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NOVEMBER 2-4, 1995

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MINUTES OF THE FOUR HUNDRED TWENTY-SIXTH MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
NOVEMBER 2-4, 1995
ROCKVILLE, MARYLAND

The 426th meeting of the Advisory Committee on Reactor Safeguards was held at Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on November 2-4, 1995. The purpose of this meeting was to discuss and take appropriate action on the items listed in the attached agenda. The meeting was open to public attendance. There were no written statements, but there was a request for time to make an oral statement from one member of the public regarding the Watts Bar Unit 1 License Application portion of the meeting.

A transcript of selected portions of the meeting was kept and is available in the NRC Public Document Room at the Gelman Building, 2120 L Street, N.W., Washington, D.C. [Copies of the transcript are available for purchase from Neal R. Gross and Co., Inc., 1323 Rhode Island Avenue, N.W., Washington, D.C. 20005.]

ATTENDEES

ACRS Members: Dr. Thomas S. Kress (Chairman), Dr. Robert L. Seale (Vice-Chairman), Dr. George Apostolakis, Mr. James C. Carroll, Dr. Ivan Catton, Dr. Mario H. Fontana, Mr. William J. Lindblad, Dr. Don W. Miller (absent from a portion of Thursday morning), Dr. Dana A. Powers, Dr. William J. Shack (absent on Saturday, November 4th), and Mr. Charles J. Wylie. [For a list of other attendees, see Appendix III.]

I. CHAIRMAN'S REPORT (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

Dr. Kress, Committee Chairman, convened the meeting at 8:30 a.m. and reviewed the schedule for the meeting. He announced that flu shots were available for Committee members, that a letter had been received from Admiral DeMars announcing the trials of the aircraft carrier John C. Stennis, and that a list of priority items would be distributed to members for comment.

II. WATTS BAR UNIT 1 OPERATING LICENSE APPLICATION (Open)

[Note: Mr. Noel Dudley was the Designated Federal Official for this portion of the meeting.]

Mr. Wylie, Chairman of the Watts Bar Subcommittee, stated that the purpose of the meeting was to continue the ACRS review of the Tennessee Valley Authority (TVA) application for an operating license for Watts Bar Nuclear Plant Unit 1. He noted that the Watts Bar Subcommittee met on November 1, 1995, and heard presenta-

tions by and held discussions with representative of the NRC staff and the TVA staff, as well as several members of the public.

TVA Presentation

Mr. John Scalice, Vice President of Watts Bar, introduced the presentation and described the safety culture and the concerns resolution program at Watts Bar. The Committee and the TVA staff discussed how changes in the safety culture were measured and how members of the TVA Office of the Inspector General were appointed. The Committee, the NRC staff, and the TVA staff discussed the history of the Watts Bar allegations resolution programs and the disparity between the TVA presentation and the presentations made by members of the public at the November 1, 1995 ACRS Subcommittee meeting.

Mr. Walter Elliott, TVA, presented information on thermo-lag fire barriers installed on safety-related cable trays, fire wall penetration seals, and fire suppression systems. The Committee and the TVA staff discussed the inspection, testing, and installation of thermo-lag material, and the definition of "largest safety-related fire."

Mr. Elliott described the result of the Watts Bar Individual Plant Examination (IPE) and how the results are used. The Committee and the TVA staff discussed the changes in the updated IPE, the uncertainty associated with the calculated core damage frequency, the training of Watts Bar personnel, and the use of IPE results by operators and maintenance planners.

Mr. Elliott presented information on the radiation monitoring systems and the Eagle 21 reactor protection system. The Committee, the NRC staff, and the TVA staff discussed the management oversight involved in the turnover of the radiation monitoring systems.

In response to questions asked during the November 1, 1995 ACRS Watts Bar Subcommittee meeting, the TVA staff provided information concerning the power available from the Watts Bar Hydroelectric Plant, the effect of spurious actuation of the carbon dioxide fire suppression system on the cooling of diesel generators, and the use of strainers in fire water pumps. The Committee and the NRC staff discussed the plans for inspecting the cement lining of safety-related pipes.

NRC Staff Presentation

Mr. Frederick Hebdon, Office of Nuclear Reactor Regulation (NRR), reviewed the status of licensing actions and activities associated with Watts Bar. He identified the most recent licensing documents and discussed the outstanding licensing issues and exemptions. Mr.

Hebdon identified the milestones for issuing an operating license. Mr. Johns Jaudon, Region II, explained how the NRC staff handles allegations from the public.

Statement By A Member of the Public

Ms. Ann Harris, a TVA employee, requested that the Committee recommend to the Commission that Watts Bar not be issued an operating license. She expressed doubts about the veracity of the information provided by TVA and stated that poor work practices exist at Watts Bar. Ms. Harris contended that the continuous changes in Watts Bar management adversely affected the organization and employee moral.

Conclusion

The Committee issued a report to The Honorable Shirley A. Jackson, Chairman, NRC, dated November 8, 1995, concerning the TVA application for an operating license for Watts Bar Nuclear Plant Unit 1.

III. BOILING WATER REACTOR (BWR) CORE POWER STABILITY/ATWS (Open)

[Note: Mr. Paul Boehnert was the Designated Federal Official for this portion of the meeting.]

Introduction

Dr. Catton, Chairman, Thermal Hydraulic Phenomena Subcommittee, introduced this topic to the Committee. He recounted the history of this issue, beginning with the March 1988 core power instability event that occurred at the La Salle plant. The ACRS review subsequently focused on the issue of core power instabilities in conjunction with an anticipated transient without scram (ATWS) event, as this represented the most significant safety concern.

The proposed modifications to the Emergency Procedure Guidelines (EPGs) developed by the Boiling Water Reactor Owners Group (BWROG) instruct the operator to initially drop the vessel water level to a point (top-of-active-fuel (TAF) plus 5 feet) below the feedwater sparger which effectively eliminates the possibility of large amplitude core power oscillations. The operator is then instructed to lower the water level below the TAF to reduce core power further while boron is injected. After injection, the water level is to be raised to remix the boron and shut down the core. The BWROG argues that lowering the level into the core gives the operators more time to take mitigative actions, should boron injection fail. Dr. Catton indicated that, in his opinion, lowering the level into the core introduces more uncertainty into the ultimate success path. He also noted that the BWROG did not present convincing arguments

to the Thermal Hydraulics Phenomena Subcommittee in defense of its positions.

Dr. Kress indicated that with the above strategy, the BWROG favored protecting the containment in the event of core damage. Dr. Powers indicated that this accident scenario may bolster the argument for the use of more robust fuel cladding, particularly for high-burnup cores. Mr. Carroll observed that the above strategy has been a part of the EPGs for some time, without the benefit of prior review by the ACRS. In response to Dr. Seale, Mr. Jones, NRR, indicated that while the strictures of the Backfit Rule preclude the staff from forcing the BWROG to revise its positions on this matter, this Rule does not preclude the staff from taking action if plant safety is threatened.

Dr. Catton also noted that the BWROG representatives declined to appear at this meeting to defend their position.

