system.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.*
* - The primary containment gaseous radioactivity monitor is not required to be operable until Operational Condition 2.

ACTIONS:

A. With the primary containment atmosphere gaseous radioactivity monitoring system inoperable, analyze grab samples of primary containment atmosphere at least once per 12 hours AND restore primary containment atmosphere gaseous radioactivity monitoring system to OPERABLE status within 30 days.

B. With the drywell floor drain sump flow monitoring system INOPERABLE, restore the drywell floor drain sump flow monitoring system to OPERABLE status within 30 days AND increase monitoring frequency of drywell unit cooler condensate flow rate (SR 4.4.3.2.1.c) to once every 8 hours.

C. With the drywell unit coolers condensate flow rate monitoring system INOPERABLE, AND the primary containment atmosphere gaseous radioactivity monitoring system OPERABLE, perform a channel check of the primary containment atmosphere gaseous radioactivity monitoring system (SR 4.4.3.1.a) once per 8 hours.

D. With the primary containment pressure and temperature monitoring system INOPERABLE, restore the primary containment pressure and temperature monitoring system to OPERABLE status within 30 days. Note: All other Tech Spec Limiting Conditions For Operation and Surveillance Requirements associated with the primary containment pressure/temperature monitoring system still apply. Affected Tech Spec Sections include: 3/4.3.7.5, 4.4.3.2.1, 3/4.6.1.6, and 3/4.6.1.7.

E. With the primary containment atmosphere gaseous radioactivity monitoring system INOPERABLE AND the drywell unit coolers condensate flow rate monitoring system INOPERABLE, restore the primary containment atmosphere gaseous radioactivity monitoring system to OPERABLE status within 30 days OR restore the drywell unit coolers condensate flow rate monitoring system to OPERABLE status within 30 days. With the primary containment atmosphere gaseous radioactivity monitoring system inoperable, analyze grab samples of primary containment atmosphere at least once per 12 hours.

ACTIONS (Continued)

.. With any other two or more leak detection systems inoperable other than ACTION E above OR with required Actions and associated Completion Time of ACTIONS A, B, C, D or E not met, be in HOT SHUTDOWN within 12 hours AND in COLD SHUTDOWN within the next 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.1 The reactor coolant system leakage detection systems shall be demonstrated operable by:

a. Perform a CHANNEL CHECK of the primary containment atmosphere gaseous radioactivity monitoring system in accordance with the Surveillance Frequency Control Program.

b. Perform a CHANNEL FUNCTIONAL TEST of required leakage detection instrumentation in accordance with the Surveillance Frequency Control Program. This does not apply to containment pressure and temperature monitoring system.

c. Perform a CHANNEL CALIBRATION of required leakage detection instrumentation in accordance with the Surveillance Frequency Control Program. This does not apply to containment pressure and temperature monitoring system.

d. Monitor primary containment pressure AND primary containment temperature in accordance with the Surveillance Frequency Control Program.

3/4.5 EMERGENCY CORE COOLING SYSTEMS

3/4.5.1 ECCS - OPERATING

LIMITING CONDITION FOR OPERATION

3.5.1 The emergency core cooling systems shall be OPERABLE with:

a. The core spray system (CSS) consisting of two subsystems with each subsystem comprised of:

1. Two OPERABLE CSS pumps, and

2. An OPERABLE flow ~~patch~~ path capable of taking suction from the suppression chamber ~~and~~ transferring the water through the spray sparger to the reactor vessel.

b. The low pressure coolant injection (LPCI) system of the residual heat removal system consisting of four subsystems with each subsystem comprised of:

1. One OPERABLE LPCI pump, and

2. An OPERABLE flow path capable of taking suction from the suppression chamber and transferring the water to the reactor vessel.

c. The high pressure coolant injection (HPCI) system consisting of:

1. One OPERABLE HPCI pump, and

2. An OPERABLE flow path capable of taking suction from the suppression chamber and transferring the water to the reactor vessel.

d. The automatic depressurization system (ADS) with at least five OPERABLE ADS valves.

APPLICABILITY: OPERATIONAL CONDITION 1, 2* ** #, and 3* ** ##.

*The HPCI system is not required to be OPERABLE when reactor steam dome pressure is less than or equal to 200 psig.

**The ADS is not required to be OPERABLE when the reactor steam dome pressure is less that or equal to 100 psig.

#See Special Test Exception 3.10.6.

##Two LPCI subsystems of the RHR system may be inoperable in that they are aligned in the shutdown cooling mode when reactor vessel pressure is less than the RHR Shutdown cooling permissive setpoint.

DRYWELL AVERAGE AIR TEMPERATURE

LIMITING CONDITION FOR OPERATION

3.6.1.7 Drywell average air temperature shall not exceed 145°F**. delete

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION: delete

With the drywell average air temperature greater than 145°F**, reduce the average air temperature to within the limit within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.7 The drywell average air temperature shall be the volumetric average of the temperatures at the following locations and shall be determined to be within the limit in accordance with the Surveillance Frequency Control Program:

	Approximate Elevation	Number of Installed Sensors*
a.	330'	3
b.	320'	3
c.	260'	3
d.	248'	6

* At least one reading from each elevation is required for a volumetric average calculation.

