

Survey Unit Release Record


Design #	EP-IT-14	Revision #	Original	Page 1 of 3
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Survey Unit #(s)	IT-14
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Description	<p>1) Embedded Pipe (EP) Survey Unit IT-14 meets the definition of embedded pipe for Plum Brook Reactor Facility (PBRF).</p> <p>2) EP IT-14 is a Class 1, Group 1 survey unit as per the PBRF Final Status Survey Plan (FSSP) and Technical Basis Document (TBD)-06-004.</p> <p>3) Surveys in EP IT-14 were performed using a scintillation detector optimized to measure gamma energies representative of Co-60. Sample #EP 3-9 from Survey Request (SR)-13 was referenced for this decision.</p> <p>4) Survey Instructions for this survey unit are incorporated into and performed in accordance with (IAW) the Babcock Services Incorporated (BSI)/LVS-002, Work Execution Package (WEP) 05-006. Survey instructions described in this document constitute "Special Methods" and the survey design used in the acquisition of survey measurements.</p> <p>5) Instrument efficiency determinations are developed in accordance with the BSI/LVS-002, WEP 05-006, these determinations are appropriate for the types of radiation involved and the media being surveyed.</p>
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Approval Signatures	Date:
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FSS/Characterization Engineer	<i>Oel R. Ball</i>	11-12-07
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Technical Reviewer (FSS/Characterization Engineer)		11-14-07
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FSS/Characterization Manager	 R. Case	11/15/07
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Survey Unit: IT-14

1.0 History/Description

- 1.1 The subject pipe system is a 4" diameter penetration located adjacent to the CRT plate within the Sub Pile Room. The system access point is located on the -34' el. of the Rx building.
- 1.2 EP IT-14 consists of 4" diameter piping that is approximately 3 feet in length.

2.0 Survey Design Information

- 2.1 EP IT-14 was surveyed IAW Procedure #BSI/LVS-002.
- 2.2 100% of the piping was accessible for survey. The accessible pipe was surveyed by static measurement at one foot increments, for a total of 3 survey measurements.
- 2.3 The total surface area for the piping system is approximately 2,919 cm² (0.3 m²) for the entire length of (3') of piping.

3.0 Survey Unit Measurement Locations/Data

- 3.1 Pipe interior radiological survey forms are provided in Attachment 2 of this release record.

4.0 Survey Unit Investigations/Results

- 4.1 None

5.0 Data Assessment Results

- 5.1 Data assessment results are provided in the EP/Buried Pipe (BP) Survey Report provided in Attachment 1.
- 5.2 All measurement results are less than the Derived Concentration Guideline Level (DCGL) for radionuclide specific EP that corresponds to the 1 mrem/yr dose goal established in Table 3-3 of the FSSP.
- 5.3 When implementing the Unity Rule, provided in Section 3.6.3 of the FSSP, and applying the Nuclide Fraction (NF), provided in TBD-06-004, the survey unit that is constituted by EP IT-14 passes FSS.
- 5.4 Background was not subtracted from the survey measurements and the Elevated Measurement Comparison (EMC) was not employed for this survey unit.

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5.5 Statistical Summary Table

Statistical Parameter	4" Pipe
Total Number of Survey Measurements	3
Number of Measurements >MDC	2
Number of Measurements Above 50% of DCGL	0
Number of Measurements Above DCGL	0
Mean	0.0579
Median	0.0290
Standard Deviation	0.0639
Maximum	0.1311
Minimum	0.0136

6.0 Documentation of evaluations pertaining to compliance with the unrestricted use limit of 25 mrem/yr and dose contributions from Embedded Pipe and radionuclides contributing 10% in aggregate of the total dose for both structural scenarios and soils.

6.1 A review of the survey results has shown that the dose contribution for EP IT-14 to be less than 1 mrem/yr. The dose contribution is estimated to be 0.058 mrem/yr based on the average of the actual gross counts measured.