NRR Presentation

Mr. Laurence E. Phillips, NRR, provided a presentation on the NRC staff's review of the modifications proposed to the BWROG EPGs to deal with core power stability. Mr. Phillips discussed the history of this issue and summarized the staff's positions, as discussed in its draft Safety Evaluation Report on this matter, as follows:

- Core cooling at the Minimum Steam Cooling Water Level (MSCWL) is adequate.
- Use of the higher level strategy (TAF plus 5 feet) is superior for BWR plants that inject boron through the lower plenum standpipe.
- Control of the vessel water level anywhere between MSCWL and TAF plus 5 feet is acceptable for all BWR plants.

NRR also noted that the results of its calculations, performed using the TRAC-BF1 and RAMONA-4B codes, support the above conclusions and also support the staff position that boron remixing will occur when the vessel water level is raised, and subsequent core flow exceeds 20% of nominal.

In conclusion, NRR noted that it has accepted the modifications proposed by the BWROG to aid vessel water level control during an ATWS event as well as those proposed to mitigate the potential for core power oscillations. For the latter, NRR, while accepting the BWROG strategy of lowering the vessel water level into the core, encourages licensees with lower plenum standpipe boron injection capability to target vessel water level control at TAF plus 5 feet.

During discussion of the code calculations performed by NRC versus those performed by the BWROG, Dr. J. March-Leuba (Oak Ridge National Laboratory) noted that the BWROG calculations have large uncertainties associated with them.

Dr. Catton complimented the NRR staff for wrapping up this issue in an orderly manner.

Conclusion

The Committee prepared a report to Chairman Jackson dated November 14, 1995, on this matter.

IV. ADVANCED CONTROL ROOM DESIGN REVIEW GUIDELINES (Open)

[Note: Dr. Medhat El-Zeftawy was the Designated Federal Official for this portion of the meeting.]

Mr. Carroll introduced the topic by stating that the NRC staff has developed NUREG-0700, Revision 1, "Human-System Interface Design Review Guideline." The objective of this NUREG is to provide guidance to the NRC staff for its human-system interface (HSI) reviews of new design submittals or as part of an inspection or other type of regulatory review.

NRC Staff Presentation

Mr. Jerry Wachtel, Office of Nuclear Regulatory Research (RES), stated that one of the first insights from studies of the Three Mile Island accident was that errors resulting from human factor deficiencies in the control room were a significant contributing factor to nuclear power plant accidents. As a result, NRC had requested all licensees and applicants for operating licenses to conduct detailed control room design reviews, including reviews of remote shutdown panels, to identify and correct human factor design deficiencies. In 1981, the NRC published NUREG-0700, "Guidelines for Control Room Design Reviews," in support of these reviews.

Since 1981, the NRC staff had focused on issues for which there were uncertainties in the scientific data needed to support the regulation. One such issue was the introduction of advanced computer-based HSI technology into control room and local control stations. The inclusion of advanced HSI technologies, such as touch-screen controls and large screens, could have significant implications for plant safety in that they will affect the overall role of personnel in the system and the requirements for personnel to understand and supervise an increasingly complex system.

The NRC has programs in place to review the design and implementation of significant changes to existing control rooms and to review

the human factors engineering aspects of advanced control rooms. However, the principal review guidance in NUREG-0700 was developed more than ten years ago and needed updating. NUREG-0700, Revision 1, provides guidance to the NRC staff for its review of advanced designs. This document is not a backfit and does not impose new requirements on current plants. The document would not apply to licensees under 10 CFR, Part 50, for the review of control room interfaces unless the licensee initiated a voluntary upgrade. The document was developed to apply primarily to advanced reactors. New plant designs submitted to the NRC under 10 CFR, Part 52, must meet 10 CFR, Part 50.34f(2)(iii), which requires a control room design that reflects the state-of-the-art human factors principles. The guidelines in NUREG-0700, Revision 1, are not mandatory. The guidelines, however, represent good human factors engineering practice based upon state-of-the-art research and validated review criteria developed in other fields, such as military and aerospace.

The staff received five sets of public comments, two of which were basically the same. General comments were received on the purpose, technical bases for the guidance, scope and format, and organization of the proposed revision. The updated guidance has been adopted for use internationally.

Conclusion

The Committee provided a letter, dated November 13, 1995, to the Executive Director for Operations, on this matter.

V. RELIABILITY OF SYSTEMS (Open)

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

Mr. Carroll, Chairman of the Plant Operations Subcommittee, introduced the topic to the Committee by noting that the Office for Analysis and Evaluation of Operational Data (AEOD) had developed a program to expand the use of probabilistic risk assessment (PRA) in their evaluation of nuclear power plant safety system performance. He noted that two reports had been completed but emphasized that the staff's presentation would cover the process (methods/means) for conducting system reliability reports. He stated that AEOD had asked to brief the Committee at a future date on the results of the system reliability reports.

AEOD Presentation

Mr. Patrick W. Baranowsky, Chief, Reliability and Risk Assessment Branch, provided an introduction and background information on the program. Mr. Steven E. Mays, Chief, Reactor Risk Assessment Section, led the discussion.

Mr. Mays reviewed the overall program for conducting system reliability studies and provided a detailed discussion of the technical approach, analysis methodology, and insights gained. Significant points made during the discussion were:

- Purpose and objectives: to evaluate actual system performance data, compare it with results estimated in licensee PRAs and IPEs, identify plant-specific outliers, and provide engineering insights.
- Scope: includes both boiling water reactor and pressurized water reactor (PWR) systems. The BWR studies include: high pressure coolant injection, reactor core isolation cooling, high pressure core spray, and isolation condenser. The PWR systems include auxiliary feedwater and high pressure safety injection. AEOD is also conducting studies on BWR and PWR low pressure injection systems, reactor trip systems, and emergency diesel generators.
- Methodology: develop system boundary limits and characterize event data according to inoperabilities where safety function was actually lost (failures) and safety function losses where actuations could be counted (demands). AEOD uses fault tree models to determine unreliabilities and uses Bayesian analysis techniques to determine failure probabilities and uncertainty intervals. The results are compared to PRA/IPE data and regulatory activities to analyze trends.
- Insights from the high pressure coolant injection study: there was no discernable trend in reliability; failure rate and unplanned demand rate were both decreasing as reflected by improved overall industry performance; there was no significant variation in reliability or failure rates due to the age of plants; there was general agreement between the study, PRAs and IPEs; there were failure differences between actual demands and routine surveillance or inspections; all failures to start from actual demands were recovered by plant personnel; and maintenance or testing out of service was an important factor but there was limited data in this area.

Mr. Lindblad questioned whether the staff analyzes according to systems or components. The staff stated that it is a top-level analysis beginning at the system-level. If the data allows, analysis can be pursued to the train-level or component-level. However, AEOD has another program for component analysis. This program focuses on components to the extent they affect system reliability or could cause a loss of safety function.

