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John F. McCann
Director
Nuclear Safety and Licensing

December 4, 2007
NL-07-149

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555-0001

Subject: Indian Point Nuclear Generating Unit No. 2
Docket No. 50-247
Indian Point Nuclear Generating Unit No. 3
Docket No. 50-286
**Annual 10 CFR 50.46 Report for Year 2006 -
Emergency Core Cooling System Evaluation Changes**

References: 1. Westinghouse letter (IPP- 07-50) Indian Point Unit 2 & 3 –
10 CFR 50.46 Annual Notification and Report, for 2006 (Revised), dated
November 12, 2007.
2. Entergy Nuclear Operations, Inc letter (NL-06-125), Indian Point Unit No.
2 & 3 - Annual 10 CFR 50.46 Report for Year 2005 - Emergency Core Cooling
System Evaluation Changes, dated December 19, 2006 (NL-06-125).

Dear Sir or Madam:

Entergy Nuclear Operations, Inc. (ENO) has reviewed the 2006 small and large break loss of coolant accident (LOCA) peak clad temperature (PCT) evaluations for Indian Point Units 2 and 3 (Reference 1). Reference 2 provided the results of ENO's review of the 2005 LOCA PCT evaluations for Indian Point Units 2 and 3 in accordance with 10 CFR 50.46. Except for a 1°F large break LOCA PCT penalty associated with changes to the containment sump strainer evaluation for Indian Point Unit 2, there have been no changes to the large break LOCA PCT for Indian Point Unit 3, nor the small break LOCA PCTs for Indian Point Units 2 and 3 since the publication of Reference 2. The Indian Point Units 2 and 3 PCTs remain below the maximum fuel clad temperature of 2200°F.

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NRK

There are no new commitments made in this letter. If you have any questions, please contact Ms. Charlene Faison at 914-272-3378.

Very truly yours,

A handwritten signature in cursive script that reads "Charlene Faison".

for John McCann
Director, Nuclear Safety & Licensing
Entergy Nuclear Operations

cc: Next page

cc:

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