7.0 Attachments

Attachment 1 – BSI EP/BP Survey Report

Attachment 2 – Pipe Interior Radiological Survey Form

Attachment 3 – DQA Worksheet

Attachment 4 –Disc containing RR for EP IT-14 & Spreadsheet

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ATTACHMENT 1
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BSI EP/BP SURVEY REPORT

Pipe ID	EP IT-14	Survey Location	Sub Pile Room Pen. -34 el.
Survey Date	24-Oct-07	2350-1 #	189094
Survey Time	16:12	Detector-Sled #	1MG1 LVS-1/ 101
Pipe Size	4"	Detector Efficiency	0.00036
DCGL (dpm/100cm ²)	2.41E+05	Pipe Area Incorporated by Detector Efficiency (in cm ²)	973
Pipe Area Incorporated by Survey Data (m ²)	0.3	Field BKG (cpm)	3.4
Routine Survey	X	Field MDCR (cpm)	10
QA Survey		Nominal MDC (dpm/100cm ²)	3,059
Survey Measurement Results			
Total Number of Survey Measurements		3	
Number of Measurements >MDC		2	
Number of Measurements Above 50% DCGL		0	
Number of Measurements Above DCGL		0	
Mean		0.0579	
Median		0.0290	
Standard Deviation		0.0639	
Maximum		0.1311	
Minimum		0.0136	
Survey Technician(s)		FOWLER	
Survey Unit Classification		1	
TBD 06-004 Piping Group		1	
SR-13 Radionuclide Distribution Sample		EP 3-9	
Measured Nuclide		Co-60	
Area Factor/EMC Used		No	
Pass/Fail FSS		Pass	
MREM/YR Contribution		<1	
COMMENTS: ACTIVITY VALUES NOT BACKGROUND CORRECTED			
RP Engineer Date		Dal Rentsall 11-12-07	

EP IT-14
4" Pipe
TBD 06-004 Group 1

Measurement #	gcpm	ncpm	Co-60 activity (total dpm)	Co-60 activity (dpm/100cm2)	Cs-137 activity (dpm/100cm2)	Eu-152 activity (dpm/100cm2)	Eu-154 activity (dpm/100cm2)	Nb-94 activity (dpm/100cm2)	Ag-108m activity (dpm/100cm2)	Unity
1	8	8	22,222	2,284	91	2,167	576	67	16	0.014
2	17	17	47,222	4,854	192	4,604	1,224	142	34	0.029
3	77	77	213,889	21,985	871	20,855	5,544	642	153	0.131
									MEAN	0.058
									MEDIAN	0.029
									STD DEV	0.064
									MAX	0.131
									MIN	0.014

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Pipe Interior Radiological Survey Form

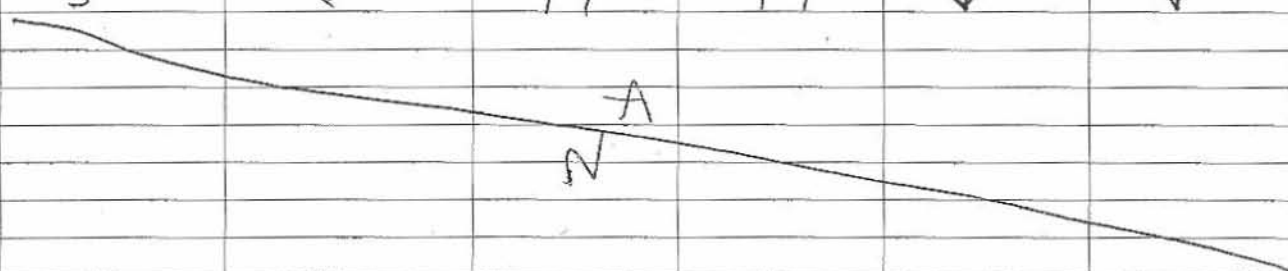
Date: 10-24-07 Time: 1617
 Pipe ID#: IT-14 Pipe Diameter: 4" Access Point Area: Sub Pk Rm
 Building: CV Elevation: -34' System: Penetration
 Type of Survey Investigation _____ Characterization _____ Final Survey X Other ✓
 Gross _____ Co60 ✓ Cs _____
 Detector ID# / Sled ID# 1m61 / LVS-1 / Sled # 101
 Detector Cal Date: 1-11-07 Detector Cal Due Date: 1-11-08
 Instrument: 2350-1 Instrument ID #: 189094
 Instrument Cal Date: 1-11-07 Instrument Cal Due Date: 1-11-08