Mr. Carroll questioned the sources of data and how the recent staff requirements memorandum (SRM), dated October 24, 1995, might affect

their ability to get data from licensees. The staff responded that most of the information for the reliability studies is derived from licensee event reports (LERs) and monthly operating reports. The SRM, on proposed rulemaking for reporting equipment reliability data (SECY-95-215), directs some changes in the statement of consideration related to systems, structures, and components in the context of the maintenance rule but approved the proposed rule for publication to solicit public comment. The staff continues to work with the industry to obtain voluntary information.

Dr. Apostolakis questioned the environments from which the data were derived (i.e. events, incidents, surveillance tests, etc.). He questioned how the operator's role was considered in system success or failure. The staff used the event fault tree model to describe the sources of plant events, the potential pathways to recovery including operator action, and the implications for system reliability. The staff highlighted the use of Bayesian probability techniques to gain insights regarding system unreliability. The staff looks for trends in system unplanned demand rate, system failure rate, and system unreliability.

Dr. Kress asked whether there were any indications that overall operating experience was higher than that seen in PRAs. The staff stated that no significant differences were seen in the early studies, but it varies according to the method by which licensees obtained PRA/IPE data.

Conclusion

The Committee expressed support for the program and the desire to review the individual studies as the results become available. The Committee took no action based on this information briefing.

VI. PROPOSED FINAL REGULATORY GUIDE 1.164, "TIME RESPONSE DESIGN CRITERIA FOR SAFETY-RELATED OPERATOR ACTIONS" (Open)

[Note: Mr. Duraiswamy was the Designated Federal Official for this portion of the meeting.]

Mr. Carroll, Chairman of the Plant Operations Subcommittee introduced the topic to the Committee and Mr. Joel J. Kramer, Control, Instrumentation, and Human Factors Branch, RES, who presented the proposed Final Regulatory Guide to resolve Generic Safety Issue B-17, "Criteria for Safety-Related Operator Actions."

NRC Staff Presentation

Mr. Kramer led the discussion of the proposed Final Regulatory Guide. Mr. Richard J. Eckenrode and Mr. James P. Bongarra, Human Factors Assessment Branch, NRR, provided supporting discussion.

Mr. Kramer reviewed the proposed Final Regulatory Guide 1.164 that endorses ANSI/ANS-58.8-1994, "Time Response Design Criteria for Safety-Related Operator Actions," without exception. He reviewed the proposed resolution to Generic Safety Issue B-17, the background that led to the development of the Standard, its application, and statistical methodology. Significant points made during the discussion were:

- The staff does not consider Regulatory Guide 1.164 to be a backfit nor an imposition of new requirements on current plants.
- Regulatory Guide 1.164 will be used to evaluate submittals by applicants for construction permits and operating licenses.
- Public comments were generally favorable and resulted in no changes to Draft Guide DG-1040, "Time Response Design Criteria for Safety-Related Operator Actions."
- ANSI/ANS-58.8-1994 was revised from an earlier 1984 version to incorporate a simplified process for calculating minimum allowable response times for manual operator actions based on simulator data collected at plants by the Electric Power Research Institute (EPRI), using symptom-based emergency operating procedures. ANSI/ANS-58.8-1994 provides an acceptable method for developing and applying timing criteria for safety-related operator actions including a methodology for determining whether or not automatic actuation would be needed to mitigate a design-basis event. The Standard sets 95% confidence estimates for crew response times to plant events.

Dr. Powers asked the staff to elaborate on how the data was collected and incorporated into ANSI/ANS-58.8-1994. The staff stated that the Standards Committee Working Group had the benefit of some EPRI documents, but the third volume containing the raw data was unavailable because it was proprietary. Dr. Powers questioned the traceability of simulator data to actual measures. He asked whether the Working Group had reviewed the raw data that had been derived from the EPRI studies. The staff expressed confidence that the results were conservative and noted that they were similar to those in the 1984 Standard. Dr. Powers stated that he could not find a traceable tie between the numbers in the Standard and the data.

Dr. Catton questioned simulator fidelity as a possible weakness in the data. The staff stated that they believed the operator reliability experiments (ORES) conducted by EPRI had been updated since the original Standard was issued in 1984. Dr. Catton acknowledged their point but described a recent site visit where an operations demonstration on the plant simulator evidenced good

performance by operators but the simulator was somewhat lacking in accurately portraying plant conditions.

Dr. Apostolakis asked if EPRI expected their results to be accepted without showing the data. The staff responded "yes" and expressed shared frustration. Dr. Powers added that the Working Group had the benefit of "bridging documents" with tables that were reported to correspond to the raw data. The staff stated that the Working Group achieved consensus on the acceptability of the Standard through an iterative review and balloting process.

Dr. Apostolakis also questioned the relevance of this guidance for future reactors. He cited 72-hour coping times for advanced reactors and indefinite coping time for passive reactors. Dr. Fontana questioned who would use the Standard if it does not apply to future plants. The staff acknowledged the Committee concerns.

Mr. Lindblad questioned how the current operating plants were classified for operator response times. The staff explained the philosophy and methodology for making determinations and stated that all plants conform to the 1994 Standard. The response times had not changed from the 1984 version of the Standard.

The Committee extensively discussed the apparent lack of a technical basis for the endorsement of ANSI/ANS-58.8-1994. The Committee expressed the view that the recommended response times developed from simulator exercises did not demonstrate that the times were appropriately conservative. The Committee expressed the concern that the Standard did not address operator response times for advanced nuclear power plants and the belief that there is a need to consider this issue in some way for evolutionary and passive plants. Members Apostolakis, Catton, and Seale provided additional comments expressing the view that it was inappropriate for the staff to accept the EPRI/ORE results without independent peer review or review of actual data collected.

Conclusion

The Committee provided a letter, dated November 14, 1995, to the Executive Director for Operations. In this letter, the Committee did not support the NRC staff endorsement of the Standard to resolve Generic Safety Issue B-17.

VII. EXECUTIVE SESSION (Open)

[Note: Dr. Larkins was the Designated Federal Official for this portion of the meeting.]

A. REPORTS, LETTERS AND MEMORANDUM

Application for Operating License for Watts Bar Nuclear Plant Unit 1 (Report to The Honorable Shirley A. Jackson, Chairman, NRC, from T. S. Kress, Chairman, ACRS, dated November 8, 1995)

Proposed Modification to the Boiling Water Reactor Owners Group Emergency Procedure Guidelines to Address Reactor Core Instabilities (Report to The Honorable Shirley A. Jackson, Chairman, NRC, from T. S. Kress, Chairman, ACRS, dated November 14, 1995)

NUREG-0700, Revision 1, "Human-System Interface Design Review Guideline (Letter to James M. Taylor, Executive Director for Operations, NRC, from T. S. Kress, ACRC Chairman, dated November 13, 1995)

Proposed Final Regulatory Guide 1.164, "Time Response Design Criteria for Safety-Related Operator Actions" to Resolve Generic Safety Issue B-17 (Letter to James M. Taylor, Executive Director for Operations, NRC, from T. S. Kress, ACRC Chairman, dated November 14, 1995)

Allegations Concerning the Application for Operating License for Watts Bar Nuclear Plant Unit 1 (Memorandum to James M. Taylor, Executive Director for Operations, NRC, from John T. Larkins, Executive Director, ACRC, dated November 8, 1995)

B. RECONCILIATION OF ACRC COMMENTS AND RECOMMENDATIONS

[Note: Mr. Duraiswamy was the Designated Federal Official for this portion of the meeting.]

