

December 31, 2007

Mr. Mark B. Bezilla
Site Vice President
FirstEnergy Nuclear Operating Company
Davis-Besse Nuclear Power Station
Mail Stop A-DB-3080
5501 North State Route 2
Oak Harbor, OH 43449-9760

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1 – UPCOMING
STEAM GENERATOR TUBE INSERVICE INSPECTION (TAC NO. MD7455)

Dear Mr. Bezilla:

The inservice inspection (ISI) of steam generator (SG) tubes plays a vital role in assuring that adequate structural integrity of the tubes is maintained. As required by the individual plant technical specifications, reporting requirements range from submitting a special report, within 15 days following completion of each ISI of SG tubes, that identifies the number of tubes plugged and/or repaired; to submitting a special report within 12 months following completion of the inspection, that provides complete results of the SG tube ISI. The special report containing the complete results shall include the following:

1. the number and extent of tubes inspected.
2. the location and percent of wall-thickness penetration for each indication of an imperfection, and
3. the identification of tubes plugged and/or repaired.

A phone conference has been arranged with members of your staff to discuss the ongoing results of the SG tube inspections to be conducted during the upcoming Davis-Besse Nuclear Power Station, Unit No. 1 (DBNPS) refueling outage. This phone call will occur after the majority of the tubes have been inspected, but before the SG inspection activities have been completed. Enclosed is a list of discussion points to facilitate this phone conference.

M. Bezilla

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The Nuclear Regulatory Commission (NRC) staff plans to document a brief summary of the conference call as well as any material that you may have provided to the NRC staff in support of the call.

Sincerely,

/RA/

Thomas J. Wengert, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosure: List of Discussion Points

cc w/encl: See next page

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STEAM GENERATOR TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

The following discussion points have been prepared to facilitate the phone conference arranged with FirstEnergy Nuclear Operating Company (the licensee), to discuss the results of the SG tube inspections to be conducted during the upcoming DBNPS refueling outage. This phone call is scheduled to occur towards the end of the planned SG tube inspection interval, but before the unit completes the inspections and repairs.

The NRC staff plans to document a brief summary of the conference call as well as any material that is provided in support of the call.

1. Discuss any trends in the amount of primary-to-secondary leakage observed during the recently completed cycle.
2. Discuss whether any secondary side pressure tests were performed during the outage and the associated results.
3. Discuss the exceptions taken to the industry guidelines.
4. For each SG provide a description of the inspections performed including the areas examined and the probes used (e.g., dents/dings, sleeves, expansion-transition), the scope of the inspection (e.g., 100 percent of dents/dings greater than 5 volts and a 20 percent sample between 2 and 5 volts), and the expansion criteria.
5. For each area examined (e.g., tube supports, dents/dings, sleeves, etc), provide a summary of the number of indications identified to-date of each degradation mode (e.g., number of circumferential primary water stress corrosion cracking indications at the expansion transition). For the most significant indications in each area, provide an estimate of the severity of the indication (e.g., provide the voltage, depth, and length of the indication). In particular, address whether tube integrity (structural and accident induced leakage integrity) was maintained during the previous operating cycle. In addition, discuss whether any location exhibited a degradation mode that had not previously been observed at this location at this unit (e.g., observed circumferential primary water stress corrosion cracking at the expansion transition for the first time at this unit).
6. Describe repair/plugging plans.
7. Describe in-situ pressure test and tube pull plans and results (as applicable and if available).

Enclosure

8. Provide the schedule for SG-related activities during the remainder of the current outage.
9. Discuss the following regarding loose parts:
 - what inspections are performed to detect loose parts
 - a description of any loose parts detected and their location within the SG
 - indications of tube damage associated with the loose parts
 - the source or nature of the loose parts if known
10. Discuss any unexpected or unusual results.
11. Discuss the results of any secondary side inspections.