From the Daily Pipe Survey Detector Control Form for the Selected Detector

Background Value 3.4 cpm
 MDCR_{static} 10 cpm
 Efficiency Factor for Pipe Diameter 0.00036 (from detector efficiency determination)
 MDC_{static} 3059 dpm/ 100 cm²
 Is the MDC_{static} acceptable? (Yes) No (if no, adjust sample count time and recalculate MDCR_{static})
 Comments: Post Decon 100% Complete

Technician Signature R. F. [Signature]

Pipe Interior Radiological Survey

Position #	Feet into Pipe from Opening	Count Time (min)	Gross Counts	Gross cpm	Net cpm	dpm/100cm ²
1	1	1	8	8	n/a	n/a
2	2	↓	17	17	↓	↓
3	3	↓	77	77	↓	↓
4						
5						
6						
7						
8						
9						
10						

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North
↓

IT-12

CRT Plate
↓

5	10
4	9
3	8
2	7
1	6

IT-14

IT-13

IT-11

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DQA Check Sheet

Design #	EP IT-14	Revision #	Original			
Survey Unit #	EP IT-14					
Preliminary Data Review						
Answers to the following questions should be fully documented in the Survey Unit Release Record				Yes	No	N/A
1. Have surveys been performed in accordance with survey instructions in the Survey Design?				X		
2. Is the instrumentation MDC for structure static measurements below the DCGL _W for Class 1 and 2 survey units, or below 0.5 DCGL _W for Class 3 survey units?						X
3. Is the instrumentation MDC for embedded/buried piping static measurements below the DCGL _W ?				X		
4. Was the instrumentation MDC for structure scan measurements, soil scan measurements, and embedded/buried piping scan measurements below the DCGL _W , or, if not, was the need for additional static measurements or soil samples addressed in the survey design?						X
5. Was the instrumentation MDC for volumetric measurements and smear analysis < 10% DCGL _W ?						X
6. Were the MDCs and assumptions used to develop them appropriate for the instruments and techniques used to perform the survey?				X		
7. Were the survey methods used to collect data proper for the types of radiation involved and for the media being surveyed?				X		
8. Were "Special Methods" for data collection properly applied for the survey unit under review?				X		
9. Is the data set comprised of qualified measurement results collected in accordance with the survey design, which accurately reflects the radiological status of the facility?				x		
Graphical Data Review						
1. Has a posting plot been created?						X
2. Has a histogram (or other frequency plot) been created?						X
3. Have other graphical data tools been created to assist in analyzing the data?						X
Data Analysis						
1. Are all sample measurements below the DCGL _W (Class 1 & 2), or 0.5 DCGL _W (Class 3)?				X		
2. Is the mean of the sample data < DCGL _W ?				X		
3. If elevated areas have been identified by scans and/or sampling, is the average activity in each elevated area < DCGL _{EMC} (Class 1), < DCGL _W (Class 2), or < 0.5 DCGL _W (Class 3)?						X
4. Is the result of the Elevated Measurements Test < 1.0?						X
5. Is the result of the statistical test (<i>S</i> + for Sign Test or <i>W</i> _r for WRS Test) ≥ the critical value?						X
Comments:						
FSS/Characterization Engineer (print/sign)				<i>Dele R. Casper</i>		Date 11-12-07
FSS/ Characterization Manager (print/sign)				<i>R. Casper</i>		Date 11/15/07

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