The Committee discussed the response from the NRC Executive Director for Operations to ACRC comments and recommendations included in recent ACRC reports:

EDO letter dated October 16, 1995, responding to the ACRC report dated September 15, 1995, concerning the Development of Improved Nondestructive Examination Techniques.

The Committee decided that it was satisfied with the EDO's response.

EDO letter dated October 30, 1995, responding to the ACRC letter dated September 15, 1995, concerning the Nuclear Energy Institute Petition for Rulemaking to Amend 10 CFR 50.48, "Fire Protection."

The Committee decided that it was satisfied with the EDO's response.

EDO letter dated October 31, 1995, responding to the ACRS letter dated October 13, 1995, concerning the National Academy of Sciences' Report on Digital Instrumentation and Control, Safety and Reliability Issues.

The Committee decided that it was satisfied with the EDO response.

C. REPORT ON THE MEETING OF THE PLANNING AND PROCEDURES SUBCOMMITTEE HELD ON NOVEMBER 1, 1995 (OPEN)

The Committee heard a report from Dr. Kress on the Planning and Procedures Subcommittee meeting held on November 1, 1995. The following items were discussed:

1. ISSUES FOR DECEMBER MEETING WITH NRC CHAIRMAN

A proposed list of priority items that the Committee plans to review in the next two years was distributed to the Members on November 2, 1995.

Conclusion

Cognizant Subcommittee Chairmen/Members will review this list for completeness/accuracy and provide comments to Sam Duraiswamy or Noel Dudley by Saturday, November 4, 1995. A final list incorporating the members' comments will be prepared and sent to the NRC Chairman. Chairman Jackson is expected to provide her views on this matter during her meeting with the ACRS in December 1995.

2. PREPAID AIRLINE TICKETS

Airlines have recently instituted a fee of \$35 for any tickets that are prepaid at the airport. If such tickets are arranged with Carlson Wagonlit Travel Services, the NRC will be billed directly. If such tickets are arranged on an emergency basis and paid for with a government credit card, the airline will charge the Member directly for the fee. To be reimbursed, the \$35 fee must be claimed on the travel voucher.

3. BUDGET IMPACT ON ACRS MEETINGS

In the event that Congress and the Executive Branch are unable to reconcile the FY 96 budget prior to November 13, 1995, the NRC will operate on FY 95 carryover funds. Should these carryover funds be exhausted prior to enactment of an FY 96 budget, all nonessential activities will cease.

Conclusion

The ACRS staff will monitor the situation to ensure sufficient funds for upcoming meetings. Members will be notified immediately if meetings will be postponed or cancelled due to insufficient funding.

4. DOMESTIC TRAVEL GUIDELINES

One set of comments was received on the draft guidelines for domestic travel.

Conclusion

A revised set of Guidelines was prepared and accepted.

5. FUTURE ACTIVITIES [See separate handout]

6. MEMBERS' ISSUES

- Comments were received concerning the purpose of the "Ad Hoc Consultation Panel." The original intent of the Ad Hoc Consultation Panel was to provide advice to the ACRS/ACNW Executive Director when requested.

Conclusion

Dr. Kress will discuss the need for the continued existence of such a panel with the Executive Director for Operations.

- Dr. Powers raised the issue of whether the ACRS should examine the issues associated with the Vessel Head Penetration Cracking.

Conclusion

The Committee will discuss the need for such an examination with Dr. Shack, who did not attend the Saturday session.

D. FUTURE MEETING AGENDA

Appendix IV summarizes the proposed items endorsed by the Committee for the 427th ACRS Meeting, December 7-9, 1995.

The 426th ACRS meeting was adjourned at 12:05 p.m. on Saturday, November 4, 1995.

related to the EIS will be available for public review, unless the information is protected from public disclosure in accordance with NRC requirements in 10 CFR § 2.790.

In the scoping process, participants are invited to speak or submit written comments, as noted above, on any or all of the areas described above. In accordance with 10 CFR 51.29, at the conclusion of the scoping process, NRC will prepare a concise summary of the determinations and conclusions reached, including the significant issues identified, and will send a copy to each participant in the scoping process.

Dated at Rockville, MD., this 13th day of October 1995.

For the U.S. Nuclear Regulatory Commission.

Michael F. Weber,

Chief, Decommissioning and Regulatory Issues Branch, Division of Low-Level Waste Management and Decommissioning, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 95-25978 Filed 10-19-95; 8:45 am]

BILLING CODE 7580-01-P

Advisory Committee on Reactor Safeguards; Meeting Agenda

In accordance with the purposes of Sections 29 and 182b. of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on November 2-4, 1995, in Conference Room T-2B3, 11545 Rockville Pike, Rockville, Maryland. The date of this meeting was previously published in the *Federal Register* on Tuesday, August 22, 1995 (60 FR 43619).

Thursday, November 2, 1995

8:30 a.m.-8:45 a.m.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting and comment briefly regarding items of current interest. During this session, the Committee will discuss priorities for preparation of ACRS reports.

8:45 a.m.-10:45 a.m.: Watts Bar Unit 1 Operating License Application (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff and the applicant (Tennessee Valley Authority) on the status of resolution of issues associated with the review of the operating license application for Watts Bar Unit 1 nuclear plant.

Representatives of the public will participate, as appropriate.

11:00 a.m.-12:30 p.m.: BWR Core Power Stability/ATWS (Open/Closed)—The Committee will hear presentations

by and hold discussions with representatives of the NRC staff and BWR Owners Group (BWROG) regarding the proposed revisions to emergency procedure guidelines developed by the BWROG for mitigation of an ATWS event compounded by core power instability.

A portion of this session may be closed to discuss General Electric Nuclear Energy proprietary information applicable to this matter.

1:30 p.m.-3:00 p.m.: Advanced Control Room Design Review Guidelines (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed Revision 1 to NUREG-0700, "Human-System Interface Design Review Guideline".

Representatives of the industry will participate, as appropriate.

3:15 p.m.-4:15 p.m.: Reliability of Safety Systems (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the methods/means used by the staff for reviewing the reliability of safety systems.

Representatives of the industry will participate, as appropriate.

4:30 p.m.-6:45 p.m.: Preparation of ACRS Reports (Open)—The Committee will discuss proposed ACRS reports on matters considered during this meeting, as well as a proposed ACRS report on the Resolution of Generic Issue 78, "Monitoring of Fatigue Transient Limits for the Reactor Coolant System".

Friday, November 3, 1995

8:30 a.m.-8:35 a.m.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting.

8:35 a.m.-9:45 a.m.: Proposed Final Regulatory Guide 1.164, "Time Response Design Criteria for Safety-Related Operator Actions" (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding proposed final Regulatory Guide 1.164.

Representatives of the industry will participate, as appropriate.

9:45 a.m.-10:30 a.m.: Report of the Planning and Procedures Subcommittee (Open/Closed)—The Committee will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS staff members.

