

Jonathan Rowley - FW: October 11 and 16 conference call summaries

From: "Devincentis, Jim" <jdevinc@entergy.com>
To: "Jonathan Rowley" <JGR@nrc.gov>
Date: 11/19/2007 11:52 AM
Subject: FW: October 11 and 16 conference call summaries

Some minor comments with one set of minutes – the other was OK as is.

Jim

From: Jonathan Rowley [mailto:JGR@nrc.gov]
Sent: Friday, November 16, 2007 10:33 AM
To: Devincentis, Jim
Subject: October 11 and 16 conference call summaries

Attached are the summaries for conference calls held on Oct. 11 and Oct. 16, 2007. Please review and comment where necessary.

Mail Envelope Properties (4741BF37.1C2 : 10 : 61890)

Subject: FW: October 11 and 16 conference call summaries
Creation Date 11/19/2007 11:51:35 AM
From: "Devincentis, Jim" <jdevinc@entergy.com>

Created By: jdevinc@entergy.com

Recipients

nrc.gov

TWGWPO03.HQGWDO01

JGR (Jonathan Rowley)

Post Office

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Summary of Telephone Conference - October 16 2007.doc		85504
Mime.822	124229	

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LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: Vermont Yankee Nuclear Power Station

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON OCTOBER 16, 2007, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND ENTERGY NUCLEAR OPERATIONS, INC., CONCERNING AUDIT QUESTIONS PERTAINING TO THE VERMONT YANKEE NUCLEAR POWER STATION LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc. (Entergy) held a telephone conference call on October 16, 2007, to discuss and clarify the staff's audit questions concerning the Vermont Yankee Nuclear Power Station (VYNPS) license renewal application asked during the October 9 - 10, 2007, audit of the VYNPS time-limited aging analyses. While attempting to address the staff's audit questions, Entergy determined that further clarification was needed on some of the questions. The telephone conference call was useful in clarifying the intent of the staff's audit questions.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a listing of the audit questions discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

Jonathan G. Rowley, Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-271

Enclosures:

1. List of Participants
2. List of Audit Questions

cc w/encls: See next page

LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: Vermont Yankee Nuclear Power Station

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cc w/encls: See next page

DISTRIBUTION: See next page

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OFFICE	LA:DLR	PM:RLRB:DLR	BC:RPB2:DLR
NAME		JRowley	RFranovich
DATE	11/ /07	11/ /07	11/ /07

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**TELEPHONE CONFERENCE CALL
VERMONT YANKEE NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION**

**LIST OF PARTICIPANTS
OCTOBER 16, 2007**

PARTICIPANTS

Jonathan Rowley
Kenneth Chang
Michael Metell
David Mannai
James Fitzpatrick
Alan Cox
James DeVincentis
David Lach
Terry Herrmann
Gary Stevens

AFFILIATIONS

U.S. Nuclear Regulatory Commission (NRC)
NRC
Entergy Nuclear Operations, Inc. (Entergy)
Entergy
Entergy
Entergy
Entergy
Entergy
Structural Integrity Associates (SIA)
SIA

ENCLOSURE 1

**REQUESTS FOR ADDITIONAL INFORMATION
VERMONT YANKEE NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION**

OCTOBER 16, 2007

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc. held a telephone conference call on October 16, 2007, to discuss and clarify the following audit questions concerning the Vermont Yankee Nuclear Power Station (VYNPS) license renewal application (LRA).

1st Audit question

The ASME Code defines that stress intensity from two temperature transients is calculated from the stress components from the two conditions. Please explain how it could be calculated from stress intensities of the two conditions derived from Greens Functions, especially at locations of geometric discontinuity. Also, please justify the validity of combining the thermal transient stress intensities with the stress intensities from the external loads and pressure loading.

Discussion: The staff provided further clarification on the question.

2nd Audit question

Provide justification for statement on page 5 of 34 of Calculation No. VY-16Q-302, that AThe Green function methodology provides identical results compared to running the input transient through the finite element model.@

Discussion: The staff provided further clarification on the question.

3rd Audit question

For the blend radius for the feedwater nozzle in Calculation No. VY-16Q-302, Table 4, Page 16, why are the Total and M+B stresses for thermal transient 3, shown in columns 3 and 4, high at 0 seconds ($t = 0$)? This question also applies to transient 4 at $t = 1801.9$ seconds, transient 9 at $t = 2524$ seconds, and transient 21-23 at $t = 20144$ seconds.

Discussion: The applicant indicated that the question is clear.

4th Audit question

Explain why there are differences in the calculated cumulative usage factor (CUF) values between Rev. A and Rev. 0 of the Structural Integrity calculations. Also, why are the CUFs calculated by Structural Integrity different from the CUFs shown in Tables 4.3.1 and 4.3.3 of the Vermont Yankee license renewal application?

ENCLOSURE 2

Vermont Yankee Nuclear Power Station

- 3 -

cc:

Discussion: The applicant indicated that the question is clear.

5th Audit question

Page 1-1 of Report VY-16Q-401 indicates that refined transient definitions for 60 years of operation are used in the computation of the CUF incorporating environmentally assisted fatigue effects. Please explain the refinements in the transient definitions.

Discussion: The applicant indicated that the question is clear.

6th Audit question

For the feedwater nozzles there are large differences between the CUFs without the environmental fatigue life correction (Fen) factors shown in Table 4.3.1 of the Vermont Yankee license renewal application and those shown in Calculation No. VY-16Q-302. Section 2.0 of the calculation on page 4 of 32 states, A...several of the conservatisms originally used in the original feedwater evaluation (such as grouping of transients) are removed...@ Please explain what conservatisms were removed.

Discussion: The applicant indicated that the question is clear.

7th Audit question

For stainless steel components listed in table 3-10 of Structural Integrity Report SIR-07-132 (VY-16Q-404), please justify that the calculated Fen values are conservative.

Discussion: The applicant indicated that the question is clear.

ENCLOSURE 2