** Operation may continue with drywell average air temperature no greater than 148°F for the remainder of the current operating cycle (Cycle 9), or until the next shutdown of sufficient duration to allow for unit cooler fan repairs, whichever comes first. delete

3/4.6.3 PRIMARY CONTAINMENT ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.6.3 Each primary containment isolation valve and ~~the~~ each instrumentation line excess flow check valve shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

a. With one or more of the primary containment isolation valves inoperable,** maintain at least one isolation valve OPERABLE in each affected penetration that is open and within 4 hours either:

1. Restore the inoperable valve(s) to OPERABLE status, or

2. Isolate each affected penetration by use of at least one de-activated automatic valve secured in the isolated position,* or

3. Isolate each affected penetration by use of at least one closed manual valve or blind flange.*

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. With one or more of the instrumentation line excess flow check valves inoperable, operation may continue and the provisions of Specification 3.0.3 are not applicable provided that within 4 hours either:

1. The inoperable valve is returned to OPERABLE status, or

2. The instrument line is isolated and the associated instrument is declared inoperable.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

c. With one or more scram discharge volume vent or drain valves inoperable, perform the applicable actions specified in Specification 3.1.3.1.

* Isolation valves closed to satisfy these requirements may be reopened on an intermittent basis under administrative control.
** Except for the scram discharge volume vent and drain valves.

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3/4.7.2 CONTROL ROOM EMERGENCY FRESH AIR SUPPLY SYSTEM - COMMON SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.2 Two independent control room emergency fresh air supply system subsystems shall be OPERABLE.

APPLICABILITY: All OPERATIONAL CONDITIONS and when RECENTLY IRRADIATED FUEL is being handled in the secondary containment, or during operations with a potential for draining the reactor vessel.

ACTION:

a. In OPERATIONAL CONDITION 1, 2, or 3:

1. With the Unit 1 diesel generator for one control room emergency fresh air supply subsystem inoperable for more than 30 days, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

2. With one control room emergency fresh air supply subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

3. With one control room emergency fresh air supply subsystem inoperable and the other control room emergency fresh air supply subsystem with an inoperable Unit 1 diesel generator, restore the inoperable subsystem to OPERABLE status or restore the Unit 1 diesel generator to OPERABLE status within 72 hours, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

4. With the Unit 1 diesel generators for both control room emergency fresh air supply subsystems inoperable for more than 72 hours, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. In OPERATIONAL CONDITION 4, 5 or when RECENTLY IRRADIATED FUEL is being handled in the secondary containment, or during operations with a potential for draining the reactor vessel:

1. With one control room emergency fresh air supply subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days, or initiate and maintain operation of the OPERABLE subsystem in the radiation isolation mode of operation.

2. With ~~both~~ delete control room emergency fresh air supply subsystem inoperable, suspend handling of RECENTLY IRRADIATED FUEL in the secondary containment and operations with a potential for draining the reactor vessel. The provisions of Specification 3.0.3 are not applicable.

ELECTRICAL POWER SYSTEMS

A.C. SOURCES - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.8.1.2 As a minimum, the following A.C. electrical power sources shall be OPERABLE:

a. One circuit between the offsite transmission network and the onsite Class 1E distribution system, and

b. Two diesel generators each with:

1. A day fuel tank containing a minimum of 200 gallons of fuel.

2. A fuel storage system containing a minimum of 33,500 gallons of fuel.

3. A fuel transfer pump.

APPLICABILITY: OPERATIONAL CONDITIONS 4, 5, and *.

ACTION:

a. With less than the above required A.C. electrical power sources OPERABLE, suspend CORE ALTERATIONS, handling of irradiated fuel in the secondary containment, operations with a potential for draining the reactor vessel and crane operations over the spent fuel storage pool when fuel assemblies are stored therein. In addition, when in OPERATIONAL CONDITION 5 with the water level less than 22 feet above the reactor pressure vessel flange, immediately initiate corrective action to restore the required power sources to OPERABLE status as soon as practical.

b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.8.1.2 At least the above required A.C. electrical power sources shall be demonstrated OPERABLE per Surveillance Requirements 4.8.1.1.1 and 4.8.1.1.2. and 4.8.1.1.3. delete

*When handling irradiated fuel in the secondary containment.

Section 3/4 8.4.1 (Deleted)

THE INFORMATION FROM THIS TECHNICAL SPECIFICATION SECTION HAS BEEN RELOCATED TO THE ~~UFSAR~~ TRM TECHNICAL SPECIFICATIONS PAGES 3/4 8-21 THROUGH 3/4 8-26 OF THIS SECTION HAVE BEEN INTENTIONALLY OMITTED.

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