A portion of this session may be closed to discuss organizational and personnel matters that relate solely to

the internal personnel rules and practices of this Advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.

10:45 a.m.-11:30 a.m.: Future ACRS Activities (Open)—The Committee will select topics for consideration during future ACRS meetings.

11:30 a.m.-11:45 a.m.: Reconciliation of ACRS Comments and Recommendations (Open)—The Committee will discuss responses expected from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports.

11:45 a.m.-12:00 Noon: Subcommittee Activities (Open)—The Committee will hear a report by the Subcommittee Chairman regarding the October 26-27, 1995 meeting on Individual Plant Examinations/ Probabilistic Risk Assessment.

1:00 p.m.-6:30 p.m.: Preparation of ACRS Reports (Open)—The Committee will continue its discussion of proposed ACRS reports on matters considered during this meeting, as well as a proposed ACRS report on the Resolution of Generic Issue 78, "Monitoring of Fatigue Transient Limits for Reactor Coolant System".

Saturday, November 4, 1995

8:30 a.m.-10:30 a.m.: Preparation of ACRS Reports (Open)—The Committee will continue its discussion of proposed ACRS reports on matters considered during this meeting and on the other matter noted above.

10:45 a.m.-12 Noon: Strategic Planning (Open)—The Committee will discuss items that are of significant importance to NRC, including rebaselining of the Committee activities for fiscal year 96-97.

12:00 Noon-12:15 p.m.: Miscellaneous (Open)—The Committee will discuss miscellaneous matters related to the conduct of Committee activities.

Procedures for the conduct of and participation in ACRS meetings were published in the *Federal Register* on October 5, 1994 (59 FR 50780). In accordance with these procedures, oral or written statements may be presented by members of the public, electronic recordings will be permitted only during the open portions of the meeting, and questions may be asked only by members of the Committee, its consultants, and staff. Persons desiring to make oral statements should notify Mr. Sam Duraiswamy, Chief, Nuclear Reactors Branch, at least five days before the meeting, if possible, so that appropriate arrangements can be made

to allow the necessary time during the meeting for such statements. Use of still, motion picture, and television cameras during this meeting may be limited to selected portions of the meeting as determined by the Chairman.

Information regarding the time to be set aside for this purpose may be obtained by contacting the Chief of the Nuclear Reactors Branch prior to the meeting. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with the Chief of the Nuclear Reactors Branch if such rescheduling would result in major inconvenience.

In accordance with Subsection 10(d) P.L. 92-463, I have determined that it is necessary to close portions of this meeting noted above to discuss matters that relate solely to the internal personnel rules and practices of this Advisory Committee per 5 U.S.C. 552b(c)(2), to discuss General Electric Nuclear Energy proprietary information per 5 U.S.C. 552b(c)(4), and to discuss matters the release of which would constitute a clearly unwarranted invasion of personal privacy per 5 U.S.C. 552b(c)(6).

Further information regarding topics to be discussed, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting Mr. Sam Duraiswamy, Chief, Nuclear Reactors Branch (telephone 301-415-7364), between 7:30 A.M. and 4:15 P.M. EDT.

ACRS meeting notices, meeting transcripts, and letter reports are now available on FedWorld from the "NRC MAIN MENU." Direct Dial Access number to FedWorld is (800) 303-9672; the local direct dial number is 703-321-3339.

Dated: October 16, 1995.

Andrew L. Bates,

Advisory Committee Management Officer.

[FR Doc. 95-25977 Filed 10-19-95; 8:45 am]

BILLING CODE 7590-01-00

Biweekly Notice; Applications and Amendments to Facility Operating Licenses Involving No Significant Hazards Consideration; Correction

AGENCY: Nuclear Regulatory Commission.

ACTION: Biweekly notice; correction.

SUMMARY: This document corrects a biweekly notice appearing in the Federal Register on October 11, 1995 (60 FR 52927). This notice is necessary

to correct placement of an individual notice.

FOR FURTHER INFORMATION CONTACT: Michael T. Lesar, Chief, Rules Review Section, Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, telephone (301) 415-7163.

SUPPLEMENTARY INFORMATION: On page 52935, in the first column, the individual notice entitled, "Rochester Gas and Electric Corporation, Docket No. 50-244, W.E. Ginna Nuclear Power Plant, Wayne County, New York" should have been printed in alphabetical order under the section entitled, "Previously Published Notices of Consideration of Issuance of Amendment to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing," which begins on page 52937. Dated at Rockville, MD, this day of October 1995.

For the Nuclear Regulatory Commission.

Michael T. Lesar,

Chief, Rules Review Section, Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration.

[FR Doc. 95-25979 Filed 10-19-95; 8:45 am]

BILLING CODE 7590-01-P

POSTAL SERVICE

Intent To Prepare a Programmatic Environmental Assessment: Priority Mail Processing System

AGENCY: Postal Service.

ACTION: Notice of intent.

SUMMARY: To aid in planning, the Postal Service intends to prepare a Programmatic Environmental Assessment (PEA) for its proposed Priority Mail Processing System, pursuant to the National Environmental Policy Act (NEPA). This PEA will assess the impact of this proposed system on the human environment, both physical and cultural, which includes the postal working environment.

DATES: Suggestions regarding issues or concerns to be addressed in the PEA must be received on or before October 31, 1995.

ADDRESSES: Mail or deliver written comments to the Manager, Operations Networks Redesign, U.S. Postal Service, 425 L'Enfant Plaza SW, Washington, DC 20260-7165.

FOR FURTHER INFORMATION CONTACT: Anthony M. Pajunas, (202) 268-3669.

SUPPLEMENTARY INFORMATION: The proposed Priority Mail Processing

System would implement a new Priority Mail processing and distribution concept in several new Priority Mail centers along the East Coast. This proposed system is needed to improve service because on-time delivery performance is below the targeted level. This system would both segregate Priority Mail from other mail classes and process and distribute Priority Mail through dedicated facilities. Although in a few instances new construction may be required, the facilities would be preferably housed in existing commercial or industrial buildings.

Stanley F. Mires,

Chief Counsel, Legislative.

[FR Doc. 95-26162 Filed 10-18-95; 1:19 pm]

BILLING CODE 7710-12-P

RAILROAD RETIREMENT BOARD

Proposed Data Collection Available for Public Comment and Recommendations

SUMMARY: In accordance with the requirement of Section 3506(c)(2)(A) of the Paperwork Reduction Act of 1995 which provides opportunity for public comment on new or revised data collections, the Railroad Retirement Board will publish periodic summaries of proposed data collections.

Comments are invited on: (a) Whether the proposed information collection is necessary for the proper performance of the functions of the agency, including whether the information has practical utility; (b) the accuracy of the RRB's estimate of the burden of the collection of the information; (c) ways to enhance the quality, utility, and clarity of the information to be collected; and (d) ways to minimize the burden related to the collection of information on respondents, including the use of automated collection techniques or other forms of information technology.

