



Entergy Nuclear Northeast  
Entergy Nuclear Operations, Inc.  
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Pete Dietrich  
Site Vice President - JAF

November 16, 2007  
JAFP-07-0130

ATTN: Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

REFERENCE: 1. Letter, Entergy to USNRC, "James A. FitzPatrick Nuclear Power Plant, Docket No. 50-333, License No. DPR-59, License Renewal Application," JAFP-06-0109, dated July 31, 2006

SUBJECT: **Entergy Nuclear Operations, Inc.  
James A. FitzPatrick Nuclear Power Plant  
Docket No. 50-333, License No. DPR-59  
License Renewal Application, Amendment 15**

Dear Sir or Madam:

On July 31, 2006, Entergy Nuclear Operations, Inc. submitted the License Renewal Application (LRA) for the James A. FitzPatrick Nuclear Power Plant (JAFNPP) as indicated by Reference 1.

During the NRC review of an LRA, 10 CFR54.21 (b) specifies that an applicant report changes to the current licensing basis (CLB) that materially affect the content of the LRA, including the Updated Final Safety Analysis Report (UFSAR) supplement. In accordance with 10 CFR54.21 (b), Entergy has completed a review and determined that the changes identified in Attachment 1 materially affect the content of the JAFNPP LRA.

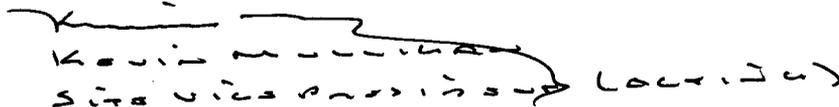
Attachment 1 deletes reference to the RHR steam condensing mode. As a result of a series of plant design changes and procedure revisions performed in accordance with 10 CFR 50.59, the steam condensing mode for the RHR system is no longer available under any operating condition. This change affected only the system descriptions in Section 2.3.2 of the LRA as shown in Attachment 1. The change has no effect on the scoping and screening or aging management review results provided in the LRA and subsequent amendments.

Should you have any questions concerning this submittal, please contact Mr. Jim Costedio at 315-349-6358

A124  
NRR

I declare under penalty of perjury that the foregoing is true and correct. Executed on the 16<sup>th</sup> day of November, 2007.

Sincerely,

A handwritten signature in black ink, appearing to read "Pete Dietrich", with a long horizontal line extending to the right.

PETE DIETRICH  
SITE VICE PRESIDENT

PD/cf

Attachment

cc:

Mr. N.B. (Tommy) Le, Senior Project Manager  
License Renewal Branch B  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Mail Stop O-11-F1  
Washington, DC 20555

Mr. Paul Eddy  
New York State Department of Public Service  
3 Empire State Plaza, 10<sup>th</sup> Floor  
Albany, NY 12223

Mr. Samuel J. Collins, Administrator  
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U. S. Nuclear Regulatory Commission  
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Mr. Paul Tonko, President  
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NRC Resident Inspector  
U. S. Nuclear Regulatory Commission  
James A. FitzPatrick Nuclear Power Plant  
P.O. Box 136  
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Mr. John P. Boska, Project Manager  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Mail Stop O-8-C2  
Washington, DC 20555

**JAFP-07-0130  
Attachment 1**

**2007 LRA Annual Update Review Results  
James A. FitzPatrick Nuclear Power Plant**

**Attachment 1**  
**2007 LRA Annual Update Review Results**  
**James A. FitzPatrick Nuclear Power Plant**

**Page 2.3-21**

Revise the fourth paragraph of Section 2.3.2.1 to read:

The RHR system is a two-loop system containing two heat exchangers ~~with steam condensing capability~~ and four main system pumps. The loops are physically separated from each other. A single header cross connects the two loops, making it possible to supply either loop from the pumps in the other loop. The system discharge piping is kept in a filled condition by its keep-full pumps to minimize time delay in LPCI actuation and to avoid water hammer on pump starts.

**Page 2.3-22**

Delete the entire second subsection, Steam Condensing, of Section 2.3.2.1:

~~*Steam Condensing*~~

~~The RHR system in the steam condensing mode may be operated in conjunction with the reactor core isolation cooling (RCIC) system in case of a loss of the main condenser as directed by emergency operating procedures. During reactor isolation, reactor steam may be relieved to the suppression chamber, via the relief valves, where it is condensed and subcooled. Decay heat is transferred to the RHR service water using the RHR heat exchangers as direct steam condensers.~~

**Page 2.3-31**

Revise the second paragraph of Section 2.3.2.5 to read:

The RCIC system consists of a steam-driven turbine-pump unit and associated valves and piping capable of delivering make-up water to the reactor vessel. RCIC normally takes suction from the demineralized water in the condensate storage tanks. A back-up supply is available from the suppression pool. ~~RCIC also connects to the RHR system, enabling RCIC alignment to the RHR system when the RHR system operates in the steam condensing mode.~~ RCIC injection to the vessel occurs through the feedwater line. RCIC shares suction points and full flow test lines with HPCI.