Title and Purpose of Information Collection

Repayment of Debt

When the Railroad Retirement Board (RRB) determines that an overpayment of Railroad Retirement Act (RRA) benefits has occurred, it initiates prompt action to notify the annuitant of the overpayment and to recover the money owed the RRB. In addition to the customary form of repayment (check, money order, annuity withholding), repayment of a debt owed the RRB can also be made by means of a credit card. To effect payment by credit card the RRB utilizes Form G-421f, Repayment by Credit Card. One form will be



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

October 23, 1995

SCHEDULE AND OUTLINE FOR DISCUSSION
426th ACRS MEETING
NOVEMBER 2-4, 1995

Thursday, November 2, 1995, Conference Room 2B3, Two White Flint North,
Rockville, Maryland

1) 8:30 - 8:⁴⁰~~45~~ A.M.

Opening Remarks by the ACRS Chairman (Open)

- 1.1) Opening Statement (TSK/SD)
- 1.2) Items of Current Interest (TSK/JTL/SD)
- 1.3) Priorities for Preparation of ACRS Reports (TSK/SD)

2) 8:⁴⁰~~45~~ - 10:⁵²~~45~~ A.M.

Watts Bar Unit 1 Operating License Application (Open) (CJW/NFD)

- 2.1) Remarks by the Subcommittee Chairman
- 2.2) Briefing by and discussions with representatives of the NRC staff and the applicant (Tennessee Valley Authority) on the status of resolution of issues associated with the review of the operating license application for Watts Bar Unit 1 nuclear plant.

Representatives of the public will participate, as appropriate.

10:⁵²~~45~~ - 11:⁰⁵~~00~~ A.M.

BREAK

3) 11:⁰⁵~~00~~ - 12:30 P.M.

BWR Core Power Stability/ATWS (Open/Closed) (IC/PAB)

- 3.1) Remarks by the Subcommittee Chairman
- 3.2) Briefing by and discussions with representatives of the NRC staff and BWR Owners Group (BWROG) regarding the proposed revisions to emergency procedure guidelines by BWROG for mitigation of an ATWS event compounded by core power instability.

[Note: A portion of this session may be closed to discuss General Electric Nuclear Energy proprietary information applicable to this matter.]

[Transcribed portions of the meeting.]

12:30 - 1:30 P.M.

LUNCH

4) 1:30 - 3:00 P.M.

Advanced Control Room Design Review Guidelines (Open) (JCC/NFD/MME)

- 4.1) Remarks by the Subcommittee Chairman
- 4.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed revision 1 to NUREG-0700, "Human-System Interface Design Review Guideline."

Representatives of the industry will participate, as appropriate.

3:00 - 3:15 P.M.

BREAK

5) 3:15 - 4:45 P.M.

Reliability of Systems (Open) (JCC/NFD/MTM)

- 5.1) Remarks by the Subcommittee Chairman
- 5.2) Briefing by and discussions with representatives of the NRC staff regarding the methods/means used by the staff for reviewing the reliability of safety systems.

Representatives of the industry will participate, as appropriate.

4:45 - 5:00 P.M.

BREAK

6) 5:00 - 6:³⁰~~45~~ P.M.

Preparation of ACRS Reports (Open)

Discussion of proposed ACRS reports on:

- 6.1) Watts Bar Unit 1 Operating License Application (CJW/NFD)
- 6.2) Proposed Resolution of Generic Issue 78, "Monitoring of Fatigue Transient Limits for the Reactor Coolant System" (RLS/NFD)
- 6.3) Advanced Control Room Design Review Guidelines (JCC/NFD/MME)
- 6.4) BWR Core Power Stability/ATWS (IC/PAB)

Friday, November 3, 1995, Conference Room 2B3, Two White Flint North,
Rockville, Maryland

7) 8:30 - 8:35 A.M.

Opening Remarks by the ACRS Chairman (Open)
(TSK/SD)

8) 8:35 - ^{10:04}~~9:45~~ A.M.

Proposed Final Regulatory Guide 1.164, "Time
Response Design Criteria for Safety-Related
Operator Actions" (Open) (JCC/NFD/MTM)

8.1) Remarks by the Subcommittee Chairman
8.2) Briefing by and discussions with
representatives of the NRC staff
regarding proposed final Regulatory
Guide 1.164.

Representatives of the industry will
participate, as appropriate.

9) ^{10:04}~~9:45~~ - ⁴⁰~~10:30~~ A.M.

Report of the Planning and Procedures
Subcommittee (Open/Closed) (TSK/JTL)

Report of the Planning and Procedures Sub-
committee on matters related to the conduct
of ACRS business, and organizational and
personnel matters relating to the ACRS staff
members.

[Note: A portion of this session may be
closed to discuss organizational and person-
nel matters that relate solely to the inter-
nal personnel rules and practices of this
Advisory Committee, and matters the release
of which would constitute a clearly unwar-
ranted invasion of personal privacy.]

⁴⁰~~10:30~~ - ^{11:00}~~10:45~~ A.M.

BREAK

10) ^{11:00}~~10:45~~ - ⁴⁵~~11:30~~ A.M.

Future ACRS Activities (Open) (TSK/SD)

Discussion of the recommendations of the
Planning and Procedures Subcommittee regard-
ing items proposed for consideration by the
full Committee during future meetings.

11) 11:⁴⁵~~30~~ - 11:⁵⁵~~45~~ A.M.

Reconciliation of ACRS Comments and Recommendations (Open) (TSK, et.al./SD, et.al.)
Discussion of responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports.

12) 11:⁵⁵~~45~~ - 12:²⁰~~00~~ Noon

Subcommittee Activities (Open) (GA/MME)
Report by the Subcommittee Chairman regarding the October 26-27, 1995, meeting on Individual Plant Examinations/Probabilistic Risk Assessment.

12:²⁰~~00~~ - 1:³⁰~~00~~ P.M.

LUNCH

13) 1:³⁰~~00~~ - 6:¹⁰~~30~~ P.M.
(3:15 - 3:30 BREAK)

Preparation of ACRS Reports (Open)
Discussion of proposed ACRS reports on:
13.1) Watts Bar Unit 1 Operating License Application (CJW/NFD)
13.2) Proposed Resolution of Generic Issue 78, "Monitoring of Fatigue Transient Limits for the Reactor Coolant System" (RLS/NFD)
13.3) Advanced Control Room Design Review Guidelines (JCC/NFD/MME)
13.4) Proposed Final Regulatory Guide 1.164, Time Response Design Criteria for Safety-Related Operator Actions (JCC/NFD/MTM)
13.5) BWR Core Power Stability/ATWS (IC/PAB)

Saturday, November 4, 1995, Conference Room 2B3, Two White Flint North, Rockville, Maryland

14) 8:30 - 10:30 A.M.

Preparation of ACRS Reports (Open) - The Committee will continue its discussion of proposed ACRS reports on matters considered during this meeting and on Proposed Resolution of Generic Issue 78.

- 15) 10:45 - 12:00 Noon Strategic Planning (Open) (TSK/JTL)
Discussion of items of significant importance to NRC, including rebaselining of the Committee activities for fiscal years 96-97.
- 16) 12:00 - 12:15 P.M. Miscellaneous (Open) (TSK/JTL)
Discussion of miscellaneous matters related to the conduct of Committee activities.

- NOTE:**
- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
 - Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX III: MEETING ATTENDEES

426TH ACRS MEETING
November 2-4, 1995

NRC STAFF

| | |
|-----------------|-----------|
| Johns P. Jaudon | Region II |
| Leonard Soffer | EDO |
| Robert Jones | NRR |
| Larry Phillips | NRR |
| Tony Ulses | NRR |
| J. Persenky | RES |
| J. Bongarra | NRR |
| J. Kramer | RES |
| F. Coffman | RES |
| Jerry Wachtel | RES |
| Greg Galletti | NRR |
| D. L. Morrison | RES |
| S. E. Mays | AEOD/SPD |
| R. J. Eckenrode | NRR |
| T. R. Wolf | AEOD/SPD |
| D. M. Rasmuson | AEOD/SPD |
| P. Baranowsky | AEOD/SPD |
| C. E. Rossi | AEOD/SPD |
| D. Allison | AEOD/SPD |
| R. Eckenrode | NRR |

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

| | |
|---------------------|-------------------------|
| Frank A. Koontz Jr. | TVA-Watts BAR |
| Robert H. Bryan Jr | TVA |
| Kent W. Brown | TVA |
| Robert C. Williams | TVA |
| M. C. Brickey | TVA |
| T. C. Grozan | TVA |
| Altheia Wyche | SERCH Licensing/Bechtel |
| Jeannine Flonicker | Public |
| R. D. Cutsinger | TVA |
| James Adair | TVA |
| Phillip W. Harris | TVA |
| David Goxtchens | TVA |
| Walt Ellicott | TVA |
| Tom O'Reilly | NVS/LIS |
| Jose March-Leuba | ORNL |
| Faith Young | Citizens-Tennessee |
| Melvin W. Gmyrek | NEI |
| John Juliano | NVS |
| John O'Haira | BNL |
| Toshiyuki Zaman | TEPCO |

APPENDIX IV: FUTURE AGENDA

The Committee agreed to consider the following during the 427th ACRS Meeting, December 7-9, 1995:



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

December 5, 1995 (Revised)

SCHEDULE AND OUTLINE FOR DISCUSSION
427th ACRS MEETING
DECEMBER 7-8, 1995

Thursday, December 7, 1995, Conference Room 2B3, Two White Flint North, Rockville, Maryland

- 1) 8:30 - 8:45 A.M. Opening Remarks by the ACRS Chairman (Open)
 - 1.1) Opening Statement (TSK/SD)
 - 1.2) Items of Current Interest (TSK/JTL/SD)
 - 1.3) Priorities for Preparation of ACRS Reports (TSK/SD)

- 2) 8:45 - 10:15 A.M. Proposed Final Generic Letter on Testing of Safety-Related Logic Circuits (Open) (DWM/NFD/MTM)
 - 2.1) Remarks by the Subcommittee Chairman
 - 2.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed final Generic Letter on Testing of Safety-Related Logic Circuits.

Representatives of the industry will participate, as appropriate.

- 10:15 - 10:30 A.M. BREAK

- 3) 10:30 - 12:00 Noon Multiple System Responses Program (MSRP) (Open) (CJW/PAB)
 - 3.1) Remarks by the Subcommittee Chairman
 - 3.2) Briefing by and discussions with representatives of the NRC staff regarding the resolution of the MSRP issues.

- 12:00 - 1:30 P.M. LUNCH

- 4) 1:30 - 3:00 P.M. Meeting with the Director of the Office of Nuclear Reactor Regulation (NRR) (Open) (TSK/MME)
 - 4.1) Remarks by the ACRS Chairman
 - 4.2) Briefing by and discussions with Mr. Russell, NRR Director, on items of mutual interest, including the following:

- Risk/Performance-Based Regulations
- Risk-Based Inspection Program
- Activities of the Nuclear industry in support of the risk/performance-based regulations
- AP600 and SBWR review status
- Staff evaluation of the 1994 Addenda to the ASME Code Section III, for Class 1, 2, and 3 Piping Systems

5) 3:00 - 3:30 P.M.

Report of the Planning and Procedures Subcommittee (Open/Closed) (TSK/JTL)
Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS staff members.

[Note: A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this Advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.]

6) 3:30 - 4:00 P.M.

Future ACRS Activities (Open) (TSK/SD)
Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings.

7) 4:00 - 4:15 P.M.

Reconciliation of ACRS Comments and Recommendations (Open) (TSK, et.al./SD, et.al.)
Discussion of the responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports.

4:15 - 4:30 P.M.

BREAK

8) 4:30 - 6:45 P.M.

Preparation of ACRS Reports (Open)
Discussion of proposed ACRS reports on:
8.1) Resolution of Generic Safety Issue 78, "Monitoring of Fatigue Transient Limits for the Reactor Coolant System" (DAP/RLS/NFD)

- 8.2) Proposed Final Generic Letter on Testing of Safety-Related Logic Circuits (DWM/NFD/MTM)
- 8.3) Resolution of Multiple System Responses Program issues (CJW/PAB)

Friday, December 8, 1995, Conference Room 2B3, Two White Flint North, Rockville, Maryland

- 9) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (TSK/SD)
- 10) 8:35 - 9:15 A.M. Preparation of ACRS Reports (Open)
Discussion of proposed ACRS reports on:
10.1) Proposed Final Generic Letter on Testing of Safety-Related Logic Circuits (DWM/NFD/MTM)
10.2) Resolution of Multiple System Responses Program issues (CJW/PAB)
- 11) 9:15 - 9:45 A.M. Preparation for Meeting with the NRC Chairman (Open) (TSK/MME)
Discussion of topics for meeting with the NRC Chairman
- 9:45 - 10:00 A.M. BREAK
- 12) 10:00 - 11:00 A.M. Meeting with the NRC Chairman (Open) (TSK/MME)
Meeting with the NRC Chairman to discuss the Chairman's regulatory agenda and philosophy.
- 11:00 - 11:15 A.M. BREAK
- 13) 11:15 - 12:15 P.M. Preparation for Meeting with the NRC Commissioners (Open) (TSK, et.al/MME)
Preparation for meeting with the NRC Commissioners to discuss the following items:
- Resolution of Generic Safety Issue 78, Monitoring Fatigue Transient Limits for the Reactor Coolant System (DAP/RLS/NFD)
 - Rulemaking to amend 10 CFR 50.48, Fire Protection (IC/GA/NFD)

- Nondestructive Examination Techniques (RLS/NFD)
- NAS Study on Digital I&C (DWM/MTM)
- Regulatory Guide 1.152, Criteria for Digital Computers in Safety Systems of Nuclear Power Plants (DWM/MTM)
- Use of IPE in the Regulatory Process - Status Report (GA/MME)

12:15 - 1:15 P.M. LUNCH

- 14) 1:30 - 3:00 P.M. Meeting with the NRC Commissioners (Open)
(TSK, et.al/JTL, et.al)
Meeting with the NRC Commissioners in the Commissioners' Conference Room, One White Flint North, to discuss the matters identified under Item 13.
- 15) 3:15 - 3:45 P.M. Election of Officers for CY 1996 (Open)
(TSK/JTL/CAH)
Election of Chairman and Vice Chairman to the ACRS, and Member-at-Large for the Planning and Procedures Subcommittee.
- 16) 3:45 - 5:30 P.M. Preparation of ACRS Reports (Open)
Discussion of proposed ACRS reports on:
16.1) Proposed Final Generic Letter on Testing of Safety-Related Logic Circuits (DWM/NFD/MTM)
16.2) Resolution of Multiple System Responses Program issues (CJW/PAB)

- NOTE:**
- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
 - Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX V
LIST OF DOCUMENTS PROVIDED TO THE COMMITTEE

[Note: Some documents listed below may have been provided or prepared for Committee use only. These documents must be reviewed prior to release to the public.]

MEETING HANDOUTS

AGENDA

DOCUMENTS

ITEM NO.

1. Opening Remarks by the ACRS Chairman
 1. Introductory Statement of the ACRS Chairman, dated November 2, 1995.
 2. Letter dated October 5, 1995 to Chairman Jackson, NRC, from Admiral DeMars, U.S. Navy, concerning the sea trials of JOHN C. STENNIS, a nuclear powered aircraft carrier.
 3. List of Priority Items, dated November 2, 1995, distributed to Members for their comments.
2. Watts Bar Unit 1 Operations License Application
 4. Pages 19-1 and 19-2, Watts Bar SSER 4, dated March 1985: Report of the ACRS: Surveillance Requirements for ERCW Cement Mortar Lining [Handout]
 5. Watts Bar Nuclear Plant Unit 1, dated November 2, 1995, presented by Tennessee Valley Authority [Viewgraphs]
 6. Watts Bar Unit 1: Status of Licensing Actions, presented by Frederick J. Hebdon, NRR, undated [Viewgraph]
3. BWR Core Power Stability/ATWS
 7. Staff Review of Modifications to BWR Emergency Procedure Guidelines, presented by Larry Phillips, dated November 2, 1995 [Viewgraphs]
 8. BWR Owners Group EPGs for ATWS/Stability - Presentation Schedule, dated November 2, 1995 [Handout 3.1]
4. Advanced Control Room Design Review Guidelines
 9. Human-System Interface Design Review Guideline, Draft NUREG-0700, Revision 1, presented by Jerry Wachtel, dated November 2, 1995 [Viewgraphs]

5. Reliability of Systems

10. AEOD System Reliability Studies, presented by Patrick W. Baranowsky and Steven E. Mays, dated November 2, 1995 [Viewgraphs]
11. Memorandum from John C. Hoyle, Secretary, to James M. Taylor, Executive Director for Operations, SECY-95-215 - Proposed Rulemaking for Reporting Equipment Reliability Data, dated October 24, 1995 [Handout]
12. Memorandum from John C. Hoyle, Secretary, to Chairman Jackson, Commissioner Rogers, Staff Requirements Memorandum, dated October 17, 1995 [Handout]
13. Report from T.S. Kress, Chairman, ACRS, to Ivan Selin, Chairman, NRC, Proposed Rulemaking on Reporting Reliability and Availability Information for Risk-significant Systems and Equipment, dated April 12, 1995 [Handout]

7. Opening Remarks by the ACRS Chairman

14. Introductory Statement of the ACRS Chairman for November 3, 1995.

8. Proposed Final Regulatory Guide 1.164, "Time Response Design Criteria for Safety-Related Operator Actions"

15. Resolution of Generic Safety Issue B-17: Criteria for Safety-related Operator Actions, presented by Joel J. Kramer, RES, dated November 3, 1995. [Viewgraphs]

9. Report of the Planning and Procedures Subcommittee

16. Final Draft Minutes of Planning and Procedures Subcommittee Meeting - November 1, 1995 [Handout #9.1]

10. Future ACRS Activities

17. Future ACRS Activities - 427th ACRS Meeting, December 7-9, 1995 [Handout #10.1]

11. Reconciliation of ACRS Comments and Recommendations

18. Reconciliation of ACRS Comments and Recommendations [Handout #11.1]

MEETING NOTEBOOK CONTENTS

TAB

DOCUMENTS

2. Watts Bar Unit 1 Operations License Application
 1. Table of Contents
 2. Proposed Agenda
 3. Project Status Report, dated February 9, 1995
 4. Letter dated August 16, 1982, from Paul Shewmon, Chairman, ACRS, to Nunzio Palladino, Chairman, NRC: ACRS Report on Watts Bar Nuclear Plant, Unit 1 and 2
 5. Memorandum from Noel Dudley, ACRS staff, to ACRS members: Trip Report for the Ad Hoc Watts Bar Subcommittee Site Visit, dated October 12, 1995
 6. Memorandum from S. Schofer, Technical Secretary, ACRS, to ACRS members: Minutes for the 415th ACRS Meeting, November 3-4, 1994, dated December 9, 1994
 7. Minutes of the ACRS Subcommittee Meeting on Watts Bar Units 1 and 2, August 10, 1982, dated October 15, 1982
 8. Meeting Minutes of the ACRS Subcommittee Meeting on Watts Bar, Knoxville, Tennessee, dated April 30, 1982
3. BWR Core Power Stability/ATWS Systems
 8. Table of Contents
 9. Project Status Report
 10. Letter from Robert A. Pinelli, BWR Owners' Group, to Gary M. Holahan, NRR, dated September 15, 1995: Request for Comment on Draft Safety Evaluation of Proposed Emergency Procedure Guidelines - Boiling Water Reactor Owners' Group (BWROG) Response.
 11. Memorandum from Robert C. Jones, NRR, to Paul Boehnert, ACRS, dated October 12, 1995: October 1995 ACRS Meeting, attaching draft Safety Evaluation Report.
4. Advanced Control Room Design Review Guidelines
 12. Table of Contents
 13. Tentative Schedule
 14. Status Report
 15. Memorandum from M. Wayne Hodges, RES, to John T. Larkins, ACRS, dated October 2, 1995: ACRS Review of NUREG-0700, Revision 1.
8. Proposed Final Regulatory Guide 1.164, "Time Response Design Criteria for Safety-Related Operator Actions
 16. Table of Contents
 17. Agenda

18. Project Status Report
19. Memorandum from M. Wayne Hodges, RES, to John T. Larkins, ACRS, dated October 4, 1995: Time Response Design Criteria for Safety-Related Operator Actions.
20. Resolution to Public Comments on Draft Regulatory Guide DG-1040: Time Response Design Criteria for Safety-Related Operator Actions.
21. Proposed Final Regulatory Guide 1.164, "Time Response Design Criteria for Safety-Related Operator Actions.
22. Memorandum from M. Wayne Hodges, RES, to John T. Larkins, ACRS, dated June 14, 1995: Issuance of Draft Regulatory Guide DG-1040, For Comment Without Prior ACRS Review.
23. ANSI/ANS-58.8-1994, Time Response Design Criteria for Safety-Related Operator Actions.
24. Generic Safety Issue B-17: Criteria for Safety-Related Operator Actions.